

UNITED STATES NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

November 3, 2014

Mr. Scott L. Batson Site Vice President Oconee Nuclear Station ON01 VP / 7800 Rochester Hwy Seneca, SC 29672

SUBJECT: REQUEST FOR ADDITIONAL INFORMATION FOR REQUEST FOR

EXEMPTION FROM 10 CFR 72.212(b)(3) AND 72.212(b)(11) (TAC NO. L24946)

Dear Mr. Batson:

By application dated August 28, 2014, Duke Energy Carolinas, LLC (Duke Energy) requested the U.S. Nuclear Regulatory Commission's (NRC) approval of an exemption to certain requirements of 10 CFR 72.212(b)(3) and 72.212(b)(11) pursuant to 10 CFR 72.7.

In connection with the staff's review of your application, we need the information identified in the enclosure to this letter. We request that you provide this information by December 1, 2014.

Please reference Docket No. 72-40 and TAC No. L24946 in future correspondence related to this request. The staff is available to meet with you to discuss your proposed responses. If you have any questions regarding this matter, I may be contacted at (301) 287-9165.

Sincerely,

John Vera, Project Manager Spent Fuel Licensing Branch Division of Spent Fuel Management Office of Nuclear Material Safety and Safeguards

Docket No. 72-40 TAC No. L24946

Enclosure: Request for Additional Information

Mr. Scott L. Batson Site Vice President Oconee Nuclear Station ON01 VP / 7800 Rochester Hwy Seneca, SC 29672

SUBJECT: REQUEST FOR ADDITIONAL INFORMATION FOR REQUEST FOR

EXEMPTION FROM 10 CFR 72.212(b)(3) AND 72.212(b)(11) (TAC NO. L24946)

Dear Mr. Batson:

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In connection with the staff's review of your application, we need the information identified in the enclosure to this letter. We request that you provide this information by December 1, 2014.

Please reference Docket No. 72-40 and TAC No. L24946 in future correspondence related to this request. The staff is available to meet with you to discuss your proposed responses. If you have any questions regarding this matter, I may be contacted at (301) 287-9165.

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/RA/

John Vera, Project Manager Spent Fuel Licensing Branch Division of Spent Fuel Management Office of Nuclear Material Safety and Safeguards

Docket No. 72-40 TAC No. L24946

Enclosure: Request for Additional Information

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ADAMS Accession No.:

Accession No. 1					
OFC:	SFM	SFM	SFM	SFM	SFM
NAME:	JVera	WWheatley	JPiotter	CAraguas	MSampson
DATE:	11/3/14	11/3/14	11/3/14	11/3/14	11/3/14

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Request for Additional Information for Oconee Nuclear Station, Units 1, 2 and 3 Docket No. 72-40

By application dated August 28, 2014, Duke Energy Carolinas, LLC (Duke Energy) requested the U.S. Nuclear Regulatory Commission's (NRC) approval of an exemption to certain requirements of 10 CFR 72.212(b)(3) and 72.212(b)(11) pursuant to 10 CFR 72.7.

This request for additional information (RAI) identifies information needed by the staff in connection with its review of the application.

1) Provide the following information:

For each affected canister, provide a comparison table illustrating both the leakage rates with the correct and incorrect temperature correction coefficients applied.

- a. Provide calculation(s), including the assumed ambient temperatures, illustrating how the temperature correction factor was applied to arrive at a revised leakage rate limit of 1.02 x 10^-07 reference cc/ sec.
- b. ANSI N14.5 identifies leak-tight as 1x10^-07 reference cc/sec and the identified revised technical specification is being identified as 1.02x10^-07 reference cc/sec. Understanding the sensitivity of the calculations to the respective temperature correction values, in light of the worst case temperature assumption, is necessary to evaluate whether the 2% increase in the Technical Specification limit has a safety implication requiring the calculation of release fractions. At this magnitude of values, there may be no discernable difference between the two leakage rate limits, given the capabilities of the measuring instruments.

This information is needed to determine the acceptability of an exemption from 10 CFR 72.212(b)(3) and 72.212(b)(11).

2) Provide the error range for the calibrated or standard leak used during the leak testing operations.

Staff is unclear as to the level of uncertainty in the measured leak testing values without complete information on the calibrated or standard leak used to provide a testing baseline.

This information is needed to determine the acceptability of an exemption from 10 CFR 72.212(b)(3) and 72.212(b)(11).