



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

November 27, 2013

Mr. Thomas Joyce
President and Chief Nuclear Officer
PSEG Nuclear
P.O. Box 236, N09
Hancocks Bridge, NJ 08038

SUBJECT: SALEM NUCLEAR GENERATING STATION, UNIT NOS. 1 AND 2 - ISSUANCE OF AMENDMENTS REGARDING CHANGE TO EMERGENCY PLAN TO REMOVE THE BACKUP R45 PLANT VENT RADIATION MONITOR INDICATIONS (TAC NOS. MF0337 AND MF0338)

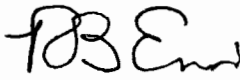
Dear Mr. Joyce:

The Commission has issued the enclosed Amendment Nos. 305 and 287 to Renewed Facility Operating License Nos. DPR-70 and DPR-75 for the Salem Nuclear Generating Station, Unit Nos. 1 and 2. These amendments approve a change to the Emergency Plan (EP) in response to your application dated November 30, 2012, as supplemented by letter dated May 31, 2013.

The amendment approves a change to the site EP to remove the backup plant vent extended range noble gas radiation monitoring (R45) indication, recording, and alarm capability in the emergency response facilities. The accident sampling function of the R45 monitoring indicator will be maintained.

A copy of our safety evaluation is also enclosed. Notice of Issuance will be included in the Commission's biweekly *Federal Register* notice.

Sincerely,

for 

John D. Hughey, Project Manager
Plant Licensing Branch I-2
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket Nos. 50-272 and 50-311

Enclosures:

1. Amendment No. 305 to Renewed DPR-70
2. Amendment No. 287 to Renewed DPR-75
3. Safety Evaluation

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

PSEG NUCLEAR, LLC

EXELON GENERATION COMPANY, LLC

DOCKET NO. 50-272

SALEM NUCLEAR GENERATING STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

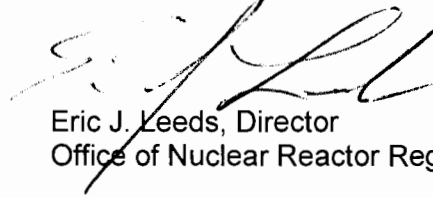
Amendment No. 305
Renewed License No. DPR-70

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment filed by PSEG Nuclear LLC, acting on behalf of itself and Exelon Generation Company, LLC (the licensees) dated November 30, 2012, as supplemented by letter dated May 31, 2013, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in Title 10 of the *Code of Federal Regulations* (10 CFR), Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

Enclosure 1

2. Accordingly, by Amendment No. 305, Renewed Facility Operating License No. DPR-70 is hereby amended to authorize revision to the Salem Nuclear Generating Station, Unit No. 1, Emergency Plan as set forth in PSEG Nuclear LLC's application dated November 30, 2012, as supplemented by letter dated May 31, 2013, and evaluated in the NRC staff's safety evaluation dated November 27, 2013. The license amendment is effective as of its date of issuance and shall be implemented within 60 days.

FOR THE NUCLEAR REGULATORY COMMISSION

A handwritten signature in black ink, appearing to read "Eric J. Leeds", is written over the typed name and title.

Eric J. Leeds, Director
Office of Nuclear Reactor Regulation

Attachment: Change to the License

Date of Issuance: November 27, 2013

ATTACHMENT TO LICENSE AMENDMENT NO. 305
RENEWED FACILITY OPERATING LICENSE NO. DPR-70
DOCKET NO. 50-272

Replace the following page of Renewed Facility Operating License No. DPR-70 with the attached revised page as indicated. The revised page is identified by amendment number and contains a marginal line indicating the area of change.

Remove
Page 3

Insert
Page 3

instrumentation and radiation monitoring equipment calibration, and as fission detectors in amounts as required;

- (5) PSEG Nuclear LLC, pursuant to the Act and 10 CFR Parts 30, 40 and 70, to receive, possess and use in amounts as required any byproduct, source or special nuclear material without restriction to chemical or physical form, for sample analysis or instrument calibration or associated with radioactive apparatus or components; and
- (6) PSEG Nuclear LLC, pursuant to the Act and 10 CFR Parts 30 and 70, to possess but not separate, such byproduct and special nuclear materials as may be produced by the operation of the facility.

C. This renewed license shall be deemed to contain and is subject to the conditions specified in the following Commission regulations in 10 CFR Chapter I: Part 20, Section 30.34 of Part 30, Section 40.41 of Part 40, Sections 50.54 and 50.59 of Part 50, and Section 70.32 of Part 70; and is subject to all applicable provisions of the Act and to the rules, regulations, and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified or incorporated below:

(1) Maximum Power Level

PSEG Nuclear LLC is authorized to operate the facility at a steady state reactor core power level not in excess of 3459 megawatts (one hundred percent of rated core power).

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 305, and the Environmental Protection Plan contained in Appendix B, are hereby incorporated in the renewed license. PSEG Nuclear LLC shall operate the facility in accordance with the Technical Specifications, and the Environmental Protection Plan.

(3) Deleted Per Amendment 22, 11-20-79

(4) Less than Four Loop Operation

PSEG Nuclear LLC shall not operate the reactor at power levels above P-7 (as defined in Table 3.3-1 of Specification 3.3.1.1 of Appendix A to this renewed license) with less than four (4) reactor coolant loops in operation until safety analyses for less than four loop operation have been submitted by the licensees and approval for less than four loop operation at power levels above P-7 has been granted by the Commission by Amendment of this renewed license.

(5) PSEG Nuclear LLC shall implement and maintain in effect all provisions of the approved fire protection program as described in the Updated Final Safety



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PSEG NUCLEAR, LLC

EXELON GENERATION COMPANY, LLC

DOCKET NO. 50-311

SALEM NUCLEAR GENERATING STATION, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 287
Renewed License No. DPR-75

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment filed by PSEG Nuclear LLC, acting on behalf of itself and Exelon Generation Company, LLC (the licensees) dated November 30, 2012, as supplemented by letter dated May 31, 2013, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in Title 10 of the *Code of Federal Regulations* (10 CFR), Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, by Amendment No. 287, Renewed Facility Operating License No. DPR-75 is hereby amended to authorize revision to the Salem Nuclear Generating Station, Unit No. 2, Emergency Plan as set forth in PSEG Nuclear LLC's application dated November 30, 2012, as supplemented by letter dated May 31, 2013, and evaluated in the NRC staff's safety evaluation dated November 27, 2013. The license amendment is effective as of its date of issuance and shall be implemented within 60 days.

FOR THE NUCLEAR REGULATORY COMMISSION



Eric J. Leeds, Director
Office of Nuclear Reactor Regulation

Attachment: Change to the License

Date of Issuance: November 27, 2013

ATTACHMENT TO LICENSE AMENDMENT NO. 287

RENEWED FACILITY OPERATING LICENSE NO. DPR-75

DOCKET NO. 50-311

Replace the following page of Renewed Facility Operating License No. DPR-75 with the attached revised page as indicated. The revised page is identified by amendment number and contains a marginal line indicating the area of change.

Remove
Page 3

Insert
Page 3

- (4) PSEG Nuclear LLC, pursuant to the Act and 10 CFR Parts 30, 40 and 70, to receive, possess and use at any time any byproduct, source or special nuclear material as sealed neutron sources for reactor startup, sealed sources for reactor instrumentation and radiation monitoring equipment calibration and as fission detectors in amounts as required;
 - (5) PSEG Nuclear LLC, pursuant to the Act and 10 CFR Parts 30, 40 and 70, to receive, possess and use in amounts as required any byproduct, source or special nuclear material without restriction to chemical or physical form, for sample analysis or instrument calibration or associated with radioactive apparatus or components; and
 - (6) PSEG Nuclear LLC, pursuant to the Act and 10 CFR Parts 30, 40 and 70, to possess but not separate, such byproduct and special nuclear materials as may be produced by the operation of the facility.
- C. This renewed license shall be deemed to contain and is subject to the conditions specified in the Commission's regulations set forth in 10 CFR Chapter I and is subject to all applicable provisions of the Act and to the rules, regulations and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified or incorporated below:
- (1) Maximum Power Level

PSEG Nuclear LLC is authorized to operate the facility at steady state reactor core power levels not in excess of 3459 megawatts (thermal).
 - (2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 287, and the Environmental Protection Plan contained in Appendix B, are hereby incorporated in the renewed license. PSEG Nuclear LLC shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NOS. 305 AND 287 TO

RENEWED FACILITY OPERATING LICENSE NOS. DPR-70 AND DPR-75

PSEG NUCLEAR, LLC

EXELON GENERATION COMPANY, LLC

SALEM NUCLEAR GENERATING STATION, UNIT NOS. 1 AND 2

DOCKET NOS. 50-272 AND 50-311

1.0 INTRODUCTION

By letter dated November 30, 2012 (Reference 1), PSEG Nuclear LLC (the licensee) submitted a license amendment application requesting U.S. Nuclear Regulatory Commission (NRC) approval for a change to the PSEG Nuclear, LLC Emergency Plan (EP) for the Salem Nuclear Generating Station, Unit Nos. 1 and 2 (Salem). The NRC staff requested additional information in a letter dated April 25, 2013 (Reference 2), and the licensee supplemented its response by letter dated May 31, 2013 (Reference 3). The proposed change removes references in the licensee's EP to the backup plant vent extended range noble gas radiation monitor R45 indication, recording, and alarm capabilities in the licensee's emergency response facilities (ERFs). The R45 monitor at each reactor unit incorporates four radiation monitor channels, all of which will be referred to collectively as R45:

- R45A–Plant Vent Noble Gas Background Subtract,
- R45B–Plant Vent Noble Gas Medium Range,
- R45C–Plant Vent Noble Gas High Range, and
- R45D–Plant Vent Filter Monitor.

The plant design originally included a low range noble gas monitor designated as R41. The licensee subsequently installed R45 to provide extended range noble gas radiation monitoring for the plant vent stack to supplement the monitoring provided by R41. The R45 monitor skid also provides a means to obtain a post-accident sample of the plant vent effluent for offline analysis. These capabilities are consistent with the guidance in Regulatory Guide (RG) 1.97, Revision 2 (Reference 4), which was issued in response to the lessons-learned from the 1979 Three Mile Island accident.

Subsequently, the licensee replaced the original R41 monitor with a wide-range noble gas monitor having the following three channels that together covered the range specified in RG 1.97, as well as that of R45 and the original R41:

- R41A–Plant Vent Low Range Noble Gas,
- R41B–Plant Vent Medium Range Noble Gas, and
- R41C–Plant Vent High Range Noble Gas.

The licensee stated in its supplement to the license amendment application (Reference 3) that the upgraded R41 monitor provided a single alarm for an overall noble gas release rate instead of individual channel alarms that were provided by R45 Channels B and C.

After installation of the upgraded R41 monitor, the R45 monitor has been maintained as a backup for the medium and high range channels of the R41 monitor. The licensee stated in its application that R45 and its supporting equipment have become obsolete and unreliable. The licensee proposes to: (1) remove the current references to R45 from the licensee's EP; and (2) remove the indication, monitoring and alarm capabilities for R45 from the ERFs. These proposed changes would enable the licensee to retire the R45 monitor, since it would no longer be relied upon in the EP. The licensee further stated that it would continue to retain a capability to perform manual sampling and analyses that the R45 skid currently provides, because this capability is not provided by R41.

The licensee determined that the proposed change constituted a potential reduction in effectiveness of the EP because obtaining the results by manual sampling and analysis, when R41 was unavailable, would take longer than referring to the redundant R45 indication, as was done in the past. Accordingly, the licensee requested prior NRC approval for the EP change as required by Title 10, of the *Code of Federal Regulations* (10 CFR), Part 50, Paragraph 50.54(q)(4). This safety evaluation documents the NRC staff's evaluation of the proposed changes to the licensee's EP and provides the requested approval.

The staff's evaluation of the proposed change to the licensee's EP was limited to the impact on the NRC's previous finding that there was reasonable assurance that adequate protective measures can and will be taken in the event of a radiological emergency at Salem. This safety evaluation makes no determination with regard to 10 CFR 50.59 or any other regulatory change process that may apply to the physical retirement of R45.

The supplemental letter dated May 14, 2013, provided additional information that clarified the application, did not expand the scope of the application as originally noticed, and did not change the NRC staff's original proposed no significant hazards consideration determination as published in the *Federal Register* on May 14, 2013 (78 FR 28252).

2.0 REGULATORY EVALUATION

Once a nuclear power plant license is issued, the licensee is required by 10 CFR 50.54(q)(2) to follow and maintain the effectiveness of an EP that meets the requirements in Appendix E to 10 CFR Part 50, and the planning standards of 10 CFR 50.47(b). The following planning standards and requirements were considered in this review:

- 10 CFR 50.47(b)(4) requires, in part, that a licensee's EP contain a standard emergency classification and action level scheme, the bases of which include facility system and effluent parameters;
- 10 CFR 50.47(b)(8) requires that a licensee's EP provide and maintain adequate emergency facilities and equipment to support the emergency response;
- 10 CFR 50.47(b)(9) requires that a licensee's EP provide and use adequate methods, systems, and equipment for assessing and monitoring actual or potential offsite consequences of a radiological emergency;
- Section IV.B.1 of Appendix E to 10 CFR Part 50 requires, in part, that the means for determining the magnitude of and for continually assessing the impact of, the release of radioactive materials be described, and that emergency action levels be based on in-plant conditions and instrumentation, in addition to onsite and offsite monitoring; and
- Section IV.E.2 of Appendix E to 10 CFR Part 50 requires, in part, that adequate provisions be made for equipment for determining the magnitude of, and continuously assessing the impact of, a release of radioactive material to the environment.

The NRC staff review was also informed by the following regulatory guidance:

- (1) NUREG-0654/FEMA-REP-1, "Criteria for Preparation and Evaluation of Radiological Emergency Response Plans and Preparedness in Support of Nuclear Power Plants" (Reference 5). This guidance was issued as an acceptable method for implementing the agency's emergency planning regulations. The following sections were considered for this review:
 - II.D Emergency Classification System,
 - II.H Emergency Facilities and Equipment, and
 - II.I Accident Assessment.
- (2) RG 1.97, "Instrumentation for Light-Water-Cooled Nuclear Power Plants to Assess Plant and Environs Conditions During and Following an Accident," Revision 2. (Reference 4).

3.0 TECHNICAL EVALUATION

3.1 Proposed Change

This license amendment application is associated with a request for NRC approval of a proposed change to the licensee's EP. The licensee proposes to remove the R45 indication, recording, and alarm capabilities from ERFs and to remove references to R45 from the EP and from affected implementing procedures. Following this change, R45 will no longer be available as an installed backup to R41, which currently serves as the primary capability for indication,

recording, and alarms for radioactive material releases to the environment via the plant vent. To enable this removal, the licensee proposes to change its EP to delete the following:

- References to "R45B/C" in Figure 6-5, and
- References to "R45A, B, C, D" for both Unit Nos. 1 and 2 in Table 10-1.

The licensee also stated that although the R45B/C monitor equipment skid will be removed, the licensee would maintain a capability in its EP to take post-accident samples from the plant vent stack, as specified by an earlier commitment to RG 1.97.

3.2 NRC Staff Evaluation

The NRC staff reviewed the licensing basis for the R41 and R45 radiation monitors by researching the correspondence between the licensee and the NRC and determined that the licensee had committed to RG 1.97 in its response to Generic Letter 82-33 (Reference 6). The Commission confirmed the licensee's commitments in a Confirming Order issued on June 12, 1984 (Reference 7). Accordingly, the staff has used RG 1.97 to perform this review.

The NRC staff evaluated the performance characteristics of the R41 and R45 channels, for both Salem Unit Nos. 1 and 2, against the applicable guidance of RG 1.97, as well as Salem Technical Specification 3/4.3.3, "Radiation Monitoring Instrumentation," and Amendments 295 (Unit 1) and 278 (Unit 2), issued on June 15, 2010 (Reference 8). As shown in the table below, the NRC staff determined that the overall range coverage of the three channels of R41 encompassed the range provided by R45 Channel B and C, the range required by RG 1.97, and the existing Salem Technical Specifications.

Channel	R41A,B,C μCi/cc	R45B,C μCi/cc	RG 1.97 μCi/cc	T/S 3/4.3.3 μCi/cc
Low range	10 ⁻⁷ to 10 ⁻¹		10 ⁻⁶ to 10 ³	
Medium range	10 ⁻⁴ to 10 ²	1.5 x 10 ⁻⁴ to 10 ³		10 ⁻³ to 10 ¹
High range	10 ⁻¹ to 10 ⁵	10 ⁰ to 10 ⁵		10 ⁻¹ to 10 ⁵

As described above, there are four channels in the R45 monitor: A, B, C, and D. Channels R45B and R45C correspond to the medium and high range channels of R41. R41 has a combined range that meets the licensee's commitment to RG 1.97. Channel R45A is a background channel, which would be used in conjunction with the indications of the other R45 channels. Channel R45D monitors the radiation level on the plant vent filter. With regard to channel R45D, the licensee's commitment to RG 1.97 was to have a capability to sample and analyze particulates and halogens; real-time monitoring was not required. Neither channel R45A or R45D is required under the licensee's commitment to RG 1.97.

Based on the above, the NRC staff finds that the removal of references to R45A, B, C and D will not reduce the licensee's ability to assess the magnitude of radioactive effluent releases during an emergency.

However, the proposed change does remove a redundant monitoring capability, although the licensee stated in the license amendment request, that the R45 indicators have become obsolete and unreliable. The NRC staff considered the impact of the reduction of the current monitor redundancy on the licensee's ability to respond to an emergency. The NRC has not required redundancy in emergency response facilities, systems, and equipment, except where deemed necessary for reasonable assurance that adequate protective measures can and will be taken in the event of a radiological emergency. Neither the planning standards of 10 CFR 50.47(b) nor 10 CFR Part 50, Appendix E establishes an explicit requirement that the monitoring capability be redundant. Similarly, the guidance in RG 1.97 does not provide for the plant vent stack noble gas monitor to be redundant. Although adequate emergency preparedness is essential, the EP comes into play only in the rare circumstances that engineered design features (including redundancy and diversity) and human capacity to take corrective actions have failed to avert a serious incident. As noted in its application, the licensee estimated that it would take about 90 minutes to obtain and analyze a sample from the time that a decision was made to obtain the sample. The NRC staff determined that a release with a magnitude warranting offsite protective actions is unlikely to occur in the absence of a degradation of the reactor coolant system, fuel, and containment fission product barriers, all of which are discoverable by other plant indications. Based on the above, the NRC staff finds that the reduction in redundancy resulting from the removal of references to R45 from the EP does not preclude timely and adequate protective measures for the public in the event that R41 is unavailable during a radiological emergency.

The R45 monitoring system currently provides the capability to obtain a sample from the plant vent stack for analysis for particulates and halogens, as identified in RG 1.97. The licensee stated in the application that, although the R45 monitor will be removed, the licensee would maintain the capability to obtain samples from the plant vent stack and to analyze such samples for halogens and particulates. The licensee supplemented its licensee amendment application with information that the sampling and analysis capability will continue to meet the guidance in RG 1.97.

The NRC staff also considered the likelihood that the manual sampling and analysis would be initiated in a timely manner in the absence of the alarm normally provided by the unavailable R41, if that monitoring capability should become unavailable. In the supplement to its license amendment application, the licensee provided the following information in this regard:

- The R41 monitor will trigger a failure alarm on the control room annunciators under certain conditions;
- An alarm response procedure directs the operators to evaluate the failure, enter the technical specification action statement, and implement the Offsite Dose Calculation Manual actions if the monitor is inoperable;
- The R41 failure detection is further enhanced by a once per shift operability check, which would also trigger the technical specification action statement, should the instrument be inoperable;

- Should the monitor become inoperable, the chemistry staff would implement procedure SC.CH-AB.ZZ-1102, including installation of a sample rig. These arrangements are capable of sampling noble gases, as well as tritium; and
- The restoration of the inoperable R41 monitor is prioritized, taking into account the technical specification requirements and the impact on the emergency response function.

The NRC staff determined that although these arrangements are for radioactivity releases comparable to routine releases, the results of this sampling could trigger post-accident sampling of the plant vent, as described in emergency plan implementing procedures. Based on the above, the NRC staff finds that there is reasonable assurance that the retirement of R45 will not preclude timely assessment of effluent releases via the plant vent.

3.3 Summary and Conclusions

The NRC staff has reviewed the licensee's technical basis for the proposed changes identified in Section 3.1 of this safety evaluation. Based on the above evaluation, the NRC staff has determined that the licensee's EP, modified as proposed, will continue to meet the requirements of 10 CFR 50.47(b) and Appendix E to 10 CFR Part 50. The NRC staff has determined that the licensee's EP, as modified, will continue to provide reasonable assurance that the licensee can and will take adequate protective measures in the event of a radiological emergency. Further, the licensee will continue to be compliant with the Confirming Order issued on June 12, 1984 (Reference 7).

4.0 STATE CONSULTATION

In accordance with the Commission's regulations, the New Jersey State official was notified of the proposed issuance of the amendments. The State Official had no comments, as stated in the letter dated January 25, 2013, from the New Jersey Department of Environmental Protection (Reference 9).

5.0 ENVIRONMENTAL CONSIDERATION

The amendments change a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The NRC staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendments involve no significant hazards consideration, and there has been no public comment on such finding (78 FR 28252). Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

6.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) there is reasonable assurance that such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: Stephen F. LaVie, NSIR/DPR

Date: November 27, 2013

References

1. Letter from PSEG Nuclear, LLC, to NRC, "License Amendment Request: Change to PSEG Nuclear LLC Emergency Plan to Remove the Salem Backup R45 Plant Vent Radiation Monitor Indications," dated November 30, 2012. (Agencywide Documents Access and Management System (ADAMS) Accession No. ML123380450).
2. Letter from NRC to PSEG Nuclear, LLC, "Salem Generating Station, Units 1 and 2 - Request for Additional Information RE: Change to PSEG Emergency Plan (TAC Nos. MF0337 and MF0338)," dated April 25, 2013. (ADAMS Accession No. ML13102A275).
3. Letter from PSEG Nuclear, LLC, to NRC, "Response to Request for Information dated April [25], 2013, 'Salem Generating Station, Units 1 and 2 - Request for Additional Information RE: Change to PSEG Emergency Plan (TAC Nos. MF0337 and MF0338),'", dated May 31, 2013. (ADAMS Accession No. ML13154A128).
4. Regulatory Guide 1.97, "Instrumentation for Light-Water-Cooled Nuclear Power Plants to Assess Plant and Environs Conditions During and Following an Accident," Revision 2, December 1980. (ADAMS Accession No. ML060750525).
5. NUREG-0654/FEMA-REP-1, "Criteria for Preparation and Evaluation of Radiological Emergency Response Plans and Preparedness in Support of Nuclear Power Plants," Final Report, September 1988. (ADAMS Accession No. ML083180169).
6. Generic Letter 82-33: Supplement 1 to NUREG-0737, "Clarification of TMI Action Plan Requirements: Requirements for Emergency Response Capability," dated December 17, 1982. (ADAMS Accession No. ML102560009).
7. Letter from NRC to Public Service Electric and Gas Company - "Issuance of Order Confirming Licensee Commitments on Emergency Response Capability," dated June 12, 1984. (ADAMS Accession No. ML011660124).

8. Letter from NRC to PSEG Nuclear, LLC, "Salem Nuclear Generating Station, Unit Nos. 1 and 2, Issuance of Amendments RE: Miscellaneous Administrative and Editorial Changes (TAC Nos. ME2229 and ME2230)," dated June 15, 2010. (ADAMS Accession No. ML101300307).
9. Letter from the New Jersey Department of Environmental Protection to the NRC, dated January 25, 2013. (ADAMS Accession No. ML13043A114).

November 27, 2013

Mr. Thomas Joyce
President and Chief Nuclear Officer
PSEG Nuclear
P.O. Box 236, N09
Hancocks Bridge, NJ 08038

SUBJECT: SALEM NUCLEAR GENERATING STATION, UNIT NOS. 1 AND 2 - ISSUANCE OF AMENDMENTS REGARDING CHANGE TO EMERGENCY PLAN TO REMOVE THE BACKUP R45 PLANT VENT RADIATION MONITOR INDICATIONS (TAC NOS. MF0337 AND MF0338)

Dear Mr. Joyce:

The Commission has issued the enclosed Amendment Nos. 305 and 287 to Renewed Facility Operating License Nos. DPR-70 and DPR-75 for the Salem Nuclear Generating Station, Unit Nos. 1 and 2. These amendments approve a change to the Emergency Plan (EP) in response to your application dated November 30, 2012, as supplemented by letter dated May 31, 2013.

The amendment approves a change to the site EP to remove the backup plant vent extended range noble gas radiation monitoring (R45) indication, recording, and alarm capability in the emergency response facilities. The accident sampling function of the R45 monitoring indicator will be maintained.

A copy of our safety evaluation is also enclosed. Notice of Issuance will be included in the Commission's biweekly *Federal Register* notice.

Sincerely,
/ra/ (REnnis for)
John D. Hughey, Project Manager
Plant Licensing Branch I-2
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket Nos. 50-272 and 50-311

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RidsNsirDprDdir Resource	SLaVie, NSIR	

Accession No.: ML13301A124

*** memo dated July 3, 2013**

OFFICE	LPL1-2/PM	LPL1-2/LA	NSIR/BC*	OGC (NLO)	LPL1-2/BC	DORL/D	NRR/D	LPL1-2/PM
NAME	JHughey	ABaxter	JAnderson	BMizuno	VRodriguez	MEvans	ELeeds	JHughey (REnnis for)
DATE	11/14/2013	10/30/2013	7/03/2013	11/08/2013	11/14/2013	11/21/2013	11/27/2013	11/27/2013

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