

Tennessee Valley Authority, Post Office Box 2000, Decatur, Alabama 35609-2000

May 1, 2013

10 CFR 50.73

ATTN: Document Control Desk U.S. Nuclear Regulatory Commission Washington, D.C. 20555-0001

> Browns Ferry Nuclear Plant, Unit 2 Facility Operating License No. DPR-52 NRC Docket No. 50-260

Subject: Licensee Event Report 50-260/2013-001-00

The enclosed Licensee Event Report provides details of an inoperable stop-check valve resulting in a condition prohibited by Technical Specifications and leading to a reactor shutdown required by Technical Specifications. The Tennessee Valley Authority is submitting this report in accordance with Title 10 of the Code of Federal Regulations (10 CFR) 50.73(a)(2)(i)(A) and 10 CFR 50.73(a)(2)(i)(B).

There are no new regulatory commitments contained in this letter. Should you have any questions concerning this submittal, please contact J. E. Emens, Jr., Nuclear Site Licensing Manager, at (256) 729-2636.

Respectfully,

K. J. Polson Vice President

Enclosure: Licensee Event Report 50-260/2013-001-00 – Inoperable Stop-Check Valve Results in Condition Prohibited by Technical Specifications and Reactor Shutdown Required by Technical Specifications

cc (w/ Enclosure):

NRC Regional Administrator - Region II NRC Senior Resident Inspector - Browns Ferry Nuclear Plant

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ENCLOSURE

Browns Ferry Nuclear Plant Unit 2

Licensee Event Report 50-260/2013-001-00

Inoperable Stop-Check Valve Results in Condition Prohibited by Technical Specifications and Reactor Shutdown Required by Technical Specifications

See Enclosed

NRC F	ORM 36	6	U.S.	NUCLEA	RR	EGULA	TORY	COMMIS	SION	APPROVED BY OMB NO. 3150-0104 EXPIRES 10/31/2013									
(10-2010	(10-2010) Estimated burden per response to comply with this mandatory collection requess 80 hours. Reported lessons learned are incorporated into the licensing process an fed back to industry. Send comments regarding burden estimate to FOIA/Privac Section (T-5 F53), U.S. Nuclear Regulatory Commission, Washington, DC 20555 0001, or by internet e-mail to infocollects.resource@nrc.gov, and to the Desk Office Office of Information and Regulatory Affairs, NEOB-10202, (3150-0104), Office of Management and Budget, Washington, DC 20503. If a means used to impose a information collection does not display a currently valid OMB control number, th NRC may not conduct or sponsor, and a person is not required to respond to, th information collection.										on request: rocess and DIA/Privacy DC 20555- esk Officer,), Office of impose an umber, the ond to, the								
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Bro	wns Fe	erry Nucl	ear Plar	nt, Unit 2	2							0500	0260				1 of 8	8	
4. TIT	4. TITLE: Inoperable Stop-Check Valve Results in Condition Prohibited by Technical Specifications and Reacto Shutdown Required by Technical Specifications										or								
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L	12. LICENSEE CONTACT FOR THIS LER																		
FACILIT Mark	FACILITY NAME TELEPHONE NUMBER (Include Area Code) Mark Acker, Licensing Engineer 256-729-7533									Code)									
ļ			13. COM	PLETE C	NEL	INE F	DR EAC	H COM	PONE	NT F	AILURE	DESC	RIBED	IN THIS R	EPO	RT	·		
CA	JSE	SYSTEM	сом	PONENT	FA	MANU- CTURER	REP	ORTABLE		CAU	JSE	SYS	STEM	COMPONE	ENT	MANU FACTUR	ER	REF 1	ORTABLE O EPIX
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		14.	SUPPLE	MENTAL	. REF	PORTE	XPECT	ED				15. EXPECTED SUBMISSION MONTH DAY			٩Y	YEAR			
ΠY	YES (If yes, complete			ECTED S	SUBN	<i>I</i> ISSIOI	V DATE) [м 🛛)		DATE		TE		N/A	N/	/ <u>A</u>	N/A
ABSTR	ABSTRACT (Limit to 1400 spaces, i.e., approximately 15 single-spaced typewritten lines) On March 2, 2013, radiography results for the Browns Ferry Nuclear Plant (BFN) Unit 2																		
	Rea	ctor Co	re Isola	tion Co	olin	g (R0	CIC) s	ystem	stop	-ch	eck v	alve,	2-HC\	/-071-0	014	, show	ed tł	ne	
	valv	e to be i	in the fu	ully ope	en p	ositio	n; the	refore	, not	me	eting	its fu	nction	as a pr	ima	ry conta	ainm	nent	
	isola	ation val	ve. Th	e RCIC	Sto	eam I	_ine C	outboa	rd Iso	olat	ion va	alve, 2	2-FCV	-071-00)03,	was cl	ose	d	
	and	deactiv	ated to	meet T	⁻ ecł	nnical	Spec	ificatio	n (T	S) 3	3.6.1.3	3 for I	Primar	y Conta	ainm	nent Isc	latic	on.	
	This	s caused	the R	CIC sys	sten	n to b	ecom	e inop	erabl	e. `	The T	S 3.5	5.3 req	juires th	ne R	CIC sy	sten	n to	
	be r	eturned	to serv	ice with	nin '	14 da	ys. B	ecaus	e val	ve 2	2-HC	V-071	1-0014	is unis	olat	ple to p	rima	iry	
	con	tainmen	t, a uni	t shutdo	own	was	requir	red to	perfo	orm	repai	rs. A	ctions	to initia	ite r	eactor			
	shutdown began on March 14, 2013, at 0800 Central Daylight Time.																		
	The Run	root ca to-Failu	use of t ure han	nis cor d valve	e wh	on is ien it	valve shoule	2-HC\ d have	/-071 bee	n cl)14 w Iassifi	as im ied as	prope a Crit	riy class tical che	sifie eck	a as a valve.			
	The corrective action to prevent recurrence is to submit preventive maintenance change requests to open, inspect, clean, and replace valve disc, dashpot, and stem, as necessary, for RCIC Turbine Exhaust Hand Control Valves, RCIC Vacuum Pump Discharge Shutoff Valves, and High Pressure Coolant Injection Turbine Exhaust Valves.																		

NRC FORM 366A (10-2010) U.S. NUCLEAR REGULATORY COMMISSION

LICENSEE EVENT REPORT (LER)

CON	NTINUATION	SHEET			
FACILITY NAME (1)	DOCKET (2)	L	ER NUMBER (6	PAGE (3)	
	05000260	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	
Browns Ferry Nuclear Plant, Unit 2		2013	001	00	2 of 8

NARRATIVE

I. Plant Operating Condition Before the Event

On March 2, 2013, at 0300 Central Standard Time (CST), Browns Ferry Nuclear Plant (BFN), Unit 2, was in Mode 1 at 88 percent rated thermal power.

II. Description of Events

A. Event:

On March 2, 2013, radiography results for the BFN, Unit 2, Reactor Core Isolation Cooling (RCIC) [BN] system turbine exhaust stop-check valve [HCV], 2-HCV-071-0014, showed the valve to be in the fully open position; therefore, not meeting the acceptance criteria of Surveillance Instruction 2-SI-3.2.3, Testing ASME Section XI Check Valves. This stop-check valve is one of two Primary Containment Isolation Valves (PCIVs) connecting the RCIC system directly to primary containment [NH]. A Past Operability Evaluation (POE) determined that valve 2-HCV-071-0014 should be considered inoperable from March 27, 2011, until March 15, 2013, at 0310 Central Daylight Time (CDT), when BFN, Unit 2, entered Mode 4.

BFN, Unit 2, Technical Specification (TS) Limiting Condition for Operation (LCO) 3.6.1.3, Primary Containment Isolation Valves (PCIVs), requires each PCIV, except reactor buildingto-suppression chamber vacuum breakers, to be Operable in Modes 1, 2, and 3. Because a PCIV in the RCIC system was determined to be inoperable, TS 3.6.1.3 Required Action A.1 requires the RCIC system to be isolated within 4 hours to prevent flow through the RCIC turbine exhaust check valve. Based on the results of the POE, check valve 2-HCV-071-0014 was inoperable for longer than allowed by the TS. The RCIC Steam Line Outboard Isolation valve [FCV], 2-FCV-071-0003, was closed and deactivated to meet TS 3.6.1.3 Required Action A.1 on March 2, 2013, at 0525 CST.

BFN, Unit 2, TS LCO 3.5.3, RCIC System, requires the RCIC system to be Operable in Mode 1 and in Modes 2 and 3 when the reactor steam dome pressure is greater than 150 pounds per square inch gauge (psig). Closing the RCIC Steam Line Outboard Isolation valve, 2-FCV-071-0003, resulted in the RCIC system being inoperable. The TS 3.5.3 Required Action A.2 requires the RCIC system to be returned to service within 14 days. Because valve 2-HCV-071-0014 is unisolable to primary containment, a unit shutdown was required to perform repairs. The TS 3.5.3 Required Action B required that BFN, Unit 2, be in Mode 3 by 1725 CDT, on March 16, 2013, and less than 150 psig by 1725 CDT, on March 17, 2013.

Actions to initiate reactor shutdown began on March 14, 2013, at 0800 CDT. On March 14, 2013, at 2203 CDT, BFN, Unit 2, entered Mode 3, and on March 15, 2013, at 0310 CDT, BFN, Unit 2, entered Mode 4.

B. Status of structures, components, or systems that were inoperable at the start of the event and that contributed to the event:

Stop-check valve 2-HCV-071-0014 was unable to perform its function as a PCIV because it was stuck in the open position.

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NARRATIVE										
C.	Dates and approximate times	of occurre	nces:							
	March 2, 2013, at 0300 CST,	Ra sh po	adiograph lowed the osition.	y results for valve to be	2-HCV-0 in the full	71-0014 y open				
1	March 2, 2013, at 0525 CST,	Bl	FN, Unit 2 ervice.	, RCIC syste	em remov	ed from				
	March 14, 2013, at 0800 CDT,	Oj sh	perations lutdown B	to						
	March 14, 2013, at 2203 CDT,	B	⁼ N, Unit 2	, entered Mo	ode 3.					
	March 15, 2013, at 0310 CDT,	Bf	BFN, Unit 2, entered Mode 4.							
D.	Manufacturer and model num that failed during the event:	nber (or oth	ber (or other identification) of each component							
	The failed component was stop a model 5312WE-8IN Stop-Che	-check valve eck Valve, m	lve 2-HCV-071-0014. The component was manufactured by the Walworth Company.							
E.	Other systems or secondary	functions affected:								
-	Valve 2-HCV-071-0014 is one of containment. Containment was RCIC system being unable to p	of two PCIVs s isolated by perform its sa	ⁱ two PCIVs connecting RCIC directly to primary isolated by closing the second PCIV resulting in the rform its safety function.							
F.	Method of discovery of each	component or system failure or procedural error:								
	On March 2, 2013, radiography valve, 2-HCV-071-0014, showe	results for t d the valve	Its for the BFN, Unit 2, RCIC system stop-check a valve to be in the fully open position.							
G.	The failure mode, mechanism	ı, and effect of each failed component, if known:								
	The disk of valve 2-HCV-071-0 configuration resulted 2-HCV-0 PCIV.	ck in the o ing unable	open positio e to perform	n. This its functic	on as a					
Н.										
Operations declared 2-HCV-071-0014 inoperable and closed RCIC Steam Line Outboard Isolation valve, 2-FCV-071-0003, to isolate primary containment, resulting in the RCIC system becoming inoperable. The RCIC system was unable to be repaired in the LCO window; therefore, Operations shutdown BFN, Unit 2, as required by TS.										
١.	Automatically and manually i	initiated sat	fety syste	m respons	es:					
	There were no safety system re	esponses.								
L	<u> </u>									

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III. Cause of the event

A. The cause of each component or system failure or personnel error, if known:

The direct cause of the valve's failure is not definitively known; however, the most likely causes of the component failing in the open position is the valve disc weight may be lighter than vendor design requirements, and the finish on disc travel guide male component, bore of valve dashpot [DPT], and stem outer diameter are excessively rough due to scoring as a result of general wear from process debris and condensing steam. Corrective actions are in place to procure the design valve disc weight, design finish on the disc travel guide male component, bore of valve dashpot, and stem outside diameter from the vendor.

B. The cause(s) and circumstances for each human performance related root cause:

The root cause of this condition is valve 2-HCV-071-0014 was improperly classified as a Run-to-Failure hand valve when it should have been classified as a Critical check valve; therefore, internal inspection and internal parts refurbishment or replacement was not performed.

There are two contributing factors.

- The Corrective Action Program (CAP) processes for problem screening did not ensure adequate cause evaluations when stop-check valve
 2-HCV-071-0014 was discovered stuck in the open position in 2002 and 2005 resulting in corrective actions that did not prevent problem recurrence.
- 2. Technical Manual for Manually Operated Valves Model 5312 for Units 1 and 2 contains insufficient information to identify valve degradation due to wear and/or diagnose reasons for valve disc sticking in the open position.

IV. Analysis of the event:

The Tennessee Valley Authority (TVA) is submitting this report in accordance with Title 10 of the Code of Federal Regulations (10 CFR) 50.73(a)(2)(i)(A), as the completion of any nuclear plant shutdown required by the plant's Technical Specifications, and 10 CFR 50.73(a)(2)(i)(B), as any operation or condition which was prohibited by the plant's Technical Specifications.

On March 2, 2013, radiography results for the BFN, Unit 2, RCIC system turbine exhaust stop-check valve, 2-HCV-071-0014, showed the valve to be stuck in the fully open position. This stop-check valve is one of two PCIVs connecting the RCIC system directly to primary containment. The POE determined that valve 2-HCV-071-0014 was inoperable from March 27, 2011, until March 15, 2013, at 0310 CDT, when BFN, Unit 2, entered Mode 4.

BFN, Unit 2, LCO 3.0.4 prohibits Mode changes when an LCO is not met except under certain conditions that were not applicable to this event. Since it was not recognized that

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a PCIV was completed, E	inoperable from Marc 3FN, Unit 2, changed	h 27, 2011, to Modes in viola	March 1 ation of L	5, 2013, un .CO 3.0.4 oi	til the POE n multiple c	was occasions.
Valve 2-HCV RCIC system state that a F risks and con being perform required. The maintenance	'-071-0014 has two N n and one associated Run-To-Failure classif nsequences of failure med and the component is classification influe on the component.	Maintenance R with primary of fication is defin without any p ent should be ences the scop	ule funct containm ned as a redictive operated be and fre	ions, one as ent isolatior component or preventiv until correc equency of p	ssociated v n. TVA pro with accep ve mainten ctive mainte preventive	vith the cedures otable ance enance is
RCIC Turbin Run-To-Failı to open is a	e Exhaust Hand-Con ure. This classificatio Maintenance Rule Rig	trol Valve, 2-H n is incorrect l sk Significant t	CV-071- because function.	0014, is cla the failure c	ssified as of stop-cheo	ck valves
should have a result, inte performed le have been ta stop-check v (PER) 69189	been classified as a criation of 241 rnal inspection and in ading to the valve be aken to change the re valves from run-to-fail 91).	Critical check Iternal parts re coming stuck Iability classif ure to critical (valve is t furbishm in the op ication of Problem	he root cause ent or repla en position. The RCIC T Evaluation	se of this e cement wa Corrective Turbine Ext Report	event. As as not e actions haust
On March 2 partially oper actions were sticking oper served as ar Order (WO) sticking oper threads in th the bore of th cause analys Level C PEF In October 2 position. A F processes a	I, 2005, radiography in position. PER 7916 ineffective in identify n. The radiography por effectiveness review 05-713361-000 and L n. With vendor concur e bore of the valve di he valve disc and the sis considered other por twas not required to 002, WO 02-009107- PER was not generate nd procedures at the	results showed 57 was written ring and correct erformed on 2 v of the correct evel C PER 7 urrence, WO 0 sc away to rer valve stem. If cotential cause be well document 000 identified ed to document	d valve 2 to addre cting the -HCV-07 tive main 9167 to 1 5-71336 nove the t cannot es becau ented wi 2-HCV-0 nt this co events co	-HCV-071-0 ss this issue cause for 2- 1-0014 on I tenance per prevent recu 1-000 mach assumed ir be determin se the caus hen the PEF 071-0014 to ndition. Ina pontributed to	0014 was s e. The corr HCV-071- March 2, 20 formed un urrence of f ined the fer ned if the P ined if the P	tuck in a rective 0014 013, der Work the valve male between ER 79167 on for a ositioned. open AP
Technical M insufficient ir reasons for missing is: ir finish specifi components tolerances.	anual for Manually Op formation to identify alve disc sticking in t iterfacing dimensions cation for internal me could impede moven This issue was detern	perated Valves valve degrada the open posit and tolerance tal component nent of parts, a mined to be a	s Model tion due ion. The es betwee s where and desig contribut	5312 for Uni to wear and specific infr en critical in friction betv gn valve dis ing factor to	its 1 and 2 d/or diagno ormation th ternal valve veen these c weight w o this event	contains se nat is e parts, ith

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Extent of Condition

The extent of condition evaluation identified eight additional stop-check valves in the Check Valve Condition Monitoring Plan potentially subject to the same failure mode as 2-HCV-071-0014. The extent of condition was limited to the Appendix J and Condition Monitoring Plan check valves because this scope addresses check valves that have safety-related, Appendix R, regulatory and/or other risk significant functions. Corrective actions are in place to procure spare parts, perform initial inspection and replacement of internal parts, and setup internal inspection preventive maintenance for the RCIC Turbine Exhaust Hand Control Valves and High Pressure Coolant Injection (HPCI) [BJ] Turbine Exhaust Valves. Additionally, corrective actions are in place to disassemble and inspect the internal parts of the RCIC Vacuum Pump Discharge Shutoff Valves for scoring or otherwise damaged surfaces between moving metal-to-metal interfaces that could impede valve disc movement and replace parts as necessary.

Extent of Cause

The scope of the extent of cause evaluation is limited to valves that could have been improperly classified as a Run-to-Failure check valve resulting in entry into any unplanned TS action statements. The RCIC Turbine Exhaust Hand Control Valves, RCIC Vacuum Pump Discharge Shutoff Valves, and HPCI Turbine Exhaust Valves were identified as meeting these criteria. Corrective Actions are in place to correct the location identification and descriptions of these valves and to submit Preventive Maintenance Change Requests (PMCRs) to open, inspect, clean, and replace valve disc, dashpot, and stem, as necessary.

V. Assessment of Safety Consequences

A POE was performed and concludes that 2-HCV-071-0014 was inoperable from March 27, 2011, at 0328 CDT, until BFN, Unit 2, was in Mode 4 on March 15, 2013, at 0310 CDT. With 2-HCV-071-0014 stuck in the open position, this valve could not perform its function as a primary containment isolation valve. However, the POE concluded that 2-CKV-071-0580, RCIC Turbine Exhaust Check valve, remained capable of isolating the penetration.

According to the POE, the first opening of 2-HCV-071-0014 following the radiography performed on February 06, 2011, is assumed to be the start of the inoperability of this valve. This assumption is conservative with respect to the actual time the valve stuck in the open position, which is unknown and cannot be positively determined.

The RCIC system injection function was not lost by the failure of 2-HCV-071-0014. With the 2-HCV-071-0014 in the open position, the RCIC system was capable of injecting and meeting its mission time. The RCIC system function was made unavailable when the RCIC Steam Line Outboard Isolation Valve, 2-FCV-071-0003, was closed to comply with TSs for isolating primary containment.

Based on this analysis, this condition is of low safety significance and there was no significant reduction to the health and safety of the public.

NRC FORM 366A U.S. NUCLEAR REGULATORY COMMISSION (10-2010) LICENSEE EVENT REPORT (LER) CONTINUATION SHEET PAGE (3) LER NUMBER (6) FACILITY NAME (1) DOCKET (2) SEQUENTIAL REVISION YEAR NUMBER NUMBER Browns Ferry Nuclear Plant, Unit 2 2013 7 of 8 05000260 -- 001 -- 00 NARRATIVE A. Availability of systems or components that could have performed the same function as the components and systems that failed during the event: With 2-HCV-071-0014 stuck in the open position, this valve could not perform its function as a primary containment isolation valve. However, 2-CKV-071-0580, RCIC Turbine Exhaust Check valve, remained capable of isolating the penetration. B. For events that occurred when the reactor was shut down, availability of systems or components needed to shutdown the reactor and maintain safe shutdown conditions, remove residual heat, control the release of radioactive material, or mitigate the consequences of an accident: This event did not occur when BFN, Unit 2, was shut down. C. For failure that rendered a train of a safety system inoperable, an estimate of the elapsed time from discovery of the failure until the train was returned to service: The POE concluded that 2-HCV-071-0014 was inoperable from March 27, 2011, at 0328 CDT, until BFN, Unit 2, was in Mode 4 on March 15, 2013, at 0310 CDT. With 2-HCV-071-0014 being stuck in the open position, this valve could not perform its function as a primary containment isolation valve. VI. **Corrective Actions** Corrective Actions are being managed by TVA's corrective action program under PER 696534. A. Immediate Corrective Actions: Check valve 2-HCV-071-0014 was declared inoperable after failing radiography. The RCIC Steam Line Outboard Isolation valve, 2-FCV-071-0003, was closed and deactivated. 3. Check Valve 2-HCV-071-0014 was dogged closed (with the valve stem and handwheel). 4. The BFN, Unit 2, reactor shutdown as required by BFN, Unit 2, TSs. 5. The internal parts of 2-HCV-071-0014 were cleaned and the valve's disk was replaced with a spare. B. Corrective Actions to Prevent Recurrence or to reduce probability of similar events occurring in the future: The root cause of this event will be addressed by submitting PMCRs to open, inspect, clean, and replace valve disc, dashpot, and stem, as necessary, for RCIC Turbine Exhaust Hand Control Valves, RCIC Vacuum Pump Discharge Shutoff Valves, and HPCI Turbine Exhaust Valves. PER corrective actions 691891-001

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and 691891-002 changed the reliability classification of the RCIC Turbine Exhaust stop-check valves from run-to-failure to critical.

The first contributing cause was addressed by previous PER corrective actions 495782-008 and 537869-001. These corrective actions revised fleet corrective action procedure NPG-SPP-03.1.4, Corrective Action Program Screening and Oversight, to include additional guidance for assigning PER Classification Levels, and to define when Level B PERs require an Upper Tier Apparent Cause Evaluation (ACE) or a Lower Tier ACE.

The second contributing cause will be addressed by revising the Walworth Technical Manual for Manually Operated Valves Model 5312 WE in accordance with NPG-SPP-09.20, Vendor Manual Control.

VII. Additional Information:

A. Previous similar events at the same plant:

A search of BFN Licensee Event Reports (LERs) for Units 1, 2, and 3 for the last several years did not identify any similar events.

A search was performed on the BFN corrective action program. Similar PERs related to the condition reported in this LER are PERs 79167 and 689932.

The corrective actions in PER 79167 were ineffective in identifying and correcting the cause for 2-HCV-071-0014 sticking open because the cause of the condition was incorrectly identified as interference caused by female threads in the bore of the valve disc.

B. Additional Information:

There is no additional information.

C. Safety System Functional Failure Consideration:

In accordance with Nuclear Energy Institute (NEI) 99-02, this condition is not considered a safety system functional failure.

D. Scram with Complications Consideration:

This event did not result in an unplanned scram with complications.

VIII. COMMITMENTS

There are no commitments.