Administrative Topics Outline ILT 1205 Facility: Browns Ferry NPP Date of Examination: 5/7/2012 Examination Level: RO/SRO Operating Test Number: Administrative Topic Туре

Describe activity to be performed Code (see Note) * Conduct of Operations 2.1.31 Verification of Off Site Power Availability to 4.16 kV D Shutdown Boards SRO/RO A1a Conduct of Operations Ρ 2.1.19 ICS Logs RO A1b 2.1.18 NRC event notification due to HPCI valve failure Ν SRO A1b Equipment Control Ν 2.2.12 Complete Primary Containment Nitrogen Leakage and Consumption Surveillance and evaluate Accentance Criteria

Form ES-301-1

1205

RO A2		Consumption Survemance and evaluate Acceptance Criteria
SRO A2	N	2.2.12 Complete Primary Containment Nitrogen Leakage and Consumption Surveillance, evaluate Acceptance Criteria, and determine Technical Specifications
Radiation Control	N	2.3.11 Calculate Airborne Effluent Release Rate iaw 0-SI-4.8.b.1.a.1
SRO/RO A3		
Emergency Plan	M	2.4.41 Classify an Event
SRO A4		
		quired for SROs. RO applicants require only 4 items unless they tive topics, when all 5 are required.
* Type Codes & Criteria		C)ontrol Room
	(I	D) itect from bank (\leq 3 for ROs; \leq 4 for SROs and RO retakes)
	()	N)ew or (M)odified from bank (≥ 1)
	(H	P)revious 2 exams (≤ 1 ; randomly selected)

(S)imulator

Administrative Topics Outline

Reactor Operator

1. Verification of Off Site Power Availability to 4.16 kV Shutdown Boards

- Direct
- 0-SR-3.8.1.A.1
- Marks 500KV and 161KV Sources as Qualified. Completes Attachment 1 for Unit 3 accurately, records indicated voltages for step 7.2[5] and does not sign acceptance criteria, does not sign acceptance criteria for 7.4[1], and marks acceptance criteria satisfied on Surveillance Task Sheet as NO.
- 2.1.31 Ability to locate control room switches, controls, and indications, and to determine that they correctly reflect the desired plant lineup. Importance RO 4.6
- 2. ICS Logs
 - Previous
 - 2-SR-2 or 3-SR-2
 - Perform Operator logs using ICS screens in accordance with 2-SR-2 Instrument Checks and Observations for log tables 1.1, 1.6, 1.25, and 1.30. Verify acceptance criteria are satisfied in accordance with notes.
 - 2.1.19 Ability to use plant computers to evaluate system or component status. RO 3.9
- 3. Complete Primary Containment Nitrogen Leakage and Consumption Surveillance and evaluate Acceptance Criteria
 - New
 - 3-SI-4.7.A.2.A, Primary Containment Nitrogen Leakage and Consumption
 - Completes Surveillance and determines that it does not meet acceptance criteria. Determines that an LCO must be entered and informs the Unit Supervisor.
 - 2.2.12 Knowledge of surveillance procedures. RO 3.7
- 4. Calculate Airborne Effluent Release Rate iaw 0-SI-4.8.b.1.a.1
 - New
 - 0-SI-4.8.B.1.a.1, 2-EOI Appendix-12
 - Calculate Stack Release Rate and Total Site Release Fraction determine it does not meet Acceptance Criteria, and determine that vent flowrate must be reduced in accordance with 2-EOI Appendix-12.
 - 2.3.11 Ability to control radiation releases RO 3.8

ILT 1205

Administrative Topics Outline

Senior Reactor Operator

- 1. Verification of Off Site Power Availability to 4.16 kV Shutdown Boards
 - Direct
 - 0-SR-3.8.1.A.1
 - Marks 500KV and 161KV Sources as Qualified. Completes Attachment 1 for Unit 3 accurately, records indicated voltages for step 7.2[5] and does not sign acceptance criteria, does not sign acceptance criteria for 7.4[1], and marks acceptance criteria satisfied on Surveillance Task Sheet as NO.
 - 2.1.31 Ability to locate control room switches, controls, and indications, and to determine that they correctly reflect the desired plant lineup. Importance SRO 4.3
- 2. NRC event notification due to HPCI Valve Failure
 - New
 - NPG-SPP-03.5, Regulatory Reporting Requirements
 - Determine NRC event notification requirements, as the Shift Manager due to a failure of the HPCI Pump Injection Valve. Determines Technical Specification actions required.
 - 2.1.18 Ability to make accurate, clear, and concise logs, records, status boards, and reports. Importance SRO 3.8
- 3. Complete Primary Containment Nitrogen Leakage and Consumption Surveillance, evaluate Acceptance Criteria, and determine Technical Specifications
 - New
 - 3-SI-4.7.A.2.A, Primary Containment Nitrogen Leakage and Consumption
 - Completes Surveillance, recognizes that it does not meet acceptance criteria, and determines Technical Specification actions required.
 - 2.2.12 Knowledge of surveillance procedures. SRO 4.1
- 4. Calculate Airborne Effluent Release Rate iaw 0-SI-4.8.b.1.a.1
 - New
 - 0-SI-4.8.B.1.a.1, 2-EOI Appendix-12
 - Calculate Stack Release Rate and Total Site Release Fraction determine it does not meet Acceptance Criteria, and determine that vent flowrate must be reduced in accordance with 2-EOI Appendix-12.
 - 2.3.11 Ability to control radiation releases SRO 4.3

ILT 1205

Administrative Topics Outline

- 5. Classify an Event
 - Modified
 - EPIP-1 and 3 Emergency Classification Procedure and Alert
 - The event is classified as an Alert 1.1-A2 and the Initial Notification appendix is completed with the correct information. Event is classified within 15 minutes and Initial Notification is completed within 15 minutes of classification.
 - 2.4.41 Knowledge of emergency action level thresholds and classifications. Importance SRO 4.6

ES-201

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Examination Outline Quality Checklist

DRAFT

Form ES-201-2

Facility:	Zowns Feery 2012 - 301 ExAm Date of Examination:	<u>{</u>	2017	
Item	Task Description	a	Initia b*	s c#
1. W	a. Verify that the outline(s) fit(s) the appropriate model, in accordance with ES-401.	pur	NA	1
R I	b. Assess whether the outline was systematically and randomly prepared in accordance with Section D.1 of ES-401 and whether all K/A categories are appropriately sampled.	3	5	46
T T	c. Assess whether the outline over-emphasizes any systems, evolutions, or generic topics.	8		lab.
E N	d. Assess whether the justifications for deselected or rejected K/A statements are appropriate.	Nk	Π	lab -
2. S	a. Using Form ES-301-5, verify that the proposed scenario sets cover the required number of normal evolutions, instrument and component failures, technical specifications, and major transients.	NA	NA	MA
U L A T	b. Assess whether there are enough scenario sets (and spares) to test the projected number and mix of applicants in accordance with the expected crew composition and rotation schedule without compromising exam integrity, and ensure that each applicant can be tested using at least one new or significantly modified scenario, that no scenarios are duplicated from the applicants' audit test(s), and that scenarios will not be repeated on subsequent days.	$\left[\right]$	$\left \right\rangle$)
O R	c. To the extent possible, assess whether the outline(s) conform(s) with the qualitative and quantitative criteria specified on Form ES-301-4 and described in Appendix D.			
3. W / T	 a. Verify that the systems walk-through outline meets the criteria specified on Form ES-301-2: (1) the outline(s) contain(s) the required number of control room and in-plant tasks distributed among the safety functions as specified on the form (2) task repetition from the last two NRC examinations is within the limits specified on the form (3) no tasks are duplicated from the applicants' audit test(s) (4) the number of new or modified tasks meets or exceeds the minimums specified on the form (5) the number of alternate path, low-power, emergency, and RCA tasks meet the criteria on the form. 			
	 b. Verify that the administrative outline meets the criteria specified on Form ES-301-1: (1) the tasks are distributed among the topics as specified on the form (2) at least one task is new or significantly modified (3) no more than one task is repeated from the last two NRC licensing examinations 			
	c. Determine if there are enough different outlines to test the projected number and mix of applicants and ensure that no items are duplicated on subsequent days.	1	4	4
4.	 Assess whether plant-specific priorities (including PRA and IPE insights) are covered in the appropriate exam sections. 	¢	Ub	103
G E	b. Assess whether the 10 CFR 55.41/43 and 55.45 sampling is appropriate.	œ		INA
N	c. Ensure that K/A importance ratings (except for plant-specific priorities) are at least 2.5.	3		lial
E R	d. Check for duplication and overlap among exam sections.	Q.	17	ial
Α	e. Check the entire exam for balance of coverage.	9	\square	Ash
L	f. Assess whether the exam fits the appropriate job level (RO or SRO).	0	T	in
c. NRC	or Kewwerth D Schart Bund Start Supervisor Michael A. JINKE / MA	7	Di 12-11 12-11 12-11 12-11	ate j
Note:	# Independent NRC reviewer initial items in Column "c"; chief examiner concurrence rec * Not applicable for NRC-prepared examination outlines	uired.		-



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{PRIVATE }	acility: Brov	vns I	Ferr	y				Da	ate c	of Ex	am:		20	12	>			
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1.	1	3	4	3				3	3	l		4	20		3	1	4	7
Emergency & Abnormal Plant	2	1	2	1		N/A		1	1		/A	1	7		1	1	2	3
Evolutions	Tier Totals	4	6	4				4	4			5	27		4		6	10
•	1	3	3	2	2	3	2	3	2	2	2	2	26		2	ſ	3	5
2. Plant	2	2	1	1	1	1	1	1	1	1	1	1	12	0	1		2	3
Systems	Tier Totals	5	4	3	3	4	3	4	3	3	3	3	38		3		5	8
3. Generic K		l Abi	lities	5		1		2	:	3	4	4 10		1	2	3	4	7
·	Categories					2	2	2	:	3	3			1	2	2	2	1
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ES-401, REV 9			T1G	1 BWR EXAMINATION OUTLINE	FORM ES-40		
KA	NAME / SAFETY FUNCTION:	IF	२	K1 K2 K3 K4 K5 K6 A1 A2 A3 A4 G	TOPIC:		
		RO	SRO				
295001AA2.03	Partial or Complete Loss of Forced Core Flow Circulation / 1 & 4	3.3	3.3		Actual core flow		
295003AK3.04	Partial or Complete Loss of AC / 6	3.0	3.2		Ground isolation		
295004AK1.02	Partial or Total Loss of DC Pwr / 6	3.2	3.4		Redundant D.C. power supplies: Plant-Specific		
295005G2.2.39	Main Turbine Generator Trip / 3	3.9	4.5		Knowledge of less than one hour technical specification action statements for systems.		
95006AA2.03	SCRAM / 1	4.0	4.2		Reactor water level		
95016AA1.04	Control Room Abandonment / 7	3.1 3	3.2		A.C. electrical distribution		
95018AK1.01	Partial or Total Loss of CCW / 8	3.5 3	3.6		Effects on component/system operations		
95019AA1.02	Partial or Total Loss of Inst. Air / 8	3.3 3	3.1		Instrument air system valves: Plant-Specific		
5021AK3.03	Loss of Shutdown Cooling / 4	2.9 2	2.9		Increasing drywell cooling		
5023AK2.05	Refueling Acc Cooling Mode / 8	3.5 3	.7		Secondary containment ventilation		
5024EK2.05	High Drywell Pressure / 5	3.9 4	.0		RPS		

IN

ES-401, RE	EV 9		T1G	1 BWR EXAMINATION OUTLINE	FORM ES-401		
KA	NAME / SAFETY FUNCTION:		IR _	K1 K2 K3 K4 K5 K6 A1 A2 A3 A4 G	TOPIC:		
		RÓ	SRO				
295025EK3.02	High Reactor Pressure / 3	3.9	4.1		Recirculation pump trip: Plant-Specific		
295026EK1.02	Suppression Pool High Water Temp. / 5	3.5	3.8	2 000000000000000000000000000000000000	Steam condensation		
95028EK2.01	High Drywell Temperature / 5	3.7	4.1		Drywell spray: Mark-I&II		
95030G2.2.22	Low Suppression Pool Wtr Lvl / 5	4.0	4.7		Knowledge of limiting conditions for operations and safety limits.		
95031EK2.15	Reactor Low Water Level / 2	3.2	3.2		A.C. distribution: Plant-Specific		
95037EA2.06	SCRAM Condition Present and Power Above APRM Downscale or Unknown / 1	4.0	4.1		Reactor pressure		
5038G2.4.46	High Off-site Release Rate / 9	4.2	4.2		Ability to verify that the alarms are consistent with the plant conditions.		
0000G2.2.40	Plant Fire On Site / 8	3.4	4.7		Ability to apply technical specifications for a system.		
00000AA1.05	Generator Voltage and Electric Grid Distrurbancecs	3.9	4.0		Engineered Safety Features		

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ES-401, REV 9			T10	32 BWR EXAMINATION OUTLINE	FORM ES-401-	
КА	NAME / SAFETY FUNCTION:		IR	K1 K2 K3 K4 K5 K6 A1 A2 A3 A4 G	TOPIC:	
		RO	SRC)		
295007AA1.05	High Reactor Pressure / 3	3.7	3.8		Reactor/turbine pressure regulating system	
295008AK2.10	High Reactor Water Level / 2	2.7	2.8		RHR (ability to drain): Plant-Specific	
295022G2.4.1	Loss of CRD Pumps / 1	4.6	4.8		Knowledge of EOP entry conditions and immediate action steps.	
295029EK2.09	High Suppression Pool Wtr Lvl / 5	3.1	3.2		RCIC: Plant-Specific	
295033EK3.01	High Secondary Containment Area Radiation Levels / 9	3.3	3.5		Emergency depressurization	
295034EK1.02	Secondary Containment Ventilation High Radiation / 9	4.1	4.4		Radiation releases	
295036EA2.03	Secondary Containment High Sump/Area Water Level / 5	3.4	3.8		Cause of the high water level	

ES-401, REV 9			T2G	1 BWR EXAMINATION OUTLINE	FORM ES-401		
KA	NAME / SAFETY FUNCTION:		IR	K1 K2 K3 K4 K5 K6 A1 A2 A3 A4 G	TOPIC:		
		RO	SRC	•			
203000K2.01	RHR/LPCI: Injection Mode	3.5	3.5		Pumps		
203000K3.04	RHR/LPCI: Injection Mode	4.6	4.6		Adequate core cooling		
205000A2.06	Shutdown Cooling	3.4	3.5		SDC/RHR pump trips		
206000A2.08	HPCI	3.9	4.2		†High suppression pool temperature: BWR-2,3,4		
209001G2.4.46	LPCS	4.2	4.2		Ability to verify that the alarms are consistent with the plant conditions.		
211000K2.01	SLC	2.9	3.1		SBLC pumps		
212000A1.01	RPS	2.8	2.9		RPS motor-generator output voltage		
215003G2.4.2	IRM	4.5	4.6		Knowledge of system set points, interlocks and automatic actions associated with EOP entry conditions.		
15003K2.01	IRM	2.5	2.7		IRM channels/detectors		
15004A1.04	Source Range Monitor	3.5	3.5		Control rod block status		
15005K4.01	APRM / LPRM	3.7	3.7		Rod withdrawal blocks		

ES-401, REV 9			T2G	1 B	W	RE	XA	M	NA	TIC	N	ou	TL	NE		FORM ES-401-		
KA	NAME / SAFETY FUNCTION:		IR	K1	κ	2 K	(3 I	K 4	K5	K6	A1	A2	A3	A4 (G	TOPIC:		
		RÓ	SRO)														
217000K5.07	RCIC	3.1	3.1] [] [Assist core cooling		
218000A4.07	ADS	3.5	3.8] [ADS valve acoustical monitor noise: Plant-Specific		
223002K6.03	PCIS/Nuclear Steam Supply Shutoff	2.9	3.1] [] [] (]	Process radiation monitoring system		
239002K1.07	SRVs	3.6	3.8	Ø	C	ם נ] []	Suppression pool		
239002K5.04	SRVs	3.3	3.5		C] [] []	Tail pipe temperature monitoring		
259002A1.05	Reactor Water Level Control	2.9	2.9] [] [<u>v</u>]	FWRV/startup level control position: Plant-Specific .		
259002K4.02	Reactor Water Level Control	2.8	3.0] 🗟	2 (Bypassing of the RWM: Plant-Specific		
261000K1.03	SGTS	2.9	3.1	2] [][]	Suppression pool		
262001A3.01	AC Electrical Distribution	3.1	3.2] [] []	Breaker tripping		
262002K3.17	UPS (AC/DC)	2.9	3.1] []	Process monitoring: Plant-Specific		
263000K1.03	DC Electrical Distribution	2.6	2.8] [] []	Battery ventilation		

ES-401, R	EV 9		T2G	1 BWR EXAMINATION OUTLINE	FORM ES-401-1
KA	NAME / SAFETY FUNCTION:		IR	K1 K2 K3 K4 K5 K6 A1 A2 A3 A4 G TOPIC	>:
		RO	SRC		
264000A3.05	EDGs	3.4	3.5		edding and sequencing
264000K5.06	EDGs	3.4	3.5		quencing
300000A4.01	Instrument Air	2.6	2.7	Pressure	a gauges
400000K6.07	Component Cooling Water	2.7	2.8	Breakers	s, relays, and disconnects

ES-401, REV 9			G2 BWR EXAMINATION OUTLINE	FORM ES-401		
KA	NAME / SAFETY FUNCTION:	IR	K1 K2 K3 K4 K5 K6 A1 A2 A3 A4 G	TOPIC:		
		RO SR	0			
201002G2.1.25	RMCS	3.9 4.2		Ability to interpret reference materials such as graphs, monographs and tables which contain performance data.		
201006A3.01	RWM	3.2 3.1		System window and light indication: P-Spec(Not-BWR6).		
214000A4.02	RPIS	3.8 3.8		Control rod position		
219000K3.01	RHR/LPCI: Torus/Pool Cooling Mode	3.9 4.1		Suppression pool temperature control		
230000K2.02	RHR/LPCI: Torus/Pool Spray Mode	2.8 2.9		Pumps		
256000K1.18	Reactor Condensate	2.9 3.0		Circulating water system		
259001A2.07	Reactor Feedwater	3.7 3.8		Reactor water level control system malfunctions		
71000A1.15	Offgas	2.7 2.8		Steam supply pressures		
72000K6.02	Radiation Monitoring	2.5 2.7		D.C. power		
38000K4.03	Plant Ventilation	2.8 2.9		Automatic starting and stopping of fans		
90001K5.01	Secondary CTMT	3.3 3.4		Vacuum breaker operation: BWR-4		

ES-401, R	EV 9	T	G2 BWR EXAMINATION OUTLINE	FORM ES-401-1
KA	NAME / SAFETY FUNCTION:	IR	K1 K2 K3 K4 K5 K6 A1 A2 A3 A4 G	TOPIC:
		RO SI	0	
290003K1.03	Control Room HVAC	2.8 2.		Remote air intakes: Plant-Specific

ES-401,	REV 9		T:	BWR EXAMINATION OUTLINE	FORM ES-401	
KA	NAME / SAFETY FUNCTION:		IR	K1 K2 K3 K4 K5 K6 A1 A2 A3 A4 G	TOPIC:	
		RO	SRC)		
G2.1.15	Conduct of operations	2.7	3.4		Knowledge of administrative requirements for temporary management directives such as standing orders, night orders, Operations memos, etc.	
32.1.38	Conduct of operations	3.7	3.8		Knowledge of the stations requirements for verbal communication when implamenting procedures	
32.2.3	Equipment Control	3.8	3.9		(multi-unit license) Knowledge of the design, procedural and operational differences between units.	
32.2.6	Equipment Control	3.0	3.6		Knowledge of the process for making changes to procedures	
32.3.15	Radiation Control	2.9	3.1		Knowledge of radiation monitoring systems	
32.3.4	Radiation Control	3.2	3.7		Knowledge of radiation exposure limits under normal and emergency conditions	
32.3.5	Radiation Control	2.9	2.9		Ability to use radiation monitoring systems	
2.4.2	Emergency Procedures/Plans	4.5	4.6		Knowledge of system set points, interlocks and automatic actions associated with EOP entry conditions.	
2.4.22	Emergency Procedures/Plans	3.6	4.4		Knowledge of the bases for prioritizing safety functions during abnormal/emergency operations.	
2.4.6	Emergency Procedures/Plans	3.7	4.7		Knowledge symptom based EOP mitigation strategies.	

ES-401,	REV 9
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SRO T1G1 BWR EXAMINATION OUTLINE

KA	NAME / SAFETY FUNCTION:		IR		K 1	K	2	КЗ	K4	K	i K	6 A	1 A	2 /	43	A4	G	TOPIC:
		RO	SRO	80														
295001AA2.04	Partial or Complete Loss of Forced Core Flow Circulation / 1 & 4	3.0	3.1] [] []] [Individual jet pump flows: Not-BWR-1&2
295005AA2.03	Main Turbine Generator Trip / 3	3.1	3.1	(C] [)	2 [Turbine valve position
295018G2.4.9	Partial or Total Loss of CCW / 8	3.8	4.2	! (C] [<u>ר</u>										 Knowledge of low power / shutdown implications in accident (e.g. LOCA or loss of RHR) mitigation strategies.
295021AA2.05	Loss of Shutdown Cooling / 4	3.4	3.5	. [С] [2 [Reactor vessel metal temperature
295026G2.1.30	Suppression Pool High Water Temp. / 5	4.4	4.0	• [] [] [] [_		 Ability to locate and operate components, including local controls.
295028G2.2.25	High Drywell Temperature / 5	3.2	4.2	[C] [] [] [Knowledge of the bases in Technical Specifications for limiting conditions for operations and safety limits.
295038G2.2.12	High Off-site Release Rate / 9	3.7	4.1]		С] [Г] [] [Knowledge of surveillance procedures.

ES-401, RE	:V 9	FORM ES-4(
KA .	NAME / SAFETY FUNCTION:		IR	K1 K2 K3 K4 K5 K6 A1 A2 A3 A4 G	TOPIC:
		RO	SRC)	
295008G2.4.18	High Reactor Water Level / 2	3.3	4.0		Knowledge of the specific bases for EOPs.
295013AA2.02	High Suppression Pool Temp. / 5	3.2	3.5		Localized heating/stratification
295020G2.2.37	Inadvertent Cont. Isolation / 5 & 7	3.6	4.6		Ability to determine operability and/or availability of safety related equipment

ES-401, RE	EV 9	FORM ES-401-			
KA	NAME / SAFETY FUNCTION:		IR	K1 K2 K3 K4 K5 K6 A1 A2 A3 A4 G	TOPIC:
		RO	SRC)	
206000A2.17	HPCI	3.9	4.3		†HPCI inadvertent initiation: BWR-2,3,4
211000G2.2.38	SLC	3.6	4.5		Knowledge of conditions and limitations in the facility license.
215003G2.1.27	IRM	3.9	4		Knowledge of system purpose and or function.
223002G2.4.20	PCIS/Nuclear Steam Supply Shutoff	· 3.8	4.3		Knowledge of operational implications of EOP warnings, cautions and notes.
239002A2.04	SRVs	4.1	4.2		ADS actuation

9/6/2011 2:58 PM

ES-401, RE	EV 9	S	RO 1	T2G2 BWR EXAMINATION OUTLINE	FORM ES-401-1
KA	NAME / SAFETY FUNCTION:		IR	K1 K2 K3 K4 K5 K6 A1 A2 A3 A4 G	TOPIC:
		RO	SRC	0	
202002A2.03	Recirculation Flow Control	2.6	2.7		Loss of D.C.
214000G2.2.42	RPIS	3.9	4.6		Ability to recognize system parameters that are entry- level conditions for Technical Specifications
233000G2.2.39	Fuel Pool Cooling/Cleanup	3.9	4.5		Knowledge of less than one hour technical specification action statements for systems.

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ES-401, REV 9			SRO	T3 BWR EXAMINATION OUTLINE	FORM ES-401-
KA	NAME / SAFETY FUNCTION:		IR	K1 K2 K3 K4 K5 K6 A1 A2 A3 A4 G	TOPIC:
		RO	SRC)	
G2.1.17	Conduct of operations	3.9	4.0		Ability to make accurate, clear and concise verbal reports.
G2.2.14	Equipment Control	3.9	4.3		Knowledge of the process for controlling equipment configuration or status
G2.2.4	Equipment Control	3.6	3.6		(multi-unit) Ability to explain the variations in control board layouts, systems, instrumentation and procedural actions between units at a facility.
G2.3.12	Radiation Control	3.2	3.7		Knowledge of radiological safety principles pertaining to licensed operator duties
G2.3.15	Radiation Control	2.9	3.1		Knowledge of radiation monitoring systems
32.4.31	Emergency Procedures/Plans	4.2	4.1		Knowledge of annunciators alarms, indications or response procedures
32.4.37	Emergency Procedures/Plans	3.0	4.1		Knowledge of the lines of authority during implamentation of an emergency plan.

ES-301

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Control Room/In-Plant Systems Outline

Form ES-301-2

Facility: Browns Ferry NPP	Date of Examination:	5/7/2012	
Exam Level: RO/SROI/ SROU	Operating Test No.:	1205	
Control Room Systems [@] (8 for RO); (7 for SR	0-I); (2 or 3 for SRO-L	J, including 1 E	SF)
System / JPM Title		Type Code*	Safety Function
a. Recirc Pump trip, power oscillations and los insert rods require manual reactor scram	ss of ability to	A, N, S	1
b. Remove RFPT 'A' from service		A, N, S	2
c. Rapid Depressurization with Turbine Bypass V Appendix-11H	Valves EOI	L, N, S	3
d. EHC Auto Cooldown		L, N, S	4
e. EOI Appendix-13 Emergency Venting Prim	ary Containment	A, EN, P, S	5
f. OI-82 Parallel D/G with Off-Site Power		D, S	6
g. Off-Gas Post-Treatment Radiation HI-HI-HI		A, D, L, S	9
h. Returning an IRM to service from Bypass		D, L, S	7(RO only)
In-Plant Systems [@] (3 for RO); (3 for SRO-I); (3	or 2 for SRO-U)		
i. Vent and Re-pressurize the Scram Pilot Air Hea	ader	D, E, R	1
j. 3-AOI-100-2, Attachment 3, Part A- Start R the Control Room	CIC from outside	D, E, R	7
k. 0-SSI-2-1, Attachment 2		A, D, E	8
All RO and SRO-I control room (and in-plant) functions; all 5 SRO-U systems must serve di may overlap those tested in the control room.	fferent safety functions; in	t and serve differe n-plant systems a	nt safety nd functions
* Type Codes	Criteria fo	r RO / SRO-I / SRC)-U

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Control Room/In-Plant Systems Outline

Form ES-301-2

A)Iternate path	4-6/2-3 (5)(5)(3)
C)ontrol room	
D)irect from bank	<u>< 9/< 8/≤4</u> (6)(5)(2)
E)mergency or abnormal in-plant	≥ 1/≥ 1/≥1 (3)(3)(2)
EN)gineered safety feature	- / - /≥1 (control room system) (1)(1)(1)
L)ow-Power / Shutdown	<u>≥1/≥1/≥1 (4)(3)(1)</u>
N)ew or (M)odified from bank including 1(A)	≥ 2/≥ 2/≥1 (4)(4)(2)
P)revious 2 exams	$\leq 3/\leq 3/\leq 2$ (randomly selected) (1)(1)(1)
R)CA	<u>≥1/≥1/≥1 (2)(2)(1)</u>
S)imulator	(8)(7)(3)

Control Room/In-Plant Systems JPM Narrative

Control Room Systems:

- a. Recirc Pump trip, power oscillations and loss of RPIS require manual reactor scram
 - Alternate Path/New/Simulator
 - 2/3-AOI-68-1A, Recirc Pump Trip/Core Flow Decrease OPRMs Operable.
 - 295001 Partial or Complete Loss of Forced Core Flow Circulation, AA2.02 Ability to determine and/or interpret the following as they apply to PARTIAL OR COMPLETE LOSS OF FORCED CORE FLOW CIRCULATION : Neutron monitoring. IMPORTANCE: RO 3.1 SRO 3.2
 - The operator will perform the actions of 2/3-AOI-68-1A, Recirc Pump Trip/Core Flow Decrease OPRMs Operable. This will direct the operator to insert control rods to less than 92.5% load line to suppress power oscillations. After the operator has inserted at least one control rod, a failure of Rod Control Power will occur. Power oscillations will ramp in and the following alarm will come in, "OPRM Pre-Trip Condition," 2-9-5A window 18. With power oscillations present, the ARP will direct the operator to manually scram the reactor in these conditions.

b. Remove RFPT 'A' from Service (Unit 2 or 3)

- Alternate Path/New/Simulator
- 2/3-OI-3, Reactor Feedwater System
- 259001 Reactor Feedwater System A4.04 Ability to manually operate and/or monitor in the Control Room: System valves. IMPORTANCE: RO 3.1 SRO 2.9
- Operator will be directed remove the 2A/3A RFPT from service in accordance with 2/3-OI-3, Reactor Feedwater System. The operator will identify that the RFPT A discharge check valve fails to close and take actions in accordance with 2/3-OI-3

c. Rapid Depressurization with Turbine Bypass Valves, EOI Appendix-11H (Unit 2 or 3)

- Low Power/New/Simulator
- 2/3-EOI Appendix-11H, Alternate RPV Pressure Control Systems Main Condenser
- 241000 Reactor/Turbine Pressure Regulating System A4.06 Ability to manually operate and/or monitor in the control room: Bypass valves IMPORTANCE: RO 3.9 SRO 3.9
- Operator is directed to perform operations necessary to establish the Main Condenser as an Alternate RPV pressure control system for Rapid Depressurization as directed by 2-EOI Appendix-11H

d. EHC Auto Cooldown (Unit 2 or 3)

- Low-Power/New/Simulator
- 2/3-OI-47, Turbine Generator System
- 239001 Main and Reheat Steam System A4.09 Ability to manually operate and/or monitor in the control room: Reactor Pressure IMPORTANCE: RO 3.9 SRO 3.3

ILT 1205

Control Room/In-Plant Systems JPM Narrative

- Operator is directed to commence an Auto Cooldown with EHC in accordance with 2/3-OI-47, Turbine Generator System. Operator must utilize the Human Machine Interface (HMI) of the EHC system to commence a cooldown as well as adjust final target pressure.
- e. EOI Appendix-13 Emergency Venting Primary Containment (Unit 2 or 3)
 - Alternate Path /ENgineered Safety Feature /Previous /Simulator
 - 2/3-EOI Appendix-13, Emergency Venting Primary Containment
 - 295024 High Drywell Pressure EA2.01 Ability to determine and/or interpret the following as they apply to HIGH DRYWELL PRESSURE: Drywell Pressure IMPORTANCE: RO 4.2 SRO 4.4
 - Operator is directed to emergency vent Primary Containment to restore and maintain Drywell Pressure below 55 psig as directed by 2/3-EOI Appendix-13, Emergency Venting Primary Containment. Emergency Venting of the Suppression Chamber through the Hardened Wetwell Vents will be unsuccessful and the operator will vent the Drywell to Secondary Containment via Primary Containment vent duct failure.

f. OI-82 Parallel D/G with Off-Site Power Source (Unit 2 or 3)

- Direct from bank/Simulator
- 0/3-OI-82, Standby Diesel Generator System
- 264000 Emergency Generators (Diesel/Jet) A4.04 Ability to manually operate and/or monitor in the control room: Manual start, loading, and stopping of emergency generator IMPORTANCE: RO 3.7 SRO 3.7
- Operator will perform actions necessary to parallel the A/3A Diesel Generator (DG) with the Off-Site power source in accordance with 0/3-OI-82, Standby Diesel Generator System.

g. Off-Gas Post-Treatment Radiation HI-HI-HI (Unit 2 or 3)

- Alternate Path/Direct from bank/Low power/Simulator
- 2/3-ARP-9-4C, Window 35 and 2/3-AOI-66-2, Offgas Post-Treatment Radiation HI-HI-HI
- 271000 Offgas System A2.04 Ability to (a) predict the impacts of the following on the OFFGAS SYSTEM; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or operations: Offgas system high radiation IMPORTANCE: RO 3.5 SRO 3.8
- Operator is directed to respond to Offgas Post-Treatment Radiation HI-HI-HI alarm in accordance with 2/3-ARP-9-4C Window 35. Operator will determine that the Offgas Isolation valve 2/3-FCV-66-28 failed to close; operator will close the valve then refer to 2/3-AOI-66-2, Offgas Post-Treatment Radiation HI-HI-HI, and perform the actions of 2/3-AOI-66-2 insert a core flow runback and reactor scram. Operator will then shut the MSIVs.

ILT 1205

h. Returning an IRM to service from Bypass (Unit 2 or 3) (RO only)

- Direct from bank/Low-Power/Simulator
- 2/3-OI-92A Intermediate Range Monitors
- 215003 Intermediate Range Monitor System A2.02 Ability to (a) predict the impacts of the following on the IRM System; and (b) based on those predictions, use procedures to correct, control, or mitigate the consequences of those abnormal conditions or operations: IRM inop condition IMPORTANCE: RO 3.5 SRO 3.7
- Operator is directed to return a bypassed IRM to service in accordance with 2/3-OI-92A, Intermediate Range Monitors. Operator must fully insert the IRM and then range the IRM to the proper scale to prevent a half scram when un-bypassed.

In-Plant Systems:

- i. Vent and Re-pressurize the Scram Pilot Air Header
 - Direct from bank/Emergency or Abnormal In-Plant/RCA Entry
 - 1-EOI Appendix-1B, Venting and Re-pressurizing the Scram Pilot Air Header
 - 295015 Incomplete SCRAM AA1.01 Ability to operate and/or monitor the following as they apply to INCOMPLETE SCRAM: CRD hydraulics IMPORTANCE: RO 3.8 SRO 3.9
 - Operator will simulate the component manipulations required to vent and subsequently re-pressurize the Scram Pilot Air Header as directed by 1-EOI Appendix 1B, Venting and Repressurizing the Scram Pilot Air Header.

j. 3-AOI-100-2, Attachment 3, Part A- Start RCIC from outside the Control Room

- Direct from bank/Emergency or Abnormal In-Plant/RCA Entry
- 3-AOI-100-2, Control Room Abandonment, Attachment 3, Part A
- 295016 Control Room Abandonment AA1.07 Ability to operate and/or monitor the following as they apply to CONTROL ROOM ABANDONMENT: Control room/local control transfer mechanisms IMPORTANCE: RO 4.2 SRO 4.3
- Operator will simulate performing operations necessary to align RCIC from outside the Control Room as directed by 3-AOI-100-2, Control Room Abandonment.

k. 0-SSI-2-1, Attachment 2

- Alternate Path/Direct from bank/Emergency or Abnormal In-Plant
- 0-SSI-2-1, Unit 2 Reactor Building Fire EL' 519 through 565 West of Column Line R11
- 600000 Plant Fire on Site AA2.16 Ability to determine and interpret the following as the apply to PLANT FIRE ON SITE: Vital equipment and control systems to be maintained and operated during a fire IMPORTANCE: RO 3.0 SRO 3.5

ILT 1205

Control Room/In-Plant Systems JPM Narrative

• Time Critical JPM for an operator to simulate performing designated steps of an SSI as directed by the Unit 2 Unit Supervisor and 0-SSI-2-1. Operator will have to simulate starting the 2D RHR pump using the manual breaker close pushbutton after the breaker fails to close.