PSEG

Public Service Electric and Gas Company P.O. Box 236 Hancocks Bridge, New Jersey 08038-0236

Nuclear Business Unit

NOV 17 1997

LR-N970753

U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555

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MONTHLY OPERATING REPORT
SALEM UNIT NO. 2
DOCKET NO. 50-311

In compliance with Section 6.9.1.6, Reporting Requirements for the Salem Technical Specifications, the original Monthly Operating Report for October, 1997, is attached. Included with this report is a table containing revised Net Electrical Energy Generated data. This table corrects 1997 data that was provided in previous reports.

Sincerely yours,

A. C. Bakken III
General Manager -
Salem Operations

RBK:tcp
Enclosures

C Mr. H. J. Miller
Regional Administrator USNRC, Region 1
475 Allendale Road
King of Prussia, PA 19046

SALEM GENERATING STATION DOCKET NO.: 50-311
UNIT: Salem 2
DATE: 11/15/97
COMPLETED BY: R. Knieriem
TELEPHONE: (609) 339-1782

MONTHLY OPERATING SUMMARY - UNIT 2
OCTOBER 1997

Salem Unit 2 began the month of October operating at full power. On October 2, at 0713, the unit was manually tripped following the loss of both operating Steam Generator Feed Pumps. The cause of the loss of feedwater was isolated to the failure of a temporary data acquisition device connected to monitor feedwater control system performance. This event was reported in Licensee Event Report 311/97-014-00 on 10/31/97.

Unit 2 returned to service on October 6, and remained in service for the remainder of the month.

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UNIT: Salem 2
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OPERATING DATA REPORT
OPERATING STATUS

1	Reporting Period OCTOBER 1997	Hours in Report Period	745
2	Currently Authorized Power Level (MWt)		3411
	Max Dependable Capacity (MWe-Net)		1106
	Design Electrical Rating (MWe-Net)		1115
3	Power level to which restricted (if any) (MWe Net)		None
4	Reason For Restriction (if any)		

		This Month	Yr To Date	Cumulative
5	No. of hours reactor was critical	656	1659	79743
6	Reactor reserve shutdown hours	0.0	0.0	0.0
7	Hours generator on line	641	1405	76635
8	Unit reserve shutdown hours	0.0	0.0	0.0
9	Gross thermal energy generated (MWH)	1853218	3465765	191246770
10	Gross electrical energy generated (MWH)	599322	1096333	79744931
11	Net electrical energy generated (MWH)	567312	934469	75637103
12	Unit Service Factor	86.0%	19.3%	48.7%
13	Unit Availability Factor	86.0%	19.3%	48.7%
14	Unit Capacity Factor (MDC)	68.9%	11.6%	43.5%
15	Unit Capacity Factor (DER)	68.3%	11.5%	43.1%
16	Unit Forced Outage Rate	14.0%	80.7%	33.8%

17 Shutdowns scheduled over next 6 months (type, date, duration):

18 If shutdown at end of report period, estimated date of Startup:

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OPERATING DATA REPORT
UNIT SHUTDOWNS AND POWER REDUCTIONS

MONTH OCTOBER 1997

NO.	DATE	TYPE F=FORCED S=SCHEDULED	DURATION (HOURS)	REASON (1)	METHOD OF SHUTTING DOWN THE REACTOR OR REDUCING POWER (2)	CORRECTIVE ACTION/COMMENT
3408	10/2 - 10/6	F	104.2	A - Feed Pump Trip	2	Manual trip following the loss of both feed pumps. Loss of feed caused by failed temporary data acquisition device. LER 311/97-014-00, 10/31/97

(1) Reason

A - Equipment Failure (Explain)
B - Maintenance or Test
C - Refueling
D - Regulatory Restriction
E - Operator Training/License Examination
F - Administrative
G - Operational Error (Explain)
H - Other

(2) Method

1 - Manual
2 - Manual Trip
3 - Automatic Trip/Scram
4 - Continuation
5 - Other (Explain)

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AVERAGE DAILY UNIT POWER LEVEL

MONTH OCTOBER 1997

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	688	17	1083
2	294	18	1088
3	0	19	1092
4	0	20	1094
5	0	21	1101
6	10	22	1101
7	212	23	1101
8	319	24	1101
9	325	25	1102
10	317	26	1103
11	305	27	1094
12	599	28	1101
13	1027	29	1103
14	1071	30	1104
15	1079	31	1103
16	1087		

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OPERATING DATA REPORT

Revised Net Electrical Energy Generated

Listed below are the monthly, year-to-date and cumulative figures for the 1997 Net Electrical Energy Generated:

	MONTH	YEAR-TO-DATE	CUMULATIVE
January	-6303	-6303	74696331
February	-9429	-15732	74686902
March	-7465	-23197	74679437
April	-7074	-30271	74672363
May	-7624	-37895	74664739
June	-13045	-50940	74651694
July	-22587	-73527	74629107
August	-17799	-91326	74611308
September	458483	367157	75069791

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SUMMARY OF CHANGES, TESTS, AND EXPERIMENTS FOR THE SALEM UNIT 2 GENERATING STATION

MONTH OCTOBER 1997

The following items completed during **October 1997** have been evaluated to determine:

1. If the probability of occurrence or the consequences of an accident or malfunction of equipment important to safety previously evaluated in the safety analysis report may be increased; or
2. If a possibility for an accident or malfunction of a different type than any evaluated previously in the safety analysis report may be created; or
3. If the margin of safety as defined in the basis for any technical specification is reduced.

The 10CFR50.59 Safety Evaluations showed that these items did not create a new safety hazard to the plant nor did they affect the safe shutdown of the reactor. These items did not change the plant effluent releases and did not alter the existing environmental impact. The 10CFR50.59 Safety Evaluations determined that no unreviewed safety or environmental questions are involved.

Design Changes Summary of Safety Evaluations

2EC-3178, Pkg. 1, Analog Feedwater Control System Replacement

This modification replaced the existing analog feedwater control system with an Advanced Digital Feedwater Control System (ADFCS). The ADFCS was installed to provide operators with a more reliable design that will reduce the number of plant trips due to feedwater control system malfunctions.

This design change does not negatively impact any accident response. This design change does not increase the probability or consequences of either an accident or a malfunction of equipment important to safety. Therefore, this design change does not involve an Unreviewed Safety Question.

2EC-3336, Pkg. 1, Feed Pump Recirculation System Upgrade

This design change modified the feed pump recirculation piping and enhanced feed pump recirculation control. This was done to minimize erosion in feed pump recirculation piping and valves; and to reduce feedwater system startup flow transients.

This design change does not negatively impact any accident response. This design change does not increase the probability or consequences of either an accident or a malfunction of equipment important to safety. Therefore, this design change does not involve an Unreviewed Safety Question.

2EC-3353, Pkg. 1, Replacement of Low Pressure Turbine Rotors With A Fully Integral Design

This design change involved the replacement of the existing Low Pressure Turbine rotors with new Mono-block Design rotors.

This design change does not negatively impact any accident response. This design change does not increase the probability or consequences of either an accident or a malfunction of equipment important to safety. Therefore, this design change does not involve an Unreviewed Safety Question.

2EC-3449, Pkg. 1, Steam Generator Tube leak Detection Main Steam Line N^{16} Monitors

This design change installed N^{16} radiation monitors that will provide operations personnel with enhanced ability to identify and respond to a Steam Generator tube leak.

This design change does not negatively impact any accident response. This design change does not increase the probability or consequences of either an accident or a malfunction of equipment important to safety. Therefore, this design change does not involve an Unreviewed Safety Question.

2EC-3554, Pkg. 1, Heater Drain Pump Start and Heater Drain Valve Open Interlock Modification

This design change modified the controls for the Bleed Steam Heater Drain pump discharge valve to prevent the Feedwater Heater and Moisture Separator Reheater Drain Tanks from draining down too rapidly during Bleed Steam Heater Drain pump startup.

This design change does not negatively impact any accident response. This design change does not increase the

probability or consequences of either an accident or a malfunction of equipment important to safety. Therefore, this design change does not involve an Unreviewed Safety Question.

2EC-3264, Pkg. 1, Steam Generator Level Sensing Line Modification

This design change modified the Steam Generator narrow and wide range instrumentation sensing lines to ensure that allowable stress limits are not exceeded.

This design change does not negatively impact any accident response. This design change does not increase the probability or consequences of either an accident or a malfunction of equipment important to safety. Therefore, this design change does not involve an Unreviewed Safety Question.

Temporary Modifications Summary of Safety Evaluations

There were no changes in this category implemented during October, 1997.

Procedures Summary of Safety Evaluations

EPlan 11, Rev. 8 & EPIP 902, Rev. 17. This procedure change alters the backup method of performing protected area accountability. The change will allow badged personnel to take their accountability card home with their site security badge.

This procedure revision does not negatively impact any accident response. This procedure revision does not increase the probability or consequences of either an accident or a malfunction of equipment important to safety. Therefore, this procedure revision does not involve an Unreviewed Safety Question.

SC.MD-EU.DG-003(Q), Rev 1, Astro-Med Recorder/Equipment Setup For Emergency Diesel Generator Related Surveillance Testing. This procedure change provides test equipment and test equipment setup that will allow an Emergency Diesel Generator (EDG) train to remain operable during its monthly Technical Specification Surveillance Test.

This procedure revision does not negatively impact any accident response. This procedure revision does not increase the probability or consequences of either an accident or a malfunction of equipment important to safety. Therefore, this procedure revision does not involve an Unreviewed Safety Question.

UFSAR Change Notices Summary of Safety Evaluations

UFSAR Change Notice 97-08, Main Steam Flow Transmitter Time Response. This Change Notice changes the UFSAR description of the steam flow transmitters to reflect a modification performed on the steam flow transmitters to provide variable dampening.

This UFSAR change does not negatively impact any accident response. This design change does not increase the probability or consequences of either an accident or a malfunction of equipment important to safety. Therefore, this design change does not involve an Unreviewed Safety Question.

Deficiency Reports Summary of Safety Evaluations

There were no changes in this category implemented during October, 1997.

Other Summary of Safety Evaluation

Safety Evaluation S97-254, Revised Technical Specification Bases 3/4.1.3, Movable Control Assemblies

This Technical Specification Bases change involves a change to the bases section to allow use of the plant computer as a rod position indicator. The current bases section describes rod position indication but is not specific as to what indicators satisfactorily meet Technical Specification Requirements. The plant computer receives the same input from the Analog Rod Position Indication system as does the Control Console indicators and provides resolution equivalent to or better than the Control Console indicators.

This Technical Specification Bases change does not negatively impact any accident response. It does not increase the probability or consequences of either an accident or a malfunction of equipment important to safety. Therefore, this Technical Specification Bases change does not involve an Unreviewed Safety Question.