



**UNITED STATES  
NUCLEAR REGULATORY COMMISSION**  
REGION II  
245 PEACHTREE CENTER AVENUE NE, SUITE 1200  
ATLANTA, GEORGIA 30303-1257

July 21, 2017

Mr. Mano Nazar  
President and Chief Nuclear Officer  
Nuclear Division  
Florida Power & Light Co.  
Mail Stop: EX/JB  
700 Universe Blvd.  
Juno Beach, FL 33408

**SUBJECT: ST. LUCIE PLANT - NRC EVALUATION OF CHANGES, TESTS, AND  
EXPERIMENTS REPORT NUMBER 05000335/2017007 AND 05000389/2017007**

Dear Mr. Nazar:

On June 9, 2017, the U.S. Nuclear Regulatory Commission (NRC) completed an inspection at your St. Lucie Plant Units 1 and 2 and discussed the results of this inspection with Mr. D. DeBoer and other members of your staff. Additional inspection results were discussed with Mr. D. DeBoer and other members of your staff on July 20, 2017. The results of this inspection are documented in the enclosed report.

The NRC inspectors did not identify any finding or violation of more than minor significance.

This letter, its enclosure, and your response (if any) will be made available for public inspection and copying at <http://www.nrc.gov/reading-rm/adams.html> and at the NRC Public Document Room in accordance with 10 CFR 2.390, "Public Inspections, Exemptions, Requests for Withholding."

Sincerely,

**/RA/**

Jonathan H. Bartley, Chief  
Engineering Branch 1  
Division of Reactor Safety

Docket Nos. 50-335, 50-389  
License Nos. DPR-67, NPF-16

Enclosure:  
Inspection Report 05000335/2017007  
and 05000389/2017007, w/Attachment:  
Supplemental Information

cc: Distribution via ListServ

SUBJECT: ST. LUCIE PLANT - NRC EVALUATION OF CHANGES, TESTS, AND  
EXPERIMENTS REPORT NUMBER 05000335/2017007 AND 05000389/2017007

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ADAMS:  Yes      ACCESSION NUMBER: \_\_\_\_\_       SUNSI REVIEW COMPLETE       FORM 665 ATTACHED

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**U. S. NUCLEAR REGULATORY COMMISSION**

**REGION II**

Docket Nos.: 050000335, 05000389

License Nos.: DPR-67, NPF-16

Report Nos.: 05000335/2017007, 05000389/2017007

Licensee: Florida Power & Light Company (FP&L)

Facility: St. Lucie Plant, Units 1 & 2

Location: 6501 South Ocean Drive  
Jensen Beach, FL 34957

Dates: June 5 – June 9, 2017

Inspectors: G. Ottenberg, Senior Reactor Inspector (Lead)  
R. Patterson, Reactor Inspector  
M. Greenleaf, Reactor Inspector  
S. Downey, Senior Reactor Inspector (Trainee)

Approved by: Jonathan H. Bartley, Chief  
Engineering Branch 1  
Division of Reactor Safety

Enclosure

## **SUMMARY**

Inspection Report (IR) 05000335/2017-007, 05000389/2017-007; 06/05/2017 – 06/09/2017; St. Lucie Plant, Units 1 and 2; NRC Evaluations of Changes, Tests, and Experiments.

The inspection activities described in this report were performed between June 5 and June 9, 2017, by four Nuclear Regulatory Commission (NRC) inspectors from Region II. The significance of inspection findings are indicated by their color (i.e., greater than Green, or Green, White, Yellow, Red) and determined using IMC 0609, "Significance Determination Process" dated April 29, 2015. Cross-cutting aspects are determined using IMC 0310, "Components Within the Cross Cutting Areas" dated December 4, 2014. All violations of NRC requirements are dispositioned in accordance with the NRC's Enforcement Policy dated November 1, 2016. The NRC's program for overseeing the safe operation of commercial nuclear power reactors is described in NUREG-1649, "Reactor Oversight Process," Revision 6.

No findings were identified.

## REPORT DETAILS

### 1. REACTOR SAFETY

#### Cornerstones: Initiating Events, Mitigating Systems, and Barrier Integrity

#### 1R17 Evaluations of Changes, Tests, and Experiments (71111.17T)

##### a. Inspection Scope

Evaluations of Changes, Tests, and Experiments: The inspectors reviewed six safety evaluations performed pursuant to Title 10, *Code of Federal Regulations* (CFR) 50.59, "Changes, tests, and experiments," to determine if the evaluations were adequate and that prior NRC approval was obtained as appropriate. The inspectors also reviewed sixteen screenings and zero applicability determinations where licensee personnel had determined that a 10 CFR 50.59 evaluation was not necessary. The inspectors reviewed these documents to determine if:

- the changes, tests, or experiments performed were evaluated in accordance with 10 CFR 50.59 and that sufficient documentation existed to confirm that a license amendment was not required;
- the safety issues requiring the changes, tests or experiments were resolved;
- the licensee conclusions for evaluations of changes, tests, or experiments were correct and consistent with 10 CFR 50.59; and
- the design and licensing basis documentation used to support the change was updated to reflect the change.

The inspectors used, in part, Nuclear Energy Institute (NEI) 96-07, "Guidelines for 10 CFR 50.59 Implementation," Revision 1, to determine acceptability of the completed evaluations and screenings. The NEI document was endorsed by the NRC in Regulatory Guide 1.187, "Guidance for Implementation of 10 CFR 50.59, Changes, Tests, and Experiments," dated November 2000.

This inspection constituted 22 evaluations, screenings, and/or applicability determination samples as defined in Inspection Procedure (IP) 71111.17-05. Documents reviewed are listed in the Attachment.

##### b. Findings

No findings were identified. However, the following unresolved item (URI) was identified.

#### 1. (Opened) Question Regarding Adequacy of Evaluation of Hot Leg Injection Actions

Introduction: The team identified an unresolved item (URI) regarding the adequacy of a 10 CFR 50.59 evaluation performed for procedures related to post-accident hot leg injection (HLI) alignment.

Description: Engineering Change (EC) 284437, "Evaluation of Actions Required to Mitigate Hot Leg Injection Single Failure Vulnerability, revision 0, was created to proceduralize actions needed to mitigate a single failure vulnerability associated with HLI

by installing temporary power jumpers to valves in the HLI flow path. The actions in the procedures were generated to provide specific direction to operators and maintenance personnel to install electrical jumper cabling to provide power to un-powered motor operated valves in the HLI flow paths in case one of the electrical power trains was disabled as a result of a postulated single failure. Certain electrical single failures could disable both the primary and alternate methods of aligning coolant injection into the reactor coolant system hot legs, which is needed to preclude boron precipitation, which could interrupt long term core cooling during postulated design basis accidents. The electrical single failures of concern were described in St. Lucie Unit 1 licensee event report 2011-003-01, Long-Term Post-LOCA Hot Leg Injection Single Failure Vulnerability.

The inspectors noted that the St. Lucie Unit 1 UFSAR sections 8.3.1.2.1, 8.3.1.2.2, and 8.3.1.2.3, "Redundancy," "Electrical Separation," and "Physical Separation," respectively, did not explicitly account for the installation of electrical jumper cables to provide power between the electrical trains as described in the newly created procedures. The inspectors also noted that the guidance in NEI 96-07, Guidelines for 10 CFR 50.59 Implementation, revision 1, section 4.3.2, "Does the Activity Result in More Than a Minimal Increase in the Likelihood of Occurrence of a Malfunction of an SSC Important to Safety?" considers a reduction in system/equipment redundancy, diversity, separation, or independence, as a more than minimal increase and would require NRC review and approval prior to implementation. The inspectors were unable to locate all relevant licensing basis documents regarding HLI action requirements for redundancy, and electrical and physical separation, or prior NRC approvals of these actions during the inspection. Review of these documents is needed to determine if the licensee's response in Question 2 of the 50.59 evaluation was adequate in its conclusion that a more than minimal increase in the likelihood of a malfunction of a structure, system, or component (SSC) important to safety previously evaluated in the UFSAR did not exist.

Additionally, the inspectors noted that the St. Lucie Unit 1 UFSAR Table 15.4.1-11, "LOCA Dose Summary," contained results of consequence analyses, but only provided results for offsite consequences and the control room. At the time of the inspection, it was unclear to inspectors how the dose consequences due to actions outside of the control room had previously been evaluated. Further, the inspectors noted that the St. Lucie Unit 1 UFSAR, Table 12.1-11, "Areas Identified in Shielding Review as Requiring Accessibility Following an Accident," did not identify the electrical bus rooms, where the jumpers would be installed, nor the operational support center (OSC) where the jumpers are stored, as requiring occupancy post-accident. The guidance in NEI 96-07, revision 1, section 4.3.4, "Does the Activity Result in More Than a Minimal Increase in the Consequences of a Malfunction?" utilizes the guidance in section 4.3.3 for determining if a more than minimal increase in the consequence of a malfunction existed. The guidance in section 4.3.3 stated, "Activities affecting on-site dose consequences that may require prior NRC approval are those that impede required actions inside or outside the control room to mitigate the consequences of reactor accidents. For changes affecting the dose to operators performing required actions outside the control room, an increase is considered more than minimal if the resultant 'mission dose' exceeds applicable GDC 19 criteria." Review of the licensee's 50.59 evaluation determined that the evaluation only addressed the potential for offsite dose increases and did not address the potential increase in dose resulting from the procedural actions for installation of the jumpers, which are performed outside of the control room. Additional

review is needed to determine if onsite dose assessment of the actions was previously addressed by the licensee or considered by the NRC.

This issue is a URI pending the determination of whether a performance deficiency or violation of NRC requirements exists. (URI 05000335/2017007-01, Question Regarding Adequacy of Evaluation of Hot Leg Injection Actions)

4OA6 Meetings, Including Exit

On June 9, 2017, the inspectors presented inspection results to Mr. D. DeBoer and other members of the licensee's staff. On July 20, 2017, a re-exit meeting was conducted via teleconference to present the final inspection results to Mr. D. DeBoer and other members of the licensee's staff. The inspectors verified no proprietary information was retained or documented in this report.

ATTACHMENT: SUPPLEMENTAL INFORMATION

## **SUPPLEMENTAL INFORMATION**

### **KEY POINTS OF CONTACT**

#### **Licensee personnel**

D. DeBoer, PSL Site Director  
D. Cecchett, Nuclear Licensing Principal  
S. Cornell, Nuclear Staff Engineer  
T. Falkiewicz, Nuclear Engineer I  
K. Frehafer, Nuclear Licensing Principal  
W. Laframboise, Nuclear Engineering Site Manager- Design  
R. Raldiris, Nuclear Engineer Principal  
M. Snyder, Nuclear Site Licensing Manager

#### **NRC personnel**

D. Beaulieu, Reactor Operations Engineer  
P. Buckberg, Project Manager  
T. Morrissey, Senior Resident Inspector, St. Lucie Plant  
L. Suggs, Chief, Projects Branch 3

### **LIST OF ITEMS OPENED, CLOSED, AND DISCUSSED**

#### **Opened**

05000335/2017007-01	URI	Question Regarding Adequacy of Evaluation of Hot Leg Injection Actions (Section 1R17.b.1)
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## LIST OF DOCUMENTS REVIEWED

### 10 CFR 50.59 Evaluations

- EC 277049, Permanent Removal of St. Lucie Unit 1 RCP 1A1 Whip (Cable), Rev. 0
- EC 278372, U2 FHB Vent Stack Rad Monitor Replacement, Rev. 1
- EC 279190, Removal of Internals from Check Valve V12174, Rev. 0
- EC 282127, Evaluation of Linear Heat Rate Limit Increase in COLR, Rev. 0
- EC 284437, Engineering Evaluation PSL-ENG-SENS-15-001, Evaluation of Actions Required to Mitigate Hot Leg Injection Single Failure Vulnerability, Rev. 0
- EC 287794, Connection of Temporary Air Compressors to Support Unit 1 Instrument Air Compressor Replacement, Rev. 0

### 10 CFR 50.59 Screenings

- EC 235776, Control Room Outside Air Intake (CROAI) Radiation Monitors Firmware Upgrade, Rev. 0
- EC 274370, Unit 2 Screen Wash System Strainer Replacement (SOER-07-2), Rev. 0
- EC 277356, Documentation Update for New Homewood Energy Services AFW Pump Motor Serial No. 2S-2EM10186, Rev. 2
- EC 278595, Agastat DSC Relay Replacement, Rev. 0
- EC 278639, PSL 1 & 2 Flex Connections For Non-Qualified Sources, Rev. 3
- EC 280780, Check and Vent valves will be added to the Nitrogen Fill Lines for the Unit 1 HCV-08-1A and HCV-08-1B MSIV Actuators, Rev. 0
- EC 280854, RCP 1A2 Replacement Motor Installation Rev. 0
- EC 282700, Unit 1 Alternate Spent Fuel Pool Cooling Evaluation, Rev. 0
- EC 283075, St. Lucie Unit 2 Permanent Reactor Cavity Seal / Shield Ring, Rev. 1
- EC 284197, 480 SWGR 2B2 and 2B5 Close and Trip Fuses, Rev. 0
- EC 284271, Unit 1 MOV Torque Switch Bypass, Rev. 0
- EC 284412, Install New Register in HVAC Duct Above Charging Pumps, Rev. 0
- EC 284575, Update Unit 1 UFSAR Containment Boundary Quality Group Designation, Rev. 0
- EC 285575, HPSI Pump 1B Motor Replacement, Rev. 0
- EC 285611, CS Pump 1B Motor Replacement, Rev. 1
- EC 287662, 120V PP-1A Undervoltage Relay Setting, Rev. 0

### 10 CFR 50.59 Applicability Determinations

None

### Procedures

- 1-EOP-04, Steam Generator Tube Rupture (SGTR), Rev. 31
- 1-EOP-99, Appendix O, Revs. 55 and 56
- 1-EOP-99, Appendix U, Local Operation of Unit 1 Atmospheric Dump Valves, Rev. 60
- 1-GME-100.03, Installation and Removal of Temporary Power Jumpers, Rev. 5
- 1-GME-100.03A, Installation and Removal of Temporary Power Jumpers, Rev. 3
- 1-GMM-04.01, PCTM – ALTERNATE SPENT FUEL POOL COOLING FOR PSL UNIT 1, Rev. 0
- 1-OSP-12.01, FLOW TEST OF THE UNIT 2 TO UNIT 1 CONDENSATE STORAGE TANK CROSS-TIE, Rev. 11
- 2-SMI-26.63A, Control Room Outside Air Intake Radiation Monitors Calibration- Channel SA, Revs. 5 & 6
- EN-AA-100-1003, Control of Design Interfaces, Rev. 2
- EN-AA-203-1201, 10 CFR Applicability and 10 CFR 50.59 Screening Reviews, Rev. 9
- EN-AA-203-1202, 10 CFR 50.59 Evaluation, Rev. 1
- EN-AA-205-1100, Design Change Packages, Rev. 21

EN-AA-205-1102, Temporary Configuration Changes, Rev. 9  
 ER-AA-123-1000, NRC Generic Letter 89-13 Program, Rev. 0  
 LI-AA-101-1003, Updated Final Safety Analysis Report (UFSAR) Revision, Rev. 5

#### Drawings

2998-G-080, Sht. 2B, Flow Diagram Feedwater and Condensate Systems, Rev. 39  
 2998-G-125, Safety Injection, Rev. 15  
 8770-9949, 8" Check Valve I-V-12-174.176 (652), Rev. 3  
 8770-9949-EC279190, 8" Check Valve I-V-12-174.176 (652), Rev. 1  
 8770-B-124, Isometric Valve Drawing SE-02-3, Rev. 9  
 8770-B-231, Sht. 29-20, Instrument Installation Detail N2 Backup System MSIV Actuators HCV-08-1A and HCV-08-1B, Revs. 0 and 1  
 8770-B-327, Shutdown Cooling Drawings, Rev. 3  
 8770-G-078, CVCS Drawing, Rev. 25  
 8770-G-078, Safety Injection System Flow Diagram, Rev. 30  
 8770-G-078, Sht. 140, Flow Diagram Fuel Pool System, Rev. 22  
 8770-G-079, Sht. 7, Flow Diagram Main Steam, Revs. 13 and 14  
 8770-G-080, Sht. 4, Flow Diagram Feedwater and Condensate Systems, Rev. 45  
 8770-G-080-SH4- EC279190, Flow Diagram Feedwater and Condensate Systems, Rev. 1  
 8770-G-083, Sht. 1A, Flow Diagram Component Cooling System, Rev. 62  
 8770-G-083, Sht. 1B, Flow Diagram Component Cooling System, Rev. 64  
 8770-G-083, Sht. 2, Flow Diagram Component Cooling System, Rev. 4  
 8770-G-085, Sht. 1A, Flow Diagram Service Air System, Rev. 41  
 8770-G-085, Sht. 1B, Flow Diagram Service Air System, Rev. 34  
 8770-G-085, Sht. 2A, Flow Diagram Instrument Air System, Rev. 40  
 8770-G-085, Sht. 2B, Flow Diagram Instrument Air System, Rev. 64  
 8770-G-085, Sht. 2C, Flow Diagram Instrument Air System, Rev. 48  
 8770-G-085, Sht. 3, Flow Diagram Instrument Air System, Rev. 25  
 8770-G-085, Sht. 4A, Flow Diagram Instrument Air System, Rev. 43  
 8770-G-170, SW Pipe Leak Repair, Rev 0  
 8770-G-275, One-Line 480V Drawings. Rev. 32  
 8770-G-794, Reactor Building Equipment Supports, Revs. 2 and 6  
 EC282700-M-001, St. Lucie Plant- Unit No. 1 Alternate Spent Fuel Pool Cooling PI&D-Configuration 1, Rev. 0  
 EC282700-M-002, St. Lucie Plant- Unit No. 1 Alternate Spent Fuel Pool Cooling PI&D-Configuration 2, Rev. 0

#### Calculations

CN-MRCDA-09-68, RCP LOCA Cable Restraint Elimination Evaluation for St. Lucie Units 1 and 2, Rev. 1  
 CN-OA-08-40, St. Lucie Unit 1 Post-LOCA Long Term Cooling ECCS Performance Analysis for Extended Power Uprate, Rev. 2  
 NSSS-026, Safety Injection Tank Discharge, Rev 0  
 PSL-2FJF-13-233, St. Lucie Unit 2 Cycle 2 Groundrules and Related Data, Rev 0

#### Corrective Action Documents

AR 1692101	AR 2126790
AR 1803405	AR 2192104
AR 1916626	CR 2008-35069

Self-Assessment Reports

AR 2072630, St. Lucie Engineering Self-Assessment Evaluation of Changes, Tests, and Experiments Inspection, May 2017

Miscellaneous Documents

06-0632-TR-009, Feedwater IST Bases Document, Rev. 0  
 1214-1978, Termination for EBASCO Spec. #1214-1978 St. Lucie II, Jan. 19, 1981  
 17859-02B, Qualification Test Program on Raychem Nuclear Cable Splices, Okonite Tape Splices, Kerite Tape Splices, Scotch Tape Splices and Butt Splices as Installed on Various Wire Insulations at Commonwealth Edison Company's LaSalle, Dresden, and Quad Cities Generating Stations Volume II of II, Rev. A  
 2998-3855, RCP Vendor Manual, Rev. 27  
 2998-A-451-1000, Equipment Qualification Report and Guidebook, Rev. 12  
 2998-A-451-6.2, Equipment Qualification Document Package 3M Corporation Splices, Rev. 8  
 2998-A-451-74.1, Equipment Qualification Document Package GE/Schulz and Homewood Auxiliary Feedwater Pump Motors, Rev. 0  
 54067TR08, Qualification Test Report for Class 1E Safety-Related 600 HP Motor Supplied by Homewood Products Corporation for Use in Nuclear Power Plants, Mar. 21, 2008  
 AREVA Document No. 77-5069878-004, Replacement Steam Generator Report for Florida Power and Light, St. Lucie Unit 2, Rev. 4  
 Change Request Notice No. 02019-14763  
 Change Request Notice No. 07127-15145  
 DBD-CVCS-1, Chemical and Volume Control System DBD, Rev. 7  
 DBD-HVAC-2, Safety Related HVAC Systems, Rev. 5  
 DBD-RCS-1, Reactor Coolant System DBD, Rev. 6  
 E-115-0870, Model RM-80 Microprocessor Software Design Document, Rev. F  
 EC 1-EOP-04, Steam Generator Tube Rupture (SGTR), Rev. 31  
 EC 1-EOP-04/2114979, Steam Generator Tube Rupture, Rev. 31  
 EC 1-PSTG-04/2114979, Emergency Operating Procedure 1-EOP-04, Steam Generator Tube Rupture, Plant Specific Technical Guideline, Rev. 31  
 EC 205864, Repair damaged NNS Service Water Piping 3-SW-69, Rev. 0  
 EC 280156, Concrete Backfill in Lieu of Class I Fill for I-30"-CW-90, Revs. 0 and 1  
 EC 283094, PSL2 UFSAR Update for OSG/RSG Structural Codes, Rev. 0  
 EC 283720, 2B2 SIT Discharge Header Piping Repair, Revs. 2 and 4  
 EC 285864, Rev. 4  
 EC-277356, Documentation Update for New Homewood Energy Services AFW Pump Motor Serial No. 2S-2EM10186, Rev. 2  
 EC-278327, SBCS DCS Conformance Modification, Rev. 3  
 EC-278372, Unit 2 Fuel Handling Building Vent Stack Radiation Monitor RM-26-12 Replacement, Rev. 7  
 EC-278595, Agastat DSC Relay Replacement, Rev. 0  
 EC-278595, Replace Auxiliary Relay 42X for V3634 Valve 480V MCC 1B6-FF1, Rev. 0  
 EC-282127, St. Lucie Unit 1 Cycle 26 Core Reload, Rev. 4  
 EC-284197, 480 SWGR 2B2 & 2B5 Close and Trip Fuses NRC Commitment – Ref NFFPA-805 LAR, Rev. 0  
 EC-285575, HPSI Pump 1B Motor Replacement, Rev. 0  
 EC-285611, CS Pump 1B Motor Replacement, Rev. 3  
 EC-287662, 120V PP-1A Undervoltage Relay Setting, Rev. 0  
 Emergency Procedure Guideline E-4, Steam Generator Tube Rupture Recovery Guideline, Rev. 6

EN-AA-100-1003-F03, Owner Acceptance review of RSG Report for Florida Power and Light St. Lucie Unit 2, Rev. 4  
 F-C5022-2, Qualification Tests of Terminal Blocks and Splice-Insulating Assemblies in a Simulated Loss-of-Coolant-Accident Environment – Phase B, Nov. 1978  
 FPL-1, Quality Assurance Topical Report, Rev. 21  
 L-76-27, Letter from R. Uhrig (FPL) to J. Davis (NRC), Re: 10 CFR 50.55(e) Interim Report Improper Containment Boundary Quality Group Designation, St. Lucie Unit 1, dated 1/22/76  
 L-76-61, Letter from R. Uhrig (FPL) to J. Davis (NRC), Re: 10 CFR 50.55(e) Final Report Improper Containment Boundary Quality Group Designation, St. Lucie Unit 1, dated 2/19/76  
 Letter from D. Thompson (NRC) to R. Uhrig (FPL), Improper Quality Group Designation for Containment Isolation Valves and Associated Piping, dated 3/4/76  
 Letter from H. Thornburg (NRC) to R. Uhrig (FPL), Improper Quality Group Designation for Containment Isolation Valves and Associated Piping, dated 2/10/76  
 Operations Training Plans for Hot Leg Injection Commitment  
 PC/M 07081, Unit 2 CROAI Radiation Monitor Noise Reduction Firmware Modification, Rev. 0  
 PCR 1692701  
 PCR-2114228, 1/2 EOP 10 – Station Blackout SBO, Mar. 1, 2016  
 PSL-ENG-SECS-07-014, Engineering Evaluation for Controlled Low-Strength Material (CLSM) for use as Class I or Lesser Classification Backfill Material, Rev. 0  
 PSL-ENG-SEIS-08-017, Control Room Outside Air (CROAI) Radiation Monitor Actuation Setpoint Basis Document, Rev. 1  
 Specification for Excavation and Backfill: Seismic Category I and Non-seismic Category, 02/1988  
 STD-C-004, Seismic Response Spectra, St. Lucie Units 1 & 2, Rev. 0  
 STD-C-008, General Civil Design Criteria for St. Lucie Units 1 & 2, Rev. 2  
 Technical Specifications Amendment 152, St. Lucie Plant Unit 2 – Issuance of Amendment \ Regarding Alternative Source Term (TAC NO. MD6202), Sep. 29, 2008  
 WO 40275887, U1: V12825- Broken Handwheel/ Stem  
 WO 40516072, U2: TS/CROAI'S RIM-26-61/62/65/66 Rad Mon Response Time Test

#### Corrective Action Documents Written as a Result of the Inspection

AR 2209110, EC 278372 Trend on Quality of Engineering Review of ECs  
 AR 2209291, 2017 NRC 50.59 Inspection- Missing Figures  
 AR 2209295, 2017 NRC 50.59 Inspection- Missing Quality Standards  
 AR 2209488, 2017 NRC 50.59 Inspection URI- Removal of V12174 Disc  
 AR 2209489, 2017 NRC 50.59 Inspection URI- CCW Piping Quality Group  
 AR 2209490, Initial Unit 1 HLI Training Frequency Not Met  
 AR 2209492, 2017 NRC 50.59 Inspection- CROAI RM Smoothing Algorithm  
 AR 2209495, Operator/Maintenance Inter-Related Task Training  
 AR 2209501, NRC50.59 Insp Question: EQ Doc-Pac Aging Basis for 3M Tape  
 AR 2211095, U2 CROAI RM Improper Time Response Test Method  
 AR 2211096, EPU Evaluation Error for CROAI RM  
 AR 2213564, 2017 NRC 50.59 Inspection URI- EC 239885 (PCM91241)  
 AR 2214470, 2017 NRC 50.59 Audit EQ Doc Pac for 3M Tape