

Tennessee Valley Authority, 1101 Market Street, Chattanooga, Tennessee 37402

August 29, 2013

10 CFR 50.4 10 CFR 50.90

ATTN: Document Control Desk U.S. Nuclear Regulatory Commission Washington, D.C. 20555-0001

Browns Ferry Nuclear Plant, Units 1, 2, and 3
Renewed Facility Operating License Nos. DPR-33, DPR-52, and DPR-68 NRC
Docket Nos. 50-259, 50-260, and 50-296

Subject: License Amendment Request Marked-up Page Corrections

References: 1. Letter from TVA to NRC, "License Amendment Request to Change Technical Specifications to Delete References to Section XI of the ASME Code and Incorporate References to the ASME OM Code and Allow Application of 25 Percent Extension of Surveillance Intervals to

Accelerated Frequencies Utilized in the IST Program - TS-475," dated

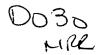
August 28, 2012 [ML12242A477]

 Technical Specifications Task Force (TSTF)-479-A, Revision 0, "Changes to Reflect Revision to 10 CFR 50.55a," dated December 19, 2005

 TSTF-497-A, Revision 0, "Limit Inservice Testing Program SR 3.0.2 Application to Frequencies of 2 Years or Less," dated August 28, 2008

In Reference 1, Tennessee Valley Authority (TVA) submitted a License Amendment Request (LAR) to modify Facility Operating Licenses DPR-33 for Browns Ferry Nuclear Plant (BFN), Unit 1; DPR-52 for BFN, Unit 2; and DPR-68 for BFN, Unit 3.

During review of the LAR, the NRC identified a minor typographical error on Technical Specification Page 5.0-12 for each of the three mark-ups provided for BFN, Units 1, 2, and 3, i.e., in the last line of 5.5.6.a, the word "are" is to be deleted. In a subsequent telephone conversation with the BFN NRC Project Manager, Ms. Farideh Saba, TVA agreed to submit the corrected pages to replace the previous pages (Page 5.0-12 for BFN, Units 1, 2, and 3) in Reference 1. The enclosures to this letter provide the three corrected pages.



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TVA has determined that correcting the typographical error does not affect the previous conclusions regarding the no significant hazards considerations and the categorical exclusion from environmental review pursuant to the provision of 10 CFR 51.22(c)(9) for the LAR submitted in Reference 1.

There are no new regulatory commitments in this letter. If you have any questions regarding these corrections, please contact Edward D. Schrull at (423) 751-3850.

I declare under penalty of perjury that the foregoing is true and correct. Executed on this 29th day of August 2013.

Respectfully,

./W. Shea

Vice President, Nuclear Licensing

Enclosures:

- 1. Corrected Marked-up Browns Ferry Nuclear Plant Technical Specification Pages 5.0-12 for Units 1, 2, and 3
- 2. Corrected Retyped Browns Ferry Nuclear Plant Technical Specification Pages 5.0-12 for Units 1, 2, and 3.

cc (Enclosures):

NRC Regional Administrator - Region II NRC Senior Resident Inspector - Browns Ferry Nuclear Plant

- Limitations on the annual and quarterly air closes resulting from noble gases released in gaseous effluents from each unit to areas beyond the site boundary, conforming to 10 CFR 50, Appendix I;
- Limitations on the annual and quarterly doses to a member of the public from iodine-131, iodine-133, tritium, and all radionuclides in particulate form with half lives > 8 days in gaseous effluents released from each unit to areas beyond the site boundary, conforming to 10 CFR 50, Appendix I; and
- j. Limitations on the annual dose or dose commitment to any member of the public beyond the site boundary due to releases of radioactivity and to radiation from uranium fuel cycle sources, conforming to 40 CFR 190.
- k. The provisions of SR 3.0.2 and SR 3.0.3 are applicable to the Radioactive Effluent Controls Program surveillance frequency.

5.5.5 Component Cyclic or Transient Limit

This program provides controls to track the FSAR Section 4.2.5, cyclic and transient occurrences to ensure that components are maintained within the design limits.

5.5.6 Inservice Testing Program

This program provides controls for inservice testing of ASME Code Class 1, 2, and 3 components. The program shall include the following:

a. Testing frequencies applicable to the ASME Code for Operations and Maintenance of Nuclear Power Plants (ASME OM Code) specified in Section XI of the ASME Boiler and Pressure Vessel Code and applicable Addenda are as follows:

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5.5 Programs and Manuals

5.5.4 Radioactive Effluent Controls Program (continued)

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Enclosure 2

Corrected Retyped Browns Ferry Nuclear Plant Technical Specification Pages 5.0-12 for Units 1, 2, and 3

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