



Consumers
Power

**POWERING
MICHIGAN'S PROGRESS**

General Offices: 1945 West Parnall Road, Jackson, MI 49201 • (517) 788-0550

June 25, 1990

Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555

DOCKET 50-255 - LICENSE DPR-20 - PALISADES PLANT
RESPONSE TO GENERIC LETTER 90-04 - STATUS OF GENERIC SAFETY ISSUES

By letter dated April 25, 1990, NRC transmitted Generic Letter 90-04 entitled, "Request for Information on Status of Licensee Implementation of Generic Safety Issues Resolved with Imposition of Requirements on Corrective Actions." This letter provides Consumers Power Company's response to the Generic Letter for Palisades. Consumers Power Company has reviewed and reported on, by use of the attached table, the status of implementation of Generic Safety Issues which are applicable to Palisades.

Brian D Johnson
Staff Licensing Engineer

CC Administrator, Region III, USNRC
NRC Resident Inspector - Palisades

Attachment

OC0690-0362-NL02

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ATTACHMENT

Consumers Power Company
Palisades Plant
Docket 50-255

ENCLOSURE 1 FROM GENERIC LETTER 90-04
STATUS OF GENERIC SAFETY ISSUES

June 25, 1990

5 Pages

OC0690-0362-NL02

FACILITY NAME: PALISADES PLANT
 DOCKET NO: 50-255
 LICENSEE: CONSUMERS POWER CO

STATUS OF LICENSEE IMPLEMENTATION OF GENERIC SAFETY ISSUES
RESOLVED WITH IMPOSITION OF REQUIREMENTS OR CORRECTIVE ACTIONS

<u>GSI/(MPA No)</u>	<u>TITLE</u>	<u>APPLICABILITY</u>	<u>STATUS</u>	<u>COMMENTS</u>
40 (B065)	Safety Concerns Associated With Pipe Breaks in the BWR Scram System	All BWRs	NA	
41 (B058)	BWR Scram Discharge Volume Systems	All BWRs	NA	
43 (B107)	Reliability of Air Systems	All Plants	C	Project complete. CPGO letter 7/15/90.
51 (L913)	Improving the Reliability of Open-Cycle Service Water Systems	All Plants	I	Projected Complete '92 REFOUT CPGO Letter 1/29/90
67.3.3 (A017)	Improved Accident Monitoring	All Plants	C	NRC SER 7/19/88
75* (B076)	Item 1.1 - Post-Trip Review (Program Description and Procedure)	All Plants	C	NRC SER 11/5/85 8/9/85
75 (B085)	Item 1.2 - Post-Trip Review Data and Information Capability	All Plants	C	NRC SER 6/30/86 Projected 9/1/90

* The 16 items listed for GSI 75 all relate to actions derived from the generic implications of Salem ATWS events. Item numbers correspond to Generic Letter 83-28 action item numbers.

<u>GSI/(MPA No)</u>	<u>TITLE</u>	<u>APPLICABILITY</u>	<u>STATUS</u>	<u>COMMENTS</u>
75 (B077)	Item 2.1 - Equipment Classification and Vendor Interface (Reactor Trip System Components)	All Plants	C	NRC SER 6/6/88 12/10/87
75 (B086)	Item 2.2.1 - Equipment Classification for Safety-Related Components	All Plants	C	NRC SER 8/21/88
75 (L003)	Item 2.2.2 - Vendor Interface for Safety-Related Components	All Plants	E	Response due 9/21/90
75 (B078)	Item 3.1.1 & 3.1.2 - Post-Maintenance Testing (Reactor Trip System Components)	All Plants	C	NRC SER 8/15/85
75 (B079)	Item 3.1.3 - Post-Maintenance Testing-Changes to Test Requirements (Reactor Trip System Components)	All Plants	C	NRC SER 4/22/86
75 (B087)	Items 3.2.1 & 3.2.2 - Post-Maintenance Testing (All Other Safety-Related Components)	All Plants	C	NRC SER 8/15/85
75 (B088)	Item 3.2.3 - Post-Maintenance Testing-Changes to Test Requirements (All Other Safety-Related Components)	All Plants	C	NRC SER 4/22/86
75 (B080)	Item 4.1 - Reactor Trip System Reliability (Vendor-Related Modifications)	All Plants	C	NRC SER 8/15/85

<u>GSI/(MPA No)</u>	<u>TITLE</u>	<u>APPLICABILITY</u>	<u>STATUS</u>	<u>COMMENTS</u>
75 (B081)	Items 4.2.1 & 4.2.2 - Reactor Trip System Reliability - Maintenance and Testing (Preventative Maintenance and Surveillance Program for Reactor Trip Breakers)	All PWRs	C	NRC SER 6/19/85
75 (B082)	Item 4.3 - Reactor Trip System Reliability - Design Modifications (Automatic Actuation of Shunt Trip Attachment for Westinghouse and B&W Plants)	All W and B&W	NA	
75 (B090)	Item 4.3 - Reactor Trip System Reliability - Tech Spec Changes (Automatic Actuation of Shunt Trip Attachment for Westinghouse and B&W Plants)	All W & B&W	NA	
75 (B091)	Item 4.4 - Reactor Trip System Reliability (Improvements in Maintenance and Test Procedures for B&W Plants)	All B&W Plants	NA	

<u>GSI/(MPA No)</u>	<u>TITLE</u>	<u>APPLICABILITY</u>	<u>STATUS</u>	<u>COMMENTS</u>
75 (B092)	Item 4.5.1 - Reactor Trip System Reliability-Diverse Trip Features (System Functional Testing)	All Plants	C	NRC SER 8/15/85
75 (B093)	Items 4.5.2 & 4.5.3 - Reactor Trip System Reliability - Test Alternatives and Intervals (System Functional Testing)	All Plants	C	NRC SER 1/12/90 NRC SER 1/12/90
86 (B084)	Long Range Plan for Dealing with Stress Corrosion Cracking in BWR Piping	All BWRs	NA	
93 (B098)	Steam Binding of Auxiliary Feedwater Pumps	All PWRs	C	NRC SER 5/25/88
99 (L817)	RCS/RHR Suction Line Valve Interlock on PWRs	All PWRs	I	Projected complete date; submittal of restructured Tech Specs - 1990. 6/28/89
124	Auxiliary Feedwater System Reliability	ANO-1&2, Rancho Seco, Prairie Island 1&2, Crystal River-3 Ft Calhoun	NA	
A-13 (B017)	Snubber Operability Assurance - Hydraulic Snubbers	All Plants	C	Tech Spec Amendment 3/12/82

<u>GSI/(MPA No)</u>	<u>TITLE</u>	<u>APPLICABILITY</u>	<u>STATUS</u>	<u>COMMENTS</u>
A-13 (B022)	Snubber Operability Assurance - Mechanical Snubbers	All Plants	C	Tech Spec Amend 3/12/82
A-16 (D012)	Steam Effects on BWR Core Spray Distribution	Oyster Creek & NMP-1	NA	
A-35 (B023)	Adequacy of Offsite Power Systems	All Plants	C	Issue to be addressed in restructured Tech Spec submitted 1990 per ltr 1/24/89
B-10	Behavior of BWR Mark III Containments	All BWR Mark III Plants	NA	
B-36	Develop Design, Testing and Maintenance Criteria for Atmosphere Cleanup System Air Filtration and Adsorption Units for Engineered Safety Features Systems and for Normal Ventilation Systems	All Plants with OL Applications	NA	
B-63 (B045)	Isolation of Low Pressure Systems Connected to the Reactor Coolant System Pressure Boundary	All Plants	C	NRC Letter 8/12/80 NRC order dated 4/20/81