



**UNITED STATES
NUCLEAR REGULATORY COMMISSION
REGION II**
245 PEACHTREE CENTER AVENUE NE, SUITE 1200
ATLANTA, GEORGIA 30303-1257

June 21, 2013

Mr. Dennis R. Madison
Vice President
Southern Nuclear Operating Company, Inc.
Edwin I. Hatch Nuclear Plant
11028 Hatch Parkway North
Baxley, GA 31513

**SUBJECT: EDWIN I. HATCH NUCLEAR PLANT - NRC EVALUATION OF CHANGES, TESTS,
AND EXPERIMENTS AND PERMANENT PLANT MODIFICATIONS BASELINE
INSPECTION REPORT 05000321/2013010 AND 05000366/2013010**

Dear Mr. Madison:

On May 23, 2013, the U.S. Nuclear Regulatory Commission (NRC) completed an inspection at your E. I. Hatch Units 1 and 2. The enclosed inspection report documents the inspection results which were discussed on May 23, 2013, with you and other members of your staff.

The inspection examined activities conducted under your licenses as they relate to safety and compliance with the Commission's rules and regulations and with the conditions of your licenses. The team reviewed selected procedures and records, observed activities, and interviewed personnel.

No findings were identified during this inspection.

In accordance with 10 CFR 2.390 of the NRC's "Rules of Practice," a copy of this letter, its enclosure, and your response (if any) will be available electronically for public inspection in the NRC Public Document Room or from the Publicly Available Records (PARS) component of NRC's Agency-wide Document Access and Management System (ADAMS).

ADAMS is accessible from the NRC Web site at <http://www.nrc.gov/reading-rm/adams.html> (the Public Electronic Reading Room).

Sincerely,

RA

Rebecca L. Nease, Chief
Engineering Branch 1
Division of Reactor Safety

Docket Nos.: 50-321, 50-366
License Nos.: DPR-57 and NPF-5

Enclosure:
Inspection Report 05000321/2013010, 05000366/2013010
w/attachment: Supplemental Information

U. S. NUCLEAR REGULATORY COMMISSION

REGION II

Docket Nos.: 50-321, 50-366

License Nos.: DPR-57 and NPF-5

Report Nos.: 05000321/2013010 and 05000366/2013010

Licensee: Southern Nuclear Operating Company, Inc.

Facility: Edwin I. Hatch Nuclear Plant

Location: Baxley, Georgia 31513

Dates: May 6 – May 23, 2013

Inspectors: E. Stamm, Senior Reactor Inspector (Team Leader)
A. Butcavage, Reactor Inspector
M. Schwieg, Resident Inspector (Brunswick)

Approved by: Rebecca L. Nease, Chief
Engineering Branch 1
Division of Reactor Safety

Enclosure

SUMMARY OF FINDINGS

IR 05000321/2013010, 05000366/2013010; 05/06/2013-05/23/2013; Edwin I. Hatch Nuclear Plant, Units 1 and 2; Evaluations of Changes, Tests, and Experiments and Permanent Plant Modifications

This report covers a two-week on-site inspection by one senior reactor inspector, one reactor inspector, and one resident inspector. No findings were identified. The NRC's program for overseeing the safe operations of commercial nuclear power reactors is described in NUREG-1649, "Reactor Oversight Process," Revision 4.

No findings were identified during this inspection.

REPORT DETAILS

1. REACTOR SAFETY

Cornerstones: Initiating Events, Mitigating Systems, and Barrier Integrity

1R17 Evaluations of Changes, Tests, and Experiments and Permanent Plant Modifications (71111.17T)

a. Inspection Scope

Evaluations of Changes, Tests, and Experiments: From May 6, 2013, through May 23, 2013, the team reviewed eight safety evaluations performed pursuant to Title 10, Code of Federal Regulations (CFR) 50.59 to determine if the evaluations were adequate and that prior NRC approval was obtained as appropriate. The team also reviewed 13 screenings where licensee personnel had determined that a 10 CFR 50.59 evaluation was not necessary. The team reviewed these documents to determine if:

- the changes, tests, or experiments performed were evaluated in accordance with 10 CFR 50.59 and that sufficient documentation existed to confirm that a license amendment was not required;
- the safety issues requiring the changes, tests or experiments were resolved;
- the licensee conclusions for evaluations of changes, tests, or experiments were correct and consistent with 10 CFR 50.59; and
- the design and licensing basis documentation used to support the change was updated to reflect the change.

The team used, in part, Nuclear Energy Institute (NEI) 96-07, "Guidelines for 10 CFR 50.59 Implementation," Revision 1, to determine acceptability of the completed evaluations, and screenings. The NEI document was endorsed by the NRC in Regulatory Guide 1.187, "Guidance for Implementation of 10 CFR 50.59, Changes, Tests, and Experiments," dated November 2000. The team also consulted Part 9900 of the NRC Inspection Manual, "10 CFR Guidance for 10 CFR 50.59, Changes, Tests, and Experiments," dated March 2001.

This inspection constituted eight samples of evaluations and 13 samples of screenings as defined in Inspection Procedure (IP) 71111.17-04.

Permanent Plant Modifications: From May 6, 2013, through May 23, 2013, the team reviewed six permanent plant modifications that had been installed in the plant during the last three years. The six modifications reviewed are listed below:

- Design Change Package (DCP) 1071694001, Modification to Isolate Unit 1 Protected Service Water from Turbine Building, Rev. 0
- DCP 1072721101, Reroute Cables out of Fire Area 0014, Ver. 3.0
- DCP 1081976501, Replacement of Upper Shroud Support Assemblies and Tie Rod Nuts, Ver. 2.0
- DCP 1092321101, Coordination Enhancement for 250 VDC Motor Control Center 1R24-S022 Cubicles 4A, 6A, and 9C, Ver. 2.0

- DCP 1101786501, Replace Fission Product Monitor Inboard Isolation Valve 1D11-F050, Ver. 1.0
- DCP 2039000201, 4.16kV Breaker Replacement, Ver. 1.1

The modifications were selected based upon risk significance, safety significance, and complexity. The team reviewed the modifications selected to determine if:

- the supporting design and licensing basis documentation was updated;
- the changes were in accordance with the specified design requirements;
- the procedures and training plans affected by the modification had been adequately updated;
- the test documentation as required by the applicable test programs had been updated; and
- post-modification testing adequately verified system operability and/or functionality.

The team also used applicable industry standards to evaluate acceptability of the modifications and performed walk downs of accessible portions of the modifications. Documents reviewed are listed in the Attachment.

This inspection constituted six permanent plant modification samples as defined in IP 71111.17-04.

b. Findings

No findings were identified.

4OA6 Meetings, Including Exit

On May 23, 2013, the team presented the inspection results to Mr. Dennis Madison and other members of the licensee's staff. The team verified that no proprietary information was retained by the inspectors or documented in this report.

ATTACHMENT: SUPPLEMENTAL INFORMATION

SUPPLEMENTAL INFORMATION

KEY POINTS OF CONTACT

Licensee personnel

B. Anderson, Health Physics Manager
S. Beverly, Licensing Engineer
T. Crowell, Site Design Engineer
E. DelRosario, Site Design Engineer
D. Hines, Site Design Supervisor
B. Hulett, Site Design Manager
C. Lane, Engineering Director
K. Long, Operations Director
D. Madison, Vice President
C. McCall, Project Engineer
M. Peacock, Designer
C. Robinson, ASD Engineer
R. Seiter, Engineering Supervisor
B. Shuman, Engineering Programs Manager
S. Tipps, Licensing Supervisor
D. Vineyard, Plant Manager
T. Wells, Principal Engineer
E. Wonn, Senior Designer
F. Zayas, Site Design Supervisor

NRC personnel

E. Morris, Senior Resident Inspector, Hatch
D. Hardage, Resident Inspector, Hatch

LIST OF ITEMS OPENED AND CLOSED

Opened and Closed

None

LIST OF DOCUMENTS REVIEWED

10 CFR 50.59 Evaluations

Licensing Document Change Request (LDCR) 2004-022, TRM/FSAR Revision to Increase RCIC Inboard Isolation Closure Time Acceptance Criteria, Ver. 1.0
Action Item (AI) 2008200410, RCIC Accident Temperature and Pressure, dated 2/14/2008
DCP 1040048401, Replacement of Reactor Recirculation Pump Motor Generator Sets with Adjustable Speed Drives, Ver. 9.0
DCP C062145201, Westinghouse SVEA-96 Optima2 Lead Use Assemblies, Ver. 1.0
Technical Evaluation (TE) 320565, LDCR for Detector Not Full in Rod Block Function, Rev. 0
AI 2007205079/RER C072813201, Movement of Heavy Loads Over Spent Fuel Pool, dated 1/9/08
Minor Design Change 1101735601, HPCI Turbine Exhaust Drain Line Removal, Ver. 1.0
DCP SNC119724, Evaluate 97F River Temperature, dated 12/15/11

10 CFR 50.59 Screenings

DCP 2072720601, Unit 2 FPCC System Heat Exchanger Replacement, Ver. 5.0
DCP 2080067101, Main Steam S/RV Replacement, Ver. 7.0
Equivalency Determination (ED) C110573201, Valve Check 14", 300 lb. Crane Swing Check Valve for RHR Service Water, Ver. 2.0
DCP SNC116958, Remove Unit 2 HPCI Turbine Exhaust Drain Line, Ver. 1.0
ED SNC340426, Relay K3 for EDG Exciter, Ver. 1.0
ED SNC341510, Replacement Parts for General Electric Type AK-25 Circuit Breaker, Ver. 1.0
ED SNC345576, AVR Assembly for EDG Exciter, Ver. 1.0
DCP SNC429939, Unit 2 Modified 3-stage SRV Replacement, Ver. 1.0
ED SNC452392, Hydraulic Actuator for MSIVs, Ver. 1.0
DCP 1042453001, Replacement Obsolete Recorders in Panel 1H11-P614, Ver. 4.0
DCP 1072724101, Replace Modified 36-05 and 44-13 Local Power Range Monitors (LPRMs) with standard LPRMs, Ver. 2.0
DCP 1081709901, CRD Pump Room Cooler Cooling Coil Replacement, Ver. 2.0
DCP 1081750401, Data Acquisition and Analysis System (DAAS) Replacement, Ver. 3.0

Calculations

BH1-M-V004-B007-0001, Compartment Long-Term Temperature Analysis, Ver. 10.0
BH2-M-0295, Compartment Long-Term Temperature Analysis, Ver. 9.0
CA-1272, Aerofin Coil Selection Program Verification & Validation Documentation, Rev. 1
CA-1481, Aerofin Performance Verification for Aerofin Type "W" Coils, Rev. 1
CA-529, Aerofin Calculation File, CRD Pump Room Cooling Coil, Ver. 2
MC-H-09-019, Southern design Calculation, Cooling Unit Support, Ver. 0
S-53596-A, Shroud Repair Seismic Design Report Unit 1, Rev. 1
S-58367, E. I. Hatch Nuclear Plant – Unit No. 1, Shroud Repair replacement Upper Support Stress Analysis Report, Ver. 2.0

Condition Reports

CAR196955
CR2002004824
CR2008107088
CR558872

Drawings

10-502, Reactor Recirculation System Logic Diagrams Sheet 1, Ver. 7.0
 10-502, Reactor Recirculation System Logic Diagrams Sheet 2, Ver. 1.0
 10-502, Reactor Recirculation System Logic Diagrams Sheet 3, Ver. 25.0
 10-502, Logic Diagrams Legend and General Notes
 H-10167, Edwin I. Hatch Nuclear Plant, Unit 1 & 2 Refueling Floor Heavy Load Paths, Ver. 11.0
 H-10171, Edwin I. Hatch Nuclear Plant, Unit 1 & 2 Refueling Floor Heavy Load Matrix, Weights and lifts, Ver. 5.0
 H-11600, Unit 1 P&ID for Service Water and Diesel Generator, Sheet 2, Rev. 36
 H-13388, Unit 1 Plant service water MO valves system diagram, Sheet 2, Rev. 4
 H-13388, Unit 1 Plant service water MO valves system diagram, Sheet 3, Rev. 5
 H-16173, Fission Products Monitoring System P&ID, Sheet 1, Ver. 14
 H-26051, Edwin I. Hatch Nuclear Plant Unit 2, Reactor Building – Plant Service Water System P&ID, Ver. 47
 HB-11004, Edwin I. Hatch Nuclear Plant Unit No. 1, RHR Service Water Outside Building ISI Boundary Diagram, Ver. 5.0
 HB-16173, Fission Products Monitoring System ISI Boundary Diagram, Ver. 2
 HB-21039, Edwin I. Hatch Nuclear Plant Unit No. 2, RHR Service Water Outside Building ISI Boundary Diagram, Ver. 11.0
 HL-16173, Fission Products Monitoring System P&ID, Sheet 1, Ver. 3
 S-15484, Reactor Shroud, Rev. 10
 S-19638, Edwin I. Hatch Nuclear Plant Unit 1, Cooling Units Design Data, Ver. 2.0
 S-28751, Edwin I. Hatch Nuclear Plant No. 2, Cast Steel Swing Check Valve, Ver. 2
 S-53593, Shroud Repair Modifications & Installation – Reactor, Sh-2, Rev. 1
 SA-A018, 20x5 MSIV Actuator, Sheets 1 thru 6 of 6, Rev. 7

Miscellaneous Documents

0000-0154-9209, SNC SRV Specification HM-S-09-001 Review, Ver. 0
 30035-99 Rev A, Wyle Laboratories Test Report, Volumes I thru VI
 34SO-B31-001, Reactor Recirculation System, Ver. 43.2
 CCN-H-09-0004, A46 Evaluation of Cable Spreading Room-cable Tray Support Anchorage Against the Base Calculation SCNH-92-034 (Version 4.0), Ver. 1.0
 Code Case N-60-5, Cases of ASME Boiler and Pressure Vessel Code, dated 2/15/94
 Copper Development Association, CDA Publication TN31, Copper-Nickel 90/10 and 70/30 Alloys Technical Data, 1982
 DOEJ-HD1072721101C001, Seismic Evaluations of Cables added to tray TL8-10, Hatch Nuclear Plant U1, Ver. 1.0
 DOEJ-HD1072721101-E002, Adequacy of R23-S004-ES8-C06A Reroute cable out of Fire Area 0014, Ver. 1.0
 DOEJ-HD1081750401-J001, EMI/RFI Evaluation for HPCI/RCIC DAAS Equipment Replacement, Rev. 0
 DOEJ-HD1081750401-M001, Heat Load Evaluation for Data Acquisition and Analysis System (DAAS) Replacement, Ver. 1.0
 DOEJ-HD1081976501-C001, Seismic Review of Impact of DCP H1081976501, Ver. 1.0
 DOEJ-HD2072720601-M001, Evaluation of Replacement Heat Exchanger 2G41-B001 Thermal Performance, Ver. 1.0
 DOEJ-HD2072720601-M001, Evaluation of NRC Regulatory Guide 1.26 Classification for 2G41-B001 Heat Exchanger, Ver. 1.0
 DOEJ-HD2080067101-M003, Evaluation of Platinum coating on Two-Stage Safety Relief Valve Discs, dated 12/08/11
 DOEJ-HRC101439201-M003, Evaluate the Effect of 97°F Plant Service Water to the Units 1

and 2 Emergency Diesel Generator Heat Exchangers, Ver. 1.0
 DOEJ-HRC101439201-M004, Evaluate the Effect of 97°F Plant Service Water to the Units 1 and 2 Plant Service Water and Residual Heat Removal Service Water Pump Motor Coolers, Ver. 1.0
 ED 1081709901, Plateau Item Status, Group Report of 50.59 Qualifications for Originators of the ED
 EE-0380-0029-0806, MPR specification and equivalency of replacement items
 Engineering Change Request (ECR) No 363, dated 10/11/04
 ES-MIS-145, Materials and Inspection Services, 10CFR 50.55a Evaluation, Ver. 1.0
 Evaluation for Reactor Core Shroud Relief Request (TAC No. MD6396), dated 3/26/08
 FCR049, Replacement of Recirculation MG sets with adjustable speed drives, Ver. 6.0
 FCR058, Replacement of Recirculation MG sets with adjustable speed drives, Ver. 6.0
 Fleet-DCM-2013, Nuclear Oversight Audit of Design Changes and Modifications, dated 03/13/13
 Focused Area Self-Assessment - Plant Site Design Process Execution, dated 07/26/12
 Focused Area Self-Assessment - 50.59/Modifications Inspection, Ver. 1
 GEH 0000-0126-6532-R1, Ultimate Heat Sink Temperature Increase to 97F Impact on DBA-LOCA Analysis and DW Equipment Qualification Analysis (S75211), Rev. 1
 H-DCM-2011, Fleet Oversight Audit of Design Changes and Modifications, dated 03/31/11
 Hatch 1& 2, Technical Requirements Manual, Unit 1-Version 91, Unit 2, Ver. 93
 Hatch UFSAR Section 18.2, Overhead Crane and Refueling Platform Inspections, Rev. 31
 Hatch UFSAR, Unit 1, Chapter 14, Safety Analysis, refers to HNP-2-FSAR-15A, Section 15A.5.6.3 RHRSW Line Cracks, Rev. 30
 HNP-1-FSAR-3, Section 3.3, Reactor Internals Mechanical Design, Rev. 19
 HNP-1-FSAR-10, Section 10.18, Equipment Cooling System, Rev. 30
 HNP-1-FSAR-C, Appendix "C", Section C.3, Rev. 19
 HNP-1-FSAR-C, Table C.3-1, Sheet 2 of 24, Rev. 19
 HNP-2-FSAR-4, Section 4.2.2, Reactor Core Support Structures and Internals Mechanical Design, Rev. 29
 LDCR 2004-035, Licensing Document Change Request, Ver. 1.1
 LDCR 2008-020, Licensing Document Change Request, Ver. 1.0
 LDCR 2008-021, Licensing Document Change Request, Ver. 1.0
 LDCR 2008-045, Heavy Loads Movement Conforming FSAR Change, dated 8/14/08
 LDCR 2009-028, Licensing Document Change Request, Ver. 1.0
 Letter to U.S. Nuclear Regulatory Commission, Subject: Edwin I. Hatch Nuclear Plant Unit 1 SVEA-96 Optima2 Lead Use Fuel Assemblies, dated 5/25/10
 MATWEB Properties of AISI 1018 Steel, Cold Drawn, dated 1/20/11
 MATWEB Properties of AISI 1019 Steel, Cold Drawn, dated 1/20/11
 MATWEB Properties of AISI 1021 Steel, Cold Drawn, dated 1/20/11
 MATWEB Properties of AISI 1030 Steel, As Rolled, dated 8/29/12
 MC-H-09-0020, Anchorage Modification, Rev. 0
 MC-H-09-0044, Fuse/fuse Coordination, Rev. 0
 MC-H-10-0142, Appendix R Protective Coordination Study of 250/125V DC Circuits
 NEI 96-07, Guidelines for 10 CFR 50.59 Implementation, Rev. 1
 NEI 98-03, Guidelines for Updating Final safety Analysis reports, Rev. 1
 NL-07-2242, SNC Letter to USNRC, Edwin I. Hatch Nuclear plant Unit No. 1, Response to NRC Request for Additional Information Regarding Modification of the Core Shroud Stabilizer Assemblies, dated 12/18/07
 NL-08-0350, SNC Letter to USNRC, Edwin I. Hatch Nuclear Plant, Core Shroud Stabilizer Assembly Update, dated 3/7/08
 NL-08-0512, SNC Letter to USNRC, Edwin I. Hatch Nuclear Plant

QFRC2012-004, Hatch Equipment Qualification Change Package, Ver. 1.0
 QFRC2012-005, Hatch Equipment Qualification Change Package, Ver. 1.0
 RER C070255901, Hatch Thermal Overload Heater Evaluation
 S-56787, Equipment qualification report of Westinghouse/Cutler-Hammer 50DHP-VR250U/1200 AMP replacement circuit breakers for safety application
 S-57861, DX1000/2000 Seismic Test Report, Ver. 2.0
 S-58360, E.I. Hatch Nuclear Plant- Unit No-1, Shroud Repair Hardware Design Specification, Ver. 1.0
 S-58361, E.I. Hatch Nuclear Plant- Unit No-1, Shroud Repair Hardware Modification Design Specification Data Sheet, Ver. 2.0
 S-58361, E.I. Hatch Nuclear Plant- Unit No-1, Shroud Repair Parts List, Ver. 2.0
 S-58637, Seismic Qualification Report TRP 5932 1" Square Body, 1" Y-Pattern, and 2" Y-Pattern Bolted Bonnet Closure Solenoid Operated Valves, dated 12/08/10
 SCNH-99-443, Seismic Evaluation of 1H21-P050, Rev. 0
 SCNH-10-064, Design Basis Seismic Qualification for MCC 1R24-S022, Ver. 1
 Specification NP-89001, Specification for Replacement Fuel Pool Cooling Heat Exchangers, Ver. 3.0
 TS Bases, B3.4.7, Residual Heat Removal (RHR) Shutdown Cooling System – Hot Shutdown, Rev. 15
 TS Bases, B3.7.1, Residual Heat Removal Service Water (RHRSW) System, Rev. 0
 USNRC Information Notice 96-17, Reactor Operation Inconsistent with the Updated Final Safety Analysis Report, dated 3/18/96
 USNRC NUREG 0612, Control of Heavy Loads, Resolution of Generic Technical Activity A-36
 USNRC Regulatory Guide 1.181, Content of the Updated Final Safety Analysis Report in Accordance with 10 CFR 50.71(e), dated 9/24/99
 USNRC Regulatory Information Summary 2005-25, Supplement 1

Modifications

DCP1071694001, Modification to Isolate Unit 1 Protected Service Water from Turbine Building, Rev. 0
 DCP1072721101, Reroute Cables out of Fire Area 0014, Ver. 3.0
 DCP1081976501, Replacement of Upper Shroud Support Assemblies and Tie Rod Nuts, Ver. 2.0
 DCP1092321101, Coordination Enhancement for 250 VDC Motor Control Center 1R24-S022 Cubicles 4A, 6A, and 9C, Ver. 2.0
 DCP1101786501, Replace Fission Product Monitor Inboard Isolation Valve 1D11-F050, Ver. 1.0
 DCP2039000201, 4.16kV Breaker Replacement, Ver. 1.1

Procedures

34SV-SUV-016-1, Cold Shutdown Valve Operability, Ver. 12.1
 42FH-ERP-001-0, Control Rod Blade Unlatching, Installation, Removal and Exchange, Ver. 8.0
 42IT-TET-014-IS, Safeguard equipment Cooler Data, Revision 0 ED1, dated 1/29/02
 42SP-08-01-09-OW-1-1, Functional Test for DCP 1072721101 – Reroute Cables out of the Fire Area 0014, Ver. 1.1
 42SV-TET-001-0, LLRT Testing Methodology, Ver. 7.3
 42SV-TET-001-1, Primary Containment Type B and Type C Leak Rate Testing, Ver. 24.1
 51GM-MLH-004-0, NUREG 0612 Heavy Load Movement, Rev. 21.9
 DI-ENG-06-0893, Southern Nuclear Plant, E. I. Hatch Design Change Package (DCP) Implementation User's Guide, Ver. 3.0
 NL-006-003, SNC Nuclear Licensing Instruction, Outgoing NRC Correspondence, Ver. 1.0

NL-006-GL04, SNC Nuclear Licensing Guideline, Guidelines for Verification of Submittals, Ver. 1.0
NMP-AD-008, Applicability Determinations, Ver. 15.2
NMP-AD-010, 10 CFR 50.59 Screenings and Evaluations, Ver. 11.0
NMP-AD-012, Operability Determinations and Functionality Assessments, Ver. 12.0
NMP-ES-034, Equivalency Determinations, Ver. 14.3
NMP-ES-034-GL01, Equivalency Determinations, Engineering Work Order Processing, Ver. 2.2
NMP-ES-038-GL02, Electrical Design Guideline, Ver. 1.0

Work Orders

SNC441401
WO1042453001
WO1071694003
WO1072721105
WO1081709910
WO1081709918
WO1081709922
WO2039000201
WOC070255901

Condition Reports generated as a result of the inspection

CR636366, Fire door 1L48-C31 sign, dated 5/8/13
CR636368, Fire door 1L48-C31 magnet, dated 5/8/13
CR637655, NRC 50.59/Modification Inspection request, dated 5/10/13
CR640166, NRC finds minor violation of 10 CFR 50.71(e) for LDCR 2004-22, dated 5/15/13
CR640204, NRC finds minor violation of 10 CFR 50.71(e) for LDCR 2004-22, dated 5/15/13
CR642894, Equivalency Determination C110573201 Lacks Answer, dated 5/21/13
CR643169, TRM not updated after MDC 1101735601, dated 5/21/13
CR643571, NRC Question, HNP-1 FSAR-C Section C.3, Vessel Internals, dated 5/22/13
CR643956, NRC 50.59/Mods Inspection Question, MSIV Documentation, dated 5/22/13
CR644213, NRC 2013 Mods Inspection Question, CRD Cooler Coils, dated 5/23/13