Facility: Point Be	each	Date of Exam: May 2017																
						RO	K/A	Cate	gory	Poin	ıts				SR	O-Onl	ly Poir	nts
Tier	Group	K 1	K 2	K 3	K 4	K 5	K 6	A 1	A 2	A 3	A 4	G*	Total		A2	C) *	Total
1.	1	3	3	3				3	3			3	18		3	;	3	6
Emergency & Abnormal	2	2	1	1		N/A		1	2	N.	/A	2	9		2	:	2	4
Plant Evolutions	Tier Totals	5	4	4				4	5			5	27		5		5	10
	1	3	3	3	3	3	2	2	2	2	2	3	28		3	:	2	5
2. Plant	2	1	0	1	1	1	1	1	1	1	1	1	10	0	1	:	2	3
Systems	I Idill						3	3	3	3	3	4	38		4	4	4	8
3. Generic k	Generic Knowledge and Abilities							2		3		4		1	2	3	4	_
	Categories							3		3		2 10		2	2	1	2	7

Note:

- 1. Ensure that at least two topics from every applicable K/A category are sampled within each tier of the RO and SRO-only outlines (i.e., except for one category in Tier 3 of the SRO-only outline, the "Tier Totals" in each K/A category shall not be less than two). (One Tier 3 Radiation Control K/A is allowed if the K/A is replaced by a K/A from another Tier 3 Category).
- 2. The point total for each group and tier in the proposed outline must match that specified in the table. The final point total for each group and tier may deviate by ±1 from that specified in the table based on NRC revisions. The final RO exam must total 75 points and the SRO-only exam must total 25 points.
- 3. Systems/evolutions within each group are identified on the associated outline; systems or evolutions that do not apply at the facility should be deleted with justification; operationally important, site-specific systems/evolutions that are not included on the outline should be added. Refer to Section D.1.b of ES-401 for guidance regarding the elimination of inappropriate K/A statements.
- 4. Select topics from as many systems and evolutions as possible; sample every system or evolution in the group before selecting a second topic for any system or evolution.
- 5. Absent a plant-specific priority, only those K/As having an importance rating (IR) of 2.5 or higher shall be selected. Use the RO and SRO ratings for the RO and SRO-only portions, respectively.
- 6. Select SRO topics for Tiers 1 and 2 from the shaded systems and K/A categories.
- 7. The generic (Ġ) K/As in Tiers 1 and 2 shall be selected from Section 2 of the K/A Catalog, but the topics must be relevant to the applicable evolution or system. Refer to Section D.1.b of ES-401 for the applicable K/As.
- 8. On the following pages, enter the K/A numbers, a brief description of each topic, the topics' importance ratings (IRs) for the applicable license level, and the point totals (#) for each system and category. Enter the group and tier totals for each category in the table above; if fuel handling equipment is sampled in a category other than Category A2 or G* on the SRO-only exam, enter it on the left side of Column A2 for Tier 2, Group 2 (Note #1 does not apply). Use duplicate pages for RO and SRO-only exams.
- 9. For Tier 3, select topics from Section 2 of the K/A catalog, and enter the K/A numbers, descriptions, IRs, and point totals (#) on Form ES-401-3. Limit SRO selections to K/As that are linked to 10 CFR 55.43.
- G* Generic K/As

ES-401			P	WR	Exai	mina	tion Outline	Form E	S-401-2
Emerge	ency	and	Abno	orma	ıl Pla	nt E	volutions - Tier 1/Group 1 (RO)		
E/APE # / Name / Safety Function	K 1	K 2	K 3	A 1	A 2	G	K/A Topic(s)	IR	#
000007 Reactor Trip - Stabilization - Recovery / 1		0					EK2.03-Knowledge of the interrelations between a reactor trip and the following: Reactor trip status panel	3.5	1
000008 Pressurizer Vapor Space Accident / 3		0					AK2.03-Knowledge of the interrelations between the Pressurizer Vapor Space Accident and the following: Controllers and positioners	2.5	2
000009 Small Break LOCA / 3			2				EK3.28-Knowledge of the reasons for the following responses as the apply to the small break LOCA: Manual ESFAS initiation requirements	4.5	3
000011 Large Break LOCA / 3			0 6				EK3.06-Knowledge of the reasons for the following responses as the apply to the Large Break LOCA: Actuation of Phase A and B during LOCA initiation	4.3	4
000015 RCP Malfunctions / 4 000017 RCP Malfunctions (Loss of RC Flow) / 4				2 2			AA1.22-Ability to operate and / or monitor the following as they apply to the Reactor Coolant Pump Malfunctions (Loss of RC Flow): RCP seal failure/malfunction	4.0	5
000022 Loss of Rx Coolant Makeup / 2				0 2			AA1.02-Ability to operate and / or monitor the following as they apply to the Loss of Reactor Coolant Makeup: CVCS charging low flow alarm, sensor, and indicator	3.0	6
000025 Loss of RHR System / 4					0 5		AA2.05-Ability to determine and interpret the following as they apply to the Loss of Residual Heat Removal System: Limitations on LPI flow and temperature rates of change	3.1	7
000026 Loss of Component Cooling Water / 8									
000027 Pressurizer Pressure Control System Malfunction / 3						02. 12	2.2.12-Knowledge of surveillance procedures.	3.7	8
000029 ATWS / 1									
000038 Steam Gen. Tube Rupture / 3	0 1						EK1.01-Knowledge of the operational implications of the following concepts as they apply to the SGTR: Use of steam tables	3.1	9
000040 Steam Line Rupture - Excessive Heat Transfer / 4		0					AK2.01-Knowledge of the interrelations between the Steam Line Rupture and the following: Valves	2.6	10
WE12 Uncontrolled Depressurization of all Steam Generators / 4									
000054 (CE/E06) Loss of Main Feedwater / 4									
000055 Station Blackout / 6									
000056 Loss of Off-site Power / 6			0 2				AK3.02-Knowledge of the reasons for the following responses as they apply to the Loss of Offsite Power: Actions contained in EOP for loss of offsite power	4.4	11
000057 Loss of Vital AC Inst. Bus / 6									
000058 Loss of DC Power / 6				0 1			AA1.01-Ability to operate and / or monitor the following as they apply to the Loss of DC Power: Cross-tie of the affected dc bus with the alternate supply	3.4	12
000062 Loss of Nuclear Svc Water / 4					0 4		AA2.04-Ability to determine and interpret the following as they apply to the Loss of Nuclear Service Water: The normal values and upper limits for the temperatures of the components cooled by SWS	2.5	13
000065 Loss of Instrument Air / 8					0 5		AA2.05-Ability to determine and interpret the following as they apply to the Loss of Instrument Air: When to commence plant shutdown if instrument air pressure is decreasing	3.4	14
W/E04 LOCA Outside Containment / 3						01. 31	2.1.31-Ability to locate control room switches, controls, and indications, and to determine that they correctly reflect the desired plant lineup.	4.6	15
W/E11 Loss of Emergency Coolant Recirc. / 4						04. 46	2.4.46-Ability to verify that the alarms are consistent with the plant conditions.	4.2	16
BW/E04; W/E05 Inadequate Heat Transfer - Loss of Secondary Heat Sink / 4	0 1						EK1.01-Knowledge of the operational implications of the following concepts as they apply to the (Loss of Secondary Heat Sink): Components, capacity, and function of emergency systems	3.8	17
000077 Generator Voltage and Electric Grid Disturbances / 6	0 2						AK1.02-Knowledge of the operational implications of the following concepts as they apply to Generator Voltage and Electric Grid Disturbances: Over-excitation	3.3	18
K/A Category Totals:	3	3	3	3	3	3	Group Point Total:	•	18

ES-401	ergen	cv an					tion Outline volutions - Tier 1/Group 2 (RO)	Form E	S-401-2
E/APE # / Name / Safety Function	K 1	K 2	K 3	A 1	A 2	G	K/A Topic(s)	IR	#
000001 Continuous Rod Withdrawal / 1									
000003 Dropped Control Rod / 1						01. 27	2.1.27-Knowledge of system purpose and/or function.	3.9	19
000005 Inoperable/Stuck Control Rod / 1						21			
000024 Emergency Boration / 1	01						AK1.01-Knowledge of the operational implications of the following concepts as they apply to Emergency Boration: Relationship between boron addition and change in T-ave	3.4	20
000028 Pressurizer Level Malfunction / 2									
000032 Loss of Source Range NI / 7									
000033 Loss of Intermediate Range NI / 7									
000036 Fuel Handling Accident / 8									
000037 Steam Generator Tube Leak / 3									
000051 Loss of Condenser Vacuum / 4									
000059 Accidental Liquid RadWaste Rel. / 9									
000060 Accidental Gaseous Radwaste Rel. / 9									
000061 ARM System Alarms / 7					01		AA2.01-Ability to determine and interpret the following as they apply to the Area Radiation Monitoring (ARM) System Alarms: ARM panel displays	3.5	21
000067 Plant Fire On-site / 8							3. 7.7		
000068 Control Room Evac. / 8						04. 18	2.4.18-Knowledge of the specific bases for EOPs.	3.3	22
000069 Loss of CTMT Integrity / 5									
W/E14 High Containment Pressure / 5									
000074 Inad. Core Cooling / 4	08						EK1.08-Knowledge of the operational implications of the following concepts as they apply to the Inadequate Core Cooling: Definition of subcooled liquid	2.8	23
W/E06 Degraded Core Cooling / 4									
W/E07 Saturated Core Cooling / 4									
000076 High Reactor Coolant Activity / 9									
W/E01 Rediagnosis / 3		01					EK2.01-Knowledge of the interrelations between the (Reactor Trip or Safety Injection/Rediagnosis) and the following:Components, and functions of control and safety systems, including instrumentation, signals, interlocks, failure modes, and automatic and manual features	3.3	24
W/E02 SI Termination / 3									
W/E13 Steam Generator Over-pressure / 4									
W/E15 Containment Flooding / 5			04				EK3.04-Knowledge of the reasons for the following responses as they apply to the (Containment Flooding): RO or SRO function within the control room team as appropriate to the assigned position, in such a way that procedures are adhered to and the limitations in the facilities license and amendments are not violated	2.9	25
W/E16 High Containment Radiation / 9									
W/E03 LOCA Cooldown - Depress. / 4				03			EA1.03-Ability to operate and / or monitor the following as they apply to the (LOCA Cooldown and Depressurization): Desired operating results during abnormal and emergency situations	3.7	26
W/E09 Natural Circulation Operations / 4									
W/E10 Natural Circulation with Steam Voide in Vessel with/without RVLIS. / 4									1
W/E08 RCS Overcooling - PTS / 4					02		EA2.02-Ability to determine and interpret the following as they apply to the (Pressurized Thermal Shock): Adherence to appropriate procedures and operation within the limitations in the facility's license and amendments	3.5	27
K/A Category Totals:	2	1	1	1	2	2	Group Point Total:		9

ES-401 PWR Examination Outline Form ES-401-2 Plant Systems - Tier 2/Group 1 (RO) System # / Name K K K K K K K K K A A A A G K A A A G K A A A G K A A A A											S-401-2			
System # / Name		K 2			K 5	Κ	Α	Α	Α	Α			IR	#
003 Reactor Coolant Pump						14			0 3			K6.14-Knowledge of the effect of a loss or malfunction on the following will have on the RCPS: Starting requirements A3.03-Ability to monitor automatic operation of the RCPS, including: Seal D/P	2.6 3.2	28 29
004 Chemical and Volume Control											04. 31	2.4.31-Knowledge of annunciator alarms, indications, or response procedures.	4.2	30
005 Residual Heat Removal										0 1		A4.01-Ability to manually operate and/or monitor in the control room: Controls and indication for RHR pumps	3.6	31
006 Emergency Core Cooling			0 2								02. 36	K3.02-Knowledge of the effect that a loss or malfunction of the ECCS will have on the following: Fuel 2.2.36-Ability to analyze the effect of maintenance activities, such as degraded power sources, on the status of limiting conditions for operations.	4.3 3.1	32 33
007 Pressurizer Relief/Quench Tank											04. 20	2.4.20-Knowledge of the operational implications of EOP warnings, cautions, and notes.	3.8	34
008 Component Cooling Water	0 4			0								K1.04-Knowledge of the physical connections and/or cause-effect relationships between the CCWS and the following systems: RCS, in order to determine source(s) of RCS leakage into the CCWS K4.01-Knowledge of CCWS design feature(s) and/or interlock(s) which provide for the following: Automatic start of standby pump	3.3 3.1	35 36
010 Pressurizer Pressure Control	0 6											K1.06-Knowledge of the physical connections and/or cause-effect relationships between the PZR PCS and the following systems: CVCS	2.9	37
012 Reactor Protection		0			0 1							K2.01-Knowledge of bus power supplies to the following: RPS channels, components, and interconnections K5.01-Knowledge of the operational implications of the following concepts as the apply to the RPS: DNB	3.3 3.3	38 39
013 Engineered Safety Features Actuation		0					0					K2.01-Knowledge of bus power supplies to the following: ESFAS/safeguards equipment control A1.01-Ability to predict and/or monitor changes in parameters (to Prevent exceeding design limits) associated with operating the ESFAS controls including: RCS pressure and temperature	3.6 4.0	40 41
022 Containment Cooling			0 1									K3.01-Knowledge of the effect that a loss or malfunction of the CCS will have on the following: Containment equipment subject to damage by high or low temperature, humidity, and pressure	2.9	42
025 Ice Condenser														
026 Containment Spray			0 2									K3.02-Knowledge of the effect that a loss or malfunction of the CSS will have on the following: Recirculation spray system	4.2	43
039 Main and Reheat Steam				0								K4.04-Knowledge of MRSS design feature(s) and/or interlock(s) which provide for the following: Utilization of steam pressure program control when steam dumping through atmospheric relief/dump valves, including T-ave. limits	2.9	44
059 Main Feedwater				1 6								K4.16-Knowledge of MFW design feature(s) and/or interlock(s) which provide for the following: Automatic trips for MFW pumps	3.1	45
061 Auxiliary/Emergency Feedwater					0 1							K5.01-Knowledge of the operational implications of the following concepts as the apply to the AFW: Relationship between AFW flow and RCS heat transfer	3.6	46
062 AC Electrical Distribution							0					A1.03-Ability to predict and/or monitor changes in parameters (to prevent exceeding design limits) associated with operating the ac distribution system controls including: Effect on instrumentation and controls of switching power supplies	2.5	47
063 DC Electrical Distribution	0 2											K1.02-Knowledge of the physical connections and/or cause-effect relationships between the DC electrical system and the following systems: AC electrical system	2.7	48
064 Emergency Diesel Generator						0 7		1 3				K6.07-Knowledge of the effect of a loss or malfunction of the following will have on the ED/G system: Air receivers A2.13-Ability to (a) predict the impacts of the following malfunctions or operations on the ED/G system; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those malfunctions or operations: Consequences of opening auxiliary feeder bus (ED/G sub supply)	2.7 2.6	49 50
073 Process Radiation Monitoring					0 2							K5.02-Knowledge of the operational implications as they apply to concepts as they apply to the PRM system: Radiation intensity changes with source distance	2.5	51
076 Service Water									0 2			A3.02-Ability to monitor automatic operation of the SWS, including: Emergency heat loads	3.7	52
078 Instrument Air		0										K2.01-Knowledge of bus power supplies to the following: Instrument air compressor	2.7	53
103 Containment								0 4		0 6		A2.04-Ability to (a) predict the impacts of the following malfunctions or operations on the containment system and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those malfunctions or operations Containment evacuation (including recognition of the alarm) A4.06-Ability to manually operate and/or monitor in the control room: Operation of the containment personnel airlock door	3.5 2.7	54 55
K/A Category Totals:	3	3	3	3	3	2	2	2	2	2	3	Group Point Total:		28

ES-401						PI						ation Outline Fo	orm E	S-401-2
System # / Name	K 1	K 2	K 3	K 4	K 5	K 6	Г	A 2	A 3	A 4	G	K/A Topic(s)	IR	#
001 Control Rod Drive									0 7			A3.07-Ability to monitor automatic operation of the CRDS, including: Boration/dilution	4.1	56
002 Reactor Coolant														
011 Pressurizer Level Control						0						K6.03-Knowledge of the effect of a loss or malfunction on the following will have on the PZR LCS: Relationship between PZR level and PZR heater control circuit	2.9	57
014 Rod Position Indication														
015 Nuclear Instrumentation				0 7								K4.07-Knowledge of NIS design feature(s) and/or interlock(s) provide for the following: Permissives	3.7	58
016 Non-nuclear Instrumentation														
017 In-core Temperature Monitor										0		A4.01-Ability to manually operate and/or monitor in the control room: Actual incore temperatures	3.8	59
027 Containment lodine Removal														
028 Hydrogen Recombiner and Purge Control														
029 Containment Purge														
033 Spent Fuel Pool Cooling											01. 32	2.1.32-Ability to explain and apply system limits and precautions.	3.8	60
034 Fuel Handling Equipment														
035 Steam Generator														
041 Steam Dump/Turbine Bypass Control					0 5							K5.05-Knowledge of the operational implications of the following concepts as they apply to the SDS: Basis for RCS design pressure limits	2.6	61
045 Main Turbine Generator														
055 Condenser Air Removal			0									K3.01-Knowledge of the effect that a loss or malfunction of the CARS will have on the following: Main condenser	2.5	62
056 Condensate														
068 Liquid Radwaste								0 4				A2.04-Ability to (a) predict the impacts of the following malfunctions or operations on the Liquid Radwaste System; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those malfunctions or operations: Failure of automatic isolation	3.3	63
071 Waste Gas Disposal														
072 Area Radiation Monitoring							0					A1.01-Ability to predict and/or monitor changes in parameters (to prevent exceeding design limits) associated with operating the ARM system controls including: Radiation levels	3.4	64
075 Circulating Water														
079 Station Air	0											K1.01-Knowledge of the physical connections and/or cause-effect relationships between the SAS and the following systems: IAS	3.0	65
086 Fire Protection														
K/A Category Totals:	1	0	1	1	1	1	1	1	1	1	1	Group Point Total:		10

ES-401	201.0	nd /					ion Outline olutions - Tier 1/Group 1 (SRO)	Form E	S-401-2
	K	K	K	mai A	A		· · · ·		
E/APE # / Name / Safety Function	1	2	3	1	2	G	K/A Topic(s)	IR	#
000007 Reactor Trip - Stabilization - Recovery / 1									
000008 Pressurizer Vapor Space Accident / 3									
000009 Small Break LOCA / 3									
000011 Large Break LOCA / 3									
000015 RCP Malfunctions / 4 000017 RCP Malfunctions (Loss of RC Flow) / 4									
000022 Loss of Rx Coolant Makeup / 2					0		AA2.03-Ability to determine and interpret the following as they apply to the Loss of Reactor Coolant Makeup: Failures of flow control valve or controller	3.6	76
000025 Loss of RHR System / 4									
000026 Loss of Component Cooling Water / 8						04. 08	2.4.8-Knowledge of how abnormal operating procedures are used in conjunction with EOPs.	4.5	77
000027 Pressurizer Pressure Control System Malfunction / 3									
000029 ATWS / 1					0		EA2.09-Ability to determine or interpret the following as they apply to a ATWS: Occurrence of a main turbine/reactor trip	4.5	78
000038 Steam Gen. Tube Rupture / 3									
000040 Steam Line Rupture - Excessive Heat Transfer / 4									
WE12 Uncontrolled Depressurization of all Steam Generators / 4									
000054 (CE/E06) Loss of Main Feedwater / 4						04. 04	2.4.4-Ability to recognize abnormal indications for system operating parameters that are entry-level conditions for emergency and abnormal operating proedures.	4.7	79
000055 Station Blackout / 6						02. 37	2.2.37-Ability to determine operability and/or availability of safety related equipment.	4.6	80
000056 Loss of Off-site Power / 6									
000057 Loss of Vital AC Inst. Bus / 6									
000058 Loss of DC Power / 6									
000062 Loss of Nuclear Svc Water / 4									
000065 Loss of Instrument Air / 8									
W/E04 LOCA Outside Containment / 3									
W/E11 Loss of Emergency Coolant Recirc. / 4									
BW/E04; W/E05 Inadequate Heat Transfer - Loss of Secondary Heat Sink / 4					0 2		EA2.02-Ability to determine and interpret the following as they apply to the (Loss of Secondary Heat Sink): Adherence to appropriate procedures and operation within the limitations in the facility's license and amendments	4.3	81
000077 Generator Voltage and Electric Grid Disturbances / 6							miniduorio in die lavilly 3 liverise and amendments		
K/A Category Totals:	0	0	0	0	3	3	Group Point Total:		6

ES-401				PWR	Exar	minat	tion Outline	Form E	S-401-2
Eme	rgenc	y and	d Abn	orma	l Plar	nt Ev	olutions - Tier 1/Group 2 (SRO)		
E/APE # / Name / Safety Function	K 1	K 2	К 3	A 1	A 2	G	K/A Topic(s)	IR	#
000001 Continuous Rod Withdrawal / 1									
000003 Dropped Control Rod / 1									
000005 Inoperable/Stuck Control Rod / 1									
000024 Emergency Boration / 1									
000028 Pressurizer Level Malfunction / 2									
000032 Loss of Source Range NI / 7					07		AA2.07-Ability to determine and interpret the following as they apply to the Loss of Source Range Nuclear Instrumentation: Maximum allowable channel disagreement	3.4	82
000033 Loss of Intermediate Range NI / 7									
000036 Fuel Handling Accident / 8									
000037 Steam Generator Tube Leak / 3									
000051 Loss of Condenser Vacuum / 4									
000059 Accidental Liquid RadWaste Rel. / 9									
000060 Accidental Gaseous Radwaste Rel. / 9							2.1.28-Knowledge of the purpose and function of major system components and controls.	4.1	83
000061 ARM System Alarms / 7									
000067 Plant Fire On-site / 8									
000068 Control Room Evac. / 8									
000069 Loss of CTMT Integrity / 5									
W/E14 High Containment Pressure / 5									1
000074 Inad. Core Cooling / 4									
W/E06 Degraded Core Cooling / 4									1
W/E07 Saturated Core Cooling / 4									
000076 High Reactor Coolant Activity / 9					04		AA2.04-Ability to determine and interpret the following as they apply to the High Reactor Coolant Activity: Process effluent radiation chart recorder	3.0	84
W/E01 Rediagnosis / 3									
W/E02 SI Termination / 3									
W/E13 Steam Generator Over-pressure / 4	1								
W/E15 Containment Flooding / 5	1								
W/E16 High Containment Radiation / 9									
W/E03 LOCA Cooldown - Depress. / 4	1								
W/E09 Natural Circulation Operations / 4									
W/E10 Natural Circulation with Steam Voide in Vessel with/without RVLIS. / 4							2.4.50-Ability to verify system alarm setpoints and operate controls identified in the alarm response manual.	4.0	85
W/E08 RCS Overcooling - PTS / 4									
K/A Category Totals:	0	0	0	0	2	2	Group Point Total:		4

ES-401						Б							orm ES	S-401-2
	1,,	1,,	1/	1,,	1/	1	Т	Ė	Г		lier	2/Group 1 (SRO)		
System # / Name	1	К 2	К 3	К 4		К 6	1	A 2	А 3	A 4	G	K/A Topic(s)	IR	#
003 Reactor Coolant Pump														
004 Chemical and Volume Control								2 8				A2.28-Ability to (a) predict the impacts of the following malfunctions or operations on the CVCS; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those malfunctions or operations: Depressurizing of RCS while it is hot	4.3	86
005 Residual Heat Removal														
006 Emergency Core Cooling														
007 Pressurizer Relief/Quench Tank														
008 Component Cooling Water														
010 Pressurizer Pressure Control								0 2				A2.02-Ability to (a) predict the impacts of the following malfunctions or operations on the PZR PCS; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those malfunctions or operations: Spray valve failures	3.9	87
012 Reactor Protection														
013 Engineered Safety Features Actuation														
022 Containment Cooling														
025 Ice Condenser														
026 Containment Spray														
039 Main and Reheat Steam								0 4				A2.04-Ability to (a) predict the impacts of the following malfunctions or operations on the MRSS; and (b) based on predictions, use procedures to correct, control, or mitigate the consequences of those malfunctions or operations: Malfunctioning steam dump	3.7	88
059 Main Feedwater														
061 Auxiliary/Emergency Feedwater														
062 AC Electrical Distribution											02. 44	2.2.44-Ability to interpret control room indications to verify the status and operation of a system, and understand how operator actions and directives affect plant and system conditions.	4.4	89
063 DC Electrical Distribution														
064 Emergency Diesel Generator														
073 Process Radiation Monitoring														
076 Service Water											04. 21	2.4.21-Knowledge of the parameters and logic used to assess the status of safety functions, such as reactivity control, core cooling and heat removal, reactor coolant system integrity, containment conditions, radioactivity release control, etc.	4.6	90
078 Instrument Air														
103 Containment														
K/A Category Totals:	0	0	0	0	0	0	0	3	0	0	2	Group Point Total:		5

ES-401						Pla						ation Outline Fo	orm E	S-401-2
System # / Name	K 1	K 2	K 3	K 4	K 5	K 6	Α	Α	A 3	Г	G	K/A Topic(s)	IR	#
001 Control Rod Drive														
002 Reactor Coolant														
011 Pressurizer Level Control														
014 Rod Position Indication														
015 Nuclear Instrumentation														
016 Non-nuclear Instrumentation											02. 42		4.6	91
017 In-core Temperature Monitor														
027 Containment lodine Removal														
028 Hydrogen Recombiner and Purge Control														
029 Containment Purge														
033 Spent Fuel Pool Cooling														
034 Fuel Handling Equipment														
035 Steam Generator											04. 41	2.4.41-Knowledge of the emergency action level thresholds and classifications.	4.6	92
041 Steam Dump/Turbine Bypass Control														
045 Main Turbine Generator														
055 Condenser Air Removal														
056 Condensate														
068 Liquid Radwaste														
071 Waste Gas Disposal														
072 Area Radiation Monitoring								0 3				A2.03-Ability to (a) predict the impacts of the following malfunctions or operations on the ARM system- and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those malfunctions or operations: Blown power-supply fuses	2.9	93
075 Circulating Water												,		
079 Station Air														
086 Fire Protection														
K/A Category Totals:	0	0	0	0	0	0	0	1	0	0	2	Group Point Total:		3

Facility Nam	e: Point	Beach Date of Exam: May 2017				
Category	K/A #	Topic		O ,,		-Only
	2.1. 14	2.1.14-Knowledge of criteria or conditions that require plant-wide announcements, such as pump starts, reactor trips, mode changes, etc.	IR 3.1	# 66	IR	#
	2.1. 38	2.1.38-Knowledge of the station's requirements for verbal communications when implementing procedures.	3.7	67		
1.	2.1.					
Conduct of Operations	2.1.					
	2.1. 02	2.1.2-Knowledge of operator responsibilities during all modes of plant operation.			4.4	94
		2.1.6-Ability to manage the control room crew during plant transients.			4.8	95
	Subtota			2		2
	2.2. 03	2.2.3-Knowledge of the design, procedural, and operational differences between units.	3.8	68		
	2.2. 14	2.2.14-Knowledge of the process for controlling equipment configuration or status.	3.9	69		
2.	2.2. 15	2.2.15-Ability to determine the expected plant configuration using design and configuration control documentation, such as drawings, line-ups, tag-outs, etc.	3.9	70		
Equipment Control	2.2.					
	2.2. 07	2.2.7-Knowledge of the process for conducting special or infrequent tests.			3.6	96
	2.2. 21	2.2.21-Knowledge of pre- and post-maintenance operability requirements.			4.1	97
	Subtota			3		2
	2.3. 07	2.3.7-Ability to comply with radiation work permit requirements during normal or abnormal conditions.	3.5	71		
	2.3. 11	2.3.11-Ability to control radiation releases.	3.8	72		
3.	2.3. 15	2.3.15-Knowledge of radiation monitoring systems, such as fixed radiation monitors and alarms, portable survey instruments, personnel monitoring equipment, etc.	2.9	73		
Radiation Control	2.3.					
	2.3.					
	2.3. 13	2.3.13-Knowledge of radiological safety procedures pertaining to licensed operator duties, such as response to radiation monitor alarms, containment entry requirements, fuel handling responsibilities, access to locked high-radiation areas, aligning filters, etc.			3.8	98
	Subtota			3		1
	2.4. 14	2.4.14-Knowledge of general guidelines for EOP usage.	3.8	74		
	2.4. 45	2.4.45-Ability to prioritize and interpret the significance of each annunciator or alarm.	4.1	75		
4. Emergency	2.4.					
Emergency Procedures / Plan	2.4.					
i iaii	2.4. 25	2.4.25-Knowledge of fire protection procedures.			3.7	99
	_	2.4.32-Knowledge of operator response to loss of all annunciators.			4.0	100
	Subtota			2		2
Tier 3 Point	Total			10		7