

Diablo Canyon Power Plant P.O. Box 56 Avila Beach, CA 93424 805/545-6000 Robert P. Powers Vice President–Diablo Canyon Operations and Plant Manager

February 18, 1997



PG&E Letter DCL-97-020

U.S. Nuclear Regulatory Commission ATTN: Document Control Desk Washington, D.C. 20555

Docket No. 50-275, DPR-80 Docket No. 50-323, DPR-82 Diablo Canyon Units 1 and 2 <u>Monthly Operating Report for January 1997</u>

Dear Commissioners and Staff:

Enclosed are the monthly operating report forms for Diablo Canyon Power Plant, Units 1 and 2, for January 1997. This report is submitted in accordance with Section 6.9.1.7 of the Units 1 and 2 Technical Specifications.

Sincerely,

Robert P. Powers

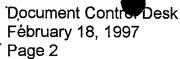
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> Mr. James E. Dyer, Acting Regional Administrator U.S. Nuclear Regulatory Commission, Region IV 611 Ryan Plaza Drive, Suite 400 Arlington, TX 76011-8064

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INPO Record Center 700 Galleria Parkway Atlanta, GA 30339-5957 Document Control Desk February 18, 1997 Page 3

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R. D. Glynn G. M. Rueger NRC Resident

77/B32 77/B14A 104/5/538

#### MONTHLY NARRATIVE REPORT OF OPERATION AND MAJOR MAINTENANCE EXPERIENCE

The monthly report describes the operating and major maintenance experience for Diablo Canyon Power Plant for the month of January 1997. This narrative report was prepared and submitted in accordance with Section 6.9.1.7 of the Units 1 and 2 Technical Specifications.

## Narrative of Daily Significant Plant Events

January 1, 1997:	Units 1 and 2 started the month in Mode 1 (Power Operation) at 100 percent power.
January 3, 1997:	An Unusual Event (UE) was declared at 1115 PST, due to a mudslide on the public access road just outside the main gate. The mudslide blocked normal access to and from the plant. The alternate access to and from the plant was verified available through the dirt road north of the site. The UE was terminated at 1430 PST after county work crews had sufficiently removed the mudslide to allow access to and from the plant through the normal route.
January 10, 1997:	At approximately 1800 PST, a 24-hour nonemergency report was made to the NRC in accordance with Section 4.1 of the Environmental Protection Plan. At approximately 1000 PST, biologists had observed a green seaturtle between the seawall and the 1-2 bar rack at the seawater Intake Structure. The turtle was removed by divers, examined by biologists, and released 1/4 mile outside the intake cove.
January 12, 1997:	Units 1 and 2 reduced power to 80 percent for modification to the discharge piping on two Unit 1 and one Unit 2 main steam safety valves (MSSVs). (For details, see Summary of Significant Safety-Related Maintenance.) Units 1 and 2 were returned to 100 percent power.
January 31, 1997:	At 1220 PST, a 1-hour, nonemergency report was made to the NRC in accordance with 10 CFR 50.72(b)(1)(ii)(B). PG&E determined that subsequent to a loss-of-coolant accident, an outside design basis condition could occur when separating cooling water trains in accordance with Emergency Operating Procedure (EOP) E-1.4, "Transfer to Hot Leg Recirculation," concurrent with loss of electrical power to a vital bus. As a conservative measure during the evaluation of this issue, EOP E-1.4 was revised to no longer require immediate train separation after the transfer to hot leg recirculation. The



decision to separate was transferred to the Technical Support Center, where it will be decided after an evaluation of plant conditions. This action precludes the postulated event from occurring. Licensee Event Report 1-97-001 will be submitted by March 3, 1997, to provide additional information on this condition.

January 31, 1997:

Units 1 and 2 ended the month in Mode 1 at 100 percent power.

## Summary of Plant Operating Characteristics, Power Reductions, and Unit Shutdowns

Unit 1 operated this month with a unit availability factor of 100 percent and a unit capacity factor (using MDC Net) of 100.43 percent. Unit 1 reduced power once this month to 80 percent for modification to the discharge piping of two MSSVs.

Unit 2 operated this month with a unit availability factor of 100 percent and a unit capacity factor (using MDC Net) of 100.64 percent. Unit 2 reduced power once this month to 80 percent for modification to the discharge piping of a MSSV.

#### Summary of Significant Safety-Related Maintenance

Units 1 and 2:

Modifications were made to the discharge piping on two Unit 1 and one Unit 2 MSSVs. Material was removed from the drip pans, welded to the outside diameter of the discharge piping, to provide clearance between the drip pans and the vent stacks. The added clearances prevent excessive load transfer from the discharge stack to the safety valve discharge piping and ultimately to the safety valve during differential movement as a result of seismic events, thermal changes, and jet reaction during valve actuation. Excessive loading of the safety valve discharge piping results in valve body distortion and could have an adverse impact on valve performance.

# Actuation of Steam Generator Safety or Pressurizer Power Operated Relief Valves

There were no challenges to the steam generator safety or pressurizer power operated relief valves.

#### **OPERATING DATA REPORT**

DOCKET NO.:	50-275
UNIT:	1
DATE:	02/18/97
COMPLETED BY:	T. Eubank/W. E. Coley
TELEPHONE:	(805) 545-4867/4741

## **OPERATING STATUS**

1.	Unit Name:	Diablo Canyon Power Plant Unit 1
2.	Reporting Period:	January 1997
З.	Licensed Thermal Power (MWt):	3338
4.	Nameplate Rating (Gross MWe):	1137
5.	Design Electrical Rating (Net MWe):	1086
6.	Maximum Dependable Capacity (Gross MWe):	1124
7.	Maximum Dependable Capacity (Net MWe):	1073.4
8.	If Changes Occur In Capacity Ratings (Items	
	3 Through 7) Since Last Report, Give Reasons:	N/A
9.	Power Level To Which Restricted, If Any (Net MWe):	N/A
10.	Reasons For Restrictions, If Any:	N/A

	This Month	<u>YTD</u>	<u>Cumulative</u>
11. Hours In Reporting Period:	744.0	744.0	102910.3
12. Number Of Hours Reactor Was Critical:	744.0	744.0	87642.3
13. Reactor Reserve Shutdown Hours:	0.0	0.0	0.0
14. Hours Generator On-Line:	744.0	744.0	86572.9
15. Unit Reserve Shutdown Hours:	0.0	0.0	0.0
16. Gross Thermal Energy Generated (MWH):	2391047	2391047	275811828
17. Gross Electrical Energy Generated (MWH):	840700	840700	92936032
18. Net Electrical Energy Generated (MWH):	802031	802031	88222288
19. Unit Service Factor:	100.00	100.00	84.12
20. Unit Availability Factor:	100.00	100.00	84.12
21. Unit Capacity Factor (Using MDC Net):	100.43	100.43	79.87
22. Unit Capacity Factor (Using DER Net):	99.26	99.26	78.94
23. Unit Forced Outage Rate:	0.00	0.00	3.64
24. Shutdowns Scheduled Over Next 6 Months			

(Type Date & Duration of Each): Fighth refueling outage (1P8)

(Type, Date, & Duration of Each): Eighth refueling outage (1R8), April 19, 1997, scheduled 36 days.

25. If Shut Down At End Of Report Period, Estimate Date of Startup:

N/A

# AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO .: 50-275 UNIT: 1 DATE: 02/18/97 COMPLETED BY: (805) 545-4867/4741 TELEPHONE:

T. Eubank/W. E. Coley

January 1997

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DAY

## AVERAGE DAILY POWER LEVEL (MWe-Net)

1	1081
2 3	1085
3	1081
4	1082
5	1086
6	1082
7	1081
8	1085
9	1081
10	1086
11	1086
12	919
13	1081
14	1080
15	1081
16	1085
17	1086
18	1086
19	1081
20	1086
21	1081
22	1087
23	1081
24	1082
25	1086
26	1086
27	1082
28	1081
29	1086
30	1081
31	1086

The average monthly electrical power level for January 1997 = 1078.00 MWe-Net

# UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO .:	50-275
UNIT:	1
DATE:	02/18/97
COMPLETED BY:	W. E. Coley
TELEPHONE:	(805) 545-4741

REPORT MONTH: January 1997

NO.	DATE	TYPE <sup>1</sup>	DURATION (HOURS)	REASON <sup>2</sup>	METHOD OF SHUTDOWN <sup>3</sup>	LICENSEE EVENT REPORT	SYSTEM CODE <sup>4</sup>	COMPONENT CODE⁵	CAUSE & CORRECTIVE ACTION TO PREVENT RECURRENCE
1	01/12/97	S	N/A	Н	5	N/A	SB	RV	Reduced power to modify discharge piping for two MSSVs. See Summary of Significant Safety- Related Maintenance for description of action taken.

1	2	3	4
Туре:	Reason:	Method:	Elis Sy
F-Forced	A-Equipment Failure (Explain)	1-Manual	-
S-Scheduled	B-Maintenance or Test	2-Manual Scram	
	C-Refueling	3-Automatic Scram	
	D-Regulatory Restriction E-Operator Training & License Examination	4-Continuation from previous month	
	F- Administrative	5-Power reduction	
	G-Operational Error (Explain) H-Other (Explain)	6-Other	

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IIS Systems List, Table 1

5 IEEE Std. 803A-1983, "IEEE Recommended Practice for Unique Identification in Power Plants and Related Facilities - Table 2"

# **OPERATING DATA REPORT**

 DOCKET NO.:
 50-323

 UNIT:
 2

 DATE:
 02/18/97

 COMPLETED BY:
 T. Eubank/W. E. Coley

 TELEPHONE:
 (805) 545-4867/4741

# **OPERATING STATUS**

1.	Unit Name:	Diablo Canyon Power Plant Unit 2
2.	Reporting Period:	January 1997
З.	Licensed Thermal Power (MWt):	3411
4.	Nameplate Rating (Gross MWe):	1164
5.	Design Electrical Rating (Net MWe):	1119
6.	Maximum Dependable Capacity (Gross MWe):	1137
7.	Maximum Dependable Capacity (Net MWe):	1087
8.	If Changes Occur In Capacity Ratings (Items	
	3 Through 7) Since Last Report, Give Reasons:	N/A
9.	Power Level To Which Restricted, If Any (Net MWe):	N/A
10.	Reasons For Restrictions, If Any:	N/A

	This Month	<u>YTD</u>	<u>Cumulative</u>
11. Hours In Reporting Period:	744.0	744.0	95469.0
12. Number Of Hours Reactor Was Critical:	744.0	744.0	82606.7
13. Reactor Reserve Shutdown Hours:	0.0	0.0	0.0
14. Hours Generator On-Line:	744.0	744.0	81406.3
15. Unit Reserve Shutdown Hours:	0.0	0.0	0.0
16. Gross Thermal Energy Generated (MW	H): 2531071	2531071	268055603
17. Gross Electrical Energy Generated (MV	VH): 850610	850610	89226442
18. Net Electrical Energy Generated (MWH	): 813866	813866	84885675
19. Unit Service Factor:	100.00	100.00	85.27
20. Unit Availability Factor:	100.00	100.00	85.27
21. Unit Capacity Factor (Using MDC Net):	100.64	100.64	81.90
22. Unit Capacity Factor (Using DER Net):	97.76	97.76	79.46
23. Unit Forced Outage Rate:	0.00	0.00	4.00
24. Shutdowns Scheduled Over Next 6 Mor	4. Shutdowns Scheduled Over Next 6 Months		
(Type, Date, And Duration Of Each):		N/A	
25. If Shut Down At End Of Report Period, I	N/A		

# AVERAGE DAILY UNIT POWER LEVEL

 DOCKET NO.:
 50-323

 UNIT:
 2

 DATE:
 02/18/97

 COMPLETED BY:
 T. Eubank/W. E. Coley

 TELEPHONE:
 (805) 545-4867/4997

January 1997	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
	1	1097
	2 3	1096
		1098
	4	1097
	5	1099
	6	1098
	7	1094
	8	1101
	9	1098
	10	1098
	11	1099
	12	1014
	13	1086
	14	1087
	15	1097
	16	1096
	17	1098
	18	1098
	19	1096
	20	1099
	21	1097
	22	1098
	23	1096
	24	1098
	25	1096
	26	1097
	27	1096
	28	1096
	29	1097
	30	1096
	31	1096

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The average monthly electrical power level for January 1997 = 1093.91 MWe-Net

### UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO .:	50-323
UNIT:	2
DATE:	02/18/97
COMPLETED BY:	W. E. Coley
TELEPHONE:	(805) 545-4741

**REPORT MONTH: January 1997** 

NO.	DATE	TYPE <sup>1</sup>	DURATION (HOURS)	REASON <sup>2</sup>	METHOD OF SHUTDOWN <sup>3</sup>	LICENSEE EVENT REPORT	SYSTEM CODE <sup>4</sup>	COMPONENT CODE⁵	CAUSE & CORRECTIVE ACTION TO PREVENT RECURRENCE
1	01/12/97	S	N/A	Η	5	N/A	SB	RV	Reduced power to modify discharge piping for one MSSV. See Summary of Significant Safety- Related Maintenance for description of action taken.

123Type:Reason:MeF-ForcedA-Equipment Failure (Explain)1-MS-ScheduledB-Maintenance or Test2-MC-Refueling3-AD-Regulatory Restriction4-CE-Operator Training & License ExaminationpF- Administrative5-PG-Operational Error (Explain)6-CH-Other (Explain)6-C

Method: 1-Manual 2-Manual Scram 3-Automatic Scram 4-Continuation from previous month 5-Power reduction 6-Other 4 EIIS Systems List, Table 1 5

IEEE Std. 803A-1983, "IEEE Recommended Practice for Unique Identification in Power Plants and Related Facilities - Table 2"

#### **REFUELING INFORMATION REQUEST**

 DOCKET NO.:
 50-275

 UNIT:
 1

 DATE:
 02/18/97

 COMPLETED BY:
 D. Farrer/W. E. Coley

 TELEPHONE:
 (805) 545-4438/4741

- 1. Name of facility: Diablo Canyon Power Plant Unit 1
- 2. Scheduled date for next refueling shutdown: April 19, 1997
- 3. Scheduled date for restart following refueling: May 24, 1997
- 4. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment? If answer is yes, what, in general, will there be? If answer is no, has the reload fuel design and core configuration been reviewed by your Plant Staff Review Committee (PSRC) to determine whether any unreviewed safety questions are associated with the core reload (Ref. 10 CFR Section 50.59)? If no such review has taken place, when is it scheduled?

No. The PSRC review should occur in April or May of 1997 before reloading the core.

- 5. Scheduled date(s) for submitting proposed licensing action and supporting information: N/A
- 6. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures: N/A
- 7. As of January 31, 1997, the number of fuel assemblies (a) in the core and (b) in the spent fuel storage pool were:
  - (a) 193 (b) 548
- 8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned, in number of fuel assemblies:

Present: 1324 Increase size by: 0

9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity: 2006 (loss of full core offload capability).

#### **REFUELING INFORMATION REQUEST**

 DOCKET NO.:
 50-323

 UNIT:
 2

 DATE:
 02/18/97

 COMPLETED BY:
 D. Farrer/W. E. Coley

 TELEPHONE:
 (805) 545-4438/4741

1. Name of facility: Diablo Canyon Power Plant Unit 2

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- 2. Scheduled date for next refueling shutdown: February 7, 1998
- 3. Scheduled date for restart following refueling: March 14, 1998
- 4. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment? If answer is yes, what, in general, will there be? If answer is no, has the reload fuel design and core configuration been reviewed by your Plant Staff Review Committee (PSRC) to determine whether any unreviewed safety questions are associated with the core reload (Ref. 10 CFR Section 50.59)? If no such review has taken place, when is it scheduled?

No. The PSRC review should occur in January or February of 1998 before reloading the core.

- 5. Scheduled date(s) for submitting proposed licensing action and supporting information: N/A
- 6. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures: N/A
- 7. As of January 31, 1997, the number of fuel assemblies (a) in the core and (b) in the spent fuel storage pool were:
  - (a) 193 (b) 580
- 8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned, in number of fuel assemblies:

Present: 1324 Increase size by: 0

9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming he present licensed capacity: 2006 (loss of full core offload capability).