

**GPU Nuclear Corporation** Post Office Box 388 Route 9 South Forked River, New Jersey 08731-0388 609 971-4000

Writer's Direct Dial Number: C321-93-2238 August 27, 1993

U.S. Nuclear Regulatory Commission Att: Document Control Desk Washington, D.C. 20555

Dear Sir:

Subject: Oyster Creek Nuclear Generating Station Docket No. 50-219 Effluent Release Report

Attached is a copy of the Oyster Creek Effluent Release Report for the period covering January, 1993 through June, 1993. This submittal is made in accordance with 10 CFR 50.36a(a)(2) and our Operating License and Technical Specifications.

If you should have any questions or require further information, please contact Brenda DeMerchant, OC Licensing Engineer at (609) 971-4642.

Very truly yours,

John J. Barton Vice President & Director Oyster Creek

JJB/BDEM:jc Attachment

cc: Administrator, Region 1
Senior NRC Resident Inspector
Chief, Bureau of Nuclear Engineering
N.J. Dept. of Environmental Protection & Energy
CN 411

Trenton, New Jersey 08623

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#### EXECUTIVE SUMMARY, 1993-1 SEMI-ANNUAL RELEASE REPORT

The Semiannual Effluent Release Report is submitted to the United States Nuclear Regulatory Commission (NRC) every six months in accordance with the Oyster Creek Nuclear Generating Station (OCNGS) Technical Specifications (Tech Specs). It Summarizes the radioactive gaseous and liquid effluents released and solid radioactive wastes shipped from OCNGS.

Attached Tables show that doses based on quantities of radioactive material released were all less than 1% of the limits allowed by the OCNGS Tech Specs. Limits for the release of radioactive effluents at OCNGS are based upon offsite exposure to members of the general public. These limits were compared to dose projections calculated using the methodology in the Offsite Dose Calculation Manual (ODCM). There were no liquid releases from OCNGS during the period. Solid waste shipments were similar to those of nuclear plants of comparable type, age and size. Concrete was used for solidification material during the reporting period. The report summarizes the fact that all effluents released were within federal regulatory requirements of the OCNGS Technical Specifications.

Included is a description of changes made to the Offsite Dose Calculation Manual (ODCM) and the Process Control Plan (PCP) during the reporting period. Effluent monitoring instruments that were inoperative as per Technical Specification 3.15 for the reporting period are also discussed.

Tech. Spec.	3.6.J.1 3.6.J.1		3.6.L.1 Air Dose	3.6.L.1	3.6.K.1	3.6.M.1 (Thyroid)	3.6.K.1
	WB mrem	Organ mrem	Beta	Gamma mrem	Body	Organ mrem	Skin mrem
Jan-June Total	0.00E+00	0.00E+00	3.39E-4	9.82E-4	4.42E-4	1.45E-2	6.61E-4
Tech.Spec. Limit	: 3	10	20	10	500	15	3000
Fraction of Limi	t 0.00E+00	0.00E+00	1.7E-5	9.82E-5	8.84E-7	9.67E-4	2.2E-7

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Maximum Offsite Dose Due to Radionuclides in Effluents Jan-June 1993

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#### **Oyster Creek Nuclear Station 1993-1 Semi Annual Effluent Report**

#### Changes to the Offsite Dose Calculation Manual

No changes were made to the ODCM during this time period.

#### Changes to the Process Control Plan

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No changes were made to the PCP during this time period.

#### Effluent Monitors Out of Service Greater than 30 Days

During the first half of 1993 three instruments were out of service for longer than 30 days:

The Overboard Discharge Monitor was declared inoperable December 7, 1992, and was returned to service March 3, 1993. No overboard discharges occurred during the period from December through March when the instrument was out of service.

The Service Water Rad. Monitor was returned to service February 16, 1993. Daily discharge samples were collected as required while the monitor was out of service

The AOG vent monitor was out of service from November, 1992 due to sample tubing leakage. It was returned to service May 21, 1993.

## OYSTER CREEK NUCLEAR GENERATING STATION LIQUID EFFLUENT RELEASES FIRST AND SECOND QUARTERS 1993

Oyster Creek Nuclear Generating Station (OCNGS) policy is to strive for zero liquid discharge of radioactive material.

As a result, there were no liquid continuous or batch releases from OCNGS in the first half of 1993.

### OYSTER CREEK NUCLEAR GENERATING STATION LIQUID EFFLUENT RELEASES FIRST AND SECOND QUARTERS 1993

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As a result, there were no liquid continuous or batch releases from OCNGS in the first half of 1993.

#### Effluent and Waste Disposal Supplemental Information

FACILITY: Oyster Creek Nuclear Generating Station LICENSEE: Owner - Jersey Central Power and Light Company Operator - GPU Nuclear Corporation

#### 1.) Regulatory Limits

a.) Fission and Activation Gases

Technical Specification 3.6.E.1

The gross radioactivity in noble gases discharged from the main condenser air ejector shall not exceed a 0.21/E Ci/sec after the holdup line ,where E is the average gamma energy (Mev per atomic transformation).

Technical Specification 3.6.K.1

The dose equivalent rate outside of the EXCLUSION AREA due to radioactive noble gas in gaseous effluent shall not exceed 500 mrem/year to the total body or 3000 mrem/year to the skin.

Technical Specification 3.6.L.1 The air dose outside of the EXCLUSION AREA due to noble gas released in gaseous effluent shall not exceed:

5 mrad/calendar quarter due to gamma radiation, 10 mrad/calendar quarter due to beta radiation, 10 mrad/calendar year due to gamma radiation, or 20 mrad/calendar year due to beta radiation

#### Technical Specification 3.6.N.1

The annual dose to a MEMBER OF THE PUBLIC due to radiation and radioactive material in effluents from the OCNGS outside of the EXCLUSION AREA shall not exceed 75 mrem to his thyroid or 25 mrem to his total body or to any other organ.

#### b. Iodines and Particulates

#### Technical Specification 3.6.K.2

The dose equivalent rate outside of the EXCLUSION AREA due to H-3, I-131, I--133, and to radioactive material in particulates having half-lives of 8 days or more in gaseous effluents shall not exceed 1500 mrem/year to any body organ when the dose rate due to H-3, Sr-89, Sr-90, and alpha-emitting radionuclides is averaged over no more than 3 months and the dose rate due to other radionuclides is averaged over no more than 31 days.

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#### Technical Specification 3.6.M.1

The dose to a MEMBER OF THE PUBLIC from iodine-131, iodine-133, and from radionuclides in particulate form having half-lives of 8 days or more in gaseous effluents, outside of the EXCLUSION AREA shall not exceed 7.5 mrem to any body organ per calendar quarter or 15 mrem to any body organ per calendar year.

#### c. Liquid Effluents

Technical Specification 3.6.I.1

The concentration of radioactive material, other than noble gases, in liquid effluent in the discharge canal at the Route 9 bridge shall not exceed the concentrations specified in 10 CFR Part 20, Appendix B, Table II, Column 2.

#### **Technical Specification 3.6.I.2**

The concentration of noble gases dissolved or entrained in liquid effluent in the discharge canal at the Route 9 bridge shall not exceed 2X10<sup>-4</sup> microcuries/milliliter.

Technical Specification 3.6.J.2

The dose to a MEMBER OF THE PUBLIC due to radioactive material in liquid effluents beyond the outside of the EXCLUSION AREA shall not exceed:

1.5 mrem to the total body during any calendar quarter,
5 mrem to any body organ during any calendar quarter,
3 mrem to the total body during any calendar year, or
10 mrem to any body organ during any calendar year.

#### 2.) Maximum Permissible Concentrations (MPC)

a. Fission and Activation Gases:

Appendix B, Table II, Column 2 of 10 CFR 20

b. Iodines and Particulates:

Appendix B, Table II, Column 2 of 10 CFR 20

c. Liquid Effluents:

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Appendix B, Table II, Column 2 of 10 CFR 20, except for dissolved or entrained noble gases where the limit is  $2 \times 10^{-4}$  uCi/ml

- 3.) Measurements and Approximation of Total Radioactivity
  - a. Fission and Activation Gases:

1. Stack

The continuous recording of gross activity and the incorporation of isotopic data obtained from a weekly grab sample analyzed using gamma spectroscopy.

#### 2. Augmented Offgas (AOG) Vent

The continuous recording of gross activity and the incorporation of isotopic data obtained from a weekly grab sample analyzed using gamma spectroscopy.

3. Turbine Building Stack and Feedpump Room Vent

The continuous recording of gross activity and the incorporation of isotopic data obtained from a monthly grab sample analyzed using gamma spectroscopy.

b. Iodines

1. Stack

Filters are changed twice weekly and analyzed using gamma spectroscopy.

2. AOG Vent

Filters are changed twice weekly and analyzed using gamma spectroscopy.

3. Turbine Building Stack and Feedpump Room Vent

Filters are changed twice weckly and analyzed using gamma spectroscopy.

c. Particulates

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1. Stack

Filters are changed twice weekly and analyzed using a low background beta counter and gamma spectroscopy.

2. AOG Vent

Filters are changed twice weekly and analyzed using gamma spectroscopy.

3. Turbine Building Stack and Feedpump Room Vent

Filters are changed twice weekly and analyzed using gamma spectroscopy.

#### d. Liquid Effluents

Analysis per batch release using gamma spectrometry with a germanium detector, a low background beta counter, and a liquid scintillation counter.

#### OYSTER CREEK NUCLEAR GENERATING STATION GASEOUS EFFLUENT ELEVATED RELEASES First Quarter 1993

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FISSION GASES	QUANTITY
	(ci )
KR85M	1.98E+00
KR87	8.07E+00
XE133	7.36E+00
XE135	1.16E+01

Total Fission Gases Released: 2.90E+01 ciGamma EBar:0.342 MevAverage Rate of Release:3.77E+00 uCi/sec

QUANTITY
(CÌ )
5.87E-03
2.87E-02
6.06E-02
1.06E-01

Total Iodines Released:2.01E-01 ciAverage Rate of Release:2.61E-02 uCi/sec

PARTICULATES	QUANTITY
	(ci )
CR51	9.90E-05
MN 54	1.24E-05
CO60	2.05E-04
SR89	1.01E-04
SR90	4.65E-06
Y91M	5.02E-02
TC99M	5.01E-04
<b>TE132</b>	4.61E-04
CS137	4.54E-05
CS138	7.81E+00
BA139	2.47E-01
BA140	1.32E-04
LA140	1.43E-04
CE144	2.15E-05
GROSSA	3.78E-06

Total Particulates Released: 8.11E+00 ci Average Rate of Release: 1.05E+00 uCi/sec

	RADIONUCLIDE					QUANTITY		
			H3			3.31E-01		
Avg.	Rate	of	Release	for	Н3:	4.31E-02	uCi/sec	

#### OYSTER CREEK NUCLEAR GENERATING STATION GASEOUS EFFLUENT GROUND-LEVEL RELEASES First Quarter 1993

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FISSION GASES	QUANTITY	
XE133	(C1) 1.44E-02	
XE135	3.74E-03	
Total Fission Gases Released: Average Rate of Release:	1.81E-02 2.35E-03	ci uCi/sec
IODINES	QUANTITY	
I131	1.70E-07	
I133	1.04E-05	
Total Iodines Released:	1.05E-05	ci
Average Rate of Release:	1.37E-06	uCi/sec
PARTICULATES	QUANTITY	
SR89	2.53E-06	
Total Particulates Poloased.	2 538-06	ci
Average Rate of Release:	3.29E-07	uCi/sec
RADIONUCLIDE	QUANTITY	
НЗ	(CI) 0.00E+00	
Avg. Rate of Release for H3:	0.00E+00	uCi/sec

#### OYSTER CREEK NUCLEAR GENERATING STATION GASEOUS EFFLUENT ELEVATED RELEASES Second Quarter 1993

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FISSION GASES	QUANTITY
	(C1 )
KR85M	2.08E+00
KR87	1.02E+01
XE135M	9.61E+00
XE135	1.46E+01
Total Fission Gases Released:	3.65E+01 ci
Gamma EBar:	0.443 Mev
Average Rate of Release:	4.69E+00 uCi/sec
IODINES	QUANTITY
	(ci)
I131	1.84E-03
I133	6.31E-03
Total Iodines Released:	8.15E-03 ci
Average Rate of Release:	1.05E-03 uCi/sec
	ΟΠΛΝΨΤΨΥ
FARICOLAIES	(Ci)

FICULATES	QUANTITY
	(ci )
NA24	3.82E-05
CR51	9.80E-05
CO60	5.84E-05
RB89	3.44E-02
SR89	3.90E-04
Y91M	2.43E-02
TC99M	2.64E-03
TE132	7.21E-06
CS138	7.66E-01
BA139	2.61E-01
BA140	5.08E-05
GROSSA	5.91E-07

Total Particulates Released: 1.09E+00 ci Average Rate of Release: 1.40E-01 uCi/sec

RADIONUCLIDE	QUANTITY		
	(ci )		
H3	1.37E+00		

Avg. Rate of Release for H3: 1.76E-01 uCi/sec

#### OYSTER CREEK NUCLEAR GENERATING TATION GASEOUS EFFLUENT GROUND-LEVEL RELEASES Second Quarter 1993

FISSION GASES	QUANTITY (ci )	*
Total Fission Gases Released:	0.00E+00	ci
Average Rate of Release:	0.00E+00	uCi/sec

IODINES QUANTITY (ci ) 3.00E-06 I131 I133 1.18E-05

Total Iodines Released: 1.48E-05 ci Average Rate of Release: 1.90E-06 uCi/sec

> PARTICULATES QUANTITY (ci ) 4.17E-07 CR51 MN54 3.97E-07 SR89 1.05E-05 CS138 4.51E-03 BA139 4.74E-04 GROSSA 2.74E-06

Total Particulates Released: 4.99E-03 ci Average Rate of Release: 6.42E-04 uCi/sec

	RADIONUCLIDE				QUANTITY		
			H3		0.00E+00		
Avg.	Rate	of	Release	for H3:	0.00E+00	uCi/sec	

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#### Solid Waste Shipped Offsite for Disposal During Period From 01/01/93 to 06/30/93

Report Date 8/19/93

### WASTE STREAM: FILTERS AND DRY ACTIVE WASTE

WASTE	VOL	UME		% ERROR (Ci)	
CLASS	Ft <sup>3</sup>	M <sup>3</sup>	SHIFFLD		
A	170.8	4.83	8.20	± 25%	
В	0	0	0	± 25%	
С	0	0	0	<b>±</b> 25%	
A11	170.8	4.83	8.20	± 25%	

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## WASTE STREAM: FILTERS AND DRY ACTIVE WASTE

### (Packaged in HIC)

Waste	Nuclide	Percent	<u>Curies</u>
<u>Class</u>	<u>Name</u>	<u>Abundance</u>	
A	Fe-55 Co-60 Cs-137 Mn-54 Cs-134 Cr-51 Ni-63 C-14 Ni-59 Tc-99 I-129 Pu-241 Cm-242 H-3 Sr-90	49.02% 34.63% 9.67% 2.48% 1.89% 1.18% 0.36% 0 0 0 0 0 0 0	4.02 E0 2.84 E0 7.93 E-1 2.03 E-1 1.55 E-1 9.66 E-2 2.95 E-2 7.30 E-4 2.43 E-3 LLD(6.09E-5 uCi/CC) LLD(1.89E-4 uCi/CC) 7.31 E-3 3.3 E-5 4.28 E-4 6.17 E-3

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### WASTE STREAM: RESINS, FILTERS SLUDGE

Waste	Nuclide	Percent	<u>Curies</u>
<u>Class</u>	<u>Name</u>	<u>Abundance</u>	
A	Name	Abundance	Curies
	Co-60	44.04%	1.11 E+2
	Fe-55	41.30%	1.04 E+2
	Cs-137	5.91%	1.49 E+1
	Mn-54	3.17%	8.00 E0
	Co-58	1.71%	4.3 E0
	Cr-51	1.46%	3.68 E0
	Cs-134	0.95%	2.4 E0
	Zn-65	0.45%	1.13 E0
	Ni-63	0.433%	1.09 E0
	Sr-90	0.10%	2.3 E-1
	C-14	0.11%	2.71 E-1
	Pu-241	0.05%	1.26 E-1
	Ni-59	0.02%	5.61 E-2
	H-3	0.001%	3.61 E-3
	Cm-242	0	5.65 E-4
	Tc-99	0	LLD(1.73 E-4 uCi/CC)
	1-129	U	LLU(2.0 E-4 U(1/(0)))

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## Estimates of Major Nuclides by Waste Class and Stream

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## WASTE STREAM: RESINS, FILTERS SLUDGE

Waste	Nuclide	Percent	<u>Curies</u>
<u>Class</u>	<u>Name</u>	<u>Abundance</u>	
A11	Co-60	48.47%	3.48 E+2
	Fe-55	30.08%	2.16 E+2
	Mn-54	7.65%	5.49 E+1
	Cs-137	4.60%	3.3 E+1
	Cr-51	3.15%	2.26 E+1
	Co-58	2.24%	1.61 E+1
	Zn-65	1.41%	1.01 E+1
	Cs-134	1.33%	9.56 E0
	Ni-63	0.39%	2.82 E0
	Sr-90	0.08%	5.65 E-1
	C-14	0.075%	5.37 E-1
	Pu-241	0.03%	2.17 E-1
	Ni-59	0.012%	8.59 E-2
	H-3	0.0007%	4.71 E-3
	Cm-242	0.0002%	1.16 E-3
	Tc-99	0	LLD(1.73E-4 uCi/CC)
	I-129	0	LLD(2.6E-4 uCi/CC)
	1-123	0	LLD(2.0L-4 UC)

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#### Solid Waste Shipped Offsite for Disposal During Period From 01/1/93 to 06/30/93

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Report Date 8/19/93

#### WASTE STREAM: IRRADIATED COMPONENTS

WASTE	VOLUME		CURIES SHIPPED	% ERROR (Ci)
CLASS	Ft <sup>3</sup>	M3		
Α	118.4	3.35	8.34	± 25%
В	0	0	0.00 E+00	± 25%
С	0	0	0.00 E+00	± 25%
A11	118.4	3.35	8.34	± 25%

Solid Waste Shipped Offsite for Disposal During Period From 01/1/93 to 06/30/93

Report Date 8/19/93

### WASTE STREAM: DRY ACTIVE WASTE SENT TO A REPROCESSOR

WASTE	VOLUME			% ERROR (Ci)
CLASS	Ft <sup>3</sup>	M <sup>3</sup>	SHIFFED	
A	15,582	440.97	2.776	± 25%
В	0	0	0	± 25%
С	0	0	0	± 25%
A11	15,582	440.97	2.776	± 25%

<u>Note:</u> This material was sent to a Reprocessor for further Volume Reduction prior to burial.

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### WASTE STREAM: DRY ACTIVE WASTE SENT TO BURIAL GROUND

Waste <u>Class</u>	Nuclide <u>Name</u>	Percent <u>Abundance</u>	<u>Curies</u>
A	Fe-55 Co-60 Cs-137 Mn-54 Cr-51 Zn-65 Cs-134 Cm-242 Pu-241 I-129 Tc-99 Sr-90 C-14 H-3 Nb-94 Ni-63 Ni-59	52.257% 32.229% 7.209% 3.582% 2.205% 1.484% 1.034% 0 0 0 0 0 0 0 0 0 0 0 0 0 0 0 0 0 0 0	1.60 E+0 9.88 E-1 2.21 E-1 1.10 E-1 6.76 E-2 4.55 E-2 3.17 E-2 0 0 0 0 0 0 0 0 0 0 0 0 0 0 0 0 0 0 0

## Solid Waste Shipped Offsite for Disposal During Period 01/01/93 to 06/30/93

Report Date 8/19/93

#### WASTE STREAM: DRY ACTIVE WASTE SENT TO BURIAL GROUND

Waste	Volume		Curies	% Error (Ci)
	Ft <sup>3</sup>	M <sup>3</sup>	Shipped	
A	360.2	10.2	3.07	± 25%
В	0	0	0	± 25%
С	0	0	0	± 25%
A11	360.2	10.2	3.07	± 25%

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#### WASTE STREAM: DRY ACTIVE WASTE SENT TO A REPROCESSOR

Waste <u>Class</u>	Nuclide <u>Name</u>	Percent <u>Abundance</u>	<u>Curies</u>
A	Fe-55 Co-60 Cs-137 Mn-54 Cr-51 Zn-65 Cs-134 Ni-63 Sr-90 Pu-241 Ni-59 H-3 C-14 Tc-99 I-129	51.30% 31.70% 7.07% 3.53% 2.17% 1.46% 1.02% 0.36% 0.06% 0.06% 0.06% 0.02% 0 0	1.42 E0 8.80 E-1 1.963E-1 9.8 E-2 6.024E-2 4.053E-2 2.832E-2 9.99 E-3 1.67 E-3 1.67 E-3 5.55 E-4 LLD(1.40E-3 uCi/CC) LLD(1.73E-4 uCi/CC) LLD(2.60E-4 uCi/CC)

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Solid Waste Shipped Offsite for Disposal During Period From 01/1/93 to 06/30/93

Report Date 8/19/93

### WASTE STREAM: CONTAMINATED METAL SENT TO REPROCESSOR

WASTE	VOLUME			% ERROR (Ci)
CLASS	Ft <sup>3</sup>	M <sup>3</sup>	SHIFFLD	
A	1,917	54.25	3.45 E-1	± 25%
В	0	0	0	± 25%
С	0	0	0	± 25%
A11	1,917	54.25	3.45 E-1	± 25%

<u>Note:</u> This material was sent to a Reprocessor for further Volume Reduction prior to burial.

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#### WASTE STREAM: CONTAMINATED METAL SENT TO REPROCESSOR

Waste	Nuclide	Percent	<u>Curies</u>
<u>Class</u>	<u>Name</u>	<u>Abundance</u>	
Α	Fe-55	51.30%	1.77 E-1
	Co-60	31.70%	1.09 E-1
	Cs-137	7.07%	2.44 E-2
	Mn-54	3.53%	1.22 E-2
	Cr-51	2.17%	7.49 E-3
	Zn-65	1.46%	5.04 E-3
	Cs-134 Ni-63	1.02%	3.52 E-3 1.24 F-3
	Sr-90 C-14	0.06%	2.07 E-4
	Pu-241 Ni-59	0.06%	2.07 E-4 6 9 F-5
	H-3 (m-242	0	LLD(1.40E-3 uCi/CC)
	Tc-99	0	LLD(1.73E-4 uCi/CC)
	I-129	0	LLD(2.60E-4 uCi/CC)

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### WASTE STREAM: SUM OF ALL CATEGORIES

Waste	Nuclide	Percent	<u>Curies</u>
<u>Class</u>	<u>Name</u>	<u>Abundance</u>	
Α	Co-60 Fe-55 Cs-137 Mn-54 Cr-51 Co-58 Cs-134 Ni-63 Sr-90 C-14 Pu-241 Ni-59 H-3 Cm-242 Tc-99 I-129 Nb-94	43.32% 41.5 % 5.86% 3.06% 2.17% 1.57% 0.95% 0.54% 0.087% 0.1% 0.05% 0.02% 0 0 0 0	119 E0 114 E0 16.1 E0 8.42 E0 5.95 E0 4.3 E0 2.62 E0 1.49 E0 2.38 E-1 2.72 E-1 1.37 E-1 6.18 E-2 4.04 E-3 5.98 E-4 LLD(1.73E-4) LLD(2.6 E-4 uCi/CC) 7.85 E-7

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## WASTE STREAM: SUM OF ALL CATEGORIES

Waste	Nuclide	Percent	
<u>Class</u>	<u>Name</u>	<u>Abundance</u>	<u>Curies</u>
В	Co-60	50.86%	2.37 E+2
	Fe-55	24.03%	1.12 E+2
	Mn-54	10.06%	4.69 E+1
	Cr-51	4.06%	1.89 E+1
	Cs-137	3.88%	1.81 E+1
	Co-58	2.53%	1.18 E+1
	Zn-65	1.92%	8.97 EO
	Cs-134	1.54%	7.16 EO
	Ni-63	0.37%	1.73 E-7
	Sr-90	0.07%	3.35 E-7
	C-14	0.06%	2.66 E-1
	Pu-241	0.02%	9.08 E-2
	Ni-59	0.006%	2.98 E-2
	H-3	0	1.1 E-3
	Cm-242	0	5.95 E-4
	Tc-99	0	LLD(1.73E-4 uCi/CC)
	I-129	0	LLD(2.6 E-4 uCi/CC)
	Nb-94	0	0

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#### WASTE STREAM: SUM OF ALL CATEGORIES

Waste	Nuclide	Percent	
<u>Class</u>	Name	<u>Abundance</u>	<u>Curies</u>
A11	Co-60	48.37%	356
	Fe-55	30.70%	226
	Cs-137	4.65%	34.2
	Mn-54	7.52%	55.32
	Cr-51	3.38%	24.85
	Co-58	2.19%	16.1
	Zn-65	1.22%	8.97
	Cs-134	1.33%	9.78
	Ni-63	0.44%	3.22
	Sr-90	0.08%	.0573
	C-14	0.07%	.0538
	Pu-241	0.03%	.228
	Ni-59	0.01%	0.0916
	H-3	0.01%	0.0967
	Cm-242	0	0.0012
	Tc-99	0	LLD(1.75E-4 uCi/CC)
	I-129	0	LLD(2.6 E-4 uCi/CC)
	Nb-94	0	7.85 E-7

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## SOLID WASTE DISPOSITION SUMMARY

NUMBER OF SHIPMENTS	MODE OF TRANSPORTATION	DESTINATION
24	Truck	Barnwell, SC
9	Truck	Oak Ridge, TN
3	Truck	Wampum, PA
0	Truck	Richland, WA
0	Truck	Beatty, NV

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### Solid Waste Shipped Offsite for Disposal During Period 01/01/93 to 06/30/93

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Report Date 8/19/93

## WASTE STREAM: RESINS, FILTERS\_SLUDGE

Waste	Volume		Curies	% Error (Ci)
Class	Ft <sup>3</sup>	M <sup>3</sup>	Shipped	
Α	2737.0	77.46	2.52 E+2	± 25%
В	873.5	24.72	4.60 E+2	± 25%
С	0	0	0	± 25%
A11	3610.5	102.18	7.12 E+2	± 25%

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#### WASTE STREAM: RESINS, FILTERS SLUDGE

Waste	Nuclide	Percent	<u>Curies</u>
<u>Class</u>	<u>Name</u>	<u>Abundance</u>	
В	Co-60 Fe-55 Mn-54 Cr-51 Cs-137 Co-58 Zn-65 Cs-134 Ni-63 Sr-90 C-14 Pu-241 Ni-59 H-3 Cm-242 Tc-99 I-129	50.86% 24.03% 10.06% 4.06% 3.88% 2.53% 1.92% 1.54% 0.37% 0.07% 0.06% 0.006% 0.02% 0.006% 0 0	2.37 E+2 1.12 E+2 4.69 E+1 1.89 E+1 1.81 E+1 1.18 E+1 8.97 E0 7.16 E0 1.73 E0 3.35 E-1 2.66 E-1 9.08 E-2 2.98 E-2 1.1 E-3 5.95 E-4 LLD(1.73E-4 uCi/CC) LLD(2.6E-4 uCi/CC)

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### WASTE STREAM: IRRADIATED COMPONENTS

Waste <u>Class</u>	Nuclide <u>Name</u>	Percent <u>Abundance</u>	<u>Curies</u>
Α	Fe-55	38.9	3.24
	Co-60	32.3	2.69
	Cr-51	24.5	2.04
	Ni-63	4.3	.358
	C-14	0	2.92 E-4
	Ni-59	0	2.64 E-3
	Tc-99	0	1.19 E-8
	I-129	N.P.	N.P.
	Pu-241	0	1.85 E-3
	Cm-242	0	2.94 E-7
	H-3	N.P.	N.P.
	Sr-90	0	3.07 E-5
	Cs-137	0	2.37 E-3
	Nb-94	Ō	7.85 E-7

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Solid Waste Shipped Offsite for Disposal During Period From 01/1/93 to 06/30/93

Report Date 8/19/93

## WASTE STREAM: SUM OF ALL CATEGORIES

WASTE	VOLUME			% ERROR (Ci)
	Ft <sup>3</sup>	M <sup>3</sup>	5.111120	
A	20885.4	591.06	275	± 25%
В	873.5	24.72	460	± 25%
С	0	0	0	± 25%
A11	21758.9	615.78	735	± 25%