

**STATUS REPORT ON THE LICENSING ACTIVITIES
AND REGULATORY DUTIES OF THE U.S. NUCLEAR REGULATORY COMMISSION**

November 2017

Enclosure

Table of Contents

GRAPHICAL METRICS

1. Staffing	1
2. Licensing	7
3. License renewal inventory and age, planned vs actual, based on 22 months for uncontested applications and 30 months for contested applications	11
4. Power Uprates, Planned vs. Actual, based on the revised metrics in SECY-13-0070	11
5. Decommissioning Plant Licensing Action Inventory and Age, monthly for one-year rolling metrics and annually for the past 10 years	12
6. Uranium Recovery license and licensing action review inventory and average age, monthly for one-year rolling metrics and annually for the past 10 years	14
7. Design certification, COL, and ESP application review inventory including age and projected completion dates	16
8. Requests for Additional Information issued by each office including the offices of Nuclear Reactor Regulation, New Reactors, Uranium Recovery, Decommissioning	16
9. Reactor Oversight Process Findings year-to-date and 3-year rolling metrics, total and by region for green, white, yellow, and red findings	20
10. Percentage of Final Significance Determinations Made within 90 Days for All Potentially Greater-Than-Green Findings, monthly for one-year rolling metrics and annually for the past 10 years	21
11. Component Design Basis Inspection (CDBI) duration, fees, and percentage of fees used to reimburse contractors – monthly averages for three-year rolling metrics	21
12. New reactor licensing and inspection status for Vogtle 3 & 4 and Summer 2 & 3 including the percentage of NRC inspections completed and the percentage of ITAAC reviews completed within 30 days	27
13. Committee for the Review of Generic Requirements (CRGR) list of issues	30

NARRATIVE INFORMATION

1. Status of License Renewal Reviews	36
2. Status of Subsequent License Renewal (SLR) readiness	37
3. Status of power uprate application reviews	38
4. Status of design certification, COL, and ESP application reviews	38
5. Status of licensing and inspection for Vogtle 3 & 4 and Summer 2 & 3 including any challenges to the timely resolution of licensing issues or Part 52 interpretations	41
6. Status of uranium recovery licensing including projected budget and timeline for both the environmental impact statement and safety evaluation report for each application review	42
7. Specific actions taken to improve efficiency of reviews conducted for compliance with the National Historic Preservation Act	45
8. Status of the pilot project on establishing flat fees for uranium recovery licensees	47
9. Status of specific actions taken or planned to ensure greater discipline and management oversight in the use of the Request for Additional Information (RAI) process associated with a regulatory requirement and limited to those RAIs necessary for making a regulatory decision	47
10. Status of specific actions undertaken to reduce corporate overhead costs including the amount of the savings and the timeframe for realizing the cost savings	49
11. Status of specific actions taken and/or planned to develop metrics for assessing the quality of cost-benefit analyses conducted in association with new requirements, backfit analyses, or rulemaking	51
12. Status of the revised guidance currently under development to clarify the use of qualitative factors	51
13. Status of the Committee to Review Generic Requirements (CRGR) review of the application of the Backfit Rule in the licensing and inspection programs across the agency	52

14. Status of Project Aim Task 19: Operating Reactor Licensing Process Improvements	55
15. Status of effort to establish clear schedules and estimated number of reviewer hours for licensing action reviews	55
16. Status of any potential changes to the Reactor Oversight Process	55
17. Status of effort to provide greater transparency and detail in invoices to applicants and licensees	55
18. Clarity in Operability Determinations	56
19. Significance Determination Process	56
20. Use of Inspection Manual Chapter 609 Appendix M, "Significance Determination Process Using Qualitative Criteria," in the Reactor Oversight Process Significance Determination Process	57
21. Engineering Inspection Programs	57
22. Technical Specifications Task Force (TSTF) process	58
23. Improving New Plant Application Review Efficiency	59
24. Unresolved policy issues with regard to the licensing of small modular reactors (SMRs)	59
25. Progress toward preparing to review non-light water reactor applications	64

GRAPHICAL METRICS¹

1. Staffing

- a. The U.S. Nuclear Regulatory Commission (NRC) Yearly Staffing (Full-Time Equivalent [FTE]) budget and actual, since Fiscal Year (FY) 2000.

NRC Budget and Actual FTE
(including the Office of the Inspector General and Reimbursable FTE)

Fiscal Year	FTE Actuals	FTE Budgeted
2000	2,777	2,814
2001	2,784	2,774
2002	2,812	2,865
2003	2,936	2,919
2004	3,034	3,058
2005	3,142	3,129
2006	3,198	3,288
2007	3,486	3,454
2008	3,715	3,729
2009	3,988	3,868
2010	4,032	3,943
2011	4,013	4,011
2012	3,846	3,977
2013	3,730	3,944
2014	3,735	3,831
2015	3,717	3,809
2016	3,549	3,628
2017	3,241	3,405
2018	3,146*	3,293

* The FY 2018 Actuals are an end-of-fiscal-year projection based upon known personnel actions as of pay period ending September 30, 2017.

- b. Monthly staffing (FTE) for preceding 12 months and projections for 12 months going forward for the offices of Nuclear Reactor Regulation (NRR), New Reactors (NRO), Uranium Recovery, Decommissioning, and for corporate support functions.

Actual/projected FTE for the period reflects utilization (or projected utilization) (i.e., approximately 1/12 of total year expenditure).

¹ The responses that provide data over 10-year periods may reflect changes in definitions, standards, size of the fleet of operating reactors, or similar changes that have occurred over the years.

U.S. Nuclear Regulatory Commission Office of Nuclear Reactor Regulation FTE Actuals and Projections 12 Months Prior and 12 Months Future Data as of 09/30/2017		
Period	Actual/ Projected FTE for the Period	Cumulative FTE
10/02/2016 - 10/29/2016	38.5	38.5
10/30/2016 - 11/26/2016	38.3	76.8
11/27/2016 - 12/24/2016	38.2	115.0
12/25/2016 - 01/21/2017	38.2	153.2
01/22/2017 - 02/18/2017	37.8	191.0
02/19/2017 - 03/18/2017	37.6	228.6
03/19/2017 - 04/15/2017	37.5	266.1
04/16/2017 - 05/13/2017	37.2	303.3
05/14/2017 - 06/10/2017	36.6	339.9
06/11/2017 - 07/08/2017	36.3	376.2
07/09/2017 - 08/05/2017	36.0	412.2
08/06/2017 - 09/02/2017	36.0	448.2
09/03/2017 - 09/30/2017	35.8	484.0
10/01/2017 - 10/28/2017	35.9	35.9
10/29/2017 - 11/25/2017	35.9	71.8
11/26/2017 - 12/23/2017	36.0	107.8
12/24/2017 - 01/20/2018	36.0	143.8
01/21/2018 - 02/17/2018	36.0	179.8
02/18/2018 - 03/17/2018	36.0	215.8
03/18/2018 - 04/14/2018	36.1	251.9
04/15/2018 - 05/12/2018	36.1	288.0
05/13/2018 - 06/09/2018	36.1	324.1
06/10/2018 - 07/07/2018	36.1	360.2
07/08/2018 - 08/04/2018	36.1	396.3
08/05/2018 - 09/01/2018	36.1	432.4
09/02/2018 - 09/29/2018	36.1	468.5

Notes:

- 1 Data are reported in two-pay-period groups because of the biweekly payroll cycle.
- 2 Actual/projected FTE for the period reflects FTE utilization (or projected utilization) (i.e., approximately 1/12 of total year FTE expenditure).
- 3 Projection is based on known future gains and losses through the end of the fiscal year.
- 4 Includes all staff in NRR.

U.S. Nuclear Regulatory Commission Office of New Reactors FTE Actuals and Projections 12 Months Prior and 12 Months Future Data as of 09/30/2017		
Period	Actual/ Projected FTE for the Period	Cumulative FTE
10/02/2016 - 10/29/2016	23.9	23.9
10/30/2016 - 11/26/2016	23.8	47.7
11/27/2016 - 12/24/2016	23.5	71.2
12/25/2016 - 01/21/2017	23.0	94.2
01/22/2017 - 02/18/2017	22.7	116.9
02/19/2017 - 03/18/2017	22.5	139.4
03/19/2017 - 04/15/2017	22.7	162.1
04/16/2017 - 05/13/2017	22.4	184.5
05/14/2017 - 06/10/2017	22.3	206.8
06/11/2017 - 07/08/2017	22.4	229.2
07/09/2017 - 08/05/2017	22.2	251.4
08/06/2017 - 09/02/2017	22.1	273.5
09/03/2017 - 09/30/2017	22.0	295.5
10/01/2017 - 10/28/2017	22.0	22.0
10/29/2017 - 11/25/2017	22.0	44.0
11/26/2017 - 12/23/2017	22.0	66.0
12/24/2017 - 01/20/2018	22.1	88.1
01/21/2018 - 02/17/2018	22.1	110.2
02/18/2018 - 03/17/2018	22.1	132.3
03/18/2018 - 04/14/2018	22.1	154.4
04/15/2018 - 05/12/2018	22.1	176.5
05/13/2018 - 06/09/2018	22.1	198.6
06/10/2018 - 07/07/2018	22.1	220.7
07/08/2018 - 08/04/2018	22.1	242.8
08/05/2018 - 09/01/2018	22.1	264.9
09/02/2018 - 09/29/2018	22.1	287.0

Notes:

- 1 Data are reported in two-pay-period groups because of the biweekly payroll cycle.
- 2 Actual/projected FTE for the period reflects FTE utilization (or projected utilization) (i.e., approximately 1/12 of total year FTE expenditure).
- 3 Projection is based on known future gains and losses through the end of the fiscal year.
- 4 Includes all staff in NRO.

U.S. Nuclear Regulatory Commission Uranium Recovery FTE Actuals and Projections 12 Months Prior and 12 Months Future Data as of 09/30/2017		
Period	Actual/ Projected FTE for the Period	Cumulative FTE
10/02/2016 - 10/29/2016	1.9	1.9
10/30/2016 - 11/26/2016	1.9	3.8
11/27/2016 - 12/24/2016	1.8	5.6
12/25/2016 - 01/21/2017	1.8	7.4
01/22/2017 - 02/18/2017	1.8	9.2
02/19/2017 - 03/18/2017	1.8	11.0
03/19/2017 - 04/15/2017	1.8	12.8
04/16/2017 - 05/13/2017	1.8	14.6
05/14/2017 - 06/10/2017	1.8	16.4
06/11/2017 - 07/08/2017	1.8	18.2
07/09/2017 - 08/05/2017	1.8	20.0
08/06/2017 - 09/02/2017	1.8	21.8
09/03/2017 - 09/30/2017	1.7	23.5
10/01/2017 - 10/28/2017	1.7	1.7
10/29/2017 - 11/25/2017	1.7	3.4
11/26/2017 - 12/23/2017	1.7	5.1
12/24/2017 - 01/20/2018	1.7	6.8
01/21/2018 - 02/17/2018	1.7	8.5
02/18/2018 - 03/17/2018	1.7	10.2
03/18/2018 - 04/14/2018	1.7	11.9
04/15/2018 - 05/12/2018	1.7	13.6
05/13/2018 - 06/09/2018	1.7	15.3
06/10/2018 - 07/07/2018	1.7	17.0
07/08/2018 - 08/04/2018	1.7	18.7
08/05/2018 - 09/01/2018	1.7	20.4
09/02/2018 - 09/29/2018	1.7	22.1

- Notes:
- 1 Data are reported in two-pay-period groups because of the biweekly payroll cycle.
 - 2 Actual/projected FTE for the period reflects FTE utilization (or projected utilization) (i.e., approximately 1/12 of total year FTE expenditure).
 - 3 Projection is based on known future gains and losses through the end of the fiscal year.
 - 4 Includes all staff in the Uranium Recovery Branch of the Office of Nuclear Material Safety and Safeguards (NMSS), and relevant staff in the following:
Environmental Review Branch, NMSS; Division of Materials Safety, State, Tribal, and Rulemaking Programs, NMSS; Fuel Cycle and Decommissioning Branch, Region IV; Office of General Counsel (OGC); and Atomic Safety Licensing Board Panel (ASLB)

U.S. Nuclear Regulatory Commission Decommissioning FTE Actuals and Projections 12 Months Prior and 12 Months Future Data as of 09/30/2017		
Period	Actual/ Projected FTE for the Period	Cumulative FTE
10/02/2016 - 10/29/2016	2.0	2.0
10/30/2016 - 11/26/2016	2.0	4.0
11/27/2016 - 12/24/2016	2.1	6.1
12/25/2016 - 01/21/2017	2.0	8.1
01/22/2017 - 02/18/2017	2.0	10.1
02/19/2017 - 03/18/2017	2.0	12.1
03/19/2017 - 04/15/2017	2.0	14.1
04/16/2017 - 05/13/2017	2.0	16.1
05/14/2017 - 06/10/2017	2.1	18.2
06/11/2017 - 07/08/2017	2.1	20.3
07/09/2017 - 08/05/2017	2.1	22.4
08/06/2017 - 09/02/2017	2.1	24.5
09/03/2017 - 09/30/2017	2.1	26.6
10/01/2017 - 10/28/2017	2.1	2.1
10/29/2017 - 11/25/2017	2.1	4.2
11/26/2017 - 12/23/2017	2.1	6.3
12/24/2017 - 01/20/2018	2.1	8.4
01/21/2018 - 02/17/2018	2.1	10.5
02/18/2018 - 03/17/2018	2.1	12.6
03/18/2018 - 04/14/2018	2.1	14.7
04/15/2018 - 05/12/2018	2.1	16.8
05/13/2018 - 06/09/2018	2.1	18.9
06/10/2018 - 07/07/2018	2.1	21.0
07/08/2018 - 08/04/2018	2.1	23.1
08/05/2018 - 09/01/2018	2.1	25.2
09/02/2018 - 09/29/2018	2.1	27.3

Notes:

- 1 Data are reported in two-pay-period groups because of the biweekly payroll cycle.
- 2 Actual/projected FTE for the period reflects FTE utilization (or projected utilization) (i.e., approximately 1/12 of total year FTE expenditure).
- 3 Projection is based on known future gains and losses through the end of the fiscal year.
- 4 Includes all staff in the Reactor and Materials Decommissioning Branches of NMSS only. No mission support staff, second level and above supervisory staff, or staff support from other offices is included.

U.S. Nuclear Regulatory Commission Corporate Support Functions FTE Actuals and Projections 12 Months Prior and 12 Months Future Data as of 09/30/2017		
Period	Actual/ Projected FTE for the Period	Cumulative FTE
10/02/2016 - 10/29/2016	41.6	41.6
10/30/2016 - 11/26/2016	41.0	82.6
11/27/2016 - 12/24/2016	40.9	123.5
12/25/2016 - 01/21/2017	40.8	164.3
01/22/2017 - 02/18/2017	40.5	204.8
02/19/2017 - 03/18/2017	40.4	245.2
03/19/2017 - 04/15/2017	40.1	285.3
04/16/2017 - 05/13/2017	39.3	324.6
05/14/2017 - 06/10/2017	38.4	363.0
06/11/2017 - 07/08/2017	38.2	401.2
07/09/2017 - 08/05/2017	37.9	439.1
08/06/2017 - 09/02/2017	37.4	476.5
09/03/2017 - 09/30/2017	37.3	513.8
10/01/2017 - 10/28/2017	37.1	37.1
10/29/2017 - 11/25/2017	37.1	74.2
11/26/2017 - 12/23/2017	37.1	111.3
12/24/2017 - 01/20/2018	37.1	148.4
01/21/2018 - 02/17/2018	37.1	185.5
02/18/2018 - 03/17/2018	37.1	222.6
03/18/2018 - 04/14/2018	37.1	259.7
04/15/2018 - 05/12/2018	37.1	296.8
05/13/2018 - 06/09/2018	37.1	333.9
06/10/2018 - 07/07/2018	37.1	371.0
07/08/2018 - 08/04/2018	37.1	408.1
08/05/2018 - 09/01/2018	37.1	445.2
09/02/2018 - 09/29/2018	37.1	482.3

- Notes:
- 1 Data are reported in two-pay-period groups because of the biweekly payroll cycle.
 - 2 Actual/projected FTE for the period reflects FTE utilization (or projected utilization) (i.e., approximately 1/12 of total year FTE expenditure).
 - 3 Projection is based on known future gains and losses through the end of the fiscal year.
 - 4 Includes all staff in the following corporate support offices: Office of the Chief Financial Officer, Office of the Chief Information Officer, Office of Administration, Office of Small Business and Civil Rights, and Office of the Chief Human Capital Officer

2. Licensing

- a. Size and median age of Licensing Action Inventory, monthly for 1-year rolling metrics and annually for the past 10 years.

1-Year Rolling Metric – Size and Median Age of Licensing Action Inventory		
Month	Inventory Total (Note 1)	Median Age (in months)
October 2016	548	3
November 2016	586	4
December 2016	635	4
January 2017	637	5
February 2017	626	3
March 2017	615	3
April 2017	631	3
May 2017	617	3
June 2017	548	3
July 2017	594	3
August 2017	625	4
September 2017	571	4
Annual Size and Median Age of Inventory – Last 10 Years		
Fiscal Year	End of FY Inventory Total	End of FY Median Age (in months)
FY 2008	669	5
FY 2009	600	5
FY 2010	721	5
FY 2011	489	5
FY 2012	491	5
FY 2013	486	5
FY 2014	606	5
FY 2015	559	4
FY 2016	546	3
FY 2017	571	4

Note 1: Similar to the licensing actions reported in the yearly Congressional Budget Justification (CBJ), the inventory does not include unusually complex or Fukushima related licensing actions.

Comments:

The charts show information on the size and median age of the licensing action inventory. The size of the inventory is defined as the number of licensing actions undergoing NRC staff review at the end of each month or fiscal year. The median age corresponds to the open inventory for the respective month or fiscal year.

- b. Licensing Actions Performance, Planned vs Actual, monthly for 1-year rolling metrics and annually for the past 10 years.

1-Year Rolling Metric for Licensing Actions Completions				
Month	CBJ Metric Target (FY Total)	Licensing Actions Submitted in Previous Year (Note 1)	Planned (Cumulative Monthly Target)	Actual (Cumulative Monthly Total)
October 2016	900	754	63	71
November 2016	900	754	126	118
December 2016	900	754	189	178
January 2017	900	754	251	250
February 2017	900	754	314	352
March 2017	900	754	377	434
April 2017	900	754	440	510
May 2017	900	754	503	604
June 2017	900	754	566	756
July 2017	900	754	628	836
August 2017	900	754	691	901
September 2017	900	754	754	967
10-Year Annual Completions for Licensing Actions				
Year	CBJ Metric Target	Licensing Actions Submitted in Previous Year (Note 1)	Actual	
FY 2008 (Note 2)	1465	1263	1054	
FY 2009	1150	993	1002	
FY 2010	950	928	988	
FY 2011	950	1182	849	
FY 2012	950	660	770	
FY 2013 (Note 3)	950	802	668	
FY 2014 (Note 3)	900	936	607	
FY 2015	900	737	792	
FY 2016	900	736	837	
FY 2017	900	754	967	

Note 1: As discussed below, the number of licensing actions submitted in the previous year establishes the target for the number of licensing actions to be completed in the current year.

Note 2: The incoming licensing actions declined and the submitted licensing actions were more complex and required longer to review.

Note 3: Issuance of licensing actions was less than planned due to redirection of resources to higher priority Fukushima-related work.

Comments:

Each year, the NRC staff establishes metrics for licensing actions and reports them in the CBJ. Over the years, the CBJ metric target for licensing actions completions changed to reflect the actual number of licensing actions submitted and to reflect any new categorization of regulatory actions.

- c. Other Licensing Task Performance, Planned vs Actual, monthly for 1-year rolling metrics and annually for the past 10 years.

1-Year Rolling Metric for Other Licensing Task Completions (OLTs)				
Month	CBJ Metric Target (FY Total)	OLTs Submitted in Previous Year (Note 1)	Planned (Cumulative Monthly Target)	Actual (Cumulative Monthly Total)
October 2016 (Note 2)	500	597	25	42
November 2016	500	597	50	61
December 2016	500	597	75	69
January 2017	500	597	100	116
February 2017	500	597	125	147
March 2017	500	597	150	352
April 2017	500	597	175	364
May 2017	500	597	200	368
June 2017	500	597	225	386
July 2017	500	597	250	444
August 2017	500	597	275	494
September 2017	500	597	300	644
10-year Annual Completions for Other Licensing Tasks				
Year	CBJ Metric Target	OLTs Submitted in Previous Year (Note 1)	Actual	
FY 2008	600	679	678	
FY 2009	600	541	541	
FY 2010	600	433	625	
FY 2011	600	329	465	
FY 2012	600	591	674	
FY 2013 (Note 3)	600	577	529	
FY 2014 (Note 4)	500	1002	765	
FY 2015 (Note 3)	500	577	461	
FY 2016	500	602	641	
FY 2017	500	597	644	

- Note 1: As discussed below, the number of OLTs submitted in the previous year establishes the target for the number of licensing actions to be completed in the current year.
- Note 2: The FY 2017 CBJ states that target for OLTs is 500 actions. However, this target was since redefined to exclude items from this metric that are not licensing activities, such as Task Interface Agreements and 2.206 Petitions. The revised OLT target is 300 actions. This change will be reflected in future revisions of the CBJ.
- Note 3: Fewer OLTs were issued than planned due to redirection of resources to higher priority Fukushima-related work.
- Note 4: The significant increase in OLTs submitted in FY 2013 is attributed to Fukushima-related actions.

Comments:

Each year, the NRC staff sets metrics for OLT completions and reports them in the CBJ. Currently, the CBJ targets are based on the number of actions initiated the previous year or 300, whichever is lower (See Note 2).

- d. Size and median age of topical report reviews, monthly for 1-year rolling metrics and annually for the past 10 years.

1-Year Rolling Metric – Size and Median Age of Topical Report Inventory		
Month	Inventory Total *	Median Age (in months)
October 2016	49	30
November 2016	49	28
December 2016	48	29
January 2017	50	29
February 2017	48	25
March 2017	45	25
April 2017	45	21
May 2017	45	21
June 2017	46	21
July 2017	42	21
August 2017	40	22
September 2017	37	22

*Topical report inventory includes topical reports currently under review requiring a Safety Evaluation Report (SER).

Comments:

The annual topical report inventory and topical report median age information for the past 10 years is not readily retrievable in the NRC’s Replacement Reactor Program System database.

3. License renewal inventory and age, planned vs actual, based on 22 months for uncontested applications and 30 months for contested applications.

License Renewal Applications Currently Under Review

Plant Name and Unit(s)	Application Receipt Date	Application Review Time (Months)	Currently or Previously Contested	Notes
Indian Point 2 & 3	04/30/2007	122	yes	1, 2
Diablo Canyon 1 & 2	11/24/2009	77	yes	1, 2, 3
Seabrook 1	06/01/2010	87	yes	2
Waterford 3	03/23/2016	16	no	1
River Bend	05/31/2017	2	yes	4

Note 1: These numbers changed from the previous report to align them with other entries to count review time from date of acceptance rather than date of receipt.

Note 2: Previously contested.

Note 3: Review suspended as of July 2016 at licensee's request.

Note 4: Currently contested.

Comments:

- Indian Point delays were associated with adjudicatory issues, reviews of substantial new information submitted by the licensee, and review of extensive public comments on NRC staff environmental review documents. The applicant recently submitted an amendment to its license renewal application to request a 10-year rather than 20-year renewal period, and the pending adjudicatory actions have been voluntarily dismissed. Seabrook is addressing a significant technical issue.

4. Power Uprates Review Times, Planned vs Actual, based on the revised metrics in SECY-13-0070*.

Plant Name	Issue Date	Uprate Type (Note 1)	Planned Review Duration (Months)	Actual Review Duration (Months)	Notes
Fermi 2	02/10/14	MUR	9	10	
Peach Bottom 2	08/25/14	EPU	18	17	
Peach Bottom 3	08/25/14	EPU	18	17	
Catawba 1	04/29/16	MUR	9	21	Note 2
Columbia Generating Station	05/11/17	MUR	9	11	
Browns Ferry 1	8/14/17	EPU	18	19	Note 3
Browns Ferry 2	8/14/17	EPU	18	19	Note 3
Browns Ferry 3	8/14/17	EPU	18	19	Note 3

* Completed power uprate reviews for applications dated August 1, 2012, or later

Note 1: MUR = measurement uncertainty recapture power uprate, EPU = extended power uprate.

Note 2: The Catawba MUR power uprate review was delayed due to unanticipated significant technical issues identified by the NRC staff during the review. Specifically, the staff

identified that the methodology the licensee used to calculate neutron fluence values at MUR conditions was based on a computer code that was not approved for use in this scenario. Upon identification of the issue, the licensee requested the NRC to include usage of the new neutron fluence methodology in the MUR review.

Note 3: The NRC staff's safety review of the Browns Ferry EPU safety review was a complex evaluation that took slightly longer than the 18 month planned review duration. However, the review was completed ahead of the date requested by the licensee.

5. Decommissioning Plant Licensing Action Inventory and Age, monthly for 1-year rolling metrics and annually for the past 10 years.

Size and Median Age of Decommissioning Transition Inventory		
Month	Inventory Total (Note 1) (Note 2)	Median Age (in months)
October 2016	18	2
November 2016	19	2.5
December 2016	15	4
January 2017	17	5
February 2017	14	2.5
March 2017	18	2.5
April 2017	18	2.5
May 2017	20	3
June 2017	23	4
July 2017	16	4
August 2017	19	4
September 2017	18	5

Annual Size and Median Age of Decommissioning Transition Inventory (Note 3)		
Fiscal Year	End of FY Inventory Total	End of FY Median Age (in months)
FY 2013	32	2
FY 2014	65	6
FY 2015	14	9
FY 2016	20	1
FY 2017	18	5

Note 1: The inventory includes licensing actions and other licensing tasks specifically related to an operating reactor plant transitioning into a decommissioning plant.

Note 2: Similar to the licensing actions and other licensing tasks reported in the yearly CBJ, the inventory does not include unusually complex licensing actions.

Note 3: There were no operating reactor decommissioning transition licensing actions in FY 2006 through FY 2012.

Comments:

Size and Median Age of Decommissioning Licensing Inventory		
Month	Open Licensing Actions (Note 4)	Median Age (Months) (Note 5)
October 2016	25	6
November 2016	26	7
December 2016	29	7
January 2017	28	8
February 2017	29	9
March 2017	30	9
April 2017	30	9.5
May 2017	31	10
June 2017	26	7
July 2017	27	8
August 2017	27	9
September 2017	28	10

The above charts provide information on decommissioning transition licensing actions. This includes a series of licensing actions needed to support operating reactor plants transitioning into decommissioning status. The inventory totals reflect the number of decommissioning transition licensing actions undergoing NRC staff review at the end of each month or fiscal year. The median age corresponds to the open inventory for the respective month or fiscal year.

10-year Inventory of Open Licensing Actions (Decommissioning)		
Fiscal Year	Open Licensing Actions (Note 4)	Median Age (Months) (Note 5)
FY 2008	3	4
FY 2009	4	4.5
FY 2010	6	9.5
FY 2011	11	10
FY 2012	14	4.5
FY 2013	15	6
FY 2014	22	7
FY 2015	26	6
FY 2016	26	8
FY 2017	28	10

Note 4: The table reflects data for all licensing actions related to shut down power reactor plants that have generally completed transitioning from operating to decommissioning status. Minor licensing tasks, such as reviews of reports not requiring NRC approval, were not included.

Note 5: The program goal is to complete major licensing actions in 1 year.

Comments:

These two charts provide information on decommissioning licensing actions for sites that have generally completed the transition from operating to decommissioning status. The totals reflect

the number of decommissioning licensing actions undergoing NRC staff review at the end of each month or fiscal year. The median age corresponds to the open actions for the respective month or fiscal year.

6. Uranium Recovery license and licensing action review inventory and average age, monthly for 1-year rolling metrics and annually for the past 10 years.

Major Uranium Recovery Licensing Action Inventory and Average Age Monthly for 1 Year Rolling (Note 1)		
Month	Number of Actions (Note 2)	Average Age in Months (Notes 3, 4, and 5)
October 2016	7	35
November 2016	7	36
December 2016 (Note 6)	6	41
January 2017	6	42
February 2017	5	42.5
March 2017	4	49.5
April 2017	4	50.5
May 2017	5	41.5
June 2017	5	42.5
July 2017	5	43.5
August 2017	5	44.5
September 2017	5	45.5
Major Uranium Recovery Licensing Action Inventory and Average Age for 10 Years Rolling		
Fiscal Year (FY)	Number of Actions	Average Age in Months
FY 2008	6	9
FY 2009	7	16.5
FY 2010	8	24
FY 2011	5	30.5
FY 2012	8	28.5
FY 2013	8	30
FY 2014	6	32
FY 2015	7	23.5
FY 2016	7	34
FY 2017	5	45.5 (Note 7)
Minor Uranium Recovery Licensing Action Inventory and Average Age Monthly for 1 Year Rolling (Note 8)		
Month	Number of Actions	Average Age in Months
October 2016	30	13
November 2016	29	12.5
December 2016	31	12
January 2017	30	13.5

February 2017	29	14.0
March 2017	29	15
April 2017	27	16
May 2017	27	16
June 2017	25	16.5
July 2017	25	17
August 2017	24	18
September 2017	24	18
Minor Uranium Recovery Licensing Action Inventory and Average Age for 10 Years Rolling		
Fiscal Year (FY)	Number of Actions	Average Age in Months
FY 2008	6	5
FY 2009	8	6
FY 2010	4	8.5
FY 2011	10	8
FY 2012	8	10
FY 2013	9	8
FY 2014	14	8
FY 2015	21	10
FY 2016	28	12.5
FY 2017	24	18 (Note 7)

- Note 1: "Major licensing actions" include new facility applications, license renewals, facility expansions, and restarts.
- Note 2: The size of the inventory is defined as the number of licensing actions undergoing NRC staff review at the end of each month or fiscal year.
- Note 3: The average age corresponds to the age of the inventory open at the end of the respective month or fiscal year, using the date the request was accepted for review as the start date. The average age is rounded to the nearest half of a month.
- Note 4: For major licensing actions, the NRC staff's goal is to complete the reviews in 36 months. This goal assumes that there are no licensee delays in responding to NRC staff requests for additional information (RAIs).
- Note 5: The average age is not strictly limited to the NRC staff's time spent in reviewing the actions; it also includes licensee response times to NRC staff RAIs, which have been up to 24 months for major licensing actions and 11 months for minor licensing actions. Furthermore, the average age does not reflect shifts in allocation of staff resources based on the NRC's priority system for reviews. At times, the NRC staff has had to re-allocate resources from new licensing reviews to support follow-up on emergent issues at operating facilities.
- Note 6: The NRC staff removed the Kendrick expansion application from the inventory in December 2016 following a request from the licensee that the NRC cease all activities related to review. To that point, the NRC staff had performed 10 months of review. The increased average age after December 2016 and in FY 2016, is influenced by significant applicant delays in responding to RAIs, technical complexities in the more recent reviews, and additional efforts necessary to respond to increasing tribal interest during cultural resources reviews. Additional information on these reviews is provided in the NRC staff's response to narrative question 6.

Note 7: the increase in the average age of major and minor uranium recovery licensing actions from FY 2016 to FY 2017 is due to hearings, unique aspects of the review, and availability of NRC staff with pertinent expertise.

Note 8: "Minor licensing actions" include routine amendments, financial surety reviews, transfers of control, and exemption requests.

7. Design certification (DC), combined license (COL), and early site permit (ESP) application review inventory including age and projected completion dates.

Project Name	Project Type	Application Review Start Date	Project Age/Duration as of September 2017 (in months)	Projected or Actual Completion of Staff Review
US-APWR (1)	DC	02/2008	115	TBD
APR1400	DC	03/2015	309	09/2018
ABWR Renewal (2)	DC	02/2011	79	Under Development
Turkey Point (3)	COL	09/2009	96	11/2016
Clinch River	ESP	01/2017	9	08/2019
NuScale	Small Modular Reactor (SMR) DC	03/2017	6	09/2020

Comments:

1. The NRC staff has been performing a review of the US-APWR DC application at a reduced pace under a Mitsubishi Heavy Industries, Ltd. (MHI)-initiated coordinated slowdown of NRC licensing activities for US-APWR.
2. The staff's review of the ABWR DC renewal application has been impacted by the applicant's 2-year delay in submitting Revision 6 of its application and open items that remain unresolved by the applicant.
3. The staff's review of the Turkey Point COL application was completed, the final safety evaluation was issued on schedule, and the contested proceeding was completed and terminated in July. The Commission's mandatory hearing on the COL application previously scheduled for October 5, 2017, was delayed at Florida Power & Light Company's request to allow its personnel time to focus on preparing for and responding to Hurricane Irma. The hearing has been rescheduled for December 12, 2017.

See narrative item #4 for additional details on the status of other applications under review as of September 2017.

8. RAIs issued by each office including NRR, NRO, Uranium Recovery, Decommissioning:

- Number of RAI's issued during each month for each office;
- Number of RAI's completed during each month for each office;
- Number of RAI's open at the end of each month for each office;
- 12-month rolling average number and 3-year rolling average number for each office;
- Amount of contractor hours charged as Part 170 fees preparing and/or reviewing RAI responses; and

- NRC staff hours charged as Part 170 fees preparing and/or reviewing RAI responses.

Office of Nuclear Reactor Regulation

Month	No. of RAIs issued (Note 1)	No. of RAIs completed (Note 2)	No. of RAIs open (Note 3)	Rolling average (Note 4)
December 2016	16	23	34	
January 2017	17	13	38	
February 2017	25	21	30	
March 2017	23	20	36	
April 2017	18	15	30	
May 2017	13	16	24	
June 2017	11	24	21	
July 2017	23	22	27	
August 2017	31	11	36	
September 2017	13	14	39	

Note 1: The number of RAIs issued for NRR reflects the number of RAI transmittals to licensees each month. Each transmittal (e.g., letter, e-mail) may contain multiple RAI questions for the same regulatory action.

Note 2: The number of RAIs completed for NRR reflects the number of RAI responses from licensees each month. Each response (e.g., letter) may contain responses to multiple RAI questions.

Note 3: The number of RAIs open for NRR reflects the number of RAI transmittals that licensees have not provided a response to as of the end of each month.

Note 4: NRR does not have a data system capable of providing a historical 12-month rolling average number and 3-year rolling average number of RAIs. Compiling this information would have to be done via manual document searches and analysis which would have a significant impact on staff resources. We can calculate the rolling average going forward.

Office of New Reactors

Project Name	Project Type	No. of RAIs Issued in September 2017	No. of RAIs Completed in September 2017	No. of RAIs Open at the end of September 20 17	Rolling Average (Note 1)
US-APWR	DC	0	0	115	
APR1400	DC	3	18	297	
ABWR Renewal	DC	0	0	7	
Turkey Point	COL	0	0	0	
Vogtle	License Amendment	2	2	2	
Clinch River	ESP	1	0	5	
NuScale (Note 2)	SMR DC	47	6	251	
NuScale	Topical Reports	16	0	56	
Westinghouse	Topical Reports	0	0	0	

Project Name	Project Type	No. of RAIs Issued in September 2017	No. of RAIs Completed in September 2017	No. of RAIs Open at the end of September 2017	Rolling Average (Note 1)
(Note 3)					

Comments:

Note 1: The NRC’s NRO does not have readily available a historical 12-month rolling average number and 3-year rolling average number of RAIs. Accurately compiling 12-month rolling averages and 3-year rolling averages would require manual document searches and analysis for several applications, including many that are no longer under review.

Note 2: NuScale requested 60 days to respond to RAIs, which is an extension from the NRC’s normal 30-day RAI response time.

Note 3: At the applicant’s request, the NRC staff’s review of the AP1000 Specialized Seismic Option Topical Report has been suspended. The two RAIs that were previously issued to the applicant have not been formally closed; however, these RAIs will no longer be tracked in this metric while the topical report is in a suspended status.

Office of Nuclear Material Safety and Safeguards

Number of Uranium Recovery Licensing Action RAIs Issued, Closed, and Open				
Month	No. of RAIs Issued (Note 1)	No. of RAIs Completed (Note 2)	No. of RAIs Open (Note 3)	Rolling Average (Note 4)
December 2016	0	1	9	
January 2017	0	0	9	
February 2017	0	0	9	
March 2017	0	1	8	
April 2017	3	2	9	
May 2017	0	0	9	
June 2017	3	0	12	
July 2017	1	2	11	
August 2017	1	2	10	
September 2017	0	0	10	

Office of Nuclear Material Safety and Safeguards (continued)

Number of Decommissioning Power Reactor Licensing Action RAIs Issued, Closed, and Open				
Month	No. of RAIs Issued (Note 1)	No. of RAIs Completed (Note 2)	No. of RAIs Open (Note 3)	Rolling Average (Note 4)
December 2016	0	0	1	
January 2017	0	0	1	
February 2017	0	0	1	

March 2017	0	0	1	
April 2017	0	0	1	
May 2017	2	0	3	
June 2017	0	1	2	
July 2017	0	2	0	
August 2017	2	0	2	
September 2017	0	1	1	

Note 1: In this table, the number of RAIs issued reflects the number of RAI transmittals to licensees or applicants each month. Each transmittal (e.g., letter, e-mail) may contain multiple RAI questions for the same licensing action.

Note 2: The number of RAIs closed reflects the number of RAI responses received from licensees or applicants each month. Each response (e.g., letter) may contain responses to multiple RAI questions.

Note 3: The number of RAIs open reflects the number of RAI transmittals to which licensees or applicants have not provided a response as of the end of each month.

Note 4: NMSS does not have a data system capable of providing a historical 12-month rolling average number, or a 3-year rolling average number of RAIs. Compiling this information would have to be done via manual document searches and analysis, which would have a significant impact on staff resources. We can calculate the rolling average going forward once sufficient data have been collected.

Part 170 Fees

For all offices, staff and contractor review of licensee applications is charged to specific cost activity codes (CACs). However, the same CAC is used for all aspects of the review and does not differentiate between time spent preparing and/or reviewing RAI responses from time spent performing other aspects of the review (e.g., time spent preparing the safety evaluation).

9. Reactor Oversight Process Findings year-to-date and 3-year rolling metrics, total and by region for green, white, yellow, and red findings.

Location	# of Findings	2014	2015	2016	2017 YTD
Nationally	Total	824	821	704	284
	NSIR (all regions)	18	26	19	0
RI	Green	167	169	155	59
	White	3	4	2	2
	Yellow	0	1	0	0
	Red	0	0	0	0
	GTG Security	1	1	0	0
	Total	171	175	157	61
	# OP Units	26	25	25	25
R2	Green	148	159	151	49
	White	4	1	0	2
	Yellow	0	0	0	0
	Red	0	0	0	0
	GTG Security	0	0	1	1
	Total	152	160	152	52
	# OP Units	32	32	33	33
R3	Green	221	202	177	69
	White	4	5	1	3
	Yellow	0	0	0	0
	Red	0	0	0	0
	GTG Security	1	1	1	0
	Total	226	208	179	72
	# OP Units	23	23	23	23
R4	Green	249	248	196	96
	White	5	2	1	2
	Yellow	2	1	0	0
	Red	0	0	0	0
	GTG Security	1	1	0	1
	Total	257	252	197	99
	# OP Units	19	19	19	19

* GTG Security: Greater-than-green security; #OP Units: Number of operating units;
 NSIR: Office of Nuclear Security and Incident Response

Comments:

Current year data lag by approximately one quarter.

10. Percentage of Final Significance Determinations Made within 90 Days for All Potentially Greater-Than-Green-Findings, monthly for one-year rolling metrics and annually for the past 10 years.

1-Year Rolling Metric	
Month	Percent Met
October 2016	N/A
November 2016	N/A
December 2016	100
January 2017	N/A
February 2017	100
March 2017	N/A
April 2017	100
May 2017	100
June 2017	N/A
July 2017	100
August 2017	100
September 2017	N/A

10-Year Annual Determinations Within 90 Days	
Year	Percent Met
2008	100
2009	100
2010	93
2011	100
2012	100
2013	100
2014	86
2015	88
2016	100
2017	100

Comments:

This metric is reported in the NRC's CBJ and measures the time from the issuance date of the first official correspondence that describes the inspection finding, until the final significance determination letter is sent to the licensee, which is expected to be 90 days or less.

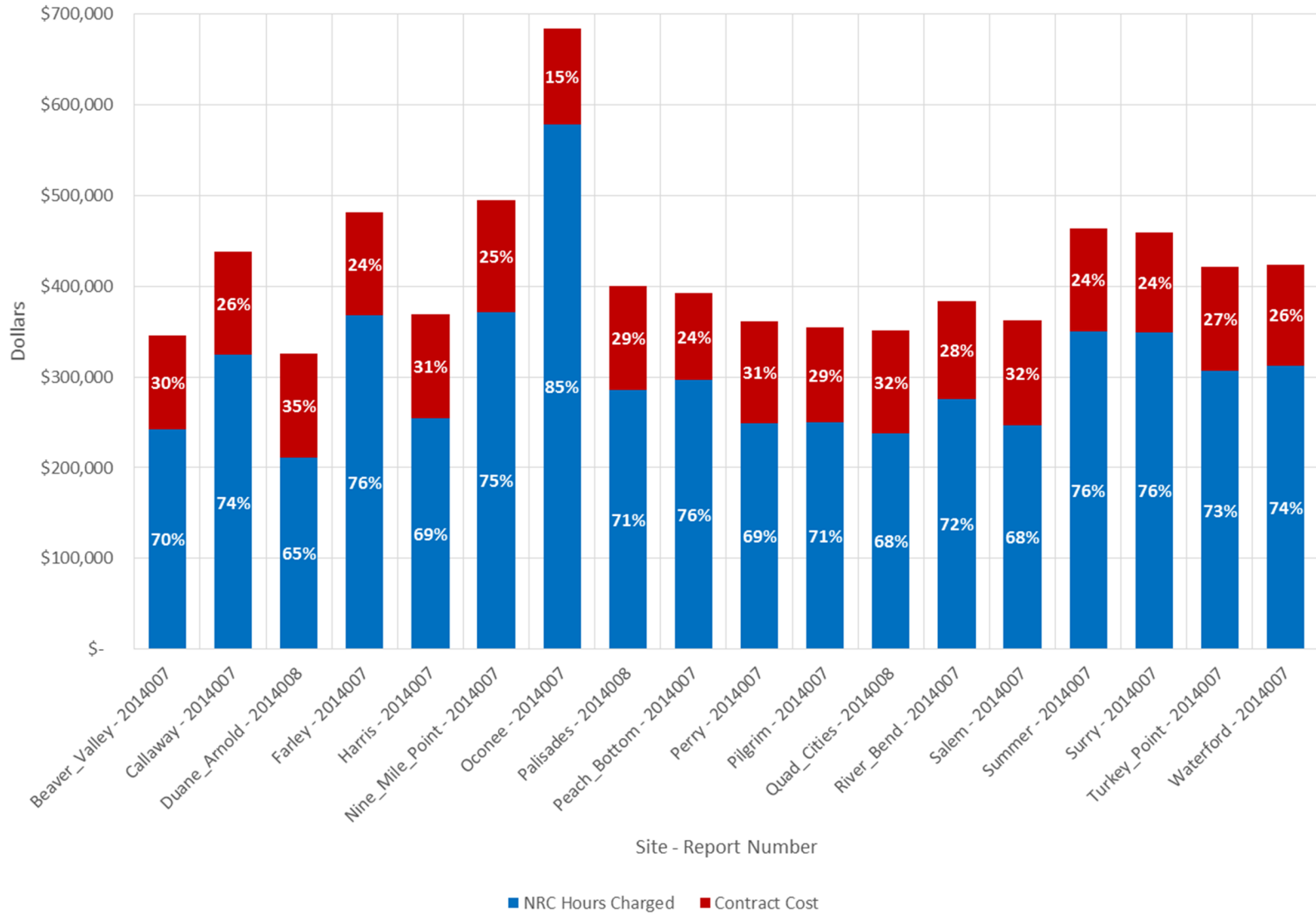
11. Component Design Basis Inspection (CDBI) duration, fees, and percentage of fees used to reimburse contractors - monthly averages for 3-year rolling metrics.

The fees associated with CDBIs are grouped per CDBI inspection in order to allow easier review by the reader and facilitate comparison between the cost of CDBIs performed at each site. Monthly comparison of CDBI fees will not provide an accurate representation of the CDBI charges for each licensee due to the inspection period of the CDBIs spanning 2 months, and because many of the CDBIs are performed during the non-outage summer months.

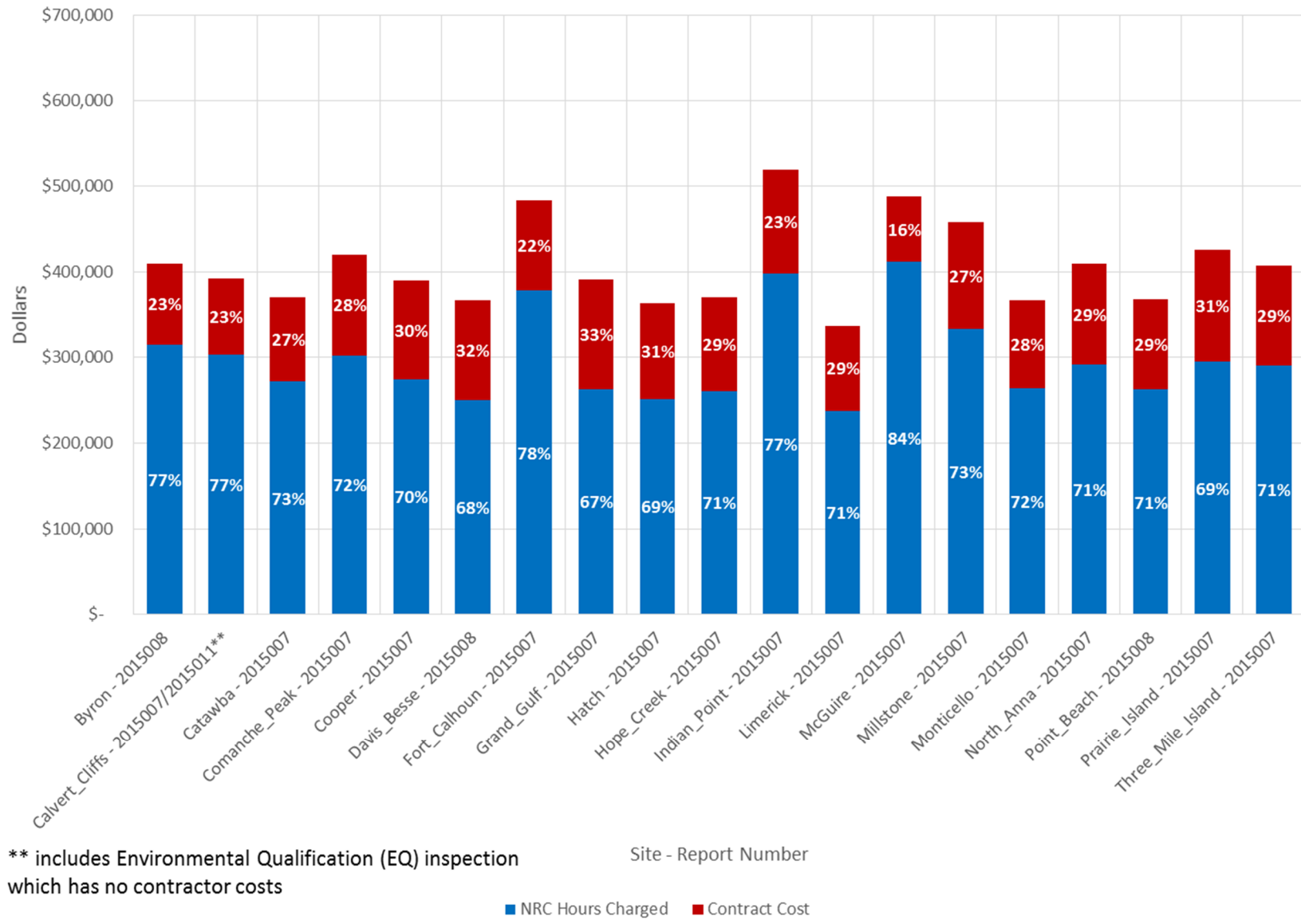
Notes:

1. CDBIs are now called Design Bases Assurance (DBA) inspections. More information can be found in the response to narrative question #21.
2. In previous versions of the plot titled "2016 CDBI Costs, By Inspection Report," it was noted that two ongoing inspections at Arkansas Nuclear One and Waterford were not shown. That note was in error and should have read "Not shown are two ongoing inspections at Arkansas Nuclear One and Brunswick." Since that time, both inspections have been completed and are now included on the 2016 plot.

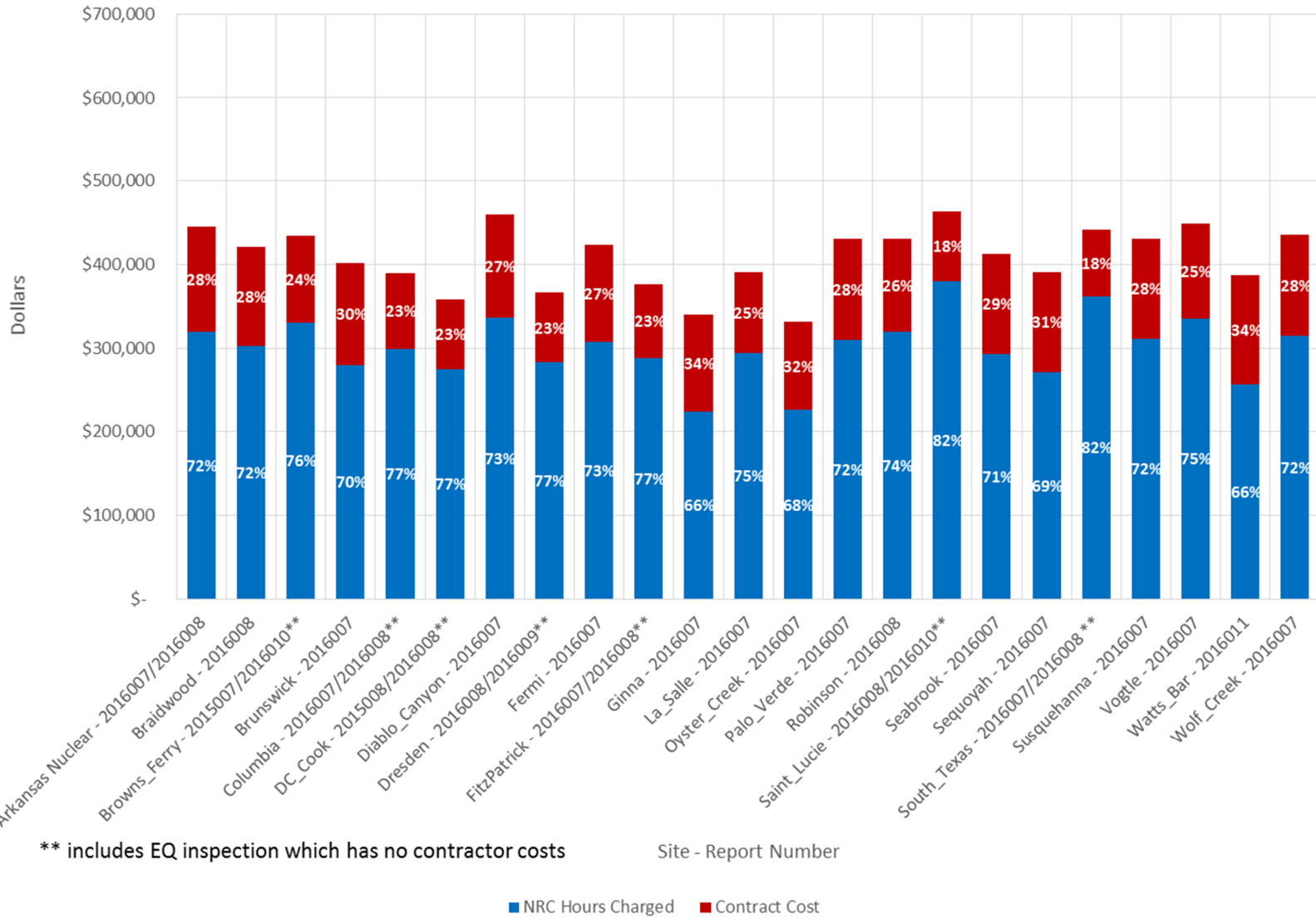
2014 Design Bases Assurance (DBA) Inspections Costs, Shown Alphabetically By Site



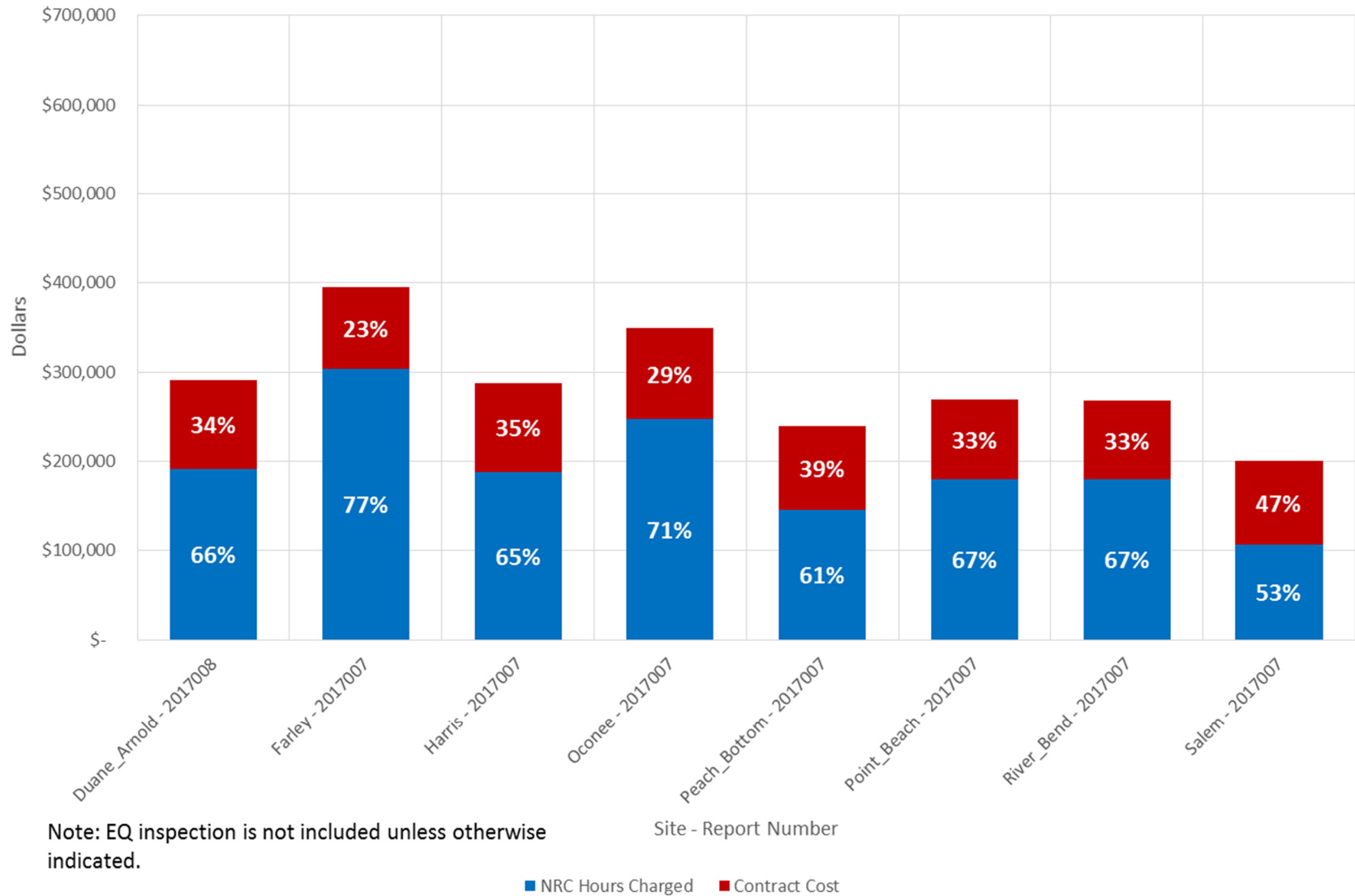
2015 Design Bases Assurance (DBA) Inspections Costs, Shown Alphabetically By Site



2016 Design Bases Assurance (DBA) Inspections Costs, Shown Alphabetically By Site



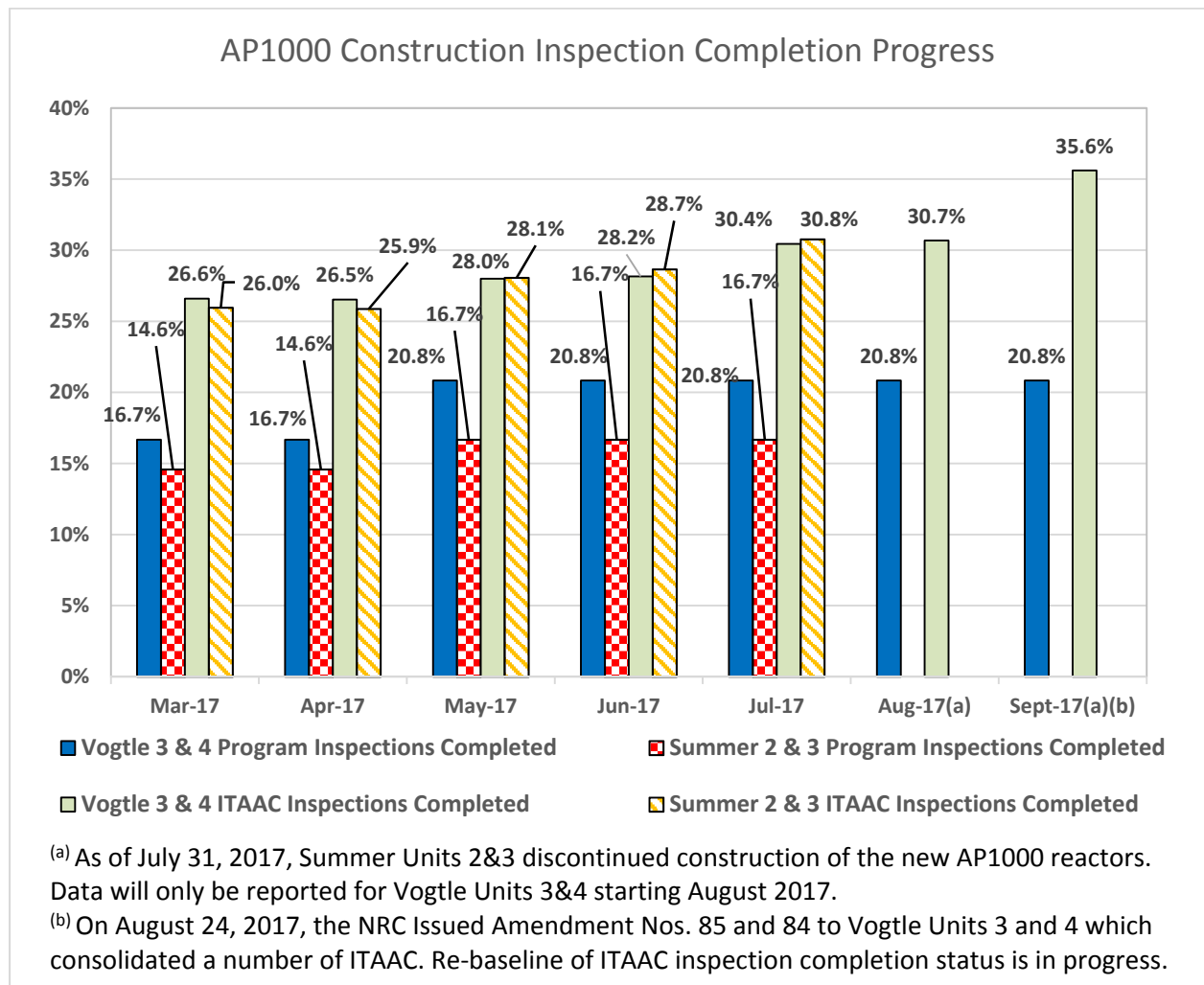
2017 Design Bases Assurance (DBA) Inspections Costs, Shown Alphabetically By Site



12. New reactor licensing and inspection status for Vogtle 3 & 4 and Summer 2 & 3 including the percentage of NRC inspections completed and the percentage of Inspections, Tests, Analysis, and Acceptance Criteria (ITAAC) reviews completed within 30 days.

Project Name	Project Type	Licensing Status
Vogtle Unit 3	Combined License Holder	COL issued on 02/10/2012
Vogtle Unit 4	Combined License Holder	COL issued on 02/10/2012
V.C. Summer Unit 2	Combined License Holder	COL issued on 03/30/2012
V.C. Summer Unit 3	Combined License Holder	COL issued on 03/30/2012

New Reactor Inspection Status:

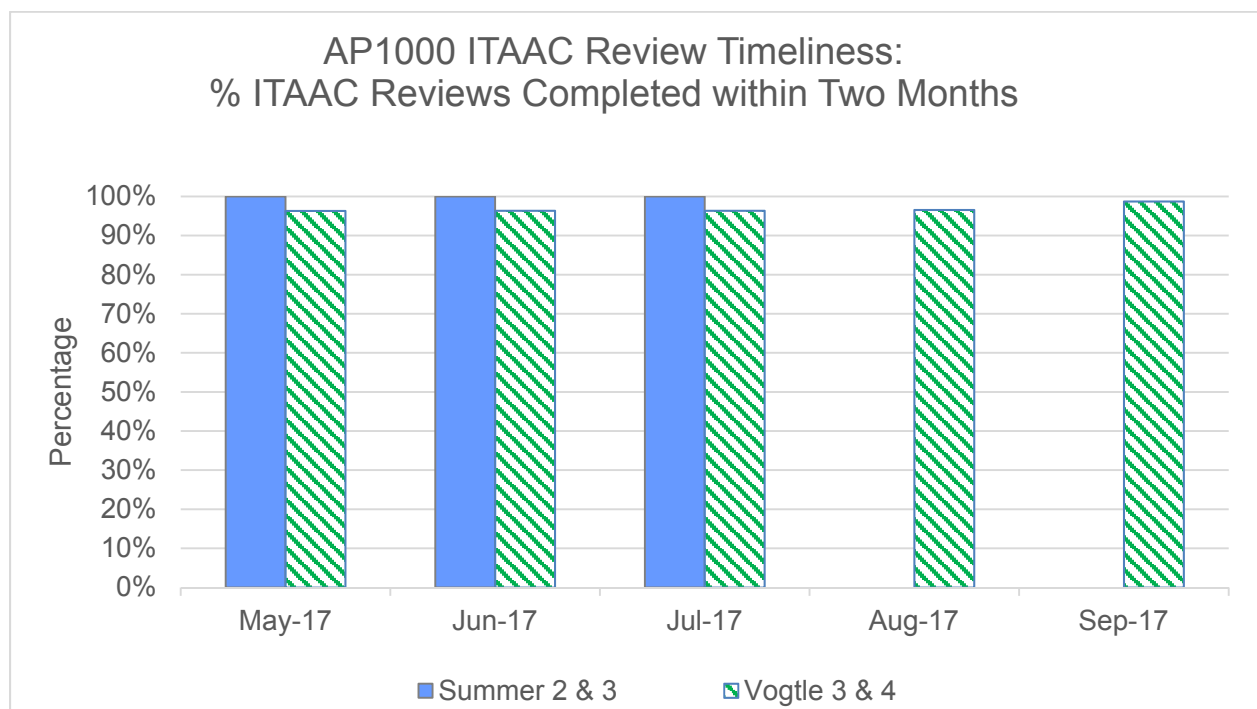


Comments:

This graph represents the percentage of NRC inspections associated with ITAAC that have been completed since the start of construction with respect to the total number of inspections required for the Vogtle and Summer facilities. As of July 31, 2017, South Carolina Electric & Gas Co. (SCE&G) and Santee Cooper discontinued construction of the new reactors at the V.C. Summer site and as a result, beginning on August 1, 2017, NRC no longer conducts ITAAC or operational program inspections for Summer Units 2 & 3.

The completed inspection status closely mirrors the completion status of the licensees' work activities associated with systems, structures, and components that have ITAAC. The ITAAC completion status is updated once inspections are completed and approved. These inspections are documented in quarterly reports. Planned inspection activities are constantly evaluated and updated to ensure they align with licensee's work activities. Specifically, on August 24, 2017, the NRC issued Amendment Nos. 85 and 84 to Vogtle Units 3 & 4, respectively. The amendments changed the Combined License (COL) Appendix C (and plant-specific design control document Tier 1) to consolidate a number of ITAAC to improve the efficiency of the ITAAC completion and closure process. As a result, a re-baseline of ITAAC inspection completion status is in progress.

This graph also reports the percentage of completed program inspections since the start of construction for these facilities, which include both programs required for construction and programs required for operation of the facilities. There are a total of five construction programs, which include Quality Assurance, Fitness for Duty, and ITAAC Management. In addition, there are a total of 20 operational programs, which include Fire Protection, Emergency Preparedness, Reactor Operator Training, and Security. The graph depicts the percentage of planned inspections that are completed, and does not account for the level of effort required for inspections.



Comments:

This graph plots the percentage of ITAAC closure notifications (ICNs) that have been completed within 2 months of submittal for Summer 2 & 3 and Vogtle 3 & 4. A two month time period was used instead of the requested 30 days to be consistent with an existing agency performance indicator. The current ICN review process provides up to 2 months to allow time for staff to perform an adequate review and to engage with the licensee in public forums as necessary. These interactions have proven to be very productive as the agency and industry gain experience with the ICN process. The agency continually assesses the ICN review

process to ensure an effective and timely review is achieved, especially as licensees approach fuel load.

On July 31, 2017 SCE&G and Santee Cooper, the licensee for V.C. Summer Units 2 & 3, announced the decision to discontinue construction on Units 2 & 3, and asked the NRC to cease work, and the NRC has complied. The NRC continues to evaluate the impact of this announcement on its programs and resources. While work in support of Summer construction activities stopped completely in August, the staff is continuing work on Vogtle construction related activities. The information represented in the graph reflects the work performed for the construction site in Georgia (Vogtle Units 3 & 4). The graph for the V.C. Summer construction site reflects the work completed as of July 31, 2017, which included 15 ICNs that were submitted for NRC review but the staff will no longer be reviewing them.

In spring 2017, the NRC completed a demonstration project to evaluate the readiness and reliability of the ITAAC inspection and verification processes. The purpose of the project was to enhance the NRC's ITAAC processes and to identify any gaps in preparation for the anticipated surge in ITAAC notifications towards the end of construction. The recommendations identified in this demonstration project are expected to result in enhancements to improve current NRC processes in support of the NRC's finding on whether the acceptance criteria in the ITAAC are met, which is required prior to fuel load. The NRC has begun to address these recommendations, and is making other refinements to the ITAAC closure verification process as discussed below.

Key enhancements were made to the public websites with information on the new units under construction to provide a convenient "portal" for stakeholders to find ITAAC-related information, including links to the ITAAC hearing procedures and, guidance on ITAAC closure, and as well as other upgrades to links for faster access to information, such as Departure Reports and License Amendments.

In anticipation of the surge in ITAAC closure notifications (ICNs) near the end of construction, additional NRO staff have successfully completed cross-qualifying as ICN reviewers, and can be utilized as needed for these reviews.

In October 2016 the NRC staff began to review "uncompleted" ITAAC notifications (UIN's). Reviewing UINs allows the staff to consider a licensee's proposed method for closing an ITAAC, which accomplishes a significant amount of the review work in advance. Staff found that the early UIN review process provides earlier communication to public stakeholders and allows earlier identification of issues related to ITAAC completion. The staff expects to expend fewer resources and less time to complete its final review of an ICN that verifies a NRC-accepted method to close an ITAAC.

On August 24, 2017, the NRC approved a Southern Nuclear Company license amendment to consolidate the number of ITAAC to improve the efficiency of the ITAAC completion and closure process. This amendment reduces the number of individual ITAAC by about 230 per plant with no reduction in safety while maintaining the technically robust nature of the ITAAC.

13. Committee for the Review of Generic Requirements (CRGR) – please provide lists of the issues formally and informally reviewed including the CRGR recommendations on each. Please provide 12-month and 3-year rolling averages for the following metrics:
- a. For the number of issues reviewed formally: the percentage accepted for imposition on industry and the percentage rejected based on cost-benefit or backfit concerns; and
 - b. For the number of issues reviewed informally: the percentage accepted for imposition on industry and the percentage rejected based on cost-benefit or backfit concerns.

Summary of CRGR Reviews Performed over the 3-Year Period					
#	TOPIC	TYPE OF REVIEW	DATE	ENDORING DOCUMENT	RESULT
1	Proposed Task Interface Agreement (TIA) Response 2014-05, “Design Analysis for Single Failure and the Integration of Class 1E Direct Current Control Cabling in Raceways with High Energy Power Cabling at the Oconee Nuclear Station Units 1, 2 and 3”. (ADAMS Accession No. ML17264A031).	Formal Review	09/26/17	CRGR #448, Pending Final Meeting Minutes for CRGR #449 , October 24, 2017 Public Meeting	Under review
2	Proposed Regulatory Issue Summary (RIS), “Disposition of Information Related to the Time Period that Safety-Related SSCs [Structures, Systems, and Components] are Installed” (ADAMS Accession No. ML17177A060).	Formal Review	09/12/17 09/14/17	CRGR #446, #447	Under review
3	Proposed Revision of Management Directive (MD) 8.4, “Management of Backfitting, Issue Finality, and Information Collection”	Formal Review	09/05/17	CRGR #445, ADAMS Accession No. ML17276B137	Endorsed
4	Proposed Rule – Cyber Security at Fuel Cycle Facilities (RIN 3150-AJ64; NRC-2015-0179) (Agencywide Documents Access and Management System (ADAMS) Accession No. ML17145A342)	Formal Review ²	07/12/17 06/27/17	CRGR #442, #444 ADAMS Accession No. ML17200A101	Endorsed (with backfitting Part 70 fuel cycle facility licensees)
5	RG 5.77, "Insider Mitigation Program" (Official Use Only – Safety Related Information)	Formal Review	07/11/17	CRGR #443 ADAMS Accession No. ML17192A480	Endorsed

² CRGR formal review is a review that is conducted in a formal setting and attendance is to include the CRGR members and the division director or his or her delegate and the supporting staff. Alternatively, an informal review is conducted by the CRGR technical staff for various generic documents such as Regulatory Issue Summaries and provides the document and recommendations to the CRGR Chairman and the members for consideration. If concerns exist or unintended backfitting requires further discussion, then a formal review will ensue that entails the staff appearing before the CRGR members in a formal setting.

6	Subsequent License Renewal (SLR) guidance documents: NUREG-2192, "Standard Review Plan for Review of Subsequent License Renewal Applications for Nuclear Power Plants," NUREG-2191, Volume 1, "Generic Aging Lessons Learned for Subsequent License Renewal (GALL-SLR) Report," and NUREG-2191, Volume 2, "Generic Aging Lessons Learned for Subsequent License Renewal (GALL-SLR) Report." (ADAMS Accession No. ML17187A031 and ML17187A204)	Informal Review	04/10/17	Internal Summary Email	Endorsed
7	Proposed Regulatory Issue Summary (RIS) 2016-11 Requests to Dispose of Very LLRW Pursuant to Title 10 of the <i>Code of Federal Regulations</i> (10 CFR) 20.2002 (ADAMS Accession No. ML16007A488)	Informal Review	09/08/16	Internal Summary Email	Endorsed
8	Proposed RIS 2016-xx, Clarifications on Security Compensatory Measures Requirements (ADAMS Accession No. ML15040A596 – March 1, 2016, Draft RIS)	Informal Review	08/11/16	Internal Summary Email	Withdrawn by requesting office
9	Proposed RIS 2016-10, "License Amendment Requests for Changes to Emergency Response Organization Staffing and Augmentation" (ADAMS Accession No. ML16124A002)	Informal Review	06/30/16	Internal Summary Email	Endorsed
10	Backfit Evaluation Document, "Evaluation for Compliance Backfit Exception: Open Phase Condition Design Vulnerability in Electric Power System" (ADAMS Accession No. ML15254A208)	Formal Review	05/17/16	CRGR #441 ADAMS Accession No. ML16145A431	Endorsed
11	Proposed RIS 2016-07, "Containment Shell or Liner Moisture Barrier Inspection" (ADAMS Accession No. ML16068A436)	Informal Review	04/19/16	Internal Summary Email	Endorsed
12	Proposed RIS 2016-04, "Clarification of 10 CFR 50.46 Reporting Requirements and Recent Issues with Related Guidance not Approved for Use by the NRC" (ADAMS Accession No. ML15324A296)	Informal Review	03/30/16	Internal Summary Email	Endorsed

13	Proposed RIS 2016-01, "NEI Guidance for the Use of Accreditation in Lieu of Commercial Grade Surveys for Procurement of Laboratory Calibration and Test Services" (ADAMS Accession No. ML15323A346)	Informal Review	02/08/16	Internal Summary Email	Endorsed
14	Interim Staff Guidance, "Guidance for the Evaluation of Acute Chemical Exposures and Quantitative Standards" (ADAMS Accession No. ML15293A314)	Formal review	01/28/16	CRGR #440 ADAMS Accession No. ML16032A047	Endorsed
15	Proposed RIS 2015-15, "Information Regarding a Specific Exemption in the Requirements for the Physical Protection of Category 1 and Category 2 Quantities of Radioactive Material" (ADAMS Accession No. ML15092A432)	Informal Review	11/20/15	Internal Summary Email	Endorsed
16	Proposed RIS 2015-15, "Information Regarding a Specific Exemption in the Requirements for the Physical Protection of Category 1 and Category 2 Quantities of Radioactive Material" (ADAMS Accession No. ML15092A432)	Informal Review	11/20/15	Internal Summary Email	Endorsed
17	Proposed RIS 2015-11, "Protective Action Recommendations for Members of the Public on Bodies of Water" (ADAMS Accession No. ML15216A300)	Informal Review	09/18/15	Internal Summary Email	Endorsed
18	Proposed RIS 2015-11, "Protective Action Recommendations for Members of the Public on Bodies of Water" (ADAMS Accession No. ML15216A300)	Informal Review	09/18/15	Internal Summary Email	Endorsed
19	Proposed RIS 2016-05, "Embedded Digital Devices in Safety-Related Systems" (ADAMS Accession No. ML115118A015)	Informal Review	09/18/15	Internal Summary Email	Predecisional at the time Issued following Commission Decision on April 29, 2016. See Item #14

20	Proposed RIS 2015-10, "Applicability of ASME Code Case N-770-1 as Conditioned in 10 CFR 50.55a, 'Codes and Standards,' to Branch Connection Butt Welds" (ADAMS Accession No. ML15068A131)	Formal review	07/07/15	CRGR #439 ADAMS Accession No. ML15189A085	Endorsed
21	Proposed RIS 2015-10, "Applicability of ASME Code Case N-770-1 as Conditioned in 10 CFR 50.55a, 'Codes and Standards,' to Branch Connection Welds" (ADAMS Accession No. ML15068A131)	Informal Review	05/11/15	Internal Summary Email	Informal Review - Elevated to Formal Review in Item #17 (CRGR Meeting #439)
22	Proposed RIS 2015-11, "Protective Action Recommendations for Members of the Public on Bodies of Water" (ADAMS Accession No. ML15216A300)	Informal Review	05/07/15	Internal Summary Email	Endorsed - Separate Informal Review Completed Following Review of Public Comment in Item #13
23	Review of Proposed RIS 2016-05, "Embedded Digital Devices in Safety-Related Systems" (ADAMS Accession No. ML15118A015)	Informal Review	04/25/15	Internal Summary Email	Endorsed
24	Generic Letter (GL) 2016-01, "Monitoring of Neutron-Absorbing Materials in Spent Fuel Pools" (ADAMS Accession No. ML16097A169)	Formal Review	04/01/15	CRGR #438 ADAMS Accession No. ML15092A656	Endorsed
25	Proposed RIS 2015-06, "Tornado Missile Protection" (ADAMS Accession No. ML15020A419)	Formal Review	03/25/15	CRGR #437 ADAMS Accession No. ML15090A373	Endorsed
26	Generic Letter (GL) 2015-01, "Treatment of Natural Phenomena Hazards in Fuel Cycle Facilities" (ADAMS Accession No. ML14328A029)	Formal Review	02/26/15	CRGR #436 ADAMS Accession No. ML14092A344	Endorsed

27	Proposed RIS 2015-10, "Applicability of ASME Code Case N-770-1 as Conditioned in 10 CFR 50.55a, 'Codes and Standards,' to Branch Connection Butt Welds" (ADAMS Accession No. ML15068A131)	Informal Review	02/03/15	Internal Summary Email	Deferred - Following Public Comments, Formal and Informal CRGR reviews were performed on 07/07/15 (Item # 17) and 05/11/15 (Item # 18), Respectively.
28	Proposed RIS 2015-01, "Qualification Requirements for Bolt and Stud Non-Destructive Examinations" (ADAMS Accession No. ML14169A612)	Informal Review	01/14/15	Internal Summary Email	Endorsed
29	Proposed RIS 2015-10, "Applicability of ASME Code Case N-770-1 as Conditioned in 10 CFR 50.55a, 'Codes and Standards,' to Branch Connection Butt Welds" (ADAMS Accession No. ML15068A131)	Informal Review	10/06/14	Internal Summary Email	Endorsed

12-Month Summary of CRGR Reviews Decisions of Potential Backfit Issues			
Review Type & Outcome	Percentage Accepted or Endorsed with Backfitting	Percentage Rejected Based on Backfit Concerns	Percentage Endorsed without Backfitting
Informal Reviews	0.0%	0.0%	100.0%
Formal Reviews	33.0%*	0.0%	67.0%*

3-Year Summary of CRGR Review Decisions of Potential Backfit Issues			
Review Type & Outcome	Percentage Accepted or Endorsed with Backfitting	Percentage Rejected Based on Backfit Concerns	Percentage Endorsed without Backfitting
Informal Reviews	0.0%	0.0%	100.0%
Formal Reviews	22.0%*	0.0%	78.0%*

*Item # 1 (CRGR #448, #449) and item # 2 (CRGR #446, CRGR #447) are not counted in the percentage calculations because they are still under review. These items will be updated in the next report.

Comments:

1. As of September 30, 2017, for the rolling 3-year period, the CRGR has conducted 29 reviews for potential backfits, including 18 informal reviews and 11 formal reviews, with two associated public meetings. During this period, CRGR reviewed two topics, which supported potentially requiring a backfit. Except for the items mentioned above, the remaining topics reviewed by CRGR were endorsed to contain no backfit implications for licensee facilities.
2. These tables provide a summary of CRGR reviews results for the rolling 3-year and 12-month periods. The percentage accepted includes CRGR endorsements of generic documents that may lead to licensee backfits, the percentage rejected are reviews in which the CRGR disapproved documents due to backfit concerns, and the percentage endorsed were reviews in which the CRGR found no backfit implications.

NARRATIVE INFORMATION

1. Status of License Renewal Reviews.

Applicant	Application Accepted for Review	Review status for long-term application reviews
Indian Point 2&3*	08/01/2007	<p>The NRC staff is working to address public comments received on its draft second supplement to the final supplemental environmental impact statement, which was issued for comment in December 2015. The staff's response to the public comments will be documented in the final supplemental environmental impact statement (FSEIS) supplement, which is currently under final review. The initial Safety Evaluation Report (SER) was issued in November 2009, with supplements issued in August 2011 and July 2015. A third SER supplement will be issued in the second quarter of FY 2018 to address new information received by the staff concerning safety issues. In January 2017, the parties to the legal proceedings reached an agreement that resulted in the withdrawal of all contentions on the license renewal application. Under the agreement, Units 2 & 3 will cease operations in April 2020 and 2021, respectively, with possible extensions to operate until April 2024 and 2025, respectively. On February 8, 2017, the State of New York Department of Environmental Conservation (NYDEC) and Riverkeeper filed an unopposed motion to the ASLB to withdraw their contentions and terminate the adjudicatory proceeding. The Licensing Board granted that motion and terminated the adjudicatory proceeding on March 13, 2017. Recently the National Marine Fisheries Service (NMFS) designated critical habitat in the Hudson River for Atlantic Sturgeon. Ongoing interactions between the NRC staff, NMFS, NYDEC, and Entergy regarding this new designation and Entergy's monitoring plan for sturgeon are expected to be completed in the second quarter of FY 2018. Resolution of this issue will be documented in the Record of Decision issued in conjunction with the renewed operating licenses for the units. A decision on the renewed operating licenses for both units is expected to be issued in the 4th quarter of FY 2018.</p>
Diablo Canyon 1&2	01/21/2010	<p>In May 2011 and July 2016 the NRC suspended the license renewal review. In June 2016 Pacific Gas and Electric announced that it had reached an agreement with interested parties not to seek license renewal for Units 1 and 2 and asked the NRC to suspend its review, pending approval of the agreement by the California Public Utilities Commission (CPUC). The applicant has stated that if the CPUC approves the agreement, it will withdraw its LRA.</p>

Seabrook 1	07/21/2010	The NRC staff continues to work with NextEra to ensure technical issues for closure of the alkali silica reaction (ASR) open item in the SER are properly addressed. In August 2016 NextEra submitted a license amendment request to revise the current licensing basis to adopt a methodology for the analysis of seismic Category I structures with concrete affected by ASR. This methodology is the basis for the aging management program being evaluated during the license renewal application review. The review of this license amendment is ongoing and has a direct impact on the schedule for the license renewal review. In addition, on October 6, 2017, the Licensing Board granted a hearing request admitting one contention on NextEra's license amendment request. This issue remains pending before the Board. Therefore, the current schedule for remaining license renewal milestones is to be determined.
Waterford	05/31/2016	The NRC staff continues work on the safety and environmental reviews and has been working on specific questions regarding the Waterford neutron fluence time-limited aging analysis. The NRC staff continues to work with the applicant to resolve this issue.
River Bend	08/07/2017	The River Bend application was received on May 31, 2017, and accepted for review on August 14, 2017, following a supplement from the applicant. The application review schedule has been established of approximately 18 months. The staff continues to work on the safety and environmental reviews. A petition to intervene and request for hearing has been filed. A Licensing Board has been established to rule on the petition and to preside over any hearing that may be held.

2. Status of SLR Readiness.

The Commission has affirmed that no revisions to either the safety or environmental regulations are needed to support the assessment of a SLR application. However, the Commission directed the staff to continue to update license renewal guidance, as needed, to provide additional clarity on the implementation of the license renewal regulatory framework. The main guidance documents for license renewal are:

- Standard Review Plan for Review of License Renewal Applications for Nuclear Power Plants (SRP-LR), Revision 2;
- GALL Report, Revision 2; and
- Standard Review Plan for Environmental Reviews for Nuclear Power Plants, Supplement 1: Operating License Renewal (Revision 1).

The guidance in these documents is based on plant operation up to 60 years. The staff evaluated this guidance to determine what, if any, revisions were necessary to address issues up to 80 years of plant operation for SLR. The staff determined that no revisions were needed to the NRC guidance document entitled, "Standard Review Plans for Environmental Reviews for Nuclear Power Plants," to support environmental reviews from 60 to 80 years. However, the staff determined that the GALL Report and the SRP-LR should be updated to facilitate the more effective and efficient review of SLR applications.

On July 14, 2017, the NRC published “Generic Aging Lessons Learned for Subsequent License Renewal (GALL-SLR) Report” (NUREG-2191, Volumes 1 and 2), and “Standard Review Plan for Review of Subsequent License Renewal Applications for Nuclear Power Plants” (SRP-SLR) (NUREG-2192).

On May 17, 2017, an unnamed licensee submitted a proprietary letter of intent to submit a SLR application by the end of 2017. On November 6, 2015, Dominion Virginia Power notified the NRC of its intent to submit an SLR application in early 2019 for Surry Power Station (ADAMS Accession No. ML15314A078). On June 7, 2016, Exelon Generation Company, LLC notified the NRC of its intent to submit an application for Peach Bottom Atomic Power Station in the third quarter of 2018 (ADAMS Accession No. ML16159A115).

3. Status of power uprate application reviews.

The NRC staff currently has the following power uprate applications under review:

- The Browns Ferry, Units 1, 2 & 3, extended power uprate application was accepted for review on January 11, 2016. The NRC’s Advisory Committee on Reactor Safeguards (ACRS) concluded its review of the application during the June 2017 full committee meeting with a recommendation to the Commission to approve the licensees proposed extended power uprate. The NRC staff completed its safety evaluation and the amendments were issued on August 14, 2017.
- The Peach Bottom Units 2 & 3, measurement uncertainty recapture uprate application was accepted for review on March 20, 2017. The NRC staff is currently performing its safety evaluation. As noted in graphical item #4, the NRC approved an EPU for Peach Bottom Units 2 & 3, in 2014.
- The Hope Creek Generating Station, measurement uncertainty recapture uprate application was accepted for review on August 9, 2017. The staff is currently performing an acceptance review of the application.

4. Status of DC, COL, and ESP applications.

DC Applications

US-APWR

MHI submitted its US-APWR DC application on December 31, 2007. By letter dated November 5, 2013, MHI initiated a coordinated slowdown of NRC licensing activities in order to focus its resources towards supporting the restart of Mitsubishi designed reactors in Japan following the Fukushima event. The NRC staff has been performing the review of the US-APWR DC application at a reduced pace and will continue at this reduced pace until further notice from the applicant. A completion date is not known at this time.

APR1400

On December 23, 2014, Korea Electric Power Corp. and Korea Hydro & Nuclear Power Co., Ltd., submitted to the NRC its application for the certification of the APR1400 standard plant design for use in the U.S. domestic energy market. The Phase 2 review (issuing an SER with open items) for all chapters of the application was completed by the staff in May 2017 and the Phase 3 review (ACRS review of SER with open items) was completed in June 2017. The staff

is currently in Phase 4 of its review (issuing the advanced SER with no open items). The final SER is projected to be issued in September 2018.

NuScale

On January 6, 2017, NuScale submitted the first SMR DC application for review by the NRC. On March 15, 2017, the NRC completed its acceptance review and concluded that the application was acceptable for review and docketed the application. The staff issued the acceptance review letter to NuScale on March 23, 2017, and developed a full review schedule with public milestones that was transmitted to NuScale on May 22, 2017. The staff's review is currently in Phase 1 (preparing the preliminary SER and issuing RAIs) and Phase 2 (issuing an SER with open items). The staff has identified 17 critical issues that require resolution. The final SER is scheduled to be completed in September 2020.

DC Renewal Applications

ABWR Renewal (General Electric-Hitachi (GEH))

On December 7, 2010, GEH submitted an application for renewal of the ABWR DC. The NRC staff issued a letter to GEH on July 20, 2012, describing certain design changes (28 items) that GEH should have included in the application. By letter dated September 17, 2012, GEH stated it planned to address the 28 items and submit Revision 6 of the ABWR DCD no later than the second quarter of 2014. On March 17, 2014, GEH submitted a subsequent letter to the NRC stating that it would not be providing Revision 6 of the DCD earlier than May 2015. By letter dated February 19, 2016, GEH submitted its revised application (Revision 6) incorporating changes to the ABWR DCD. On August 30, 2016, the staff issued a schedule letter to GEH based on resolving all open items by January 2017. However, some open items associated with the review of the application remain unresolved. As a result, on August 3, 2017, the staff issued a letter to GEH indicating that the NRC will not be able to meet the original schedule. The letter also stated that the NRC will issue a revised schedule letter to GEH after additional interactions with the applicant are held to resolve these issues.

COL Applications

Turkey Point Units 6 and 7

On June 30, 2009, Florida Power & Light Company (FPL) submitted a COL application for two AP1000 units at the existing Turkey Point Nuclear Generating Station site in Miami-Dade County, FL. On September 4, 2009, the NRC staff issued a letter to FPL indicating the Turkey Point COL application was acceptable for docketing and indicated that as a subsequent COL applicant referencing the AP1000 design, the Turkey Point COL review schedule would also be dependent on the review schedules for the AP1000 DC application as well as the Vogtle reference COL (R-COL) application. In addition, the staff indicated that additional information was needed in the areas of geology, hydrology, and structural engineering in order to develop a complete and integrated review schedule and that review of Section 2.5 of the application would not begin until the information requested had been provided. On May 28, 2010, the staff issued a schedule letter to FPL projecting a final SER completion date of December 2012 and a final environmental impact statement (EIS) completion date of October 2012. In this letter, the staff reiterated its concern that it still had not received the additional information from FPL related to Section 2.5 of its application and that the review of Section 2.5 would not begin until this information was received by the staff. By letter dated October 27, 2011, the NRC issued a revised schedule to FPL due to the dependence of the Turkey Point COL review on the reviews of the AP1000 DC amendment and Vogtle R-COL applications' reviews. The revised schedule

projected final SER and final EIS completion dates of November 2013 and February 2014, respectively.

On May 4, 2012, the NRC issued a letter to FPL identifying two significant issues that were affecting the staff's ability to complete its safety and environmental reviews of the Turkey Point COL application: (1) geology, seismology, and geotechnical engineering and (2) the alternative sites analyses. Based on the significant issues identified above, the NRC indicated to FPL that the staff's safety and environmental reviews in these areas of the Turkey Point COL application would be suspended until FPL made substantial modifications to its COL application.

In a letter dated April 17, 2014, the staff informed FPL that publication of the final EIS would be re-evaluated based on the number and complexity of comments received (approximately 11,000 comment letters) on the draft EIS from Federal, State, and local agencies, members of the public, and interested stakeholders. The revised schedule projected final EIS completion in October 2016.

By letter dated August 26, 2014, the staff issued a letter to FPL indicating sufficient quality information had been submitted such that the NRC staff could schedule the review of Sections 2.5.1 through 2.5.5. The new schedule projected issuance of the final SER in October 2016.

Also in an October 27, 2015, letter the staff informed FPL that the staff continued to actively engage with the AP1000 licensees as well as Westinghouse Electric Company to resolve several generic design issues. Since then the NRC staff has reviewed the additional information provided by Duke Energy Florida under the Levy Nuclear Plant docket and the information was subsequently reviewed by the ACRS in April 2016.

In a letter dated April 14, 2016, FPL endorsed departures related to changes in the AP1000 DC that had also been submitted by Duke Energy Florida for the Levy COL application. On May 13, 2016, the NRC issued a revised schedule letter to FPL for the Turkey Point COL application contingent upon FPL providing the required information related to the departures by May 16, 2016, and all such proposed departures being equally and fully applicable to Turkey Point Units 6 & 7, and not requiring any additional staff review. The new schedule projected issuance of the final SER in November 2016. The NRC staff completed its safety review and presented the advanced final SER to ACRS on August 19, 2016. The final SER for Turkey Point was issued on November 10, 2016. The NRC issued the final EIS on October 28, 2016.

Per a Commission Order, the mandatory hearing was delayed to allow environmental consultations with other government agencies to proceed. Those consultations have concluded. The Commission's mandatory hearing on the COL application previously scheduled for October 5, 2017, was delayed at FPL's request to allow its personnel time to focus on preparing for and responding to Hurricane Irma. The mandatory hearing is currently rescheduled for December 12, 2017.

On May 2-3, 2017, the ASLB conducted an evidentiary hearing in Homestead, FL, in the contested proceeding involving the Southern Alliance for Clean Energy, National Parks Conservation Association, and other joint intervenors. On July 10, 2017, the ASLB ruled in favor of the NRC staff and terminated the contested proceeding. No appeal was filed within the appeal deadline. On April 18, 2017, the City of Miami, City of South Miami, and Village of Pinecrest (petitioners) filed a new petition seeking a hearing. On July 31, 2017, the ASLB rejected the pending contention and terminated the contested proceeding involving those petitioners. The petitioners filed an appeal that is pending before the Commission.

Early Site Permit Applications

Clinch River

On May 12, 2016, the Tennessee Valley Authority (TVA) submitted an ESP application for the Clinch River Nuclear Site located in Oak Ridge, TN. By letter dated August 11, 2016, TVA identified certain aspects of the application for which it intended to provide supplemental information. The NRC responded to TVA in a letter dated August 19, 2016, and informed TVA that its application would remain in a tendered but not docketed status until all of the supplemental information identified was provided to NRC. By December 15, 2016, TVA had provided the supplemental information in support of its application, and by letter dated January 5, 2017, the NRC staff informed TVA that its application, as supplemented, was accepted for docketing and detailed technical review.

NRC staff began its detailed technical review of the ESP application the first week of January 2017 and developed a full review schedule with public milestones that was transmitted to TVA on March 17, 2017. The Phase A review (preparing the preliminary SER and issuing RAls) for all chapters of the application was completed by the staff on August 4, 2017. The staff is currently in Phase B (issuing the advanced SER with no open items) of its review. The final SER is projected to be issued in September 2019.

On June 12, 2017, the Southern Alliance for Clean Energy, Tennessee Environmental Coalition, and Blue Ridge Environmental Defense League filed petitions seeking a hearing. On September 12, 2017, the ASLB conducted oral argument on these petitions. On October 10, 2017, the Licensing Board issued a decision that denied the Blue Ridge Environmental Defense League's petition to intervene and granted the joint petition to intervene by Southern Alliance for Clean Energy and the Tennessee Environmental Coalitions', and admitted two contentions.

5. Status of licensing and inspection status for Vogtle 3 & 4 and Summer 2 & 3.

The NRC issued COLs to Southern Nuclear Operating Co. and several co-owners on February 10, 2012, for two AP1000 units at the Vogtle site near Augusta, GA; and to SCE&G and Santee Cooper on March 30, 2012, for two AP1000 units at the V.C. Summer site near Columbia, SC. As construction progresses, the NRC has increased the pace of construction inspections to verify compliance with the agency's regulations and to ensure that the new plants are constructed in accordance with their COLs. On July 31, 2017, SCE&G and Santee Cooper, the licensees for V.C. Summer Units 2 & 3, announced their decision to discontinue construction on Units 2 & 3. As of August 1, 2017, the NRC has stopped inspections and licensing reviews at V.C. Summer.

The graphs provided in Item 12 of the Graphical Metrics section of this report represent completed inspections of safety-related components and construction activities. The completion of these ITAAC-related inspections closely mirrors the completion status of the licensees' work activities associated with the ITAAC. The graphs also report the percentage of completed program inspections, which are separate from the ITAAC-related inspections, and include both construction and operational programs. Program inspection status also closely mirrors the licensees' completion status of program development and implementation.

6. Status of uranium recovery licensing including projected budget and timeline for both the EIS and SER for each application review.

The table below provides the status of major uranium recovery licensing actions currently under review, the timeline for completing the associated EISs and SERs, and the total projected budget per project.

The NRC does not formulate its budget at the project level. The budget for the Uranium Recovery Program is formulated at a higher level using budget models for the number, type, and complexity of reviews anticipated. The projected budget information reported below includes the program staff and contract support resource estimates to perform the safety and environmental reviews from submittal to licensing decision, excluding resources for OGC's reviews, hearings, mission support, supervisory support, travel, and allocated agency corporate support resources. The estimates are based on budget models for different types (such as expansions, renewals, and new licenses) and complexities of major licensing action reviews. The NRC staff's goal is to complete the review of major licensing actions within 3 years; however, the staff estimates that smaller, less complex applications may be reviewed in 2 years, while larger, more complex, applications may require up to 4 years to review.

Uranium Recovery Applicant	Application Accepted for Review	Review Status and Projected Budget
Cameco North Trend Expansion	08/28/07	<p>The applicant requested the NRC staff to stop its review of the North Trend application and to focus its efforts on the review of the Marsland expansion. The SER for the North Trend expansion was completed in July 2013. The NRC staff has suspended its work related to the development of the draft Environmental Assessment (EA) and conduct of Section 106 consultations pursuant to the National Historic Preservation Act. In addition, the hearing to address contentions related to groundwater is on hold, pending completion of the NRC staff's environmental review. The current schedule for remaining milestones will be determined after the NRC staff has completed its review for the Marsland expansion.</p> <p>The projected total budget to conduct the review is 3.0 FTE and \$600K.</p>
Uranium One Ludeman Expansion	05/16/12	<p>NRC environmental and safety reviews are in progress for the Ludeman Project, which is an expansion to the existing Willow Creek Project. The licensee is working to resolve safety and environmental issues. NRC met with the licensee on February 22, 2017, to discuss these issues and the licensee's plan to submit an amended application that addresses a major change of design planned by the licensee. On March 28, 2017, the NRC staff issued a letter to the licensee requesting</p>

		<p>an updated schedule of when they would submit the required information necessary for the staff to complete its review. In April 2017 the applicant provided all information except for the amended application. The staff received the amended technical report in June 2017 and the amended environmental report in August 2017. The NRC staff is on schedule to complete the SER by March 2018. Based on the amended environmental report, the cultural resources review can now be completed. The NRC staff is on schedule to issue the final EA by August 3, 2018, and reach a decision on the license amendment on August 31, 2018.</p> <p>The projected total budget to conduct the review is 3.0 FTE and \$600K.</p>
Cameco Smith Ranch License Renewal	07/05/12	<p>Environmental and safety reviews are in progress. Open issues are currently being addressed. On May 2, 2013, the NRC staff issued RAIs on safety and environmental aspects of the renewal request. On April 21, 2015, the licensee submitted its responses to the RAI. The NRC staff is working with the licensee to close remaining open issues. On May 2, 2016, the staff communicated to the licensee that its response to the RAI was incomplete. In December 2016 the staff requested the licensee to provide a schedule for completing its response to the remaining aspects of the RAI. On January 10, 2017, the licensee provided a subsequent update on when it expects to respond to part, but not all, of the NRC staff's RAI. The NRC staff responded to Cameco's letter on April 14, 2017. The NRC staff and Cameco also held the first of several public meetings to address Cameco's development of sufficient RAI responses. The NRC staff's SER completion date of July 2018 and EA completion date of August 2018 were based on receipt of Cameco's RAI responses by June 30, 2017. However, the responses were not received by June 30, 2017, therefore, the NRC staff's schedule for completing this action will be impacted. The NRC staff has informed Cameco and will revise the SER and EA completion dates once Cameco determines its schedule for responding.</p> <p>The projected total budget to conduct the review is 3.5 FTE.</p>
Crow Butte Marsland Expansion	10/05/12	<p>Environmental and safety reviews are in progress. The NRC staff issued RAIs in July 2013. The licensee responded in November 2015. After reviewing the responses, the NRC determined that additional information was required to resolve the RAIs. Cameco</p>

		<p>submitted responses to the open issues in June and August of 2017. At this time, the NRC staff is proceeding with its review and is currently on schedule to complete the SER in January 2018, the final EA in April 2018, and make a final licensing decision in May 2018. The Marsland expansion review has an admitted contention that will go to hearing after completion of the NRC staff's review.</p> <p>The projected total budget to conduct the review is 3.0 FTE and \$600K.</p>
Hydro Resources, Inc. (HRI) License Renewal	06/24/13	<p>The sites, located very close to Navajo Nation lands, were licensed in 1998. Construction has not yet commenced. The license renewal review was placed in abeyance on November 13, 2014, while HRI continues its work with the Navajo Nation Council. In March 2016 the NRC approved the transfer of control of the license from the HRI parent company, Uranium Resources, Inc., to Laramide Resources. The parties finalized the transaction in January 2017. The schedule for remaining milestones associated with the licensing review is to be determined.</p> <p>The projected total budget to conduct the review is 2.6 FTE.</p>
Kennecott Sweetwater License Renewal	11/25/14	<p>The licensee has maintained the facility in stand-by since 1983, waiting on better market conditions to recommence operations. Environmental and safety reviews are in progress. On October 18, 2016, the licensee submitted supplemental information related to groundwater detection monitoring. On November 14, 2016, the licensee submitted revised environmental information. The NRC staff has identified issues with these submittals and will be working with the licensee to resolve them. The issues identified with this revised environmental information impacted the SER completion date, the final EA date, and the final licensing decision completion date originally planned for August 2017. By letter dated July 26, 2017, the licensee identified its schedule for addressing the remaining safety review RAIs. Based on the licensee's schedule, the staff plans to complete its SER and final EA in February 2018. The staff is on schedule to reach a licensing decision in August 2018.</p> <p>The projected total budget to conduct the review is 0.5 FTE.</p>
Strata Kendrick Expansion	01/14/16	<p>On May 27, 2016, and September 14, 2016, the NRC staff issued RAIs for the environmental review and for the safety review, respectively. On December 15, 2016, the licensee requested that the NRC cease all</p>

		<p>activities related to this review. As a result of the licensee's request, the NRC staff is no longer reviewing this licensing action. The staff's safety and environmental reviews, including development of the Supplemental EIS, are on hold.</p> <p>The projected total budget to conduct the review is 3.5 FTE and \$1500K, which includes completing the EIS.</p>
<p>Lost Creek KM Horizon/East Expansion</p>	<p>05/02/17</p>	<p>By letter dated February 27, 2017, the applicant resubmitted a revised application. The NRC staff has accepted the application for review on May 2, 2017. The NRC staff issued its initial RAI on July 27, 2017, and its second RAI on August 28, 2017. The final safety evaluation report is scheduled to be completed in August 2018. The NRC staff continues to coordinate with the U.S. Bureau of Land Management (BLM) in its preparation of the EIS in accordance with the BLM/NRC Memorandum of Understanding (MOU) and the letter of December 4, 2014, designating BLM as the lead agency and the NRC as a cooperating agency. The BLM is scheduled to publish the final EIS in December 2018.</p> <p>The projected total budget to conduct the review is 3.0 FTE.</p>

7. Specific actions taken to improve efficiency of reviews conducted for compliance with the National Historic Preservation Act.

The Section 106 process under the National Historic Preservation Act (NHPA) requires Federal agencies to take into account the effects of their undertakings on historic properties and afford the Advisory Council on Historic Preservation (ACHP) a reasonable opportunity to comment. The NRC carries out its Section 106 obligations in consultation with a number of parties, including the State Historic Preservation Officer (or Tribal Historic Preservation Officer, where appropriate), local government agencies, Indian Tribes and Native Hawaiian organizations, the licensee or applicant, and the public. The Section 106 regulations require that the NRC make a reasonable and good faith effort to identify historic properties that may be affected by the undertaking, including those of traditional and religious significance to Tribes. The NRC must complete the Section 106 process prior to making its licensing decision. For efficiency, the NRC's goal is to conduct the Section 106 process in coordination with the National Environmental Policy Act (NEPA) review process.

Over the past several years, the number of uranium recovery licensing reviews has increased. In addition, the complexity of the Section 106 reviews associated with these licensing actions has grown significantly and, as a result, the NRC's consultation efforts with respect to its obligations under Section 106 have also increased. The complexity of these Section 106 consultations can vary from project to project due to a number of factors. First, the NRC has seen a significant increase in the number of Tribes interested in each licensing review – from a few Tribes prior to 2009 to a current average of 20 Tribes per project. Also, the siting of proposed facilities in areas that are known to be the aboriginal homelands of Tribes, or near

sites that are considered sacred by Tribes, can influence the nature and complexity of the Section 106 consultations. Therefore, it has taken an increased amount of time and level of effort to identify historic and cultural properties, as well as to determine the eligibility of these properties for listing in the National Register of Historic Places; this has impacted the timeliness of the NRC staff's review activities. Tribes have requested field surveys at the proposed project sites to identify properties of traditional religious and cultural importance to them. Responding to survey requests has taken a significant amount of staff time due, in part, to extensive discussions with a large number of consulting parties (e.g., Tribes and other Federal and State agencies) on the format, scope, and extent of the field surveys.

Based upon lessons learned in the uranium recovery licensing functional area, the NRC has taken a number of actions to facilitate and enhance its Section 106 reviews. These actions include developing the NRC's Tribal Policy Statement, revising the Tribal Protocol Manual, conducting Tribal workshops, partnering with the ACHP, and issuing guidance documents. These actions are discussed in detail below.

On December 2, 2016, the NRC approved the final Tribal Policy Statement to guide the NRC's government-to-government interactions with Tribes. The final Tribal Policy Statement was published in the *Federal Register* (FR) on January 9, 2017 (82 FR 2402). The policy statement is intended to encourage and facilitate Tribal involvement in activities under the NRC's jurisdiction including Section 106 consultations. Along with the Tribal Policy Statement, the NRC is also revising its Tribal Protocol Manual, NUREG-2173 (ADAMS Accession No. ML14274A014). The Tribal Protocol Manual is intended to facilitate effective consultations and interactions between the NRC and Tribes. Additionally, in 2013, NRC established an interagency partnership with the ACHP.

Through this partnership, the ACHP established a dedicated liaison that works directly with the NRC by providing technical assistance with Section 106 reviews of specific licensing actions, as well as providing relevant training and guidance. In the area of training, the NRC has developed and conducted training courses for staff involved in consultations. The NRC has also increased its tribal outreach activities. In 2014 and 2015, the NRC completed five workshops where the NRC staff shared information with a number of Tribes on uranium recovery, NEPA reviews, Tribal consultation under the Section 106 process of the NHPA, and health physics.

With respect to the Section 106 reviews for uranium recovery licensing actions, in 2015 and 2016, the NRC visited with several Tribes in the Northern Plains, who have been involved in the licensing process of uranium recovery activities, to gather information about the Tribes' concerns and recommendations with respect to the NRC's consultation and communication efforts. The NRC also developed draft Interim Staff Guidance (ISG) for conducting the Section 106 process specific to uranium recovery licensing actions, namely, "Guidance for Conducting the Section 106 Process of the National Historic Preservation Act for Uranium Recovery Licensing Actions" [FSME-ISG-02 (ADAMS Accession No. ML14163A049)].

In the area of operating nuclear reactors, the NRC follows well-established guidance in carrying out its NHPA Section 106 obligations using Office Instruction LIC-203 (Rev. 3), "Procedural Guidance for Preparing Categorical Exclusions, Environmental Assessments, and Considering Environmental Issues" (ADAMS Accession No. ML12234A708). This guidance provides a framework for fulfilling the NRC's NHPA Section 106 responsibilities for operating reactor licensing reviews, including identification of historic properties, assessment of effects, and resolution of any identified adverse effects. This guidance also includes procedures to efficiently streamline NHPA Section 106 compliance activities by using the NEPA process. In

the area of new reactor licensing, the NRC has engaged with industry through the Nuclear Energy Institute (NEI) to develop guidance for early interaction with other agencies including State and Tribal governments in accordance with the Section 106 process [see NEI 10-7, Revision 1, "Industry Guideline for Effective Pre-Application Interactions With Agencies Other than NRC During the Early Site Permit Process," (ADAMS Accession Nos. ML13028A392) and ACHP's slides on "National Historic Preservation Act: Overview for the NRC and Nuclear Energy Institute," dated September 11, 2012 (ADAMS Accession Nos. ML12257A450 and ML12258A114)]. Additionally, the NRC will continue early actions with industry, other agencies, Tribal governments, and State Historic Preservation Officers, to enhance subsequent operating nuclear reactor license renewal reviews.

8. Status of the pilot project on establishing flat fees for uranium recovery licensees.

As directed by the Commission, the NRC is undertaking a flat fee pilot program for uranium recovery licensees. As described in the staff paper SECY-16-0097, "Fee Setting Improvements and Fiscal Year 2017 Proposed Fee Rule," this pilot will involve evaluation of data to collect a representative sample of the costs for various licensing reviews. The agency has developed a new data recording structure that was deployed in early October 2017, and staff has been trained to record the data using this new data structure. After a period of recording data using the new data structure, the staff will analyze the data and develop recommendations. The staff will be engaging with the Agreement States with uranium recovery licensees to understand their fee schedule development process. The staff will also be engaging with the licensee community. These recommendations will continue to address requirements under the Omnibus Budget Reconciliation Act of 1990 to collect approximately 90 percent of the NRC's annual budget through fees and under the Independent Offices Appropriation Act, 1952 to assess user fees that are fair and based on the costs to the government and certain other factors. The staff is scheduled to submit recommendations to the Commission for approval in FY 2019, and implement the Commission's direction in FY 2020.

9. Status of specific actions taken or planned to ensure greater discipline and management oversight in the use of the RAI process associated with a regulatory requirement and limited to those RAIs necessary for make a regulatory decision. These actions should describe management oversight, management accountability, and the training necessary to provide stable and sustainable improvement among the applicable program business lines.

Operating Reactors

The Commission has recently taken specific actions to ensure greater discipline and management oversight in the RAI process.

On June 30, 2014, the Commission issued a SRM (ADAMS Accession No. ML14181B402) directing the staff to consider, in the context of Project Aim, ways to reduce the licensing action backlog and get back on target with respect to timeliness. In response the SRM, NRR, as the lead for the operating reactor business line, launched several initiatives and took other actions to study how the NRC can leverage or revise its existing licensing processes to enhance agency efficiency, effectiveness, and predictability as a regulator, while maintaining a continued strong safety focus. These initiatives have analyzed the issues that caused the backlog, including issues related to the RAI process, and provided recommendations to NRR management regarding enhancements to the licensing review process. In part, as a result of recommendations from the initiatives, NRR management issued interim guidance to the staff in

January 2015, and updated interim guidance in April 2016, that provides expectations to help ensure consistency of the licensing review process, sound decision-making, and discipline of schedule. In January 2017 this interim guidance was incorporated into NRR procedures. Some of the key items in the procedures that have added discipline and management oversight to the RAI process include the following:

- NRR staff review of an application will be limited to the scope of the licensing action and RAIs should have a clear nexus to information required to make a safety determination regarding the licensing action.
- At the point when RAIs are transmitted from the technical staff to the NRR project manager, the technical staff are expected to have developed a draft safety evaluation (SE). In addition to ensuring that the RAIs contain both a sound technical and regulatory basis, the technical staff should be able to correlate each RAI to a “hole” in the draft SE that the licensee response is intended to fill.
- NRR management will maintain a focus on RAIs. Prior to sending a second (and any subsequent) round of RAIs in a specific technical area, NRR division level management will apply additional oversight to discuss the need for the RAIs and whether alternative methods, such as a public meeting or audit, may be more effective and efficient for determining the necessary information that the licensee needs to submit.
- NRR project managers are expected to track licensee timeliness and adherence to RAI response schedules. Any significant delays in licensee responses will be brought to NRR management attention.

Training sessions were held with the technical and project management staff on RAI quality and process. In addition, following issuance of the finalized NRR guidance in this area in January 2017 an online training package was developed and provided to the NRR staff. This training covers expectations regarding added discipline and management oversight of the RAI process.

Other actions taken that help provide a stable and sustainable improvement in the RAI process and add accountability to the process include:

- In November 2014 NRR management began holding periodic meetings to discuss open licensing actions, develop alignment on the best approaches to completing those actions, and monitor licensing performance.
- In October 2016 NRR replaced the existing software used to manage and monitor licensing reviews with a newly developed software package called the Reactor Protection System - Licensing/Workload Management software. This system has the capability to better track RAI issuance and status.
- NRR performed an internal audit of a sample of RAIs issued between April and December 2016 and found that the overall adherence to quality, timeliness, and process expectations was satisfactory. The audit team identified areas for continued improvement and recommended increased staff training on the RAI guidance, development of staff job aids, and consideration of modifications to staff guidance to better reflect the reactor license renewal and non-power utilization facilities licensing processes.

Decommissioning and Low-Level Waste

The Division of Decommissioning, Uranium Recovery, and Waste Programs' internal guidance includes the expectation that RAIs will be developed in conjunction with the draft SER to ensure that the RAI is necessary to reach a safety finding. In addition, the guidance contains the expectation to include a reference in the RAI to the specific requirement that has not been met, and encourages staff to conduct telephone conferences with licensees and applicants to efficiently resolve technical issues on RAIs. The NRC staff recently finalized an internal self-assessment that identifies possible efficiency improvements within the Uranium Recovery Program. The self-assessment includes recommendations for improving the efficiency of the RAI process, such as issuing RAIs as they are written rather than as a group, and reemphasizing the expectation that staff develop the draft safety evaluation and RAIs in concert. The staff has prioritized the recommendations and is developing an implementation plan.

New Reactors

The NRC provided information to the Government Accountability Office (GAO) in support of an audit currently underway by GAO on the NRC's RAI process and related enhancements. The NRC provided several documents to GAO that specifically explain its RAI process and steps NRO has taken to ensure that RAIs issued from the office are consistently of high quality and are necessary to make a safety finding. On October 7, 2016, the NRO Director issued the memorandum, "Effective Use of Request for Additional Information, Audit, and Confirmatory Analysis in New Reactor Licensing Review" (ADAMS Accession No. ML16278A574), to all NRO staff, to promote focus on safety, efficiency, consistency, and clarity, in the ongoing and future reviews of new reactor licensing applications.

In 2008 NRO published an RAI job aid document to be used as guidance by NRC staff when preparing RAIs. The RAI job aid document provided best practices information for preparing RAIs. This past summer, senior managers in NRO reexamined the 2008 RAI job aid and the overall process for issuing RAIs and made additional modifications to incorporate best practices learned throughout the course of many licensing reviews. In October 2016, NRO revised the RAI process (ADAMS Accession No. ML16280A389) to include a new quality check audit process where, in addition to the technical branch's supervisor, the division management of both the technical and project management organizations review an RAI before it is issued to the applicant or licensee. In addition, the Office Director will review RAIs on a sampling basis to keep abreast of high-priority issues identified in reviews and to support the office's emphasis on efficiency as we focus on safety, security, and environmentally significant matters.

10. Status of specific actions undertaken to reduce corporate overhead costs including the amount of the savings and the timeframe for realizing cost savings.

The NRC's FY 2017 enacted budget for Corporate Support activities was \$300.1 million. The FY 2017 enacted budget included resources for the five recognized overhead activities of acquisitions, real property, human capital, financial management, and information technology. Additionally, the Corporate Support budget includes the NRC's small business outreach efforts, as well as resources to support the Office of the Commission.

As part of the agency's Project Aim effort to plan and execute the agency's mission in an effective and efficient manner, the Commission approved a staff recommendation to re-baseline the agency's workload — focusing on statutory mandates, as well as work pertaining to the agency's safety and security mission in SRM "Project Aim 2020 Report and Recommendations"

(ADAMS Accession No. ML15159A234). In SRM-SECY-16-0009 (ADAMS Accession No. ML16104A158), “Recommendations Resulting from the Integrated Prioritization and Re-baselining of Agency Activities,” the Commission approved a total of \$8.4 million, including 24.3 FTE, in reductions from the Corporate Support area. Re-baselining reductions totaling \$4.8 million, including 13.0 FTE, were taken from Corporate Support in the FY 2017 request, as detailed in the FY 2017 CBJ. The balance of \$3.6 million, including 11.3 FTE, in re-baselining savings was reflected in the agency’s FY 2017 enacted budget.

In addition to the work of SECY-16-0009 listed above, in SECY-16-0035 (ADAMS Accession No. ML16077A184), “Additional Re-baselining Items,” the NRC staff identified other activities that could provide additional savings in the long term. Additional re-baselining cost savings, as well as possible areas for future savings in the Corporate Support area, are also included in the table below. In particular, the NRC recently awarded two complementary contracts to reduce the total cost of ownership of the agency’s existing fleet of printers, scanners, and copiers by using Multi-Functional Devices and Managed Print Services. The NRC expects to realize annual savings of \$0.7M beginning in FY 2018. Notably, the anticipated savings exceed the agency’s initial projection of \$0.2M per year.

Consistent with SECY-16-0035 referenced above, the “Additional Re-baselining Item” titled *Review of Corporate Offices’ Full-Time Equivalent (FTE) Utilization and Workload* (marked as complete in report for the month ending March 31, 2017), the agency took a number of actions to mitigate the need for involuntary reductions to meet our FY 2018 workload needs. Also, the agency initiated involuntary reductions in a small number of corporate support offices. As of September 30, 2017, eleven employees had received reduction in force notices. NRC senior managers will continue to work with the affected individuals to identify other funded higher priority work that matches their skill sets, which could reduce the final number of involuntary separations that occur.

Product Line	Description	Total \$ (M)*	FTE	Status	Fiscal Year
Additional Re-baselining Items (SECY-16-0035)					
Administrative Services	Reduce Office Space in Three White Flint North	TBD	TBD	In process	FY 2019
Administrative Services	Reduce Office Space in the Regions	TBD	TBD	In process	FY 2019
Financial Management	Standardize Budget Formulation and Execution across Business Lines	TBD	TBD	In process	FY 2019
Financial Management	Use a Federal Shared Service Provider for Accounts Payable	TBD	TBD	In process	FY 2019
Administrative Services and Information Technology	Workstation Efficiencies	TBD	TBD	In process	FY 2019
Subtotal – Additional Re-baselining Savings		TBD	TBD		

Other Corporate Support Savings

Information Technology	IT Infrastructure Support - the agency expects to realize a 10 to 15 percent drop in contract expenses resulting from a new acquisition strategy.	-3.6	0	In process	FY 2018
Information Technology	Reduce the total ownership of the agency's existing fleet of printers, scanners, and copiers using Multi-Functional Devices and Managed Print Services.	-0.7	0	Complete	FY 2018
Subtotal – Other Corporate Support		-\$4.3	0		
Total		-\$4.3	0		

*Total includes FTE cost.

The NRC remains committed to continuing to identify efficiencies in the Corporate Support area that will lead to cost savings.

11. Status of specific actions taken and/or planned to develop metrics for assessing the quality of cost-benefit analyses conducted in association with new requirements, backfit analyses, or rulemaking.

The staff has not yet taken any action to develop specific metrics for assessing the quality of its cost benefit analyses. As described in narrative item 12, the staff is in the process of revising its existing guidance pertaining to cost-benefit analyses in two phases. Phase 1 primarily involves consolidation and harmonization of existing guidance across business lines including administrative and methodology enhancements. Phase 2 will address potential policy issues and methodology changes. Depending on the nature of the policy issues and methodology changes, Commission approval may be necessary. These efforts, in conjunction with the CRGR efforts, described in narrative item 13 below, to review the application of the Backfit Rule in the licensing and inspection programs across the agency, will inform the development of future metrics for assessing the quality of cost-benefit analyses.

12. Status of the revised guidance currently under development to clarify the use of qualitative factors. In addition to this revised guidance, please list and briefly describe any actions taken and/or planned that would maximize the use of quantitative factors in regulatory analyses required for rulemaking, in the regulatory analyses required under the Backfit Rule, and in the Reactor Oversight Process Significance Determination Process.

The NRC staff released the draft cost-benefit guidance for a 60-day public comment period, which ended on June 16, 2017. Comments received during the public comment period are currently being addressed by the staff in order to finalize the guidance. This update consolidates guidance documents, incorporates recommendations from the GAO's 2014 report on the NRC's cost-estimating practices and cost-estimating best practices from the GAO's guide, and captures best practices for the consideration of qualitative factors in accordance with Commission direction in the SRM for SECY-14-0087.

The NRC staff is using the improvements in cost estimating and cost-benefit analysis as this guidance is developed. With regard to actions taken or planned that would maximize the use of quantitative factors in the regulatory analyses required for rulemaking or backfitting, the staff

makes every effort to quantify the estimates of benefits and costs to the extent possible. However, the staff acknowledges that some attributes in regulatory analyses are difficult to quantify, and thus would require additional resources to develop a strictly quantitative analysis (which might still entail such large uncertainty so as to be of limited practical value). The draft updated cost-benefit guidance includes an appendix that identifies best practices for the consideration of qualitative factors and describes a number of methods that can be used to support the NRC's evidence-based, quantitative, and analytical approach to decisionmaking. This appendix provides a toolkit to enable analysts to clearly present analyses of qualitative results in a transparent way that decisionmakers, stakeholders, and the general public can understand. However, this updated guidance clearly states that these methods (1) should only be used when quantification may not be practical, (2) are not a substitute for collecting accurate information to develop realistic cost estimates, and (3) do not constitute an expansion of the consideration of qualitative factors in regulatory, backfit, or environmental analyses.

With regards to action taken and/or planned that would maximize the use of quantitative factors in the Reactor Oversight Process (ROP) Significance Determination Process, the staff continues to enhance methods, models, data, and analytical tools that it relies upon to enhance the use and quality of quantitative factors. Specifically, in accordance with a User Need developed by the NRR (ADAMS Accession No. ML15110A210), NRC's Office of Nuclear Regulatory Research continues to upgrade the plant-specific probabilistic risk assessment (PRA) models for internal events to reflect changes to plant design and procedures and continues to develop PRA models for external initiators. In addition, NRR staff continues to enhance the methods that NRC uses in support of the Significance Determination Process as well as other reactor oversight processes (e.g., incident response) and updates NRC's Risk Assessment Standardization Project guidance.

13. Status of the Committee to Review Generic Requirements (CRGR) review of the application of the Backfit Rule in the licensing and inspection programs across the agency. The review should include the following as a minimum:

- a. The need for training on the requirements and application of 10 CFR 50.109;
- b. The need for a process, training, and/or oversight in addressing inspection issues that may redefine or reinterpret the original licensing basis (e.g., unresolved issues, task interface agreements, disputed violations) to ensure that new requirements are not imposed through the inspection program
- c. A review of proposed regulatory changes that are currently in process to ensure that regulatory actions are appropriately informed by the requirements of 10 CFR 50.109, in light of the Executive Director's recent decision on the backfit appeal. Examples of such actions could include but are not limited to the following:
 - i. The Draft Regulatory Issue Summary on Service Life addressing the treatment of vendor recommendations within the regulatory framework
 - ii. 10 CFR 50.46(c) rulemaking for which the justification utilizes the adequate protection provisions of the backfit rule to obviate the need to compare the benefits of public health and safety with the cost of compliance for the three major portions of the rule
 - iii. Use of the compliance exception backfit as proposed by the NRC staff to address the "open phase condition" issue
 - iv. Possible alteration of the risk reduction credit given for Incipient Fire Protection after the modifications have been installed and received approval from the NRC crediting the technology
- d. Please report your progress in the monthly report

a & b. The CRGR was requested by the NRC Executive Director of Operations (EDO) in tasking memoranda dated June 9, 2016, and December 15, 2016³, to review the implementation of agency backfitting guidance. In addition, the CRGR examined its charter⁴ to assess whether it should be expanded beyond its current roles and responsibilities to include review of rulemaking and facility-specific actions under certain circumstances.

The CRGR provided the results of this assessment to the EDO on June 27, 2017 (ADAMS Accession No. ML17174B161). The CRGR provided a series of recommendations: (1) make the CRGR report publicly available; (2) update general and office-level implementing guidance to incorporate recent policy developments and Commission instruction; (3) issue a policy announcement on recent Commission policy changes; (4) update and develop training requirements to provide initial and refresher training to managers and NRC staff with backfitting and issue finality responsibilities; (5) implement knowledge management procedures and a backfitting Community of Practice; and (6) conduct an effectiveness review of actions taken in response to the CRGR recommendations. By letter dated July 19, 2017 (ADAMS Accession No. ML17198C141), the EDO agreed with the CRGR and approved the three near-term actions recommended in the report. The staff was tasked to develop due dates and resource estimates to complete all the recommendations identified by the CRGR.

c. The CRGR has incorporated the recent lessons learned from the Exelon backfit appeal decision and the Commission’s direction in SRM-COMSECY-16-0020 in its review of proposed regulatory changes and decision making.

The table below provides a summary of the status of regulatory changes and issues as of September 30, 2017. The table also contains planned CRGR review activities.

Status of Select Regulatory Activities		
Title	Status of Regulatory Change	CRGR Review Activities
RIS on Service Life - “Disposition of Information Related to the Time Period That Safety-Related Structures, Systems, or Components are Installed”	RIS (ADAMS Accession No. ML17177A060) was issued for public comment and the public comments have been dispositioned. This RIS was sent to CRGR June 29, 2017.	Under review (see Graphics Metrics response #13 above, Item #2, CRGR meetings #446 & #447)
10 CFR 50.46(c) Rulemaking	The NRC staff prepared a regulatory analysis for the 10 CFR 50.46c draft final rule (ADAMS Accession No. ML15323A122) to identify the benefits and costs of the particular regulatory approach for addressing emergency core cooling system performance. The regulatory analysis focuses on the marginal difference in benefits and costs for each alternative relative to the “no action” baseline alternative for the three major portions of the rule, which is consistent	Based on established criteria at the time, the CRGR was not required to review the rulemaking to assess potential backfits. The rulemaking is currently with the Commission for review and vote.

³ The EDO tasking memorandum dated June 9, 2016, and December 15, 2016 (ADAMS Accession Nos. ML16133A575 and ML16344A004), respectively.

⁴ The CRGR Charter, Revision 8 (ADAMS Accession No. ML110620618).

Status of Select Regulatory Activities		
Title	Status of Regulatory Change	CRGR Review Activities
	with the requirements of the backfit rule (10 CFR 50.109), Commission direction, and the ongoing revisions to the agency's cost-benefit guidance (e.g., NUREG/BR-0058, Revision 5).	
Alteration of Credit for Incipient Fire Detection in Prior Approvals	For licensees that have implemented risk-informed fire protection standard NFPA-805 ⁵ and use incipient fire detection. The program obligates licensees to maintain PRA models to consider updated information on the performance and reliability of plant systems periodically and update accordingly.	No CRGR review or request has been identified for potential backfit consideration.
RIS on "AP1000 Certified Design Information Errors Discovered During Design Finalization and Construction Activities"	The purpose of this RIS is to provide guidance, to present and future applicants for a COL or COL renewal referencing the AP1000 DC, regarding previous applicant-identified errors contained in the AP1000 DC (DCD Revision 19) and how these errors may be addressed by current and future applicants.	CRGR requested a formal review of the RIS to ensure adherence to the issue finality provisions in 10 CFR Part 52. The staff has placed the issuance of the AP1000 RIS on hold pending additional input from industry.
Proposed Rule, 10 CFR 73.53, "Requirements for Cyber Security at Nuclear Fuel Cycle Facilities" and associated draft regulatory guidance, DG-5062 "Cyber Security Programs for Nuclear Fuel Cycle Facilities"	The proposed rule (ADAMS Accession No. ML17145A342), if approved, would require certain Fuel Cycle Facility licensees to establish, implement, and maintain a cyber security program that can detect, protect against, and respond to a cyber-attack capable of causing one or more of the consequences of concern as defined in the proposed rule.	CRGR completed its review in two meetings, June 27 and July 12, 2017. This rule contained backfitting and was endorsed by the CRGR.
Regulatory Guide 5.77, "Insider Mitigation Program"	This regulatory guide describes an approach that the staff of the NRC considers acceptable for an insider mitigation program for nuclear power reactors that contain protected or vital areas.	A CRGR review was conducted on July 11, 2017, and was endorsed with no backfitting.

d. The list of recent CRGR reviews is reported monthly under Graphical Metrics response #13. CRGR conducted three reviews in September 2017.

⁵ NFPA 805 "Performance-Based Standard for Fire Protection for Light Water Reactor Electric Generating Plants."

14. Status of Project Aim Task 19: Operating Reactor Licensing Process Improvements.

On January 24, 2017, the NRC staff finalized a recommendation for the Commission on Project Aim Task No. 19 regarding the licensing business process improvement (BPI) activity. The staff recommended closing Task No. 19 because the desired outcomes of the BPI review – improving predictability, timeliness, and efficiency of licensing reviews – have been achieved without the need to expend the additional time and cost of a formal BPI. This recommendation is publicly available (ADAMS Accession No. ML16340A115).

On March 2, 2017, the Commission approved the staff's recommendation to close Project Aim Task No. 19. The Commission's approval and voting record is available (ADAMS Accession Nos. ML17061A631 and ML17061A636), respectively.

15. Status of effort to establish clear schedules and estimated number of review hours for licensing action reviews.

The revised Expectations Memo (ADAMS Accession No. ML16202A029) issued in April 2016 provided the NRC staff additional guidance on establishing clear schedules and providing accurate estimates for the number of review hours. This included finalizing the review hours and estimated schedule following the acceptance review, and additional engagement between the staff and management for milestones that cannot be achieved. On October 1, 2016, NRR implemented additional guidance for licensing actions regarding schedules and review hours. For licensing actions received after October 1, 2016, NRR has been providing the licensees an estimate of the predicted staff hours and forecasted number of months the staff anticipates it will need to complete the review. This information is annotated at the completion of the staff's acceptance review of the licensing action. If there are significant changes to the schedule or estimated hours, the staff will communicate the reasons for the changes, along with the new estimates during the routine interactions with the licensees. NRR is monitoring schedule and resource utilization adherence through the monthly workload management process.

16. Status of any potential changes to the ROP.

Significant potential changes to the ROP include the following:

- Changes to structure of inspection reports

The staff is revising Inspection Manual Chapter (IMC) 0611, "Power Reactor Inspection Reports," to improve the readability and understandability of inspection reports, to eliminate redundant or unnecessary language and to streamline the report writing process for the staff.

- IMC 0609, "Significance Determination Process," Appendix M, "Significance Determination Process Using Qualitative Criteria." The staff has received stakeholder feedback on its initially proposed changes to Appendix M, is preparing a revised approach, and will re-engage with industry stakeholders in the coming months to review the changes and plan next steps.

17. Status of effort to provide greater transparency and detail in invoices to applicants and licensees.

The Commission approved staff recommendations identified in SECY-15-0015, "Project AIM 2020 Report and Recommendations," to undertake an effort to: (1) simplify how the NRC calculates its fees, (2) improve fees transparency, and (3) improve the timeliness of the NRC's communications about fee changes. Under this initiative as well as other improvement activities, several recommendations to improve invoices are currently under review and appropriate ones will be implemented as systems and processes improvements/enhancements can be achieved. For example, the NRC has already doubled the length of the cost activity description field on Part 170 invoices. This improvement provides licensees with increased and better quality information about the work activities performed by NRC staff and contractors for which they are being billed. Another example is the current effort to improve the descriptions associated with the cost activity codes so more precise information can be placed on invoices. Additionally, for all fee-recoverable work, the agency will provide licensees with additional information on their invoices and also group all charges associated with a single licensing action or inspection together for increased clarity. This change will take effect on invoices beginning in January 2018.

18. Clarity in Operability Determinations. The predictability and stability of the regulatory framework could be improved if there was greater clarity on operability determinations with regard to the entry conditions for triggering a review and the optimum use of risk insights for evaluating operability. Please describe the feasibility of utilizing an industry consensus document as a means of accomplishing predictability and repeatability in operability determinations.

The NRC agrees that using an industry consensus document to bring added clarity to operability determinations is feasible. Historically, the nuclear industry has used guidance in IMC 0326 as a guide for performing operability determinations in lieu of developing their own guidance. Over the past year, industry launched an effort to develop a consensus document for operability determinations. The NRC is engaged with the nuclear industry on this effort. The agency held a public meeting on June 1, 2017, to identify the fundamental concepts used in operability determinations and participated in an operability panel at the NEI Licensing Forum on August 23, 2017. Until the industry effort matures to a point that is appropriate for NRC to consider endorsement, the NRC will continue to rely upon existing guidance in IMC 0326, "Operability Determinations & Functionality Assessments for Conditions Adverse to Quality or Safety," to help NRC inspectors in consistently assessing operability determinations. The NRC envisions that this effort could inform future revisions of IMC 0326.

19. Significance Determination Process. Licensees maintain detailed, plant-specific PRA models that are accessible to the NRC. Please describe the potential to utilize these more detailed models in lieu of the NRC's Standardized Plant Analysis Risk (SPAR) model as a means of reaching quantitative regulatory decisions that are more efficient and timely. Please also describe the actions taken and/or planned to address this opportunity.

NRC staff uses plant-specific SPAR models, developed and maintained by NRC staff, in a number of risk-informed applications. The SPAR models utilize standardized conventions and modeling methods to improve staff efficiency and, in some cases, are more detailed than the associated licensee-maintained, non-standardized models. In 2015, NRC's Risk-Informed Steering Committee (RISC), which comprises NRC's senior leadership, directed the NRC staff to evaluate the costs and benefits associated with using licensees' PRA models in lieu of the SPAR models.

The staff identified and evaluated a number of technical, regulatory, cost, and other related factors pertinent to use of licensees' PRA models in lieu of the SPAR models. These included, but were not limited to, fixed and variable costs, ease of use for NRC staff (including training costs), potential legal issues (including loss of the ability to perform independent confirmatory analysis), and licensee willingness to participate. The results from the cost analysis indicated a significant cost for transition to licensee models with a potential for longer-term small cost savings once full transition was complete.

The NRC staff also worked with NEI to gauge licensees' willingness to participate, since the viability of the proposal depended upon full NRC access to licensee PRA models (which are not normally submitted to the NRC under the current regulatory framework). While some licensees were supportive of the proposal, there was considerable resistance with allowing NRC staff full access to the licensee PRA models.

Based on these considerations, the NRC staff recommended that the NRC should continue to rely on SPAR models in implementing its risk-informed regulatory activities. Based on the staff's recommendation, the RISC made the decision to discontinue the evaluation and continue to use SPAR models for operating reactor oversight programs.

20. On a monthly basis, please report each instance where Inspection Manual Chapter 609 Appendix M, "Significance Determination Process Using Qualitative Criteria," has been applied in the Reactor Oversight Process Significance Determination Process, including the justification for doing so.

Appendix M was not used to disposition any inspection findings in September 2017.

21. Engineering Inspection Programs. In a rolling three year period, the NRC performs multiple inspections of engineering programs (e.g., Component Design Basis Inspection (CDBI), 10 CFR 50.59 and Modifications Inspection, Ultimate Heat Sink (UHS) Inspection, Tri-Annual Fire Protection Inspection). The CDBI and UHS inspections predominantly look at the original licensing basis information on a recurring basis. This previously NRC-approved design basis information is the least likely to change and, without sufficient management oversight, could be subject to unjustified and post hoc reinterpretation by NRC inspectors and consultants. Please evaluate the potential benefits of utilizing the CDBI and UHS inspections as reactive inspection tools to be used only when issues are identified with current performance. Please provide a summary of your conclusions and any actions planned to address this issue.

Staff recently made changes to the CDBI inspection procedure effective January 1, 2017. This was done, in part, to address industry feedback that the current level of inspection resources being applied to verify licensees' compliance with their original licensing basis were excessive, as the NRC had already reviewed the most risk-significant components associated with the licensee's mitigation system through the CDBI inspections over the last 10 years. Additionally, inspection guidance was added to the revised CDBI inspection procedure to remind NRC inspectors and consultants that issues that could result in different interpretation of the plant licensing bases should be referred to NRC management and technical staff for resolution. To emphasize that NRC inspectors and consultants should not reinterpret the original licensing bases while performing CDBI inspections, the CDBI inspection procedure was renamed DBA inspection. Additionally, the 10 CFR 50.59 and Modifications Inspection was reduced in scope, and the modification samples moved to the new DBA inspection procedure. This revision will allow recent changes to mitigation systems to be sampled for inspection to ensure that

mitigation systems will still meet their design requirements. The inspection resources saved by revisions to the CDBI and the 10 CFR 50.59 and Modifications Inspection were moved to a new programmatic design basis program inspection procedure.

No significant revisions were made to the UHS inspection procedure or the Tri-Annual Fire Protection inspection in calendar year (CY) 2016, although the agency is considering performing a more holistic review of all engineering inspections during CY 2017 to evaluate what engineering inspections are needed and their basis and what inspection resources should be applied to these inspections and at what frequency. The nuclear industry is also planning to perform a similar, independent review during CY 2017.

Changing the categorization of the CDBI and the UHS inspection to reactive would result in these activities being removed from the baseline inspection program.

The NRC staff believes that it is prudent to periodically verify that the designed capabilities of mitigation systems are maintained by licensees.

22. Please describe the actions planned and/or taken to ensure that the Technical Specifications Task Force (TSTF) process achieves the regulatory efficiencies that were initially projected. Please include progress reports with regard to any TSTF "travelers" adopted by the industry.

Industry, through the TSTF, proposes changes to the Standard Technical Specifications (STS) via a "traveler" submitted for NRC staff review and approval. The traveler process was collaboratively developed between NRC and the nuclear industry 20 years ago as a means for industry to revise the STS to gain regulatory efficiencies. Since then, the NRC has approved over 340 travelers that have streamlined the process for NRC review and approval of plant specific license amendment requests to adopt the approved STS changes. To ensure the TSTF traveler process functions efficiently, the NRC has recently addressed lessons learned by introducing two process changes: (1) improved documentation of NRC's approval of the traveler through publication of a safety evaluation documenting key review findings; and (2) established work plans during traveler reviews to ensure that all appropriate technical branches are involved during the traveler review and to ensure consistency between reviewers. Additionally, the NRC has requested that licensees submit requests to adopt travelers soon after they are approved, to ensure review continuity. The NRC is seeing early successes from the process enhancements in the reviews of licensees' adoption of TSTF-542, "Reactor Pressure Vessel Water Inventory Control."

The NRC is committed to continuing to work with industry on travelers to make improvements to the STS. In recent years the requested STS changes from industry have shifted to more complex items (e.g., risk-informed STS changes). To ensure the traveler process achieves the regulatory efficiencies that were initially intended, and to align on priorities, the NRC holds quarterly public meetings and monthly status calls with the TSTF. Additionally, the status of TSTF travelers are discussed by both NRC and industry senior management at the quarterly Regulatory Issues Task Force meeting.

In 2016, three travelers were approved by the NRC. Currently six travelers are under review by the NRC staff. The TSTF has indicated that there may be as many as 12 new travelers submitted this calendar year. The latest status report of travelers currently under review is available (ADAMS Accession No. ML17132A095).

23. Improving New Plant Application Review Efficiency. Please review new-plant application reviews to identify necessary changes in practices and guidance to ensure the appropriate level of detail for application acceptance and review. Please describe any justifications for increasing the level of detail required beyond that of previous applications such as Vogtle 3 and 4, Summer 2 and 3, and the AP1000.

Following the completion of the AP1000 DC review and the issuance of the Vogtle and Summer COLs, to 10 CFR Part 52 (“Licenses, Certifications, and Approvals for Nuclear Power Plants”) licensing process and contribute to more effective and efficient reviews of future applications. The NRC staff drew on previous assessments of portions of the new reactor licensing process, lessons shared at the NRC’s 2012 Regulatory Information Conference, feedback received at a public meeting on lessons learned, and the results of internal and external surveys on the new reactor licensing process. As a result of this review, in April 2013 the NRC issued its “New Reactor Licensing Process Lessons Learned Review: 10 CFR Part 52” (ADAMS Accession No. ML13059A239).

On December 18, 2014, the NRC issued Revision 2 of NRO-REG-100, “Acceptance Review Process for Early Site Permit, Design Certification, and Combined License Applications” (ADAMS Accession No. ML14078A152), which provides guidance to NRC staff who conduct acceptance reviews for ESP, DC, and COL applications submitted under 10 CFR Part 52. The changes made in Revision 2 include 1) changing the standard for accepting an application from enough information to “begin” the review to enough information to “conduct” the review; 2) adding criteria to support the new standard for acceptance; 3) adding a flow chart and supporting discussion to clarify the acceptance review process; 4) expanding the applicability of this office instruction to ESP applications; 5) clarifying text to indicate that acceptance reviews will be performed in 60 days; 6) adding text to describe pre-application interactions; and 7) incorporating lessons learned from the APR1400 DC application acceptance review.

24. Please provide a list of any unresolved policy issues with regard to the licensing of small modular reactors (SMRs). Please include an approximate date for when each issue was first raised, any plans or actions taken to resolve the issue, and the projected date of resolution.

Issue Title/Applicability	Status	References
I. Appropriate Source Term, Dose Calculations, and Siting for SMRs	<p>In the Commission Memorandum dated December 29, 2011, the staff stated it would remain engaged with SMR stakeholders regarding applications of a mechanistic source term (MST) methods, review of pre-application white papers and topical reports it receives from potential SMR applicants concerning source term issues that discuss design-specific proposals to address MST, and considerations of research and development in this area. If necessary, the staff would propose revised review guidance or regulations, or propose new guidance to support reviews of SMRs.</p> <p>In Commission Memos dated May 30, 2013, and June 20, 2014, the staff provided updates on</p>	<p>SECY-16-0012 (02/07/16) Commission Memo (06/20/14) Commission Memo (05/30/13) Commission Memo (12/29/11)</p>

Issue Title/Applicability	Status	References
	<p>interactions with U.S. Department of Energy (DOE) and nuclear industry organizations regarding MST. NRO developed Information SECY 16-0012, dated February 7, 2016, which addressed this item. The paper concluded that (1) SMR and non-light water reactor (non-LWR) applicants can employ modern analysis tools to demonstrate quantitatively the safety features of those designs, (2) MST analysis methods can also be used by applicants to demonstrate the ability of the enhanced safety features of plant designs to mitigate accident releases allowing future COL applicants to consider reduced distances to Exclusion Area Boundaries and Low Population Zones and potentially increased proximity to population centers.</p> <p>Disposition: As discussed in SECY-16-0012, the staff will engage with interested stakeholders on this issue in 2017 and inform the Commission, as necessary.</p>	
<p>II. Offsite Emergency Planning (EP) Requirements for SMRs</p>	<p>In SECY-11-0152, staff identified a possible approach for a scalable emergency planning zone for SMRs. The NRO staff is working with NSIR and NRR on an internal working group to review these issues further. The Office of the Secretary stated that the staff would liaise with other stakeholders (Department of Homeland Security/Federal Emergency Management Agency, the Environmental Protection Agency, Department of State, Department of Commerce, NEI, American Nuclear Society, and the public) to consider industry position papers on this topic, and develop recommendations.</p> <p>In a 2013 Commission Memorandum dated May 30, 2013, the staff provided updates on staff activities. The staff stated that it would not go further in proposing new policy or revising guidance for specific changes to EP requirements absent specific proposals from industry. On December 23, 2013, NEI submitted a white paper on this topic. The staff conducted a public meeting to discuss the white paper on April 8, 2014, issued follow-up questions to NEI on June 11, 2014, and NEI responded in November 2014. The SECY-15-0077 regarding EP for SMRs and non-LWRs, was issued on May 29, 2015, and the SRM was issued on August 4, 2015. The Commission approved the staff's recommendation to initiate a rulemaking. Staff developed notation</p>	<p>SRM-SECY-16-0069 (06/22/16) SECY-16-0069 (05/31/16) SRM-SECY-15-0077 (08/04/15) SECY-15-0077 (05/29/15) NEI Response to NRC Questions on White Paper (11/19/14) NRC Letter to NEI (R. Bell) (06/11/14) NEI White Paper (12/23/13) Commission Memo (05/30/13) SECY-11-0152 (10/28/11)</p>

Issue Title/Applicability	Status	References
	<p>vote SECY-16-0069, which discussed the rulemaking plan and schedule. On June 22, 2016, the Commission approved the staff's plan and schedule for the rulemaking pertaining to emergency preparedness for SMRs and other new technologies.</p> <p>Disposition: The rulemaking will disposition EP issues for future SMRs, non-LWR, and other new design technologies such as medical isotope producing facilities. The Commission directed the staff to utilize exemptions in the interim (e.g., for the TVA ESP) until completion of the EP rulemaking. The draft regulatory basis was published for public comment in the Federal Register on April 13, 2017. A public meeting was held May 10, 2017, to discuss the draft regulatory basis. The public comment period closed on June 27, 2017. The final regulatory basis was provided to the Commission on September 29, 2017, and is scheduled to be published in November 2017.</p>	
<p>III. Insurance and Liability for SMRs</p> <p><i>This issue only applies to multi-module designs with electrical power generation less than 100 MWe per module, such as the NuScale design or small non-LWR designs; or for reactors designed for process heat generation with a rated output greater than 10 MWe.</i></p>	<p>In SECY-11-0178, the staff identified a potential inequity between the insurance requirements for power reactors producing electrical power equal or greater than 100 MWe per unit and those SMR designs with individual modules producing less than 100 MWe. Specifically, staff raised the question of whether there would be insurance and indemnity coverage sufficient to pay all public claims in the case of an insurable event for an SMR with an individual module sized at less than 100 MWe under the current Price-Anderson Act and associated regulatory language.</p> <p>Since completing that paper, staff prepared a comparative analysis of different SMR designs to further explore the potential inequity. Staff is using this analysis, and other inputs, to develop a SECY paper for this topic. In the paper, staff will identify whether rulemaking or a change to the current interpretation of the definitions given in the Price-Anderson Act is recommended.</p> <p>Disposition: This is a narrowly focused issue and is related to other multi-module issues, such as the multi-module licensing process, and differences in potential consequences from non-LWR designs. Staff is engaging stakeholders in</p>	<p>SECY-11-0178 (12/22/11)</p>

Issue Title/Applicability	Status	References
	2017, and will assess the need for continuation or modification of the Price-Anderson provisions.	
IV. Security and Safeguards Requirements for SMRs	<p>Staff determined in SECY-11-0184 that the current regulatory framework is adequate to certify, approve, and license light-water SMRs, the manufacturing of SMR fuel, transportation of special nuclear material and irradiated fuel, and the interim storage of irradiated fuel proposed for light-water SMRs under 10 CFR Parts 50, 52, 70, 71, and 72, respectively. The staff also determined that security and material control and accounting requirements in 10 CFR Parts 72, 73, and 74, respectively, are also adequate.</p> <p>In the case of non-LWRs, the staff's preliminary conclusion is that the current security regulatory framework is comprehensive and sufficiently robust to certify, approve, and license non LWRs. Sufficient provisions are available to provide flexibility for designers and applicants to meet performance-based and prescriptive security requirements and to apply methods or approaches to achieve the objective of high assurance that activities involving special nuclear materials are not inimical to the common defense and security, and do not constitute an unreasonable risk to public health. On December 14, 2016, NEI submitted a white paper on a "Proposed Consequence-Based Physical Security Framework for Small Modular Reactors and Other New Technologies." This paper, "... proposes an approach to security that appropriately considers the enhanced safety and security incorporated into these designs and provides a more effective and efficient means to protect the public health and safety." In the transmittal letter, NEI requests that "... the NRC establish regulatory positions on this approach and the associated policy and technical issues." NEI submitted a fee waiver request for NRCs review of this white paper.</p> <p>Disposition: The NRC has approved NEI's fee waiver request and met with NEI on May 3, 2017, to discuss the review of their submittal. The NRC provided feedback on NEI's white paper in July 2017, and met with NEI again on October 12, 2017.</p>	<p>NEI White Paper (12/14/16)</p> <p>SECY-11-0184 (12/29/11)</p>

Issue Title/Applicability	Status	References
<p>V. Security and Safeguards Requirements for SMRs</p>	<p>Staff determined in SECY-11-0184 that the current regulatory framework is adequate to certify, approve, and license light-water SMRs, the manufacturing of SMR fuel, transportation of special nuclear material and irradiated fuel, and the interim storage of irradiated fuel proposed for light-water SMRs under 10 CFR Parts 50, 52, 70, 71, and 72, respectively. The staff also determined that security and material control and accounting requirements in 10 CFR Parts 72, 73, and 74, respectively, are also adequate.</p> <p>In the case of non-LWRs, the staff's preliminary conclusion is that the current security regulatory framework is comprehensive and sufficiently robust to certify, approve, and license non LWRs. Sufficient provisions are available to provide flexibility for designers and applicants to meet performance-based and prescriptive security requirements and to apply methods or approaches to achieve the objective of high assurance that activities involving special nuclear materials are not inimical to the common defense and security, and do not constitute an unreasonable risk to public health. On December 14, 2016, NEI submitted a white paper on a "Proposed Consequence-Based Physical Security Framework for Small Modular Reactors and Other New Technologies." This paper, "... proposes an approach to security that appropriately considers the enhanced safety and security incorporated into these designs and provides a more effective and efficient means to protect the public health and safety." In the transmittal letter, NEI requests that "... the NRC establish regulatory positions on this approach and the associated policy and technical issues." NEI submitted a fee waiver request for NRCs review of this white paper.</p> <p>Disposition: The NRC has approved NEI's fee waiver request and met with NEI on May 3, 2017, to discuss the review of their submittal. The NRC provided feedback on NEI's white paper in July 2017, and met with NEI again on October 12, 2017.</p>	<p>NEI White Paper (12/14/16)</p> <p>SECY-11-0184 (12/29/11)</p>

25. Please describe the process toward preparing to review non-light water reactor applications.

The agency has developed a vision and strategy to assure NRC readiness to conduct its mission for these technologies effectively and efficiently. The staff described the vision and strategy in “NRC Vision and Strategy: Safely Achieving Effective and Efficient Non-Light Water Reactor Mission Readiness,” which was published in the *Federal Register* on July 21, 2016, for stakeholder input. The NRC updated its vision and strategy document (ADAMS Accession No. ML16356A670) to reflect stakeholder feedback and made it publicly available in December of 2016.

The NRC’s non-LWR vision and strategy has three strategic objectives—enhancing technical readiness, optimizing regulatory readiness, and optimizing communication. The NRC has developed implementation action plans (IAPs) to identify the specific activities the NRC will conduct in the near-term (0-5 years), mid-term (5-10 years), and long-term (beyond 10 years) timeframes to achieve non-LWR readiness. In the fall of 2016, the NRC released its draft near-term IAPs to obtain stakeholder feedback. The staff has also developed draft mid- and long-term IAPs, which were released to the public in February of 2017. The staff updated its IAPs to reflect stakeholder feedback in July of 2017 (ML17165A069 and ML17164A173).

As part of its activities related to the regulatory readiness strategic objective, the NRC will seek to optimize the regulatory framework for non-LWR reviews and licensing processes. In the near term (0–5 years), the staff will examine opportunities for flexibilities within the existing regulatory framework. Potential examples of these flexibilities include the use of a staged-review process and the use of conceptual design assessments during the pre-application period. The NRC described these approaches in “A Regulatory Review Roadmap for Non-Light Water Reactors” (ADAMS Accession No. ML16391A248), which was released as a draft on October 25, 2016, to facilitate stakeholder feedback. The NRC plans to finalize the regulatory review roadmap in the fall of 2017. Over the longer term, the NRC will examine whether a new risk-informed, performance-based regulatory framework for non-LWRs would be beneficial, effective, and efficient.

In June 2017, the NRC issued a draft document, “Nuclear Power Reactor Testing Needs and Prototype Plants for Advanced Reactor Designs,” to solicit stakeholder feedback (ADAMS Accession No. ML17025A353). This draft describes the relevant regulations governing the testing requirements for advanced reactors, the process for determining testing needs under the regulations, when a prototype plant might be needed and how it might differ from the proposed standard plant design, and licensing strategies and options that include the use of a prototype plant to meet the NRC’s testing requirements. The draft document was discussed during a public meeting on August 3, 2017. The NRC plans to address stakeholder feedback and incorporate this document into the regulatory review roadmap to be issued in the fall of 2017.

The NRC is also developing advanced reactor design criteria. As part of that effort, DOE completed a report entitled, “Guidance for Developing Principal Design Criteria for Advanced (Non-Light Water) Reactors,” and submitted it to the NRC in December 2014. The NRC reviewed DOE’s report and published draft design criteria for advanced reactors on the NRC’s public web site on April 7, 2016, to facilitate stakeholder feedback. The informal public comment period closed on June 8, 2016. After consideration of stakeholder input, the NRC issued draft regulatory guide DG-1330, “Guidance for Developing Principal Design Criteria for Non-Light Water Reactors” for formal public comment. DG-1330 was published in the *Federal*

Register on February 3, 2017, and the comment period closed on April 3, 2017. The NRC plans to issue a final regulatory guide in 2018.

The NRC is also engaged in the Licensing Modernization Project (LMP) being led by Southern Company, and coordinated by NEI, with costs-shared by DOE. The LMP's objective is to inform the NRC as the agency develops technology-inclusive, risk-informed, and performance based regulatory guidance for licensing advanced non-LWRs for NRC to review and potentially endorse. The NRC is currently reviewing two LMP white papers: Modernization of Technical Requirements for Licensing of Advanced Non-Light Water Reactors - Selection of Licensing Basis Events, Draft Report, Revision 0, (ADAMS Accession No. ML17104A254) and Modernization of Technical Requirements for Licensing of Advanced Non-Light Water Reactors - Probabilistic Risk Assessment Approach (ADAMS Accession No. ML17158B543). Additional LMP white papers regarding safety system classification and defense-in-depth are expected to be submitted in the fall of 2017.

As part of its activities related to the communications strategic objective, the NRC is conducting public meetings with stakeholders every 4 to 6 weeks. These stakeholder meetings are used by the NRC to solicit input on policy and process issues related to the possible licensing and regulation of non-LWR technologies. The NRC engaged in discussions on a utility-led licensing modernization project. White papers are being prepared by the utility-led working group and provided to the NRC staff to support preparation of an industry guidance document for potential NRC endorsement in a regulatory guide for non-LWR developers. The NRC staff is currently reviewing the first white paper on risk-informed performance-based licensing basis event selection (ADAMS Accession No.: ML17104A254) and the second white paper on probabilistic risk assessment (ADAMS Accession No. ML17158B543). The NRC and DOE also hosted a series of three Advanced Non-LWR Workshops. The last of these workshops was held on April 25 and 26, 2017. The focus of this series of workshops was to open a dialogue between key stakeholders to discuss challenges in the commercialization of non-LWR technologies and to discuss possible solutions. In addition, the NRC continues to meet with potential applicants upon request.

On November 10, 2016, the NRC and DOE signed a MOU (ADAMS Accession No. ML16215A382) on the Gateway for Accelerated Innovation in Nuclear (GAIN) Initiative. This MOU describes the roles, responsibilities, and processes related to the implementation of the DOE GAIN initiative. GAIN is an initiative that is intended to provide the nuclear energy community with increased access to the technical, regulatory, and financial support necessary to move new or advanced nuclear reactor designs toward commercialization while ensuring the continued safe, reliable, and economic operation of the existing nuclear fleet. As described in the MOU, the NRC is responsible for providing DOE and the nuclear energy community with accurate, current information on the NRC's regulations and licensing processes. DOE is then responsible for sharing that information with the prospective applicants, as appropriate.

The NRC will also continue to share information with various international groups, including the Organization for Economic Co-operation and Development's Nuclear Energy Agency (NEA), the International Atomic Energy Agency, the Generation IV International Forum, and the NRC's international regulatory counterparts. The NRC chairs NEA's ad hoc group for international regulators of non-LWRs known as the Group on the Safety of Advanced Reactors. The purpose of the Group is to bring interested regulators together to discuss common interests, practices, and problems and address both the regulatory interests and research needs in support of nuclear safety and security.