



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

March 11, 2015

Mr. Bryan C. Hanson
President and Chief Nuclear Officer
Exelon Nuclear
4300 Winfield Road
Warrenville, IL 60555

SUBJECT: PEACH BOTTOM ATOMIC POWER STATION, UNITS 2 AND 3 - ISSUANCE OF AMENDMENTS RE: REVISE FACILITY OPERATING LICENSE AND THE TECHNICAL SPECIFICATIONS TO INCORPORATE ADMINISTRATIVE CHANGES (TAC NOS. MF4401 AND MF4402)

Dear Mr. Hanson:

The Commission has issued the enclosed Amendment Nos. 296 and 299 to Renewed Facility Operating License Nos. DPR-44 and DPR-56 for Peach Bottom Atomic Power Station, Units 2 and 3. These amendments consist of changes to the Technical Specifications (TSs) and Facility Operating Licenses (FOLs) in response to your application dated July 11, 2014, as supplemented by letter dated December 1, 2014.

The amendments incorporate several administrative changes to the FOLs and TSs such as deleting historical items that are no longer applicable, correcting errors, and removing references that are no longer valid.

A copy of the safety evaluation is also enclosed. Notice of Issuance will be included in the Commission's Biweekly *Federal Register* Notice.

Sincerely,

A handwritten signature in black ink, appearing to read "RBE", written over a light blue horizontal line.

Richard B. Ennis, Senior Project Manager
Plant Licensing Branch I-2
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket Nos. 50-277 and 50-278

Enclosures:

1. Amendment No. 296 to Renewed DPR-44
2. Amendment No. 299 to Renewed DPR-56
3. Safety Evaluation

cc w/encls: Distribution via Listserv



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

EXELON GENERATION COMPANY, LLC

PSEG NUCLEAR LLC

DOCKET NO. 50-277

PEACH BOTTOM ATOMIC POWER STATION, UNIT 2

AMENDMENT TO RENEWED FACILITY OPERATING LICENSE

Amendment No. 296
Renewed License No. DPR-44

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Exelon Generation Company, LLC (Exelon Generation Company), and PSEG Nuclear LLC (the licensees), dated July 11, 2014, as supplemented by letter dated December 1, 2014, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

Enclosure 1

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C(2) of Renewed Facility Operating License No. DPR-56 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 296, are hereby incorporated in the license. Exelon Generation Company shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 60 days.

FOR THE NUCLEAR REGULATORY COMMISSION



Douglas A. Broaddus, Chief
Plant Licensing Branch I-2
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical Specifications
and Facility Operating License

Date of Issuance: March 11, 2015

ATTACHMENT TO LICENSE AMENDMENT NO. 296
RENEWED FACILITY OPERATING LICENSE NO. DPR-44
DOCKET NO. 50-277

Replace the following pages of the Renewed Facility Operating License with the attached revised pages. The revised pages are identified by amendment number and contain marginal lines indicating the areas of change.

<u>Remove</u>	<u>Insert</u>
3	3
7	7
7a	7a

Replace the following pages of the Appendix A Technical Specifications with the attached revised pages. The revised pages are identified by amendment number and contain marginal lines indicating the areas of change.

<u>Remove</u>	<u>Insert</u>
4.0-3	4.0-3
5.0-21	5.0-21
5.0-22	5.0-22
5.0-22a	-----

- (5) Exelon Generation Company, pursuant to the Act and 10 CFR Parts 30 and 70, to possess, but not to separate, such byproduct and special nuclear material as may be produced by operation of the facility, and such Class B and Class C low-level radioactive waste as may be produced by the operation of Limerick Generating Station, Units 1 and 2.

C. This renewed license shall be deemed to contain and is subject to the conditions specified in the following Commission regulations in 10 CFR Chapter I: Part 20, Section 30.34 of Part 30, Section 40.41 of Part 40, Section 50.54 of Part 50, and Section 70.32 of Part 70; all applicable provisions of the Act and the rules, regulations, and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified below:

- (1) Maximum Power Level

Exelon Generation Company is authorized to operate the Peach Bottom Atomic Power Station, Unit 2, at steady state reactor core power levels not in excess of 3951 megawatts thermal.

- (2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 296, are hereby incorporated in the license. Exelon Generation Company shall operate the facility in accordance with the Technical Specifications.

- (3) Physical Protection

Exelon Generation Company shall fully implement and maintain in effect all provisions of the Commission-approved physical security, training and qualification, and safeguards contingency plans including amendments made pursuant to provisions of the Miscellaneous Amendments and Search Requirements revisions to 10 CFR 73.55 (51 FR 27817 and 27822), and the authority of 10 CFR 50.90 and 10 CFR 50.54(p). The combined set of plans¹, submitted by letter dated May 17, 2006, is entitled: "Peach Bottom Atomic Power Station Security Plan, Training and Qualification Plan, Safeguards Contingency Plan, and Independent Spent Fuel Storage Installation Security Program, Revision 3." The set contains Safeguards Information protected under 10 CFR 73.21.

Exelon Generation Company shall fully implement and maintain in effect all provisions of the Commission-approved cyber security plan (CSP), including changes made pursuant to the authority of 10 CFR 50.90 and 10 CFR 50.54(p). The Exelon Generation Company CSP was approved by License Amendment No. 283.

- (4) Fire Protection

The Exelon Generation Company shall implement and maintain in effect all provisions of the approved fire protection program as described in the Updated Final Safety Analysis Report for the facility, and as approved in the NRC Safety Evaluation Report (SER) dated May 23, 1979, and Supplements dated August 14, September 15, October 10 and November 24, 1980, and in the NRC SERs dated September 16, 1993, and August 24, 1994, subject to the following provision:

¹ The Training and Qualification Plan and Safeguards Contingency Plan are Appendices to the Security Plan.

(11) Mitigation Strategy License Condition

Develop and maintain strategies for addressing large fires and explosions and that include the following key areas:

- (a) Fire fighting response strategy with the following elements:
 - 1. Pre-defined coordinated fire response strategy and guidance
 - 2. Assessment of mutual aid fire fighting assets
 - 3. Designated staging areas for equipment and materials
 - 4. Command and control
 - 5. Training of response personnel
 - (b) Operations to mitigate fuel damage considering the following:
 - 1. Protection and use of personnel assets
 - 2. Communications
 - 3. Minimizing fire spread
 - 4. Procedures for implementing integrated fire response strategy
 - 5. Identification of readily-available pre-staged equipment
 - 6. Training on integrated fire response strategy
 - 7. Spent fuel pool mitigation measures
 - (c) Actions to minimize release to include consideration of:
 - 1. Water spray scrubbing
 - 2. Dose to onsite responders
- (12) The licensee shall implement and maintain all Actions required by Attachment 2 to NRC Order EA-06-137, issued June 20, 2006, except the last action that requires incorporation of the strategies into the site security plan, contingency plan, emergency plan and/or guard training and qualification plan, as appropriate.
- (13) Deleted

Renewed License No. DPR-44
Revised by letter dated August 9, 2007
Amendment 296

(14) Spent Fuel Pool Criticality Considerations

- (a) Use of spent fuel pool storage cells without NETCO-SNAP-IN® rack inserts shall be restricted as follows:
- 1) Minimum panel Boron-10 areal density of a storage cell shall be greater than or equal to 0.014 grams per square centimeter to store fuel assemblies with the maximum in-core cold k-infinity of up to 1.235. The minimum panel Boron-10 areal density shall be evaluated by assuming that the panel areal density was initially equal to a value of 0.0235 grams per square centimeter.
 - 2) A storage cell shall not contain any fuel assembly if the minimum panel Boron-10 areal density of a storage cell is less than 0.014 grams per square centimeter. The minimum panel Boron-10 areal density shall be evaluated by assuming that the panel areal density was initially equal to a value of 0.0235 grams per square centimeter.
- (b) Until the installation of NETCO-SNAP-IN® rack inserts are completed in the Peach Bottom Unit 2 spent fuel pool, Boraflex degradation shall be monitored analytically every 6 months.
- (c) Boraflex degradation shall be monitored by in-situ testing in the Peach Bottom Unit 2 spent fuel pool no later than December 31, 2014, unless installation of the NETCO-SNAP-IN® rack inserts for Unit 2 have been completed prior to this date.
- (d) Installation of NETCO-SNAP-IN® rack inserts shall be completed by December 31, 2016.

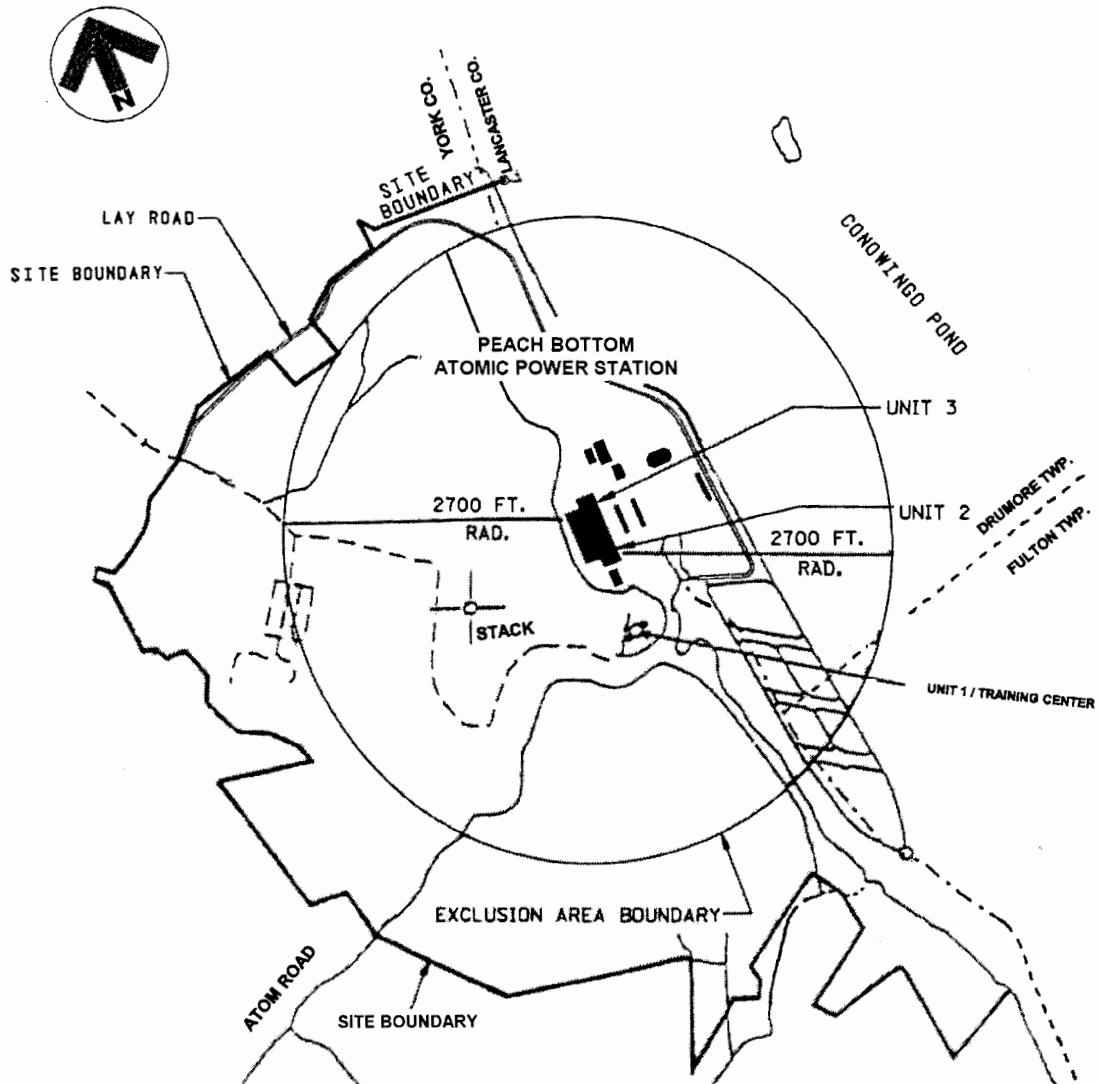


Figure 4.1-1 (page 1 of 1)
Site and Exclusion Area Boundaries

5.6 Reporting Requirements (continued)

5.6.5 CORE OPERATING LIMITS REPORT (COLR)

- a. Core operating limits shall be established prior to each reload cycle, or prior to any remaining portion of a reload cycle, and shall be documented in the COLR for the following:
 1. The Average Planar Linear Heat Generation Rate for Specification 3.2.1;
 2. The Minimum Critical Power Ratio for Specifications 3.2.2 and 3.3.2.1;
 3. The Linear Heat Generation Rate for Specification 3.2.3;
 4. The Control Rod Block Instrumentation for Specification 3.3.2.1; and
 5. The Oscillation Power Range Monitor (OPRM) Instrumentation for Specification 3.3.1.1.
- b. The analytical methods used to determine the core operating limits shall be those previously reviewed and approved by the NRC, specifically those described in the following document:
 1. NEDE-24011-P-A, "General Electric Standard Application for Reactor Fuel" (latest approved version as specified in the COLR).
- c. The core operating limits shall be determined such that all applicable limits (e.g., fuel thermal mechanical limits, core thermal hydraulic limits, Emergency Core Cooling Systems (ECCS) limits, nuclear limits such as SDM, transient analysis limits, and accident analysis limits) of the safety analysis are met.
- d. The COLR, including any midcycle revisions or supplements, shall be provided upon issuance for each reload cycle to the NRC.

5.6 Reporting Requirements (continued)

5.6.6 Post Accident Monitoring (PAM) Instrumentation Report

When a report is required by Condition B or F of LCO 3.3.3.1, "Post Accident Monitoring (PAM) Instrumentation," a report shall be submitted within the following 14 days. The report shall outline the preplanned alternate method of monitoring, the cause of the inoperability, and the plans and schedule for restoring the instrumentation channels of the Function to OPERABLE status.

5.6.7 Reactor Coolant System (RCS) PRESSURE AND TEMPERATURE LIMITS REPORT (PTLR)

- a. RCS pressure and temperature limits for heatup, cooldown, low temperature operation, criticality, and hydrostatic testing as well as heatup and cooldown rates shall be established and documented in the PTLR for the following:
 - i) Limiting Conditions for Operation Section 3.4.9, "RCS Pressure and Temperature (P/T) Limits"
 - ii) Surveillance Requirements Section 3.4.9, "RCS Pressure and Temperature (P/T) Limits"
 - b. The analytical methods used to determine the RCS pressure and temperature limits shall be those previously reviewed and approved by the NRC, specifically those described in the following document:
 - i) NEDC-33178P-A, "GE Hitachi Nuclear Energy Methodology for Development of Reactor Pressure Vessel Pressure-Temperature Curves," Revision 1, June 2009
 - c. The PTLR shall be provided to the NRC upon issuance for each reactor vessel fluence period and for any revision or supplement thereto.
-



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

EXELON GENERATION COMPANY, LLC

PSEG NUCLEAR LLC

DOCKET NO. 50-278

PEACH BOTTOM ATOMIC POWER STATION, UNIT 3

AMENDMENT TO RENEWED FACILITY OPERATING LICENSE

Amendment No. 299
Renewed License No. DPR-56

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Exelon Generation Company, LLC (Exelon Generation Company), and PSEG Nuclear LLC (the licensees), dated July 11, 2014, as supplemented by letter dated December 1, 2014, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

Enclosure 2

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C(2) of Renewed Facility Operating License No. DPR-56 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 299, are hereby incorporated in the license. Exelon Generation Company shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 60 days.

FOR THE NUCLEAR REGULATORY COMMISSION



Douglas A. Broaddus, Chief
Plant Licensing Branch I-2
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical Specifications
and Facility Operating License

Date of Issuance: March 11, 2015

ATTACHMENT TO LICENSE AMENDMENT NO. 299

RENEWED FACILITY OPERATING LICENSE NO. DPR-56

DOCKET NO. 50-278

Replace the following pages of the Renewed Facility Operating License with the attached revised pages. The revised page are identified by amendment number and contain marginal lines indicating the areas of change.

Remove

3

7

7a

Insert

3

7

7a

Replace the following pages of the Appendix A Technical Specifications with the attached revised pages. The revised page are identified by amendment number and contain marginal lines indicating the areas of change.

Remove

4.0-3

5.0-21

5.0-22

5.0-22a

Insert

4.0-3

5.0-21

5.0-22

- (5) Exelon Generation Company, pursuant to the Act and 10 CFR Parts 30 and 70, to possess, but not to separate, such byproduct and special nuclear material as may be produced by operation of the facility, and such Class B and Class C low-level radioactive waste as may be produced by the operation of Limerick Generating Station, Units 1 and 2.

C. This renewed license shall be deemed to contain and is subject to the conditions specified in the following Commission regulations in 10 CFR Chapter I: Part 20, Section 30.34 of Part 30, Section 40.41 of Part 40, Section 50.54 of Part 50, and Section 70.32 of Part 70; all applicable provisions of the Act and the rules, regulations, and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified below:

- (1) Maximum Power Level

Exelon Generation Company is authorized to operate the Peach Bottom Atomic Power Station, Unit No. 3, at steady state reactor core power levels not in excess of 3951 megawatts thermal.

- (2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 299, are hereby incorporated in the license. Exelon Generation Company shall operate the facility in accordance with the Technical Specifications.

- (3) Physical Protection

Exelon Generation Company shall fully implement and maintain in effect all provisions of the Commission-approved physical security, training and qualification, and safeguards contingency plans including amendments made pursuant to provisions of the Miscellaneous Amendments and Search Requirements revisions to 10 CFR 73.55 (51 FR 27817 and 27822), and the authority of 10 CFR 50.90 and 10 CFR 50.54(p). The combined set of plans¹, submitted by letter dated May 17, 2006, is entitled: "Peach Bottom Atomic Power Station Security Plan, Training and Qualification Plan, Safeguards Contingency Plan, and Independent Spent Fuel Storage Installation Security Program, Revision 3." The set contains Safeguards Information protected under 10 CFR 73.21.

Exelon Generation Company shall fully implement and maintain in effect all provisions of the Commission-approved cyber security plan (CSP), including changes made pursuant to the authority of 10 CFR 50.90 and 10 CFR 50.54(p). The Exelon Generation Company CSP was approved by License Amendment No. 283.

¹The Training and Qualification Plan and Safeguards Contingency Plan and Appendices to the Security Plan.

Renewed License No. DPR-56
Revised by letter dated October 28, 2004
Revised by letter dated November 5, 2004
Revised by letter dated May 29, 2007
Amendment No. 299

plant-specific program. Before July 2, 2014, the licensee will notify the NRC of its decision to implement the core shroud inspection and evaluation guidelines program or a plant-specific program, and provide the appropriate revisions to the Updated Final Safety Analysis Report Supplement summary descriptions of the core shroud inspection and evaluation guidelines program.

(11) Mitigation Strategy License Condition

Develop and maintain strategies for addressing large fires and explosions and that include the following key areas:

- (a) Fire fighting response strategy with the following elements:
 - 1. Pre-defined coordinated fire response strategy and guidance
 - 2. Assessment of mutual aid fire fighting assets
 - 3. Designated staging areas for equipment and materials
 - 4. Command and control
 - 5. Training of response personnel
 - (b) Operations to mitigate fuel damage considering the following:
 - 1. Protection and use of personnel assets
 - 2. Communications
 - 3. Minimizing fire spread
 - 4. Procedures for implementing integrated fire response strategy
 - 5. Identification of readily-available pre-staged equipment
 - 6. Training on integrated fire response strategy
 - 7. Spent fuel pool mitigation measures
 - (c) Actions to minimize release to include consideration of:
 - 1. Water spray scrubbing
 - 2. Dose to onsite responders
- (12) The licensee shall implement and maintain all Actions required by Attachment 2 to NRC Order EA-06-137, issued June 20, 2006, except the last action that requires incorporation of the strategies into the site security plan, contingency plan, emergency plan and/or guard training and qualification plan, as appropriate.
- (13) Deleted

Renewed License No. DPR-56
Revised by letter dated August 9, 2007
Amendment No. 299

(14) Spent Fuel Pool Criticality Considerations

(a) Use of spent fuel pool storage cells without NETCO-SNAP-IN® rack inserts shall be restricted as follows:

- 1) Minimum panel Boron-10 areal density of a storage cell shall be greater than or equal to 0.014 grams per square centimeter to store fuel assemblies with the maximum in-core cold k-infinity of up to 1.235 (except as noted in a.3 below for restricted cells). The minimum panel Boron-10 areal density shall be evaluated by assuming that the panel areal density was initially equal to a value of 0.0235 grams per square centimeter.
- 2) A storage cell shall not contain any fuel assembly if the minimum panel Boron-10 areal density of a storage cell is less than 0.014 grams per square centimeter (except as noted in a.3 below for restricted cells). The minimum panel Boron-10 areal density shall be evaluated by assuming that the panel areal density was initially equal to a value of 0.0235 grams per square centimeter.

Renewed License No. DPR-56
Revised by letter dated August 9, 2007
Amendment No. 299

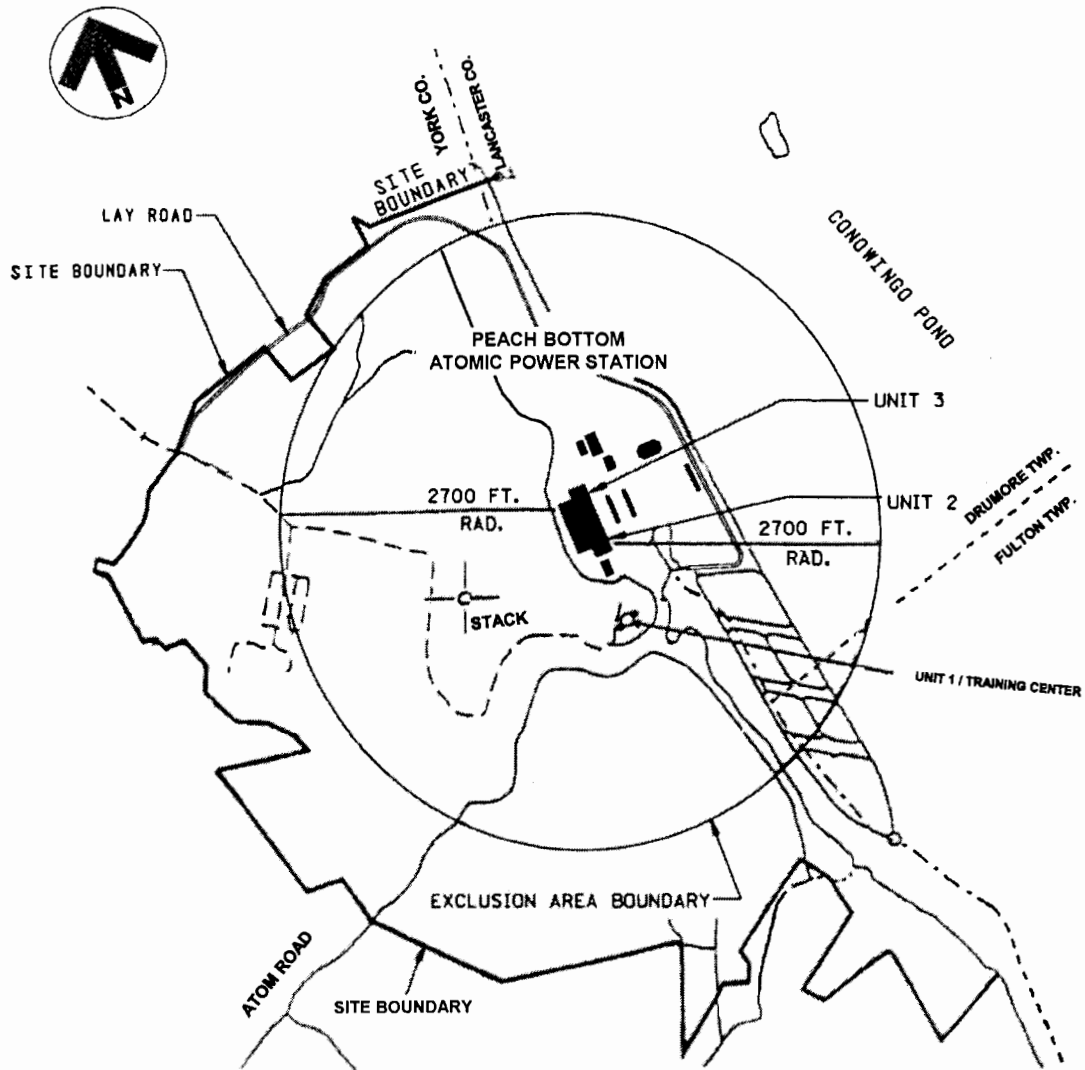


Figure 4.1-1 (page 1 of 1)
Site and Exclusion Area Boundaries

5.6 Reporting Requirements (continued)

5.6.5 CORE OPERATING LIMITS REPORT (COLR)

- a. Core operating limits shall be established prior to each reload cycle, or prior to any remaining portion of a reload cycle, and shall be documented in the COLR for the following:
 1. The Average Planar Linear Heat Generation Rate for Specification 3.2.1;
 2. The Minimum Critical Power Ratio for Specifications 3.2.2 and 3.3.2.1;
 3. The Linear Heat Generation Rate for Specification 3.2.3;
 4. The Control Rod Block Instrumentation for Specification 3.3.2.1; and
 5. The Oscillation Power Range Monitor (OPRM) Instrumentation for Specification 3.3.1.1.
- b. The analytical methods used to determine the core operating limits shall be those previously reviewed and approved by the NRC, specifically those described in the following document:
 1. NEDE-24011-P-A, "General Electric Standard Application for Reactor Fuel" (latest approved version as specified in the COLR).
- c. The core operating limits shall be determined such that all applicable limits (e.g., fuel thermal mechanical limits, core thermal hydraulic limits, Emergency Core Cooling Systems (ECCS) limits, nuclear limits such as SDM, transient analysis limits, and accident analysis limits) of the safety analysis are met.
- d. The COLR, including any midcycle revisions or supplements, shall be provided upon issuance for each reload cycle to the NRC.

5.6 Reporting Requirements (continued)

5.6.6 Post Accident Monitoring (PAM) Instrumentation Report

When a report is required by Condition B or F of LCO 3.3.3.1, "Post Accident Monitoring (PAM) Instrumentation," a report shall be submitted within the following 14 days. The report shall outline the preplanned alternate method of monitoring, the cause of the inoperability, and the plans and schedule for restoring the instrumentation channels of the Function to OPERABLE status.

5.6.7 Reactor Coolant System (RCS) PRESSURE AND TEMPERATURE LIMITS REPORT (PTLR)

- a. RCS pressure and temperature limits for heatup, cooldown, low temperature operation, criticality, and hydrostatic testing as well as heatup and cooldown rates shall be established and documented in the PTLR for the following:
 - i) Limiting Conditions for Operation Section 3.4.9, "RCS Pressure and Temperature (P/T) Limits"
 - ii) Surveillance Requirements Section 3.4.9, "RCS Pressure and Temperature (P/T) Limits"
 - b. The analytical methods used to determine the RCS pressure and temperature limits shall be those previously reviewed and approved by the NRC, specifically those described in the following document:
 - i) NEDC-33178P-A, "GE Hitachi Nuclear Energy Methodology for Development of Reactor Pressure Vessel Pressure-Temperature Curves," Revision 1, June 2009
 - c. The PTLR shall be provided to the NRC upon issuance for each reactor vessel fluence period and for any revision or supplement thereto.
-



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NOS. 296 AND 299

TO RENEWED FACILITY OPERATING LICENSE NOS. DPR-44 AND DPR-56

EXELON GENERATION COMPANY, LLC

PSEG NUCLEAR LLC

PEACH BOTTOM ATOMIC POWER STATION, UNITS 2 AND 3

DOCKET NOS. 50-277 AND 50-278

1.0 INTRODUCTION

By application dated July 11, 2014, as supplemented by letter dated December 1, 2014 (Agencywide Documents Access and Management System (ADAMS) Accession Nos. ML14192B143 and ML14337A294, respectively), Exelon Generation Company, LLC (Exelon, the licensee), requested changes to the Facility Operating Licenses (FOLs) and Technical Specifications (TSs) for Peach Bottom Atomic Power Station (PBAPS), Units 2 and 3.

The proposed amendment would incorporate several administrative changes as follows:

- 1) PBAPS Unit 3, FOL page 3: Footnote 1 in license condition 2.C(2), which specified requirements for implementation of a measurement uncertainty recapture (MUR) power uprate, would be deleted since it was a one-time requirement that has already been fulfilled and is no longer needed.
- 2) PBAPS Units 2 and 3, FOLs pages 7 and 7a: License Condition 2.C(13), which specified requirements related to control room habitability, would be deleted since it specifies one-time requirements that have already been fulfilled and are no longer needed.
- 3) PBAPS Units 2 and 3, TS page 4.0.3: Figure 4.1-1, "Site and Exclusion Area Boundaries," would be revised to correct an error regarding location of a road and make other minor changes to more accurately reflect the current physical configuration.
- 4) PBAPS Units 2 and 3, TS pages 5.0-21 and 5.0-22: TS 5.6.5, "Core Operating Limits Report (COLR)," would be revised to delete references that are no longer applicable.

The supplement dated December 1, 2014, provided additional information that clarified the application, did not expand the scope of the application as originally noticed, and did not change the Nuclear Regulatory Commission (NRC or the Commission) staff's original proposed no significant hazards consideration determination as published in the *Federal Register* on September 2, 2014 (79 FR 52062).

2.0 REGULATORY EVALUATION

In Section 50.36 of Title 10 of the *Code of Federal Regulations* (10 CFR), "Technical specifications," the NRC established its regulatory requirements related to the content of TSs. Pursuant to 10 CFR 50.36, TSs are required to include items in the following five specific categories: (1) safety limits, limiting safety system settings, and limiting control settings; (2) limiting conditions for operation; (3) surveillance requirements; (4) design features; and (5) administrative controls. The regulation does not specify the particular requirements to be included in a plant's TSs.

3.0 TECHNICAL EVALUATION

3.1 License Conditions 2.C(2) and 2.C(3) (Unit 3) - Delete Historical Footnote

On November 22, 2002, the NRC staff approved Amendments Nos. 247 and 250, for PBAPS, Units 2 and 3, respectively (ADAMS Accession No. ML031010365). The amendments approved a MUR power uprate to increase the licensed power level by 1.62%. As part of Amendment No. 250 for Unit 3, the NRC included the following implementation requirements:

This license amendment is effective as of its date of issuance, and shall be implemented upon startup following the Unit 3 14th Refueling Outage, currently scheduled for fall 2003.

On May 7, 2003, the NRC issued renewed licenses for PBAPS, Units 2 and 3 (ADAMS Accession No. ML031150073). Since the MUR had not yet been implemented for Unit 3 at the time the renewed license was issued, a footnote was added to the PBAPS Unit 3 renewed FOL (ADAMS Accession No. ML031180025), to address the MUR implementation requirements. Specifically, footnote "1" was added to license condition 2.C(2), "Technical Specifications." The footnote read as follows:

Licensed power level was revised by Amendment No. 250, dated November 22, 2002, and will be implemented following the 14th refueling outage currently scheduled for Fall 2003.

The licensee has proposed to delete this footnote since it was a one-time requirement that has already been fulfilled and is no longer needed.

The NRC staff concludes that this change is acceptable because the footnote proposed for deletion does not contain any requirements that are applicable to current or future operations.

License condition 2.C(3), "Physical Protection," currently has a footnote designated as footnote "2," which reads as follows:

The training and Qualification Plan and Safeguards Contingency Plan and Appendices to the Security Plan.

Due to the deletion of the footnote in license condition 2.C(2), the licensee has proposed to renumber footnote "2" in license condition 2.C(3) as footnote "1." In addition, the word "training" in the footnote has been changed to "Training," consistent with the footnote as originally issued via NRC letter dated October 28, 2004 (ADAMS Accession No. ML043120008). The NRC staff concludes that these changes are administrative in nature, and therefore, are acceptable.

3.2 License Condition 2.C(13) (Units 2 and 3) - Delete Historical License Condition

On October 31, 2007, the NRC staff approved amendments for a number of Exelon plants, including PBAPS, Units 2 and 3, which modified TS requirements, related to control room habitability (ADAMS Accession No. ML072550382). As part of these amendments, license condition 2.C(13) was added to PBAPS, Units 2 and 3, FOLs pages 7 and 7a. The new license condition included requirements regarding the first performance of new surveillance, assessment, and measurement requirements. The license condition specified time requirements for completion of these 3 items.

The licensee's application stated that the first performance of the surveillance, assessment, and measurement requirements were completed within the time allowances specified in license condition 2.C(13). As such, the licensee has proposed to delete license condition 2.C(13) since it specifies one-time requirements that have already been fulfilled, and are no longer needed.

The NRC staff concludes that this change is acceptable because license condition 2.C(13) does not contain any requirements applicable to current or future operations.

3.3 TS Figure 4.1-1, "Site and Exclusion Area Boundaries" (Units 2 and 3) - Revise Figure to Reflect Current Physical Configuration

The licensee's application proposed to revise TS Figure 4.1-1, "Site and Exclusion Area Boundaries," to correct errors and to make other minor changes to more accurately reflect the current site configuration. The revision would include correcting the details pertaining to the main access road in relation to the PBAPS facility. Additionally, the correction would include adjusting the details of the parking areas, correcting the access road within the Site Area Boundary (SAB), and changing the Susquehanna River label to Conowingo Pond. The route of the main access road has not physically changed, nor has the location of the SAB. Once within the SAB, the main access road runs along the bank of Conowingo Pond, as stated in the UFSAR. Therefore, for consistency, the name would be changed from Susquehanna River to Conowingo Pond. Extra lines that depict old parking areas would be removed from the drawing. In addition, railroad tracks that were in place during the construction phase of PBAPS, and have since been removed, would be deleted from the drawing. For clarity and consistency purposes, the licensee stated the correction of these items is appropriate.

The purpose of Figure 4.1-1 is to define the site and exclusion area boundaries. The NRC staff concludes that the proposed changes are appropriate since they more accurately reflect the current physical configuration. Therefore, the proposed changes to Figure 4.1-1 are acceptable.

3.4 TS 5.6.5.b (Units 2 and 3) - Delete COLR References

The NRC approved analytical methods for determining the core operating limits, as reported in the COLR, are specified in TS 5.6.5.b. The licensee's application stated that TS 5.6.5.b contains references that are no longer applicable for determining COLR values and should be deleted. The licensee has proposed to delete references 2 through 9, as discussed below.

With respect to Reference 2, General Electric (GE) topical report NEDC-32162P, "Maximum Extended Load Line Limit and ARTS [Average Power Range Monitor, Rod Block Monitor, Technical Specifications] Improvement Program Analyses for Peach Bottom Atomic Power Station Units 2 and 3," Revision 2, dated March 1995, the licensee's application stated that this reference is an application report, not a methodology. Furthermore, the licensee stated that the licensing basis for ARTS/Maximum Extended Load Line Limit Analysis (MELLLA) is specified in NEDE-24011-P-A, "General Electric Standard Application for Reactor Fuel," (referred to as GESTAR II), which is included as TS 5.6.5.b, Reference 1. The NRC staff concludes that the proposed deletion of Reference 2 is acceptable since the analytical methods pertaining to ARTS/MELLLA are described in GESTAR II, which is included as TS 5.6.5.b, Reference 1.

References 3 through 8 pertain to Philadelphia Electric Company (PECO) methodologies. The licensee's application stated that these references were approved in the 1988 time frame and are no longer used. Since these methods are no longer used, the NRC staff concludes that the deletion of TS 5.6.5.b, References 3 through 8, is acceptable.

With respect to Reference 9, GE topical report NEDO-32465-A, "Reactor Stability Detect and Suppress Solution Licensing Basis Methodology and Reload Applications," dated August 1996, the licensee's application stated that this topical report is referenced within GESTAR II (i.e., TS 5.6.5.b, Reference 1). As such, the licensee stated there is no need to reference it separately in the TSs. The NRC staff concludes that the proposed deletion of Reference 9 is acceptable since the associated topical report is referenced within GESTAR II, which is included as TS 5.6.5.b, Reference 1.

3.5 Technical Evaluation Conclusion

Based on the considerations in Sections 3.1 through 3.4 above, the NRC staff concludes that the proposed amendment is acceptable.

4.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Pennsylvania State official was notified of the proposed issuance of the amendments. The State official had no comments.

5.0 ENVIRONMENTAL CONSIDERATION

The amendment makes editorial, corrective or other minor revisions; including the updating of NRC approved references. Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(10). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need to be prepared in connection with the issuance of the amendment.

6.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) there is reasonable assurance that such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributors: T. Lamb
R. Ennis

Date: March 11, 2015

Mr. Bryan C. Hanson
President and Chief Nuclear Officer
Exelon Nuclear
4300 Winfield Road
Warrenville, IL 60555

SUBJECT: PEACH BOTTOM ATOMIC POWER STATION, UNITS 2 AND 3 - ISSUANCE OF AMENDMENTS RE: REVISE FACILITY OPERATING LICENSE AND THE TECHNICAL SPECIFICATIONS TO INCORPORATE ADMINISTRATIVE CHANGES (TAC NOS. MF4401 AND MF4402)

Dear Mr. Hanson:

The Commission has issued the enclosed Amendment Nos. 296 and 299 to Renewed Facility Operating License Nos. DPR-44 and DPR-56 for Peach Bottom Atomic Power Station, Units 2 and 3. These amendments consist of changes to the Technical Specifications (TSs) and Facility Operating Licenses (FOLs) in response to your application dated July 11, 2014, as supplemented by letter dated December 1, 2014.

The amendments incorporate several administrative changes to the FOLs and TSs such as deleting historical items that are no longer applicable, correcting errors, and removing references that are no longer valid.

A copy of the safety evaluation is also enclosed. Notice of Issuance will be included in the Commission's Biweekly *Federal Register* Notice.

Sincerely,

/RA/

Richard B. Ennis, Senior Project Manager
Plant Licensing Branch I-2
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket Nos. 50-277 and 50-278

Enclosures:

1. Amendment No. 296 to Renewed DPR-44
2. Amendment No. 299 to Renewed DPR-56
3. Safety Evaluation

cc w/encls: Distribution via Listserv

DISTRIBUTION:

PUBLIC	RidsAcrsAcnw_MailCTR Resource
LPL 1-2 R/F	RidsRgn1MailCenter Resource
RidsNrrDoriDpr Resource	RidsNrrDssStsb Resource
RidsNrrDoriLpl1-2 Resource	RidsNrrDssSrxs Resource
RidsNrrPMPeachBottom Resource	
RidsNrrLAABaxter Resource	

ADAMS Accession No: ML14363A227

OFFICE	LPL1-2/PM	LPL1-2/PM	LPL1-2/LA	SRXB/BC	STSB/BC	OGC	LPL1-2/BC	LPL1-2/PM
NAME	TLamb	REnnis	ABaxter	CJackson	RElliott	AGhosh	DBroaddus	REnnis
DATE	1/9/15	2/9/15	1/23/15	1/28/15	2/3/15	2/6/15	3/11/15	3/11/15

OFFICIAL RECORD COPY