



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

June 29, 2016

Mr. Peter P. Sena, III
President
PSEG Nuclear LLC - N09
P.O. Box 236
Hancocks Bridge, NJ 08038

SUBJECT: SALEM NUCLEAR GENERATING STATION, UNIT NOS. 1 AND 2 - ISSUANCE OF AMENDMENTS TO EXTEND IMPLEMENTATION PERIOD FOR UNIT NO. 1 LICENSE AMENDMENT NO. 311 AND UNIT NO. 2 LICENSE AMENDMENT NO. 292 (CAC NOS. MF7697 AND MF7698)

Dear Mr. Sena:

The U.S. Nuclear Regulatory Commission (the Commission) has issued the enclosed Amendment Nos. 314 and 295 to Renewed Facility Operating License Nos. DPR-70 and DPR-75 for the Salem Nuclear Generating Station (Salem), Unit Nos. 1 and 2, respectively. These amendments consist of changes to the Renewed Facility Operating Licenses in response to your application dated May 10, 2016.

The amendments extend the implementation period for the Salem, Unit No. 1, License Amendment No. 311, and the Salem, Unit No. 2, License Amendment No. 292, which were effective as of their date of issuance (i.e., March 7, 2016). Specifically, the implementation period for the above amendments has been extended from July 5, 2016 (i.e., 120 days from the date of issuance), to prior to entry into Mode 6 for the Salem, Unit No. 1, Fall 2017 refueling outage (1R25), and prior to entry into Mode 6 for the Salem, Unit No. 2, Spring 2017 refueling outage (2R22), to align with the outages for which the replacement of the source range and intermediate range detectors is scheduled.

A copy of our safety evaluation is also enclosed. Notice of Issuance will be included in the Commission's biweekly *Federal Register* notice.

Sincerely,

A handwritten signature in black ink, appearing to read "Carleen J. Parker".

Carleen J. Parker, Project Manager
Plant Licensing Branch I-2
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket Nos. 50-272 and 50-311

Enclosures:

1. Amendment No. 314 to Renewed DPR-70
2. Amendment No. 295 to Renewed DPR-75
3. Safety Evaluation

cc w/enclosures: Distribution via Listserv



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

PSEG NUCLEAR LLC

EXELON GENERATION COMPANY, LLC

DOCKET NO. 50-272

SALEM NUCLEAR GENERATING STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 314
Renewed License No. DPR-70

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment filed by PSEG Nuclear LLC, acting on behalf of itself and Exelon Generation Company, LLC (the licensees), dated May 10, 2016, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in Title 10 of the *Code of Federal Regulations* (10 CFR), Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, Renewed Facility Operating License No. DPR-70 is amended to extend the implementation period of Amendment No. 311, issued on March 7, 2016, to prior to entry into Mode 6 for the Salem, Unit No. 1, Fall 2017 refueling outage (1R25).
3. This license amendment is effective as of its date of issuance and shall be implemented by July 5, 2016.

FOR THE NUCLEAR REGULATORY COMMISSION

A handwritten signature in black ink, appearing to read "Doug Broaddus". The signature is written in a cursive style with a large initial "D".

Douglas A. Broaddus, Chief
Plant Licensing Branch I-2
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Renewed Facility
Operating License

Date of Issuance: June 29, 2016

ATTACHMENT TO LICENSE AMENDMENT NO. 314
RENEWED FACILITY OPERATING LICENSE NO. DPR-70
DOCKET NO. 50-272

Replace the following page of Renewed Facility Operating License No. DPR-70 with the attached revised page as indicated. The revised page is identified by amendment number and contains a marginal line indicating the area of change.

Remove
Page 3

Insert
Page 3

instrumentation and radiation monitoring equipment calibration, and as fission detectors in amounts as required;

- (5) PSEG Nuclear LLC, pursuant to the Act and 10 CFR Parts 30, 40 and 70, to receive, possess and use in amounts as required any byproduct, source or special nuclear material without restriction to chemical or physical form, for sample analysis or instrument calibration or associated with radioactive apparatus or components; and
- (6) PSEG Nuclear LLC, pursuant to the Act and 10 CFR Parts 30 and 70, to possess but not separate, such byproduct and special nuclear materials as may be produced by the operation of the facility.

C. This renewed license shall be deemed to contain and is subject to the conditions specified in the following Commission regulations in 10 CFR Chapter I: Part 20, Section 30.34 of Part 30, Section 40.41 of Part 40, Sections 50.54 and 50.59 of Part 50, and Section 70.32 of Part 70; and is subject to all applicable provisions of the Act and to the rules, regulations, and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified or incorporated below:

(1) Maximum Power Level

PSEG Nuclear LLC is authorized to operate the facility at a steady state reactor core power level not in excess of 3459 megawatts (one hundred percent of rated core power).

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 314, and the Environmental Protection Plan contained in Appendix B, are hereby incorporated in the renewed license. PSEG Nuclear LLC shall operate the facility in accordance with the Technical Specifications, and the Environmental Protection Plan.

(3) Deleted Per Amendment 22, 11-20-79

(4) Less than Four Loop Operation

PSEG Nuclear LLC shall not operate the reactor at power levels above P-7 (as defined in Table 3.3-1 of Specification 3.3.1.1 of Appendix A to this renewed license) with less than four (4) reactor coolant loops in operation until safety analyses for less than four loop operation have been submitted by the licensees and approval for less than four loop operation at power levels above P-7 has been granted by the Commission by Amendment of this renewed license.

(5) PSEG Nuclear LLC shall implement and maintain in effect all provisions of the approved fire protection program as described in the Updated Final Safety



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PSEG NUCLEAR LLC

EXELON GENERATION COMPANY, LLC

DOCKET NO. 50-311

SALEM NUCLEAR GENERATING STATION, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 295
Renewed License No. DPR-75

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment filed by PSEG Nuclear LLC, acting on behalf of itself and Exelon Generation Company, LLC (the licensees), dated May 10, 2016, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in Title 10 of the *Code of Federal Regulations* (10 CFR), Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

Enclosure 2

2. Accordingly, Renewed Facility Operating License No. DPR-75 is amended to extend the implementation period of Amendment No. 292, issued on March 7, 2016, to prior to entry into Mode 6 for the Salem, Unit No. 2, Spring 2017 refueling outage (2R22).
3. This license amendment is effective as of its date of issuance and shall be implemented by July 5, 2016.

FOR THE NUCLEAR REGULATORY COMMISSION

A handwritten signature in black ink, appearing to read "Doug A. Broaddus". The signature is written in a cursive style with a large initial "D".

Douglas A. Broaddus, Chief
Plant Licensing Branch I-2
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Renewed Facility
Operating License

Date of Issuance: June 29, 2016

ATTACHMENT TO LICENSE AMENDMENT NO. 295

RENEWED FACILITY OPERATING LICENSE NO. DPR-75

DOCKET NO. 50-311

Replace the following page of Renewed Facility Operating License No. DPR-75 with the attached revised page as indicated. The revised page is identified by amendment number and contains a marginal line indicating the area of change.

Remove
Page 3

Insert
Page 3

- (4) PSEG Nuclear LLC, pursuant to the Act and 10 CFR Parts 30, 40 and 70, to receive, possess and use at any time any byproduct, source or special nuclear material as sealed neutron sources for reactor startup, sealed sources for reactor instrumentation and radiation monitoring equipment calibration and as fission detectors in amounts as required;
 - (5) PSEG Nuclear LLC, pursuant to the Act and 10 CFR Parts 30, 40 and 70, to receive, possess and use in amounts as required any byproduct, source or special nuclear material without restriction to chemical or physical form, for sample analysis or instrument calibration or associated with radioactive apparatus or components; and
 - (6) PSEG Nuclear LLC, pursuant to the Act and 10 CFR Parts 30, 40 and 70, to possess but not separate, such byproduct and special nuclear materials as may be produced by the operation of the facility.
- C. This renewed license shall be deemed to contain and is subject to the conditions specified in the Commission's regulations set forth in 10 CFR Chapter I and is subject to all applicable provisions of the Act and to the rules, regulations and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified or incorporated below:
- (1) Maximum Power Level

PSEG Nuclear LLC is authorized to operate the facility at steady state reactor core power levels not in excess of 3459 megawatts (thermal).
 - (2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 295, and the Environmental Protection Plan contained in Appendix B, are hereby incorporated in the renewed license. PSEG Nuclear LLC shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.



UNITED STATES
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SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NOS. 314 AND 295 TO

RENEWED FACILITY OPERATING LICENSE NOS. DPR-70 AND DPR-75

PSEG NUCLEAR LLC

EXELON GENERATION COMPANY, LLC

SALEM NUCLEAR GENERATING STATION, UNIT NOS. 1 AND 2

DOCKET NOS. 50-272 AND 50-311

1.0 INTRODUCTION

By letter dated May 10, 2016 (Agencywide Documents Access and Management System (ADAMS) Accession No. ML16131A555), PSEG Nuclear LLC (PSEG, the licensee) submitted a request for changes to the Salem Nuclear Generating Station (Salem), Unit Nos. 1 and 2, Renewed Facility Operating Licenses. The requested changes would extend the implementation period for the Salem, Unit No. 1, License Amendment No. 311, and the Salem, Unit No. 2, License Amendment No. 292 (ADAMS Accession No. ML16035A087), which were effective as of the date of issuance (i.e., March 7, 2016). Specifically, the licensee has proposed to extend the implementation period for the above amendments from July 5, 2016 (i.e., 120 days from the date of issuance), to prior to entry into Mode 6 for the Salem, Unit No. 1, Fall 2017 refueling outage (1R25), and prior to entry into Mode 6 for the Salem, Unit No. 2, Spring 2017 refueling outage (2R22), to align with the outages for which the replacement of the source range and intermediate range detectors is scheduled.

2.0 REGULATORY EVALUATION

In issuing an amendment to an operating license, the U.S. Nuclear Regulatory Commission (NRC) staff specifies when the amendment is effective and by when the amendment must be implemented. Enclosures 1 and 2 to the NRC's letter dated March 7, 2016, contain License Amendment No. 311 for Salem, Unit No. 1, and License Amendment No. 292 for Salem, Unit No. 2, respectively. The amendments state, in part, that:

This license amendment is effective as of its date of issuance and shall be implemented within 120 days.

There are no specific regulatory requirements regarding the time period for amendment implementation. The NRC staff generally specifies an implementation period consistent with

that requested in the licensee's application insofar as the implementation period does not affect the staff's conclusion that the license amendment may be issued.

For License Amendment No. 311 for Salem, Unit No. 1, and License Amendment No. 292 for Salem, Unit No. 2, the 120-day implementation period was requested in PSEG's letter dated February 3, 2016 (ADAMS Accession No. ML16034A265). Subsequently, on March 7, 2016, the NRC issued the license amendment with an implementation period of 120 days.

Implementation periods requested by licensees may vary based on the extent of the physical changes needed to the plant and/or associated documentation that needs to be revised. Sometimes the licensee requests that it be allowed to implement the amendment during the next refueling outage to support changes that the licensee wants to make when the plant is shut down.

Since the implementation period specified by the NRC in an amendment is considered part of the operating license, the licensee is required to fully implement the amendment by a date no later than that specified otherwise, the licensee will be in violation of its operating license. Any extension of the implementation period needs prior NRC approval as a license amendment pursuant to Title 10 of the *Code of Federal Regulations* (10 CFR) Section 50.90.

3.0 TECHNICAL EVALUATION

3.1 Background

By letter dated March 7, 2016, the NRC issued License Amendment No. 311 for Salem, Unit No. 1, and License Amendment No. 292 for Salem, Unit No. 2. The amendments created a new Technical Specification (TS) Limiting Condition for Operation (LCO) 3.9.2.1 for unborated water source isolation valves in Mode 6. TS LCO 3.9.2.1 isolates unborated water sources in Mode 6, which precludes a boron dilution event. The amendment also revised the neutron flux instrumentation requirements in Mode 6 by removing the existing requirement for one source range neutron flux monitor with audible indication in the containment and in the control room during Mode 6.

The amendment was effective as of its date of issuance and was required to be implemented within 120 days (i.e., July 5, 2016).

In its original March 9, 2015, license amendment request (ADAMS Accession No. ML15068A359), PSEG requested the approval of the Salem, Unit No. 1, License Amendment No. 311 by March 9, 2016, to align with the Salem, Unit No. 1, Spring 2016 refueling outage (1R24) for which the replacement of the source range and intermediate range detectors was scheduled. However, prior to the Salem 1R24 outage, PSEG elected to move the replacement of the source range and intermediate range detectors from the 1R24 outage to the Fall 2017 refueling outage (1R25). Therefore, by letter dated February 3, 2016 (ADAMS Accession No. ML16034A265), PSEG updated the requested implementation schedule to a period of 120 days to allow implementation following the 1R24 refueling outage. The NRC staff determined that this implementation period was acceptable because the implementation of the license amendment to isolate all unborated water sources in Mode 6 is not dependent on the replacement source range and intermediate range detectors.

3.2 NRC Evaluation

The Salem, Unit No. 1, License Amendment No. 311, and the Salem, Unit No. 2, License Amendment No. 292, were effective as of their date of issuance (i.e., March 7, 2016), and were required to be implemented within 120 days (i.e., July 5, 2016). Due to unforeseen changes in the outage scope, PSEG requested that the implementation period be extended.

Specifically, PSEG requested that implementation not be required until prior to entry into Mode 6 for the Salem, Unit No. 1, Fall 2017 refueling outage (1R25), and prior to entry into Mode 6 for the Salem, Unit No. 2, Spring 2017 refueling outage (2R22). PSEG noted that implementing the amendments within the required 120-day period could divert operations, maintenance, and engineering resources from essential outage activities.

There are two methods to preclude boron dilution events in pressurized-water reactors (PWRs) such as Salem:

- 1) An analysis that assumes a maximum unborated water flow and determines there is adequate time for operator action to mitigate the event. Plants that use this method are not required to isolate all unborated water sources by closing the associated valves. For this method, the audible count rate instrumentation provides the operator with prompt and definite indication of a boron dilution event. A high count rate alarm is provided in the containment and control room. This is the current method used at Salem, Unit Nos. 1 and 2, and operability of the audible count rate is currently required by TS LCO 3.9.2.
- 2) Isolate all sources of unborated water. When this method is used, no analyses are required. In addition, when this method is used, an audible count rate is not required. By License Amendment No. 311 for Salem, Unit No. 1, and License Amendment No. 292 for Salem, Unit No. 2, the NRC staff approved the use of this method to prevent reactor coolant system boron dilution during refueling (Mode 6) at Salem, Unit Nos. 1 and 2.

Salem, Unit Nos. 1 and 2, will continue to comply with the existing Mode 6 TS 3.9.2 requirements until Amendment No. 311 for Salem, Unit No. 1, and Amendment No. 292 for Salem, Unit No. 2, are implemented. This includes the current 1R24 outage at Salem, Unit No. 1, and any potential entries into Mode 6 at Salem, Unit Nos. 1 or 2, between now and implementation. Current TS 3.9.2 requirements will continue to ensure that the units are operated consistent with the Salem Updated Final Safety Analysis Report (UFSAR) accident analysis for a boron dilution event during Mode 6. Not implementing Salem, Unit No. 1, License Amendment No. 311, and Salem, Unit No. 2, License Amendment No. 292, has no effect on safety, as the NRC staff has determined that either method is acceptable to preclude boron dilution events at Salem. Since the current method will continue to ensure that the Salem, Unit Nos. 1 and 2, are operated consistent with the current licensing basis requirements for preventing a boron dilution event in Mode 6, extending the implementation period of Salem, Unit No. 1, Amendment No. 311, and Salem, Unit No. 2, Amendment No. 292, does not pose any nuclear safety impact.

Based on the above, the NRC staff finds that the proposed extension of the implementation period for Salem, Unit No. 1, License Amendment No. 311, and Salem, Unit No. 2, License Amendment No. 292, is purely administrative in nature. The proposed amendments do not

involve any physical changes to the plant and do not involve any changes in the operation of the plant. Operation of the plant with the currently required method for precluding boron dilution events in Mode 6 prior to the implementation of Amendment Nos. 311 and 292 does not raise any safety concerns. Therefore, the NRC staff concludes that the proposed amendments are acceptable.

4.0 FINAL NO SIGNIFICANT HAZARDS CONSIDERATION DETERMINATION

The NRC regulations in 10 CFR 50.92 state that the NRC may make a final determination that a license amendment involves no significant hazards consideration if operation of the facility, in accordance with the amendment, would not: (1) involve a significant increase in the probability or consequences of an accident previously evaluated; or (2) create the possibility of a new or different kind of accident from any accident previously evaluated; or (3) involve a significant reduction in a margin of safety.

As required by 10 CFR 50.91(a), the licensee, in its application dated May 10, 2016, provided its analysis of the issue of no significant hazards consideration, using the standards in 10 CFR 50.92, which is presented below.

1. Do the proposed changes involve a significant increase in the probability or consequences of an accident previously evaluated?

Response: No.

The proposed amendment implementation schedule extension is administrative in nature and does not require any modifications to or change in operation of plant systems or components. The extension of the amendment implementation period does not increase the probability or consequences of an accident previously evaluated in the Updated Final Safety Analysis Report (UFSAR). Current Technical Specification (TS) 3.9.2 requirements will continue to ensure the plant is operated consistent with the UFSAR accident analysis for a boron dilution event.

Therefore, the proposed change does not involve a significant increase in the probability or consequences of an accident previously evaluated.

2. Do the proposed changes create the possibility of a new or different kind of accident from any accident previously evaluated?

Response: No.

The proposed amendment implementation schedule extension is administrative in nature. The extension of the amendment implementation does not require any physical plant modifications, does not alter any plant systems or components, and does not change the operation of the plant. Current TS 3.9.2 requirements will continue to ensure the plant is operated consistent with the UFSAR accident analysis for a boron dilution event.

Therefore, the proposed change does not create the possibility of a new or different kind of accident from any previously evaluated.

3. Do the proposed changes involve a significant reduction in a margin of safety?

Response: No.

Margin of safety is related to the confidence in the ability of the fission product barriers to perform their intended functions. These barriers include the fuel cladding, the reactor coolant system pressure boundary, and the containment. The proposed TS change is administrative in nature and does not affect any of these barriers. Current TS 3.9.2 requirements will continue to ensure the plant is operated consistent with the UFSAR accident analysis for a boron dilution event.

Therefore, the proposed change does not involve a significant reduction in the margin of safety.

The NRC staff reviewed the licensee's no significant hazards consideration analysis. Based on this review and on the NRC staff's safety evaluation of the underlying license amendment request, the NRC staff concludes that the three standards of 10 CFR 50.92(c) are satisfied. Therefore, the NRC staff had made a final determination that no significant hazards consideration is involved for the proposed amendment and that the amendment should be issued as allowed by the criteria contained in 10 CFR 50.91.

5.0 STATE CONSULTATION

In accordance with the Commission's regulations, the New Jersey State official was notified of the proposed issuance of the amendments. The State official had no comments.

6.0 ENVIRONMENTAL CONSIDERATION

The amendments change a requirement with respect to the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The NRC staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendments involve no significant hazards consideration, and there has been no public comment on such finding published in the *Federal Register* on May 23, 2016 (81 FR 32351). The Commission has made a final determination that no significant hazards consideration is involved for the proposed amendments as discussed in Section 4.0 of the safety evaluation. Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

7.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) there is reasonable assurance that such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: Alan Wang

Date: June 29, 2016

June 29, 2016

Mr. Peter P. Sena, III
President
PSEG Nuclear LLC - N09
P.O. Box 236
Hancocks Bridge, NJ 08038

SUBJECT: SALEM NUCLEAR GENERATING STATION, UNIT NOS. 1 AND 2 - ISSUANCE OF AMENDMENTS TO EXTEND IMPLEMENTATION PERIOD FOR UNIT NO. 1 LICENSE AMENDMENT NO. 311 AND UNIT NO. 2 LICENSE AMENDMENT NO. 292 (CAC NOS. MF7697 AND MF7698)

Dear Mr. Sena:

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A copy of our safety evaluation is also enclosed. Notice of Issuance will be included in the Commission's biweekly *Federal Register* notice.

Sincerely,
/RA/
Carleen J. Parker, Project Manager
Plant Licensing Branch I-2
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket Nos. 50-272 and 50-311

Enclosures:

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2. Amendment No. 295 to Renewed DPR-75
3. Safety Evaluation

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ADAMS Accession No.: ML16137A579

OFFICE	LPL4-2/PM	LPL1-2/PM	LPL1-2/LA	OGC	LPL1-2/BC	LPL1-2/PM
NAME	AWang	TWengert	LRonewicz	JWachutka NLO w/comments	DBroaddus	CParker
DATE	5/18/2016	5/18/2016	5/17/2016	6/28//2016	6/27/2016	6/29/2016

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