



NIAGARA MOHAWK POWER CORPORATION/301 PLAINFIELD ROAD, SYRACUSE, N.Y. 13212/TELEPHONE (315) 474-1511

July 11, 1989
NMP1L 0419

U.S. Nuclear Regulatory Commission
Attn: Document Control Desk
Washington, D.C. 20555

Re: Nine Mile Point Unit 1
Docket No. 50-220
DPR-63

Nine Mile Point Unit 2
Docket No. 50-410
NPF-69

Gentlemen:

The purpose of this letter is to provide you our response to Generic Letter 89-06, "Task Action Plan Item I.D.2 - Safety Parameter Display System".

Generic Letter 89-06 was issued by the Commission to all licensees to obtain the implementation status of the Safety Parameter Display Systems (SPDS) as required by NUREG-0737, Supplement 1. Attached to Generic Letter 89-06 was NUREG-1342, "A Status Report Regarding Industry Implementation of Safety Parameter Display Systems". NUREG-1342 describes methods used by some licensees/applicants to implement SPDS requirements in a manner found acceptable by the Staff.

Specifically, Generic Letter 89-06 requested each licensee to provide one of the following: 1) Certification that the SPDS fully meets the requirements of NUREG-0737, Supplement 1, taking into account the information provided in NUREG-1342; 2) Certification that the SPDS will be modified to fully meet the requirements of NUREG-0737, Supplement 1, taking into account the information provided in NUREG-1342, and the modification implementation schedule; 3) If a certification cannot be provided, the reasons for the finding and a discussion of the compensatory actions to be taken.

To meet the requirements of Generic Letter 89-06, Niagara Mohawk submits the enclosed certification which states the Nine Mile Point Unit 1 SPDS system meets the requirements of NUREG-0737, Supplement 1, (Category I) and the Nine

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Mile Point Unit 2 (NMP2) SPDS system will be modified to meet the requirements of NUREG-0737, Supplement 1 (Category II). A list of the required NMP2 modifications/evaluations and the scheduled implementation dates is also included.

Very truly yours,

NIAGARA MOHAWK POWER CORPORATION



C. D. Terry
Vice President

Nuclear Engineering & Licensing

JT/mlf
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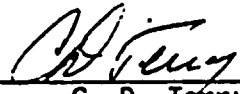
xc: Regional Administrator, Region I
Mr. R. A. Capra, Director
Ms. M. M. Slosson, Project Manager
Mr. W. A. Cook, Resident Inspector
Records Management

UNITED STATES
NUCLEAR REGULATORY COMMISSION

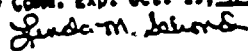
In the matter of)	Unit 1
)	Docket No. 50-220
Niagara Mohawk Power Corporation)	and
)	Unit 2
Nine Mile Point Nuclear Station)	Docket No. 50-410

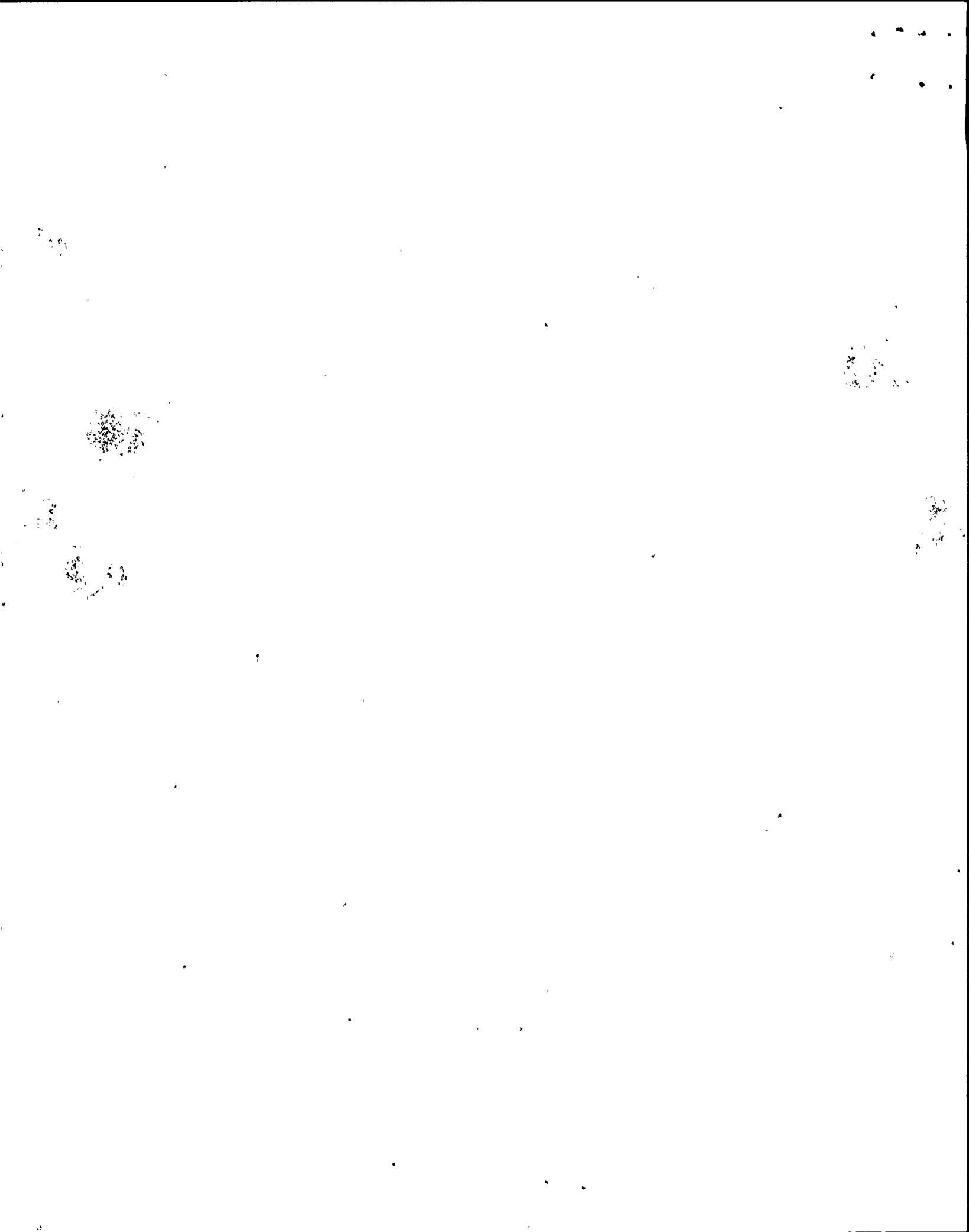
CERTIFICATION

C. D. Terry, being duly sworn, states that he is Vice President, Nuclear Engineering and Licensing of Niagara Mohawk Power Corporation, that he is authorized on the part of said Company to sign and file with the Nuclear Regulatory Commission this certification; and that the following statements/information attached hereto are accurate.

BY 
C. D. Terry
Vice President
Nuclear Engineering and Licensing

Subscribed and Sworn to
Before me this 11th day
of July, 1989

LINDA M. SCHOND
Notary Public State of NY
Qual. Onon. Co., No. 4908015
My Comm. Exp. Oct. 19, 89




NMP1

Nine Mile Point Unit 1 (NMP1) Safety Parameter Display System (SPDS) meets the requirements of NUREG-0737, Supplement 1, taking into account the information provided in NUREG-1342. A brief discussion is provided below to clarify NMP1's position on NUREG-1342 Items III.A.1/III.B.1.

- NUREG-1342 Sections III.A.1/III.B.1 require all SPDS-related information be physically displayed such that the information can be clearly read from the SPDS user's typical position. In the past, the main SPDS terminal and the Emergency Operating Procedures (EOP's) were located at the same work location. However, converting the EOP's from procedure to flow chart format has resulted in locating the EOP's away from the SPDS terminals. The NMPC Operations and Computer Departments are tasked with evaluating means to rectify this situation.

NMP2

The Nine Mile Point Unit 2 (NMP2) Safety Parameter Display System (SPDS) will be modified to meet the requirements of NUREG-0737, Supplement 1, taking into account the information provided in NUREG-1342. The implementation schedule for the modifications required to bring about compliance with NUREG-0737, Supplement 1, are provided on Attachment 1.

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ATTACHMENT 1

NMP2

<u>Description</u>	<u>Checklist Section</u>	<u>Scheduled Completion</u>
1) Radioactivity control will be added to the SPDS displays. This includes plant overview display (Level I) and additional detail (Level II) display. All five required safety functions will then be displayed on the SPDS	2.1.4, 2.2.9, 3.1, 3.3	By end of first refueling outage
2) Drywell temperature will be added as a containment integrity parameter	2.2.10, 2.2.7	By end of first refueling outage
3) The SPDS reactor water level parameter has displayed invalid/erroneous information or has been in a false alarm condition greater than 5 percent of operation. A work request (WR 162049) has been issued to troubleshoot.	4.7, 4.8	By end of first refueling outage
4) High and Low Level setpoints will be re-evaluated as part of the overall SPDS modification.	5.10	By end of first refueling outage.
5) Complete verification and validation (V&V) requirements (as identified in the pre-implementation audit) i.e. man-in-the-loop testing, collect and centrally locate all necessary V&V documentation, validation test	4.5, 5.15	By end of first refueling outage.

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ATTACHMENT 1

NMP2

<u>Description</u>	<u>Checklist Section</u>	<u>Scheduled Completion</u>
6) The SPDS screens cannot be clearly read from the EOP flowcharts. A Problem Report has been issued to evaluate options to correct this condition.	5.3	Complete preliminary evaluation by end of first refueling outage.
7) Limit marks will be added on the SPDS Level II displays.	5.11	By end of first refueling outage.
8) The SPDS computer points are not included in routine instrument loop surveillances. A Problem Report has been issued to identify all SPDS points and to include these points into applicable surveillances.	4.4	Identify points and begin process of including points in surveillances by end of first refueling outage

Note: The changes to the SPDS defined by the NMPC response to the pre-implementation audit will be completed by the end of the first refuel outage. Other changes, resulting from the SPDS design evaluation and/or from the man-in-the loop testing will be scheduled as appropriate.

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REGULATORY GUIDE 1.97
RMR TYPE A VARIABLES
38 UNITS

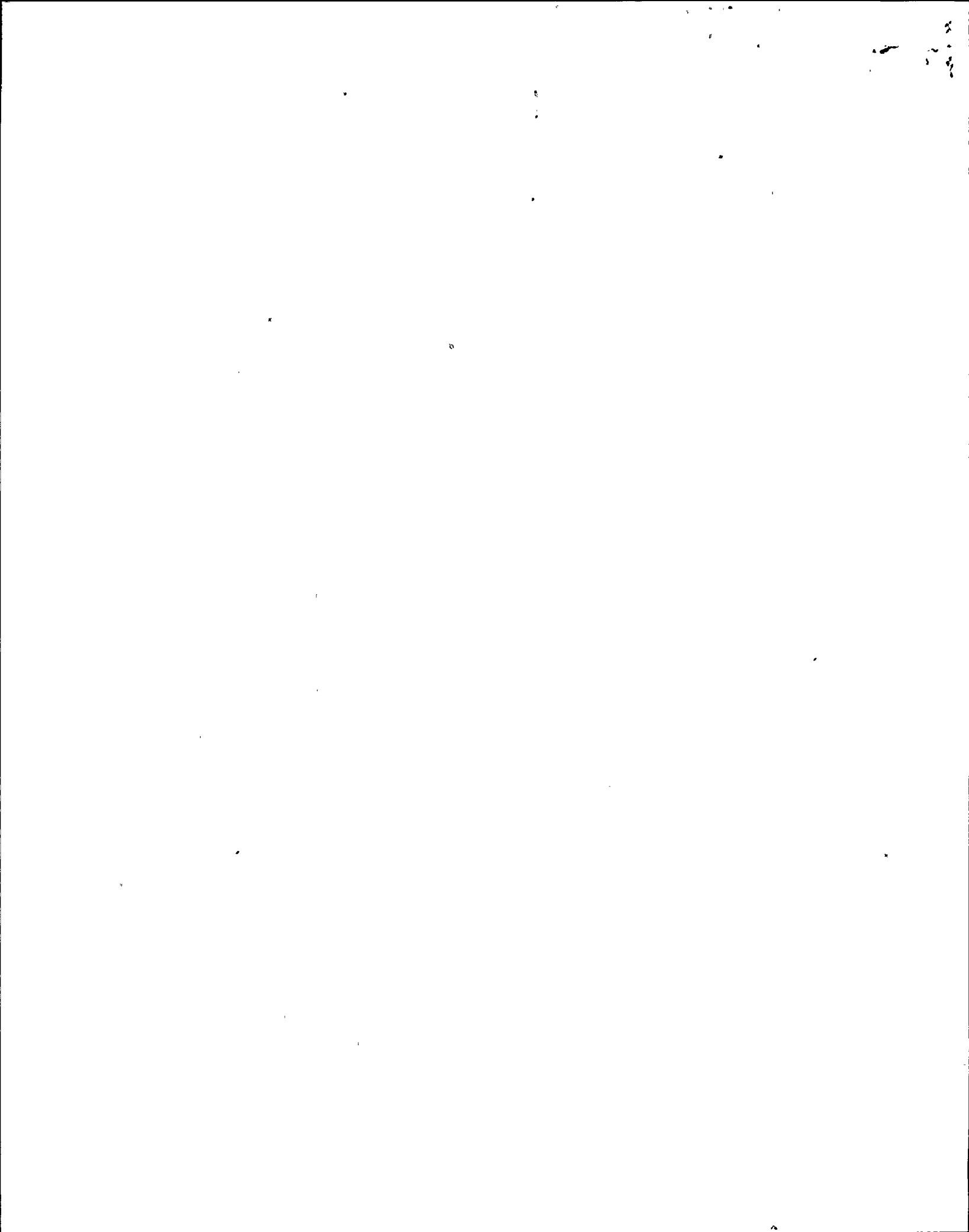
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CONTAINMENT HYDROGEN CONCENTRATION	A		A	A	A	A	A	A	A	A	A	A	A	A	A	A	A	A	A	A	A	A	A	A	A	A	A	A	A	A	A	A	A	A	A	A	A	A	A	A	191	
CONTAINMENT OXYGEN CONCENTRATION			A						A																																	111
CONTAINMENT PRESSURE			A						A																																	41
CONTAINMENT TEMPERATURE			A						A																																	31
CORE SPRAY FLOW																																										11
CORE SPRAY PRESSURE																																										31
DIESEL GENERATOR BATTERY VOLTAGE																																										21
DIESEL GENERATOR OUTPUT CURRENT																																										21
DIESEL GENERATOR OUTPUT POWER																																										21
DIESEL GENERATOR OUTPUT VOLTAGE																																										21
DRYWELL HYDROGEN CONCENTRATION	A		A	A					A	A	A																														101	
DRYWELL OXYGEN CONCENTRATION			A						A																																	41
DRYWELL PRESSURE	A	A	A	A	A	A	A	A	A	A	A	A	A	A	A	A	A	A	A	A	A	A	A	A	A	A	A	A	A	A	A	A	A	A	A	A	A	A	A	A	A	311
DRYWELL TEMPERATURE			A						A																																	151
MAIN STEAM VALVE POSITION			A						A																																	11
NEUTRON FLUX									A	A																																41
REACTOR COOLANT LEVEL	A	A	A	A					A	A	A	A	A	A	A	A																									321	
REACTOR COOLANT SYSTEM PRESSURE			A						A	A	A																														121	
REACTOR VESSEL PRESSURE	A	A	A		A	A	A	A	A				A	A	A																										151	
RHR SERVICE WATER SYSTEM FLOW																																									11	
RHR SYSTEM FLOW																																									31	
SUMP LEVEL			A																																						21	
SUPPRESSION CHAMBER PRESSURE																																									71	
SUPPRESSION POOL LEVEL	A	A	A	A	A	A	A	A	A	A	A	A	A	A	A																										291	
SUPPRESSION POOL TEMPERATURE	A	A	A	A	A	A	A	A	A	A	A	A	A	A	A																										331	
TORUS AIR TEMPERATURE																																									11	
NON-R.G. 1.97 VARIABLES																																									11	
NO TYPE A VARIABLES																																									11	
NUMBER OF TYPE A VARIABLES	1	71	51	121	91	51	51	71	71	111	111	81	71	71	71	31	31	31	31	91	91	51	61	61	0	1	131	111	111	51	21	101	71	41	41	51	31	41	71	1		

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REGULATORY GUIDE 1.97
BWR TYPE A VARIABLES
KEY TO PLANTS

FILE NO.	PLANT NAME
OR05 UNIT 1	BROWNS FERRY 1
OR05 UNIT 3	BROWNS FERRY 3
OR05 UNIT 2	BROWNS FERRY 2
OR06 UNIT 2	BRUNSWICK 2
OR06 UNIT 1	BRUNSWICK 1
LP09	CLINTON 1
OR09	COOPER 1
OR12 UNIT 2	DRESDEN 2
OR12 UNIT 3	DRESDEN 3
OR13	DUANE ARNOLD
LP12	FERMI 2
OR15	FITZPATRICK 1
LP15	GRAND GULF 1
OR20 UNIT 1	HATCH 1
OR20 UNIT 2	HATCH 2
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LP19 UNIT 1	LASALLE 1
LP19 UNIT 2	LASALLE 2
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OR35	PILGRIM 1
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LP31	RIVER BEND 1
LP34	SHOREHAM 1
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LP39 UNIT 2	SUSQUEHANNA 2
OR49	VERMONT YANKEE 1
LP42	WNP 2



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	LP09	LP12	LP15	LP18	LP19	LP19	LP20	LP20	LP25	LP29	LP29	LP31	LP34	LP39	LP39	LP42	OR05	OR05	OR05	OR06	OR06	OR09	OR12	OR12	OR13	OR15	OR20	OR20	OR26	OR28	OR29	OR32	OR34	OR34	OR35	OR38	OR38	OR49	VARI				
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CONTAINMENT HYDROGEN CONCENTRATION	A	I	I	A	I	A	I	A	I	A	I	A	I	A	I	A	I	A	I	A	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	19			
CONTAINMENT OXYGEN CONCENTRATION	I	I	I	A	I	I	A	I	A	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	11			
CONTAINMENT PRESSURE	I	I	A	I	I	I	I	I	I	A	I	A	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	4		
CONTAINMENT TEMPERATURE	I	I	A	I	I	I	I	I	I	A	I	A	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	3		
CORE SPRAY FLOW	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	A	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	1	
CORE SPRAY PRESSURE	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	A	A	A	I	I	I	I	I	I	I	I	I	I	I	I	I	3	
DIESEL GENERATOR BATTERY VOLTAGE	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	A	A	A	I	I	I	I	I	I	I	I	I	I	I	I	2	
DIESEL GENERATOR OUTPUT CURRENT	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	A	A	A	I	I	I	I	I	I	I	I	I	I	I	I	I	2
DIESEL GENERATOR OUTPUT POWER	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	A	A	A	I	I	I	I	I	I	I	I	I	I	I	I	I	2
DIESEL GENERATOR OUTPUT VOLTAGE	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	A	A	A	I	I	I	I	I	I	I	I	I	I	I	I	I	2
DRYWELL HYDROGEN CONCENTRATION	A	I	A	A	I	I	I	I	A	A	A	I	I	I	I	I	I	A	A	A	I	I	I	I	I	I	A	A	A	I	I	I	I	I	I	I	I	I	I	I	I	I	10
DRYWELL OXYGEN CONCENTRATION	I	I	A	I	I	I	I	I	A	A	A	I	I	I	I	I	I	A	A	A	I	I	I	I	I	I	A	A	A	I	I	I	I	I	I	I	I	I	I	I	I	I	4
DRYWELL PRESSURE	A	A	A	A	A	A	A	A	A	A	A	A	A	A	A	A	A	A	A	A	A	A	A	A	A	A	A	A	A	A	A	A	A	A	A	A	A	A	A	A	A	A	31
DRYWELL TEMPERATURE	I	I	A	I	I	I	I	I	A	A	A	I	I	I	I	I	A	A	A	A	A	A	A	A	A	A	A	A	A	A	A	A	A	A	A	A	A	A	A	A	A	A	15
MAIN STEAM VALVE POSITION	I	I	A	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	1
NEUTRON FLUX	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	4
REACTOR COOLANT LEVEL	A	A	A	A	I	I	A	A	A	A	A	A	A	A	A	A	I	I	I	A	A	A	A	A	A	A	A	A	A	A	A	A	A	A	A	A	A	A	A	A	A	A	32
REACTOR COOLANT SYSTEM PRESSURE	I	I	A	I	I	I	I	I	A	A	A	I	I	I	I	I	A	I	I	A	A	A	A	A	A	A	A	A	A	A	A	A	A	A	A	A	A	A	A	A	A	A	12
REACTOR VESSEL PRESSURE	A	A	A	A	A	A	A	A	A	I	I	A	A	A	A	I	I	I	I	A	A	A	A	A	A	A	A	A	A	A	A	A	A	A	A	A	A	A	A	A	A	A	15
RHR SERVICE WATER SYSTEM FLOW	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	A	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	1
RHR SYSTEM FLOW	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	A	A	A	I	I	I	I	I	I	I	I	I	I	I	I	I	I	3
SUMP LEVEL	I	I	A	I	I	I	I	I	I	I	I	I	A	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	2	
SUPPRESSION CHAMBER PRESSURE	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	A	A	A	A	A	A	A	A	I	I	I	I	I	I	I	I	I	I	I	I	I	I	A	7
SUPPRESSION POOL LEVEL	A	A	A	A	A	A	A	A	A	A	A	A	A	A	A	A	I	I	I	A	A	A	A	A	A	A	A	A	A	A	A	A	A	A	A	A	A	A	A	A	A	A	29
SUPPRESSION POOL TEMPERATURE	A	A	A	A	A	A	A	A	A	A	A	A	A	A	A	A	I	I	I	A	A	A	A	A	A	A	A	A	A	A	A	A	A	A	A	A	A	A	A	A	A	A	33
TORUS AIR TEMPERATURE	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	1	
NON-R.G. 1.97 VARIABLES	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	1	
NO TYPE A VARIABLES	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	I	1
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