		of Examination:		
Control Room Systems <sup>@</sup> (8 for RO); (7 for SRO-I);	(2 or 3 for SRO-U,	including 1 ESF)		
System / JPM Title		Type Code*	Safety Function	
S1. ANO-2-JPM-NRC-SIT09 006 A4.02 RO-4.0/SRO-3.8 Isolate SITs following SIAS with SIAS reset		A/D/EN/L/S	2 Inventory Control	
S2. ANO-2-JPM-NRC-ELEC05 062 A4.01 RO3.3/SRO3.1 Transfer Auxiliaries from SU#2 to SU#3 for 2A-1		N/S	6 Electrical	
S3. ANO-2-JPM-NRC-SDBC2 041 A4.05 RO-3.1/SRO-3.3 Manually Operate SDBCS valves		A/L/N/S	4 Heat Removal	
S4. ANO-2-JPM-NRC-CCW01 008 A4.01 RO-3.3/SRO-3.1 Secure CCW system using EOP		D/L/S	8 Plant service systems	
S5. ANO-2-JPM-NRC-CNMT2 103 A3.01 RO-3.9/SRO-4.2 Drain the containment sump		A/D/P/S	5 Containment Integrity	
S6. ANO-2-JPM-NRC-CEA01 001 A4.03 RO-4.0/SRO-3.7 Perform a CEA exercise		D/P/S	1 Reactivity control	
S7. ANO-2-JPM-NRC-CPC06 012 A4.02 RO-3.3/SRO-3.4 Place #1 CEAC to inop on "B" CPC		D/S	7 Instrumentation	
S8. ANO-2-JPM-NRC-AOP3 010 A2.02 RO-3.9/SRO-3.9 Isolate failed PZR spray valve		A/D/P/S	3 Pressure Control	
In-Plant Systems <sup>®</sup> (3 for RO); (3 for SRO-I); (3 or 2	2 for SRO-U)			
P1. ANO-2-JPM-NRC-EDDCB 064 A4.01 RO-4.0/SRO-4.3 Start EDG 2K-4B without DC		A/D/E/L	6 Electrical	
P2. ANO-2-JPM-NRC-MGSU 001 A4.08 RO-3.7/SRO-3.4 Start the First MG set		D/L	1 Reactivity control	
P3. ANO-2-JPM-NRC-FPEM1 033 A2.03 RO-3.1/SRO-3.5 Perform emergency SW make up to SFP		D/E/R	8 Plant service systems	
@ All RO and SRO-I control room (and in-plant) systems must be different and serve different safety functions; all 5 SRO-U systems must serve different safety functions; in-plant systems and functions may overlap those tested in the control room.				
* Type Codes Criteria for RO / SRO-I / SRO-U			RO-U	
(A)Iternate path (C)ontrol room (D)irect from bank (E)mergency or abnormal in-plant (EN)gineered safety feature (L)ow-Power / Shutdown (N)ew or (M)odified from bank including 1(A) (P)revious 2 exams (R)CA (S)imulator	$4-6/4-6/2-3$ $\leq 9/\leq 8/\leq 4$ $\geq 1/\geq 1/\geq 1$ $-/-/\geq 1$ (control room system) $\geq 1/\geq 1/\geq 1$ $\leq 2/\geq 2/\geq 1$ $\leq 3/\leq 3/\leq 2$ (randomly selected) $\geq 1/\geq 1/\geq 1$			

Facility: Arkansas Nuclear One Examination Level: RO X	SRO	Date of Examination: 08/27/2012 Operating Test Number: 2012-1	
Administrative Topic (see Note)	Type Code*	Describe activity to be performed	
A1. Conduct of Operations 2.1.25 RO (3.9)	D/P/R	Determine any limits for CEA positions using the COLR PDIL  ANO-2-JPM-NRC-ADMIN-PDIL	
A2. Conduct of Operations 2.1.23 RO (4.3)	D/R	Calculate Time to Boil using computer program  ANO-2-JPM-NRC-ADMIN-TTBCRO	
A3. Equipment Control 2.2.42 RO (3.9)	D/R	Review 2P-89B surveillance to determine operability ANO-2-JPM-NRC-ADMIN-2P89BSURV	
A4. Radiation Control 2.3.4 RO (3.2)	N/R	Calculate expected dose for Re-entry during emergency and if applicable limits will be exceeded  ANO-2-JPM-NRC-ADMIN-EMGRESP	
Emergency Procedures/Plan			
NOTE: All items (5 total) are required for SROs. RO applicants require only 4 items unless they are retaking only the administrative topics, when all 5 are required.			
* Type Codes & Criteria:	(D)irect from (N)ew or (M	om, (S)imulator, or Class(R)oom  n bank (≤ 3 for ROs; ≤ 4 for SROs & RO retakes) )odified from bank (≥ 1) exams (≤ 1; randomly selected)	

Facility: Arkansas Nuclear One Examination Level: RO	Unit 2 SRO X	Date of Examination: 08/27/2012 Operating Test Number: 2012-1	
Administrative Topic (see Note)	Type Code*	Describe activity to be performed	
A5. Conduct of Operations 2.1.32 SRO (4.0)	D/R	Review performing transferring Unit Auxiliary Calculation and determine if it they can be transferred.  ANO-2-JPM-NRC-ADMIN-SU2LOADSRO	
A6. Conduct of Operations 2.1.23 SRO (4.4)	D/R	Review Time to Boil Calculation.  ANO-2-JPM-NRC-AMDIN-TTBCSRO	
A7. Equipment Control 2.2.14 SRO (4.3)	D/R	Supervisory review of maintenance activities for configuration control  ANO-2-JPM-NRC-ADMIN-MAINT	
A8. Radiation Control 2.3.4 SRO (3.7)	N/R	Calculate expected dose for Re-entry during an emergency and determine if entry is allowed.  ANO-2-JPM-NRC-ADMIN-EMGRESPSRO	
A9. Emergency Procedures/Plan 2.4.41 SRO (4.6)	D/P/R	Determine Emergency Action Level for given conditions.  ANO-2-JPM-NRC-ADMIN-EAL11	
NOTE: All items (5 total) are required for SROs. RO applicants require only 4 items unless they are retaking only the administrative topics, when all 5 are required.			
* Type Codes & Criteria: (C)ontrol room, (S)imulator, or Class(R)oom (D)irect from bank (≤ 3 for ROs; ≤ 4 for SROs & RO retakes) (N)ew or (M)odified from bank (≥ 1) (P)revious 2 exams (≤ 1; randomly selected)			

## ANO-2-JPM-NRC-ADMIN-PDIL

## Page 1 of 5

## ADMINISTRATIVE JOB PERFORMANCE MEASURE

UNIT: <u>2</u>	REV #: 002	<u> </u>	DATE:	
SYSTEM/DUTY AREA:	Conduct of Operation	ns A1		
TASK: Determine limits	for CEA positions usi	ng the COLR PDIL.		
JTA#: ANO-2-RO-OPRO	OC-NORM-62			
KA VALUE RO:	3.9 SRO:	4.2 KA REFE	ERENCE:	2.1.25
APPROVED FOR ADMINIS	TRATION TO: RO:	χ SRO:		
TASK LOCATION:	INSIDE CR:	OUTSIDE CR:	вотн:	X
SUGGESTED TESTING EN	IVIRONMENT AND MET	HOD (PERFORM OR	SIMULATE):	
PLANT SITE:	SIMULATOR:	Perform	CLASSROOM:	Perform
POSITION EVALUATED:	RO:	SRO:		
ACTUAL TESTING ENVIRO	ONMENT: SIMULATO	R: PLANT S	SITE: CLA	ASSROOM:
TESTING METHOD: SI	MULATE: F	PERFORM:		
APPROXIMATE COMPLET	TON TIME IN MINUTES:	15 Minutes	<u> </u>	
REFERENCE(S): OP-210	)2.004, Unit 2 Tech Spe	cs, and Unit 2 COLR	ł .	
EXAMINEE'S NAME:		Ba	dge #:	
EVALUATOR'S NAME:				
THE EXAMINEE'S PERFOR		TED AGAINST THE S	STANDARDS CON	TAINED IN THIS
SATISFACTORY:	UNSATISFAC	TORY:		
PERFORMANCE CHECKL	ST COMMENTS:			
Start Time Sto	op Time To	otal Time		
SIGNED:	·	DATE:	<del></del>	

SIGNATURE INDICATES THIS JPM HAS BEEN COMPARED TO ITS APPLICABLE PROCEDURE BY A QUALIFIED INDIVIDUAL (NOT THE EXAMINEE) AND IS CURRENT WITH THAT REVISION.

#### THE EXAMINER SHALL REVIEW THE FOLLOWING WITH THE EXAMINEE:

The examiner shall ensure that the examinee has been briefed on NUREG 1021 Appendix E.

#### JPM INITIAL TASK CONDITIONS:

- The plant has been at 100% since January 1<sup>st</sup> of this year with CEAs at the program insertion limit.
- The following conditions were established on 8-20-12 at 12:00 which was 250 EFPD:
  - Plant power was lowered to 70% power due to 'A' Main Feedwater pump outage for bearing replacement.
  - Group P CEAs are full out.
  - Group 6 CEAs are being used for ASI control and are 75" withdrawn.
  - COLSS ASI is -.07.
  - #1 CEAC is inoperable for Surveillance testing.
  - #2 CEAC is operable.

**TASK STANDARD**: Determine that group 6 CEAs are inserted into the long term steady state insertion limits and that the 5 EFPD limit has not been exceeded.

TASK PERFORMANCE AIDS: OP-2102.004, Unit 2 Tech Specs, and Unit 2 COLR

#### **INITIATING CUE:**

It is now 8-23-12 at 02:00 (251.8 EFPD) and the SM/CRS directs "Using OP-2102.004A for the conditions stated above determine applicable CEA insertion limits (if any) and if any time limits for the current configuration have been exceeded."

	PERFORMANCE CHECKLIST	STANDARD	(Circle One)		
	Using OP-2102.004     determines that the COLR     must be referenced to     determine if any limits are     applicable.	Examinee correctly derived from OP-2102.004 that the COLR must be referenced to determine if any limits apply.	N/A SAT UNSAT		
(C)	Using cycle 22 COLR figure 3 determines that Group 6 CEA's are inserted into the long term steady state insertion limit of operation.	Examinee correctly derived from graph based on given values that CEAs are inserted into the long term steady state insertion limits (LTSSIL).	N/A SAT UNSAT		
(C)	3. Using Tech Spec 3.1.3.6 or OP-2102.004 determines the limits for present CEA configuration.	Examinee determines that the current CEA configuration is limited to $\leq$ 5 EFPD per 30 EFPD and $\leq$ 14 EFPD per calendar year.	N/A SAT UNSAT		
(C)	4. Using Tech Spec 3.1.3.6 or OP-2102.004 determines if any time limits have been exceeded for LTSSIL.	Examinee determines that 1.8 EFPD is less than 5 EFPD and no limits have been exceeded	N/A SAT UNSAT		
	END				

## **EXAMINER'S COPY**

#### JPM INITIAL TASK CONDITIONS:

- The plant has been at 100% since January 1<sup>st</sup> of this year with CEAs at the program insertion limit.
- The following conditions were established on 8-20-12 at 12:00 which was 250 EFPD:
  - Plant power was lowered to 70% power due to 'A' Main Feedwater pump outage for bearing replacement.
  - Group P CEAs are full out.
  - Group 6 CEAs are being used for ASI control and are 75" withdrawn.
  - COLSS ASI is -.07.
  - #1 CEAC is inoperable for Surveillance testing.
  - #2 CEAC is operable.

#### **INITIATING CUE:**

It is now 8-23-12 at 02:00 (251.8 EFPD) and the SM/CRS directs "Using OP-2102.004A for the conditions stated above determine applicable CEA insertion limits (if any) and if any time limits for the current configuration have been exceeded."

Applicable CEA insertion limits (If any) _	Long Term Steady State Insertion Li	mits
Have any time limits for the current confi	iguration been exceeded?	Yes No Circle One

Circle One

### **ADMINISTRATIVE JOB PERFORMANCE MEASURE**

## **EXAMINEE'S COPY**

#### JPM INITIAL TASK CONDITIONS:

- The plant has been at 100% since January 1<sup>st</sup> of this year with CEAs at the program insertion limit.
- The following conditions were established on 8-20-12 at 12:00 which was 250 EFPD:
  - Plant power was lowered to 70% power due to 'A' Main Feedwater pump outage for bearing replacement.
  - Group P CEAs are full out.
  - Group 6 CEAs are being used for ASI control and are 75" withdrawn.
  - COLSS ASI is -.07.
  - #1 CEAC is inoperable for Surveillance testing.
  - #2 CEAC is operable.

#### **INITIATING CUE:**

the conditions stated above determine applicable CEA insertion limits time limits for the current configuration have been exceeded."	(if any) and if any
Applicable CEA insertion limits (If any)	
Have any time limits for the current configuration been exceeded?	Yes / No

It is now 8-23-12 at 02:00 (251.8 EFPD) and the SM/CRS directs "Using OP-2102.004A for

# ANO-2-JPM-NRC-ADMIN-TTBCRO

PAGE 1 OF 5

## ADMINISTRATIVE JOB PERFORMANCE MEASURE

UNIT: 2		REV #: 0	002	DA	TE:	
	TY AREA: Conduc	-				
TASK: Ca	Iculate Time to Boil u	sing compu	ter program			
JTA#: AN	IO2-RO-NORM4					
KA VALUE	RO: 4.3	_ SRO:	4.4	KA REFEREN	CE:	2.1.23
APPROVED	FOR ADMINISTRATION	ON TO: RO	D: X	SRO:		
TASK LOCA	TION: INSIDE	CR:	OUTSIE	DE CR:	вотн:	<u>x</u>
SUGGESTE	D TESTING ENVIRON	MENT AND	METHOD (PE	ERFORM OR SI	MULATE):	
PLANT SITE				rform Cla :	ssroom Per	form
POSITION E	VALUATED: RO:	X	SRO:			
ACTUAL TES	TING ENVIRONMENT:	SIMULATOR	R:	PLANT SITE:	Classro	om:
TESTING ME	ETHOD: SIMULAT	E:	PERFORM	:		
APPROXIMA	ATE COMPLETION TIME	ME IN MINUT	ES: <u>15</u>	Minutes		
REFERENCE	E(S): 1015.008 Attac	hment E				
EXAMINEE'S	S NAME:			KCN#:		
EVALUATOR	R'S NAME:					
	NEE'S PERFORMANC ND IS DETERMINED 1		UATED AG	AINST THE STA	NDARDS CO	NTAINED IN
SATISFACT	ORY:	UNSATISF	ACTORY: _			
PERFORMA	NCE CHECKLIST COI	MMENTS:				
Start Time	Stop Time		Total Time			
SIGNED:			DATE:			

SIGNATURE INDICATES THIS JPM HAS BEEN COMPARED TO ITS APPLICABLE PROCEDURE BY A QUALIFIED INDIVIDUAL (NOT THE EXAMINEE) AND IS CURRENT WITH THAT REVISION.

## PAGE 2 OF 5

THE EXAMINER SHALL REVIEW THE FOLLOWING WITH THE EXAMINEE:
The examiner shall ensure that the examinee has been briefed on NUREG 1021 Appendix E.
JPM INITIAL TASK CONDITIONS:
Plant shutdown for repair of Steam Generator Tube Leak. Making preparations for draining the RCS to
24 inches above the bottom of the hotleg to install nozzle dams. PZR level is 40%.
Time after shutdown = 50 hours; one (1) PZR code safety valve is removed; PZR manway is installed;
ECCS vent valves are de-energized OPEN; NO RCP seal work inprogress; NO RCS cold or hot leg openings;
RCS Temperature is 105°F. Nozzle Dams are not installed.
TASK STANDARD:
Time to boil calculation(TTB) and time to core uncovery (TTCU)calculated using the computer program and
Values calculated are TTB = 15 min. to 16 min. and TTCU=2hrs to 2hrs 10 min.
TASK PERFORMANCE AIDS:
1015.008 attachment E, Computer operational with the current revision of LOSDC2 installed
(SP-94-C-0001-01, Rev. 13), set up shortcut to program on the desktop.
SIMULATOR SETUP:
NA
EXAMINER'S NOTES:

#### **INITIATING CUE:**

CRS/SM directs: "Perform 1015.008 Attachment E steps 1 through 4, Time to boiling/Core Uncovery Estimate, for the projected level using data given."

	PERF	ORMANCE CHECKLIST	STANDARDS	(Circle One)
	1.	Start a time to boil (TTB) calculation using LOSDC2 program.	Computer program started	N/A SAT UNSAT
	2.	Time since shutdown: 2days 2 hours	Recorded time 50 hours	N/A SAT UNSAT
	3.	Decay Heat Produced 12,765.59 BTU/Sec	Recorded decay heat 12,765.59 BTU/Sec	N/A SAT UNSAT
(C)	4.	Time until coolant boils: 15 min	Recorded to boil 14.5 to 15.5 min.	N/A SAT UNSAT
(C)	5.	Time until core uncovery: 2 hours 5 min 15 sec	Recorded time to core uncovery 2 hours to 2 hours 10 min.	N/A SAT UNSAT
	6.	Makeup rate: 86 gpm	Recorded makeup rate 86 gpm	N/A SAT UNSAT
	4.	Heat Up rate: 7.13 °F/min	Recorded heat up rate 7.13 °F/min	N/A SAT UNSAT
	5.	RCS eq. Press: 47.1 psia	Recorded RCS eq. Press 47.1 psia	N/A SAT UNSAT
	6.	Time until 10 F from bulk boiling: 13.6 min	Recorded time to 10° from bulk boiling 13.6 min	N/A SAT UNSAT
EXAMINER'S NOTE:				
Prompt the Examinee that the file should not be printed or saved.				
			END	

#### **EXAMINER'S COPY**

#### JPM INITIAL TASK CONDITIONS:

Given the following Plant conditions:

- Plant shutdown for repair of Steam Generator Tube Leak
- Making preparations for draining the RCS to 24 inches above the bottom of the hot leg to install nozzle dams
- This is considered the initial calculation
- PZR level is 40% and open to atmosphere
- Time after shutdown = 50 hours
- One (1) PZR code safety valve is removed
- PZR manway is installed
- ECCS vent valves are de-energized OPEN
- NO RCP seal work in progress
- NO RCS cold or hot leg openings
- RCS Temperature is 105°F
- Nozzle Dams are not installed

#### **INITIATING CUE:**

Record calculations below:

CRS/SM directs: "Perform 1015.008 Attachment E steps 1 through 4, Time to boiling/Core Uncovery Estimate, for the projected level using data given." (Do not print data or save file.)

Time since shutdown:	
Decay Heat Produced:	_
Time until coolant boils:	_
Time until core uncovery:	
Makeup rate:	
Heat Up rate:	
RCS eq. Press:	
Time until 10 °F from bulk boiling:	

#### **EXAMINEE'S COPY**

#### JPM INITIAL TASK CONDITIONS:

Given the following Plant conditions:

- Plant shutdown for repair of Steam Generator Tube Leak
- Making preparations for draining the RCS to 24 inches above the bottom of the hot leg to install nozzle dams
- This is considered the initial calculation
- PZR level is 40% and open to atmosphere
- Time after shutdown = 50 hours
- One (1) PZR code safety valve is removed
- PZR manway is installed
- ECCS vent valves are de-energized OPEN
- NO RCP seal work in progress
- NO RCS cold or hot leg openings
- RCS Temperature is 105°F
- Nozzle Dams are not installed

#### **INITIATING CUE:**

CRS/SM directs: "Perform 1015.008 Attachment E steps 1 through 4, Time to boiling/Core Uncovery Estimate, for the projected level using data given." (Do not print data or save file.)

#### Record calculations below:

Time since shutdown:	
Decay Heat Produced:	<u> </u>
Time until coolant boils:	
Time until core uncovery:	
Makeup rate:	
Heat Up rate:	
RCS eq. Press:	
Time until 10 °F from bulk boiling:	

UNIT: 2 REV #: 002
SYSTEM/DUTY AREA: _ Equipment Control A3
TASK: Review 2P-89B surveillance to determine operability
JTA#: ANO2ROHPSISURV16
KA VALUE RO: 3.7 SRO: 4.1 KA REFERENCE: 2.2.12
APPROVED FOR ADMINISTRATION TO: RO: X SRO:
TASK LOCATION: INSIDE CR: OUTSIDE CR: BOTH: X
SUGGESTED TESTING ENVIRONMENT AND METHOD (PERFORM OR SIMULATE):
PLANT SITE:         SIMULATOR:         LA           Perform         B:
POSITION EVALUATED: RO: SRO:
ACTUAL TESTING ENVIRONMENT: SIMULATOR: PLANT SITE: LAB:
TESTING METHOD: SIMULATE: PERFORM:
APPROXIMATE COMPLETION TIME IN MINUTES: 15 Minutes
REFERENCE(S): OP 2104.039 Supplement 2
EXAMINEE'S NAME: KCN#:
EVALUATOR'S NAME:
THE EXAMINEE'S PERFORMANCE WAS EVALUATED AGAINST THE STANDARDS CONTAINED IN THIS JPM AND IS DETERMINED TO BE:
SATISFACTORY: UNSATISFACTORY:
SATISFACTORY: UNSATISFACTORY: PERFORMANCE CHECKLIST COMMENTS:

SIGNATURE INDICATES THIS JPM HAS BEEN COMPARED TO ITS APPLICABLE PROCEDURE BY A QUALIFIED INDIVIDUAL (NOT THE EXAMINEE) AND IS CURRENT WITH THAT REVISION.

THE EXAMINER SHALL REVIEW THE FOLLOWING WITH THE EXAMINEE:
The examiner shall ensure that the examinee has been briefed on NUREG 1021 Appendix E.
JPM INITIAL TASK CONDITIONS:
Plant is at 100% power and all ESF equipment is in normal alignment.
2P89C is aligned to the GREEN side.
The 2P-89B Quarterly test has been completed using OP 2104.039 Supplement 2.
TASK STANDARD:
Determine the following for 2P-89B surveillance results:
Pump ΔP is out of LIMITING RANGE FOR OPERABILITY.
Correct the calculation of pump $\Delta P$ .
Determine a Condition report must be initiated due to mechanical seal leakage.
Determine that 2P-89B in inoperable.
TASK PERFORMANCE AIDS:
Marked-up copy of Supplement 2 2P-89B quarterly test
SIMULATOR SETUP:
NA
EXAMINER'S NOTES:

#### **Initiating CUE:**

The Control Room Supervisor directs determination of the operability of the 'B' HPSI pump, 2P89B, by reviewing the completed surveillance data OP 2104.039 Supplement 2 step 3.1 through 3.3. Complete the remainder of section 3 through step 3.6 and correct any errors identified. If any actions are required then note them below.

	PE	RFORMANCECHECKLIST	<u>STANDARD</u>	CIRCLE ONE
NOTE	: Prov	ride a marked-up copy of Supplemer	nt 2, 2P-89B quarterly test	
	1.	Perform review of the surveillance procedure results.	Reviews the surveillance procedure provided.	N/A
(C)	2.	Pump ΔP is out of LIMITING RANGE FOR OPERABILITY.	Pump D/P is out of LIMITING RANGE FOR OPERABILITY.  'NO' should be circled instead of 'YES' on far right column.	N/A SAT UNSAT
(C)	3.	Calculation for determining lower pump $\Delta P$ limit is in error.  This calculation is below the table in supplement 2 section 3 and recorded in the table on the line for pump $\Delta P$ .	Corrects the calculation for determining lower pump D/P limit to between 1380 - 1382 psid.	N/A SAT UNSAT
(C)	4.	Pump inboard mechanical seal leakage is greater than 17 drops per minute.	Notes that a CR should be initiated due to the pump inboard mechanical seal leakage.	N/A SAT UNSAT
(C)	5.	High Pressure Safety injection pump inoperability	Notes that 2P-89B must be declared inoperable.	N/A SAT UNSAT

#### **EXAMINER's COPY**

## **INITIAL PLANT CONDITIONS**

Plant is at 100% power and all ESF equipment is in normal alignment. 2P89C is aligned to the GREEN side. The 2P-89B Quarterly test has been completed using OP 2104.039 Supplement 2.

## **Initiating CUE:**

The Control Room Supervisor directs determination of the operability of the 'B' HPSI pump, 2P89B, by reviewing the completed surveillance data OP 2104.039 Supplement 2 step 3.1 through 3.3. Complete the remainder of section 3 through step 3.6 <u>AND</u> correct any errors identified. If any actions are required then note them below.

#### **EXAMINEE's COPY**

## **INITIAL PLANT CONDITIONS**

Plant is at 100% power and all ESF equipment is in normal alignment. 2P89C is aligned to the GREEN side. The 2P-89B Quarterly test has been completed using OP 2104.039 Supplement 2.

## **Initiating CUE:**

The Control Room Supervisor directs determination of the operability of the 'B' HPSI pump, 2P89B, by reviewing the completed surveillance data OP 2104.039 Supplement 2 step 3.1 through 3.3. Complete the remainder of section 3 through step 3.6 <u>AND</u> correct any errors identified. If any actions are required then note them below.

PROC./WORK PLAN NO. PROCEDURE/WORK PLAN TITLE: PAGE: 123 of 244
2104.039 HPSI SYSTEM OPERATION CHANGE: 069

#### SUPPLEMENT 2

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#### 3.0 ACCEPTANCE CRITERIA

3.1 Record values observed during 2P-89B operation below AND compare with limiting range of values for operability.

TABLE 1					
TEST QUANTITY	INSTRUMENT (INCLUDE TEST INST)	MEASURED VALUE	ACCEPTABLE NORMAL RANGE	LIMITING RANGE FOR OPERABILITY	IS DATA IN LIMITING RANGE?
Running Suct. Pressure	2PI-5100 (local)	33.5 psig	≥ 8 psig	≥ 8 psig	YES NO
	2PI-5101 (local)	1389 psig	N/A	N/A	N/A
Discharge Pressure	2PI-5109 (2C16)	1379 psig	N/A	N/A	N/A
RWT Temperature	2TIS-5675 (2C17)	90 °F	40 to 110°F	N/A	N/A
Pump ΔP {4.3.1}	2PI-5101 - 2PI-5100	1355.5 psid	N/A	1360.81(1)to 1612.8 psid	YES NO
Mechanical Seal leakage (inboard)	N/A	18 DPM	≤17 DPM	N/A	N/A
Mechanical Seal leakage (outboard)	N/A	0 DPM	≤17 DPM	N/A	N/A
Motor Running Amps	Ammeter at 2A-406	ØA 34 Amps	N/A	N/A	N/A
(CR-1-96-0272-07)		ØB 33 Amps			
		ØC 35 Amps			

Note 1: For RWT temperature (T), minimum acceptable pump  $\Delta P$  is: 1372.9 + 20.41 (Instrument error) - [(T-40)/4] psid  $\text{Minimum } \Delta P = \textbf{1393.31} - [(\underline{90} - 40)/4] = \underline{1360.81} \text{ psid}$ 

3.2 Independently verify pump  $\Delta P$  calculation.

Performed By: \_\_\_\_\_Jimmy Reactor\_\_\_\_\_

PROC./WORK PLAN NO. PROCEDURE/WORK PLAN TITLE: PAGE:
2104.039 HPSI SYSTEM OPERATION CHANGE:

SUPPLEMENT 2

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3.3 Document observation of check valve stroke in Table 2.

TABLE 2						
CHECK	TEST	TEST CRITERIA	IS PROPER VALVE			
VALVE	DIRECTION		STROKE INDICATED?			
2SI-10C	Closed	2P-89C NOT rotating	YES NO N/A			
2SI-10B	Closed	2P-89B NOT rotating	YES NO N/A			

- 3.4  $\underline{\text{IF}}$  NO circled in Table 1 OR 2, THEN perform the following:
  - Declare affected component inoperable.
  - Refer to Tech Spec 3.5.2 OR 3.5.3.
  - Notify Shift Manager.
  - Initiate WR/WO as applicable.
- 3.5 IF mechanical seal leakage greater than 5 DPM, THEN initiate a Work Request.
- 3.6 <u>If</u> mechanical seal leakage greater than 17 DPM, THEN initiate a Condition Report

#### NOTE

If HPSI pump 2P-89A OR 2P-89C inoperable, then 2SI-10C/2SI-10B testing may be deferred until inoperable HPSI pump restored to operable as per IST Coordinator.

- 3.7 IF 2SI-10C AND/OR 2SI-10B NOT tested (N/A circled in Table 2), THEN perform the following:
  - Initiate notification to perform test during current quarter OR as directed by IST Coordinator.
  - Annotate on status board.
  - Annotate in Work Exceptions section of WR/WO.
- 3.8 Pump Data recorded in database AND reviewed by SRO.
- 3.9 IF this surveillance performed as PMT, THEN complete Unit 2 IST Data Collection (1015.0160).

Comments		
Performed By	Jimmy Reactor	DateTODAY
-	Sammy Reactor	

PROC./WORK PLAN NO. 2104.039		N NO.	PROCEDURE/WORK PLAN TITLE:  HPSI SYSTEM OPERATION	PAGE: CHANGE:	125 of 24 069		244
			SUPPLEMENT 2	PAGF	: 11	OF	11
4.0	SUPE	RVISO	R REVIEW AND ANALYSIS	11101		01	
	4.1		ll measured values recorded in ACCEPTANCE CRITERIA in specified LIMITING RANGE FOR OPERABILITY?	fall	YE	1 R	Ø
	4.2 Do all measured values recorded in ACCEPTANCE CRITERIA within ACCEPTABLE NORMAL RANGE? (N/A if all results out normal range also outside limiting range.)				YES	NO	NA
	4.3 IF NO answered to 4.1, THEN perform the following corrective actions:		•				
			Verify LCO Tracking Record initiated per Conduct of Operations (1015.001).				
		• 7	Verify Condition Report initiated.				
	4.4		O circled in 4.2, I perform the following corrective actions:				
		• 7	Verify WR/WO has been initiated.				
			Complete Surveillance Test Schedule Change Request (to double test frequency.	1000.0091	0)		
	Comme	nts					

4.5 Are all administrative requirements of this test satisfied?

Supervisor \_\_\_\_\_ Date \_\_\_\_

YES NO

#### **ANO-2JPM-NRC-EMGRESP**

PAGE 1 OF 5

## **ADMINISTRATIVE JOB PERFORMANCE MEASURE**

UNIT:	2	REV #:	001		DATE :	
		Radiation Contro			cy and if applicab	le limits will be
LP#:	ASLP-RO-RA	DP				
KA VAL	UE RO: _	3.2 SRO:		3.7 KA R	EFERENCE:	2.3.4
APPRO\	VED FOR ADMI	NISTRATION TO:	RO:	X SRO		
TASK L	OCATION:	INSIDE CR:		OUTSIDE CR:	ВОТН:	X
SUGGE	STED TESTING	ENVIRONMENT AN	D ME	THOD (PERFOR	M OR SIMULATE):	
PLANT S	SITE:	SIMULA	TOR:		Classroom:	Perform
POSITIO	ON EVALUATED	D: RO:		SRO:		
ACTUAL	TESTING ENVIR	ONMENT: SIMULA	TOR:	PLAN	T SITE: CI	assroom:
TESTING	G METHOD:	SIMULATE:	P	PERFORM:		
APPROX	XIMATE COMPL	ETION TIME IN MIN	IUTES	6: <u>15 Minute</u>	<u>es</u>	
REFERE	ENCE(S): 1903	3.033 and 1903.033A	form			
EXAMIN	EE'S NAME:			K	CN#:	
EVALUA	EVALUATOR'S NAME:					
THE EXAMINEE'S PERFORMANCE WAS EVALUATED AGAINST THE STANDARDS CONTAINED IN THIS JPM AND IS DETERMINED TO BE:						
SATISF	ACTORY:	UNSATI	SFAC	TORY:		
PERFORMANCE CHECKLIST COMMENTS:						
Start Time		Stop Time	To	otal Time		
SIGNED	:			DATE:		

SIGNATURE INDICATES THIS JPM HAS BEEN COMPARED TO ITS APPLICABLE PROCEDURE BY A QUALIFIED INDIVIDUAL (NOT THE EXAMINEE) AND IS CURRENT WITH THAT REVISION.

# ANO-2JPM-NRC-EMGRESP <u>ADMINISTRATIVE JOB PERFORMANCE MEASURE</u>

THE EXAMINER SHALL REVIEW THE FOLLOWING WITH THE EXAMINEE:
The examiner shall ensure that the examinee has been briefed on NUREG 1021 Appendix E.
JPM INITIAL TASK CONDITIONS:
The plant is tripped from 100% power.
300 gpm LOCA in progress.
Loss of Offsite power is in progress.
LOCA EOP is being implemented.
RDACS indicates an off site release in progress.
Whole body dose rates in area of work are 3.5 Rem/hr.
The SM and TSC director have not authorized exceeding 10CFR20 dose limits.
Site Area Emergency has been declared.
RP estimates that it will take approximately 65 minutes to complete emergency actions and stop the release to prevent exceeding RDACS General emergency criteria.
You have been selected to make the entry to stop the release.
Your ERIMS dose to date 943 mR.
TASK STANDARD:
Calculated estimated dose for the entry to be between 4443 and 4793 mR.
Determined that 10CFR20 dose limits are applicable (5R)
Determined that 10CFR20 limits would not be exceeded.
TASK PERFORMANCE AIDS:
1903.033 Protective Action Guidelines for Recue/Repair and Damage Control Teams.
SIMULATOR SETUP:
NA
EXAMINER'S NOTES:

# ANO-2JPM-NRC-EMGRESP <u>ADMINISTRATIVE JOB PERFORMANCE MEASURE</u>

#### **INITIATING CUE:**

SM directs: "Using 1903.033 Protective Action Guidelines for Rescue/Repair and Damage control Teams determine your estimated year to date total dose for repair and determine if entry is allowed".

	PERF	ORMANCE CHECKLIST	STANDARDS	(Circle One)		
(C)	1.	Determine applicable dose limits.	Using 1903.033 determines that administrative does limits do not apply and 10CFR20 limits do apply. (5 Rem)	N/A SAT UNSAT		
			Examiner note: the applicant may include a note about 200 mR allowance for exit but this is not required and is used for the EAD setpoint.			
(C)	2.	Calculate the Operator's estimated dose	Calculated the Operator's estimated dose to in the following range:  4443 to 4793 mR	N/A SAT UNSAT		
(C)	5.	Determine if the Operator will exceed dose limits for entry to stop the release.	Determined that Jimmy Operator will not exceed 10CFR20 dose limits.	N/A SAT UNSAT		
	END					

#### **EXAMINER'S COPY**

#### JPM INITIAL TASK CONDITIONS:

Given the following Plant conditions:

- The plant is tripped from 100% power.
- 300 gpm LOCA in progress.
- Loss of Offsite power is in progress.
- LOCA EOP is being implemented.
- RDACS indicates an off site release in progress.
- Whole body dose rates in area of work are 3.5 Rem/hr.
- The SM and TSC director have not authorized exceeding 10CFR20 dose limits.
- Site Area Emergency has been declared.
- RP estimates that it will take approximately 65 minutes to complete emergency actions and stop the release.
- You have been selected to make the entry to stop the release.
- Your ERIMS dose to date 943 mR.

#### **INITIATING CUE:**

SM directs: "Using 1903.033 Protective Action Guidelines for Rescue/Repair and Damage control Teams determine your estimated year to date total dose for repair and determine if entry is allowed".

Your estimated TEDE dose:	4443	<u> – 4793                                   </u>	mR
Applicable dose limits is:	5	rem	
Can you perform re-entry and co	omplete	the task v	vithin applicable dose limits? (circle one) YES/NO

#### **EXAMINEE'S COPY**

#### JPM INITIAL TASK CONDITIONS:

Given the following Plant conditions:

- The plant is tripped from 100% power.
- 300 gpm LOCA in progress.
- Loss of Offsite power is in progress.
- LOCA EOP is being implemented.
- RDACS indicates an off site release in progress.
- Whole body dose rates in area of work are 3.5 Rem/hr.
- The SM and TSC director have not authorized exceeding 10CFR20 dose limits.
- Site Area Emergency has been declared.
- RP estimates that it will take approximately 65 minutes to complete emergency actions and stop the release.
- You have been selected to make the entry to stop the release.
- Your ERIMS dose to date 943 mR.

#### **INITIATING CUE:**

SM directs: "Using 1903.033 Protective Action Guidelines for Rescue/Repair and Damage control Teams determine your estimated year to date total dose for repair and determine if entry is allowed".

Your estimated TEDE dose:	_ mR
Applicable dose limits is:	
Can you perform re-entry and complete the task within a	applicable dose limits? YES / NO

UNIT: 2 REV #: 002	DATE:
SYSTEM/DUTY AREA: Conduct of Operations	A5
TASK: Perform Transferring Unit Auxiliaries from	SU3 or Unit Aux to SU2
JTA#: ANO-2-RO-ELECD-NORM-026	
KA VALUE RO:3.8 SRO:4.0	KA REFERENCE: 2.1.32
APPROVED FOR ADMINISTRATION TO: RO:	SRO:X
TASK LOCATION: INSIDE CR: O	UTSIDE CR: BOTH:X
SUGGESTED TESTING ENVIRONMENT AND METHO	D (PERFORM OR SIMULATE):
PLANT SITE: Perform SIMULATOR:	Perform LAB:
POSITION EVALUATED: RO:	SRO:
ACTUAL TESTING ENVIRONMENT: SIMULATOR:	PLANT SITE: LAB:
TESTING METHOD: SIMULATE: PER	FORM:
APPROXIMATE COMPLETION TIME IN MINUTES:	15 Minutes
REFERENCE(S): OP 2107.001 Attachment P	
EXAMINEE'S NAME:	KCN:
EVALUATOR'S NAME:	
THE EXAMINEE'S PERFORMANCE WAS EVALUATED JPM AND IS DETERMINED TO BE:	D AGAINST THE STANDARDS CONTAINED IN THIS
SATISFACTORY: UNSATISFACTOR	RY:
PERFORMANCE CHECKLIST COMMENTS:	
PERFORMANCE CHECKLIST COMMENTS:	
PERFORMANCE CHECKLIST COMMENTS:  Start Time Stop Time Total	

SIGNATURE INDICATES THIS JPM HAS BEEN COMPARED TO ITS APPLICABLE PROCEDURE BY A QUALIFIED INDIVIDUAL (NOT THE EXAMINEE) AND IS CURRENT WITH THAT REVISION.

#### THE EXAMINER SHALL REVIEW THE FOLLOWING WITH THE EXAMINEE:

The examiner shall ensure that the examinee has been briefed on NUREG 1021 Appendix E.

#### JPM INITIAL TASK CONDITIONS:

Mode 4; 250 EFPD. All ESF systems are in normal alignment.

Startup Transformer #3 is supplying all AC electrical busses.

'A' and 'B' Condensate Pumps are running for post maintenance testing and 'C' and 'D' Condensate Pumps are in pull to lock.

'B' Circulating Water Pump is running and 'A' Circulating Water Pump is in pull to lock.

'B' and 'D' Reactor Coolant Pumps are running and 'A' and 'C' Reactor Coolant Pumps are secured.

Auto-transformer is out of service for the next 12 hours due to emergency forced outage, design engineering has completed an evaluation to allow SU<sup>#</sup>2 to be placed in service without the Auto-transformer being in service.

Unit 1 is at 100% and all busses energized from their Unit Auxiliary Transformer and with no equipment out of service.

Unit 1 SU<sup>#</sup>2 feeder to A1 (A111) is in normal after stop.

SU<sup>#</sup>2 Voltage Regulator is NOT in voltage reduction mode.

2H2 loading is 4MVA, 2H1 loading is 5 MVA

Data from Feeder breaker 2A112: Vab = 4102VAC; Vb c = 4150VAC; Vc a= 4120VAC

Data from Feeder breaker 2A113: la = 1810amps; lb = 1820amps; lc = 1810amps

Data from Feeder breaker 2A212: Vab = 4102VAC; Vbc = 4150VAC; Vca = 4120VAC

Data from Feeder breaker 2A213: la = 1600amps, lb = 1610amps; lc = 1620amps

#### TASK STANDARD: Identify 3 errors in OP-2107.001 section 11.0

Procedure step 11.6 was marked as NA when the initial conditions stated that the auto-transformer was out of service.

Procedure step 11.8.2 was marked as complete when initial conditions stated that unit 1 breaker A111 (A1) was not in pull to lock but in normal after stop.

Procedure step 11.8.5 was marked as complete when the loading of 2A1 and 2A2 exceeded the limit and precaution value for 4160VAC winding loading of 21 MVA with forced air and oil available (Calculation done on Attachment 'P is correct)

And Identified the error that SU<sup>#</sup>2 can not be placed in service in current configuration.

**TASK PERFORMANCE AIDS**: OP-2107.001 limits and precautions (section 5), completed section 11.0, and completed Attachment 'P'

#### **EXAMINER'S NOTES:**

#### **INITIATING CUE:**

The SM directs, "Maintenance on Startup transformer \*3 is planned and must be removed from service. Startup \*2 must be placed in service with load shed BYPASSED. OP-2107.001 section 11.0 has been completed up to step 11.8.6. Step 11.7 of OP-2107.001 is NOT applicable. Perform verification of steps completed, identify errors and determine if Startup transformer \*2 may be placed in service." Current Plant Conditions are the same as the initial plant conditions.

	PERFORMANCE CHECKLIST		STANDARDS		(Circle One)			
(C)	1.	Reviewed procedure OP- 2107.001 section 11.0 and Attachment 'P' for errors.	Identific	Procedure step 11.6 was marked as NA when the initial conditions stated that the auto-transformer was out of service.	N/A SAT UNSAT			
			2)	Procedure step 11.8.2 was marked as complete when initial conditions stated that unit 1 breaker A111 (A1) was not in pull to lock but in normal after stop.				
			3)	Procedure step 11.8.5 was marked as complete when the loading of 2A1 and 2A2 exceeded the limit and precaution value for 4160VAC winding loading of 21 MVA with forced air and oil available (Calculation done on Attachment 'P is correct)				
(C)	2.	Determined that SU <sup>#</sup> 2 availability.		nined that SU <sup>#</sup> 2 could not sed in service in current tration.	N/A SAT UNSAT			
	END							

#### **EXAMINER's COPY**

#### **INITIAL PLANT CONDITIONS**

- Mode 4; 250 EFPD. All ESF systems are in normal alignment.
- Startup Transformer \*3 is supplying all AC electrical busses.
- 'A' and 'B' Condensate Pumps are running for post maintenance testing and 'C' and 'D' Condensate Pumps are in pull to lock.
- 'B' Circulating Water Pump is running and 'A' Circulating Water Pump is in pull to lock.
- 'B' and 'D' Reactor Coolant Pumps are running and 'A' and 'C' Reactor Coolant Pumps are secured.
- Auto-transformer is out of service for the next 12 hours due to emergency forced outage, design engineering has completed an evaluation to allow SU<sup>#</sup>2 to be placed in service without the Auto-transformer being in service.
- Unit 1 is at 100% and all busses energized from their Unit Auxiliary Transformer and with no equipment out of service.
- Unit 1 SU<sup>#</sup>2 feeder to A1 (A111) is in normal after stop.
- SU<sup>#</sup>2 Voltage Regulator is NOT in voltage reduction mode.
- 2H2 loading is 4MVA, 2H1 loading is 5 MVA
- Data from Feeder breaker 2A112: Vab = 4102VAC; Vbc = 4150VAC; Vca = 4120VAC
- Data from Feeder breaker 2A113: la = 1810amps; lb = 1820amps; lc=1810amps
- Data from Feeder breaker 2A212: Vab = 4102VAC; Vbc = 4150VAC; Vca = 4120VAC
- Data from Feeder breaker 2A213: la = 1600amps, lb = 1610amps; lc = 1620amps

Can Startup Transformer #2 be placed in service given the initial condition information? \_\_\_\_\_No\_\_

#### **Initiating CUE:**

The SM directs, "Maintenance on Startup transformer \*3 is planned and must be removed from service. Startup \*2 must be placed in service with load shed BYPASSED. OP-2107.001 section 11.0 has been completed up to step 11.8.6. Perform verification of steps completed, identify errors and determine if Startup transformer \*2 may be placed in service." Current Plant Conditions are the same as the initial plant conditions.

<u>List any errors identified below:</u>	
Refer to performance standards on page 3	

#### **EXAMINEE's COPY**

#### **INITIAL PLANT CONDITIONS**

- Mode 4; 250 EFPD. All ESF systems are in normal alignment.
- Startup Transformer \*3 is supplying all AC electrical busses.
- 'A' and 'B' Condensate Pumps are running for post maintenance testing and 'C' and 'D' Condensate Pumps are in pull to lock.
- 'B' Circulating Water Pump is running and 'A' Circulating Water Pump is in pull to lock.
- 'B' and 'D' Reactor Coolant Pumps are running and 'A' and 'C' Reactor Coolant Pumps are secured.
- Auto-transformer is out of service for the next 12 hours due to emergency forced outage, design engineering has completed an evaluation to allow SU<sup>#</sup>2 to be placed in service without the Auto-transformer being in service.
- Unit 1 is at 100% and all busses energized from their Unit Auxiliary Transformer and with no equipment out of service.
- Unit 1 SU<sup>#</sup>2 feeder to A1 (A111) is in normal after stop.
- SU<sup>#</sup>2 Voltage Regulator is NOT in voltage reduction mode.
- 2H2 loading is 4MVA, 2H1 loading is 5 MVA
- Data from Feeder breaker 2A112: Vab = 4102VAC; Vbc = 4150VAC; Vca = 4120VAC
- Data from Feeder breaker 2A113: la = 1810amps; lb = 1820amps; lc = 1810amps
- Data from Feeder breaker 2A212: Vab = 4102VAC; Vbc = 4150VAC; Vca = 4120VAC
- Data from Feeder breaker 2A213: la = 1600amps, lb = 1610amps; lc = 1620amps

Can Startup Transformer #2 be placed in service given the initial condition information? \_\_\_\_\_

#### **Initiating CUE:**

The SM directs, "Maintenance on Startup transformer \*3 is planned and must be removed from service. Startup \*2 must be placed in service with load shed BYPASSED. OP-2107.001 section 11.0 has been completed up to step 11.8.6. Perform verification of steps completed, identify errors and determine if Startup transformer \*2 may be placed in service." Current Plant Conditions are the same as the initial plant conditions.

List any errors identified below:		

# ANO-2-JPM-NRC-ADMIN-TTBCSRO

#### PAGE 1 OF 5

## ADMINISTRATIVE JOB PERFORMANCE MEASURE

UNIT: _	2	-		REV #:	003	_		DA	TE:		
SYSTEM	/DUTY	AREA:	Conduct	of Opera	ations	A6					
TASK:	Revie	ew Time to	Boil Cald	culation							
JTA#:	ANO	2-RO-NOR	M-4								
KA VALU	JE	RO:	4.3	SRO:		4.4	KA RE	FERE	NCE:	2.1.23	
APPROV	ED FC	R ADMINI	STRATIO	N TO:	RO:		SRO:	X	-		
TASK LO	CATIO	ON:	INSIDE C	CR:		OUTSIDI	E CR:		вотн:	X	
SUGGES	STED 1	ESTING E	NVIRON	MENT AN	D MET	THOD (PE	RFORM	I OR SI	IMULATE)	):	
PLANT S	SITE:			SIMULA	TOR:	Per	form	Cla	assroom:	Perform	-
POSITIO	N EVA	LUATED:	RO:			SRO:		Х			
ACTUAL 1	TESTIN	IG ENVIROI	NMENT:	SIMULAT	ΓOR:		PLANT	SITE:		Classroom:	
TESTING	MET	HOD: S	SIMULATE	i:	PI	ERFORM:					
APPROX	IMATE	COMPLE	TION TIM	IE IN MIN	IUTES:	20 1	Minutes	<u> </u>			
REFERE	NCE(S	6): <u>OP-10</u>	15.008 At	tachmen	t E and	d OP-2103	3.011				
EXAMINE	EE'S N	IAME:					I	KCN: _			
EVALUA <sup>-</sup>	TOR'S	NAME:									
		E'S PERFO			/ALUA	TED AGA	INST T	HE ST	ANDARDS	CONTAINED IN	I
SATISFA	CTOR	Y:		UNSATI	SFAC	TORY:					
PERFOR	MANC	E CHECK	LIST COM	IMENTS:							
Start Time			op me		To	tal Time					
SIGNED:						DATE:				_	

SIGNATURE INDICATES THIS JPM HAS BEEN COMPARED TO ITS APPLICABLE PROCEDURE BY A QUALIFIED INDIVIDUAL (NOT THE EXAMINEE) AND IS CURRENT WITH THAT REVISION.

THE EXAMINER SHALL REVIEW THE FOLLOWING WITH THE EXAMINEE:
The examiner shall ensure that the examinee has been briefed on NUREG 1021 Appendix E.
JPM INITIAL TASK CONDITIONS:
Plant shutdown for repair of Steam Generator Tube Leak. Making preparations for draining the RCS to
24 inches above the bottom of the hotleg to install nozzle dams. PZR level is 40%.
Time after shutdown = 35 hours; one (1) PZR code safety valve is removed; PZR manway is installed;
ECCS vent valves are de-energized OPEN; NO RCP seal work inprogress; NO RCS cold or hot leg openings;
RCS Temperature is 115°F.
TASK STANDARD:
Identify errors in the original Time to boil calculation :
1) Initial RCS water level was recorded as 42" above the bottom of the hot leg instead of the correct 24".
2) Current RCS temperature was recorded as 80°F instead of 115°F as stated in the initial conditions.
AND
Identify that the RCS cannot be drained to lowered inventory due to TTCU is less than 2 hours or TTB <15 minutes
TASK PERFORMANCE AIDS:
Computer operational with the current revision of LOSDC2 installed.
(SP-94-C-0001-01, Rev. 13), set up shortcut to program on the desktop. Marked up copy of LOSDC2 printout.
SIMULATOR SETUP:
NA
EXAMINER'S NOTES:

#### **INITIATING CUE:**

CRS/SM directs: "Perform a supervisory review of the given time to boil and time to core uncovery calculation using LOSDC2 program and identify any errors and determine if conditions are met to drain the RCS to lowered inventory."

	PERFO	RMANCE CHECKLIST	STANDARDS	(Circle One)				
	1.	Compare time to boil (TTB) calculation using LOSDC2 program.	Computer program independently run and calculations compared	N/A SAT UNSAT				
(C)	2.	Identified errors	<ol> <li>Initial RCS water level was recorded as 42" above the bottom of the hot leg instead of the correct 24".</li> <li>Current RCS temperature was recorded as 80°F instead of the correct 115°F.</li> </ol>	N/A SAT UNSAT				
(C)	3.	Determined that the actual Time until core uncovery is < 2 hours or Time to boil: is < 15 minutes	Determined that the RCS cannot be drained to lowered inventory.	N/A SAT UNSAT				
	EXAMINER'S NOTE:							
Promp	Prompt the Examinee that the file SHOULD NOT be printed or saved.							
	END							

#### **EXAMINER'S COPY**

#### JPM INITIAL TASK CONDITIONS:

Given the following Plant conditions:

- Plant shutdown for repair of Steam Generator Tube Leak
- Making preparations for draining the RCS to 24 inches above the bottom of the hot leg to install nozzle dams
- PZR level is 40%
- Time after shutdown = 35 hours
- One (1) PZR code safety valve is removed
- PZR manway is installed
- ECCS vent valves are de-energized OPEN
- NO RCP seal work in progress
- NO RCS cold or hot leg openings
- RCS Temperature is 115°F

#### **INITIATING CUE:**

CRS/SM directs: "CRS/SM directs: "Perform a supervisory review of the given time to boil and time to core uncovery calculation using LOSDC2 program and determine if conditions are met to drain the RCS to lowered inventory IAW plant procedures."

#### Do not print data or save LOSCD2 file.

See tas	k standards	s page 3			

#### **EXAMINEE'S COPY**

#### JPM INITIAL TASK CONDITIONS:

Given the following Plant conditions:

- Plant shutdown for repair of Steam Generator Tube Leak
- Making preparations for draining the RCS to 24 inches above the bottom of the hot leg to install nozzle dams
- PZR level is 40%
- Time after shutdown = 35 hours
- One (1) PZR code safety valve is removed
- PZR manway is installed
- ECCS vent valves are de-energized OPEN
- NO RCP seal work in progress
- NO RCS cold or hot leg openings
- RCS Temperature is 115°F

CRS/SM directs: "CRS/SM directs: "Perform a supervisory review of the given time to boil and time to core uncovery calculation using LOSDC2 program and determine if conditions are met to drain the RCS to lowered inventory."

## Do not print data or save LOSCD2 file.

List any errors identified below:		

Can the RCS be drained to reduced inventory given the initial condition information?

CVCTEM										
SISIEIVI	DUTY AREA:	Equipme	ent Contr	ol	A7					
TASK:	Supervisory	review of m	aintenan	ce acti	vities for	configu	uration	control		
JTA#:	ANO-SRO-AL	MIN-NORM	-48							
KA VALU	E RO:	3.9	SRO:		4.3	KA RE	FERE	NCE:	2.2.14	
APPROV	ED FOR ADM	INISTRATIO	N TO:	RO:		SRO:	X	-		
TASK LO	CATION:	INSIDE (	CR:		OUTSIDI	E CR:		вотн:	X	
SUGGES	TED TESTING	ENVIRONI	MENT AN	ID MET	THOD (PE	RFORM	OR S	IMULATE)	):	
PLANT S	ITE:		SIMULA	TOR:	Per	form	Cla	assroom:	Perform	
POSITIO	N EVALUATE	D: RO:			SRO:		Х			
ACTUAL 1	ESTING ENVIR	ONMENT:	SIMULAT	TOR:		PLANT	SITE:		Classroom:	
TESTING	METHOD:	SIMULATE	i:	P	ERFORM:					
APPROX	IMATE COMP	LETION TIN	IE IN MIN	IUTES	: 10	Minutes	<u>s_</u>			
REFEREI	NCE(S): 102	5.003, Cond	luct of Ma	ainten	ance, 210	4.029 S	ervice '	Water sys	tem operations	
EXAMINE	EE'S NAME:						KCN: _			
EVALUA <sup>-</sup>	TOR'S NAME:									
	MINEE'S PER I AND IS DET			/ALUA	TED AGA	INST T	HE ST	ANDARDS	CONTAINED IN	
SATISFA	CTORY:		UNSATI	SFAC	TORY:					
PERFOR	MANCE CHEC	CKLIST CON	MENTS:							
Start Time		Stop Time		To	tal Time					
SIGNED:					DATE:				<del>-</del>	

SIGNATURE INDICATES THIS JPM HAS BEEN COMPARED TO ITS APPLICABLE PROCEDURE BY A QUALIFIED INDIVIDUAL (NOT THE EXAMINEE) AND IS CURRENT WITH THAT REVISION.

## A2JPM-SRO-MAINT PAGE 2 OF 5

## ADMINISTRATIVE JOB PERFORMANCE MEASURE

THE EXAMINER SHALL REVIEW THE FOLLOWING WITH THE EXAMINEE:
The examiner shall ensure that the examinee has been briefed on NUREG 1021 Appendix E.
JPM INITIAL TASK CONDITIONS:
I&C Maintenance personnel have completed troubleshooting the Spent Fuel Cooling radiation monitor 2RE-1525.
The troubleshooting required isolating AND draining the radiation monitor.
There are currently no Equipment Status Log (ESL) entries.
Maintenance Configuration Tracking Log 1025.003C was used for configuration control.
TASK STANDARD:
Identified errors.
1) 2SW-1525A 'Restoration' position is listed as CLOSED when required to be OPEN.
2) 2SW-1525M 'Require position for Maintenance' is listed as CLOSED when required to be OPEN to drain the system.
3) 2SW-1525H 'Restoration' position is listed as OPEN when required to be CLOSED.
4) 2SW-1525J Restorer and Verifier initials are the same.
TASK PERFORMANCE AIDS:
Marked up copy of form 1025.003C, Maintenance Configuration Tracking Log
Copy of OP-2104.029 or computer.
SIMULATOR SETUP:
NA
EXAMINER'S NOTES:

#### **INITIATING CUE:**

CRS/SM directs: "Perform an Operations verification of the given 1025.003C form, Maintenance Configuration Tracking Log following maintenance on SFP cooling radiation monitor 2RE-1525. List any error(s) with the form completion below"

	PERFORMANCE CHECKLIST STANDARDS (Circle One)				
Evaluator's Note: Provide examinee with a copy of 1025.003, Maintenance Configuration Tracking Log Sheet.					
	1.	Review the Maintenance Configuration Tracking Log Sheet	Examinee reviewed the Maintenance Configuration Tracking Log Sheet and compared it to the normal lineup in 2104.029, Attachment E.	N/A SAT UNSAT	
(C)	2.	Identify errors.	Identify errors  1) 2SW-1525A 'Restoration' position is listed as CLOSED when required to be OPEN.  2) 2SW-1525M 'Require position for Maintenance' is listed as CLOSED when required to be OPEN to drain the system.  3) 2SW-1525H 'Restoration' position is listed as OPEN when required to be CLOSED.  4) 2SW-1525J Restorer and Verifier initials are the same.	N/A SAT UNSAT	
END					

#### **EXAMINER'S COPY**

#### JPM INITIAL TASK CONDITIONS:

I&C Maintenance personnel have completed troubleshooting the Spent Fuel Cooling radiation monitor 2RE-1525.

The troubleshooting required isolating AND draining the radiation monitor.

There are currently no Equipment Status Log (ESL) entries.

Maintenance Configuration Tracking Log 1025.003C was used for configuration control.

#### **INITIATING CUE:**

CRS/SM directs: "Perform an Operations verification of the given 1025.003C form, Maintenance Configuration Tracking Log following maintenance on SFP cooling radiation monitor 2RE-1525. List any error(s) with the form completion below"

see task standard on page 3 for list of errors				
	_			
	_			
	_			
	_			
	_			

#### **EXAMINEE'S COPY**

#### JPM INITIAL TASK CONDITIONS:

I&C Maintenance personnel have completed troubleshooting the Spent Fuel Cooling radiation monitor 2RE-1525.

The troubleshooting required isolating AND draining the radiation monitor.

There are currently no Equipment Status Log (ESL) entries.

Maintenance Configuration Tracking Log 1025.003C was used for configuration control.

#### **INITIATING CUE:**

CRS/SM directs: "Perform an Operations verification of the given 1025.003C form, Maintenance Configuration Tracking Log following maintenance on SFP cooling radiation monitor 2RE-1525. List any error(s) with the form completion below"				

# ANO-2JPM-NRC-ADMIN-EMGRESPSRO <u>ADMINISTRATIVE JOB PERFORMANCE MEASURE</u>

#### PAGE 1 OF 5

UNIT:	2		REV #:	001		D	ATE	
_					_	•		
SYSTEM	I/DUTY AREA:	Radiatio	n Contro	l	A8			
TASK:	Calculated exallowed	xpected dos	se for Re-	entry	during an	emergency a	and determi	ne if entry is
LP#:	ASLP-RO-RA	ADP						
KA VALU	JE RO: _	3.2	SRO:		3.7	KA REFERE	ENCE:	2.3.4
APPROV	ED FOR ADM	INISTRATIC	N TO:	RO:		SRO: X		
TASK LO	OCATION:	INSIDE (	CR:		OUTSID	E CR:	_ вотн:	X
SUGGES	STED TESTING	S ENVIRONI	MENT AN	D ME	THOD (PE	RFORM OR	SIMULATE)	:
PLANT S	SITE:		SIMULA	TOR:		c	lassroom:	Perform
POSITIO	N EVALUATE	D: RO:			SRO:		_	
ACTUAL .	TESTING ENVIR	RONMENT:	SIMULAT	ΓOR:		PLANT SITE:	: CI	assroom:
TESTING	METHOD:	SIMULATE	Ē:	P	ERFORM	:	_	
APPROX	(IMATE COMP	LETION TIN	IE IN MIN	UTES	: 15	Minutes		
REFERE	NCE(S): 190	3.033 and 1	903.033A	form				
EXAMINI	EE'S NAME:					KCN:		
EVALUA	EVALUATOR'S NAME:							
THE EXAMINEE'S PERFORMANCE WAS EVALUATED AGAINST THE STANDARDS CONTAINED IN THIS JPM AND IS DETERMINED TO BE:								
SATISFA	ACTORY:		UNSATI	SFAC	TORY:		_	
PERFOR	MANCE CHEC	CKLIST CON	MENTS:					
Start Time		Stop Time		To	tal Time		-	
SIGNED:	·				DATE:			

SIGNATURE INDICATES THIS JPM HAS BEEN COMPARED TO ITS APPLICABLE PROCEDURE BY A QUALIFIED INDIVIDUAL (NOT THE EXAMINEE) AND IS CURRENT WITH THAT REVISION.

### PAGE 2 OF 5

THE EXAMINER SHALL REVIEW THE FOLLOWING WITH THE EXAMINEE:
The examiner shall ensure that the examinee has been briefed on NUREG 1021 Appendix E.
JPM INITIAL TASK CONDITIONS:
The plant is tripped from 100% power.
300 gpm LOCA in progress.
Loss of Offsite power is in progress.
LOCA EOP is being implemented.
RDACS indicates an off site release in progress.
A Site Area Emergency has been declared based on the RDACS dose rate.
RDACS dose rates are continuing to rise and the trend indicates they will exceed GE criteria if the release is not terminated.
Whole body dose rates in area of work are 7.3 Rem/hr.
RP estimates that it will take approximately 75 minutes to complete emergency actions and stop the release which will prevent exceeding RDACS General emergency criteria.
Joe Mechanic and Ralph RP have been selected to make the entry to stop the release but have not volunteered.
Joe Mechanic and Ralph RP have been briefed on the task and entry requirements.
Joe Mechanic's ERIMS dose to date is 382 mR and has badge number 20031.
Ralph RP's ERIMS dose to date 1353 mR and has badge number 20005.
TASK STANDARD:
Determine that 10CFR20 limits can be exceeded for protection of Large populations by
calculating Joe Mechanics and Ralph RP's dose and complete 1903.033A form for entry.
TASK PERFORMANCE AIDS:
1903.033 Protective Action Guidelines for Recue/Repair and Damage Control Teams.
SIMULATOR SETUP:
NA NA
EXAMINER'S NOTES:

#### **INITIATING CUE:**

The SM directs, use 1903.033 Protective Action Guidelines for Rescue/Repair and Damage control Teams determine Joe Mechanic and Ralph RP's estimated year to date total dose for repair and determine if entry is allowed and complete 1903.033A if entry is allowed.

	PERF	ORMANCE CHECKLIST	STANDARDS	(Circle One)		
(C)	1.	Calculate Joe Mechanic's estimated dose	Calculated Joe Mechanic's estimated dose to in the following range:	N/A SAT UNSAT		
			9307 to 9707 mR			
(C)	2.	Calculate Ralph RP's estimated dose	Calculated Ralph RP's estimated dose to in the following range:	N/A SAT UNSAT		
			10278 to 10678 mR			
(C)	3.	Determine if Joe Mechanic's is allowed to make the entry to stop the release.	Determined that Joe Mechanic is allowed to make the entry for protection of Large populations.	N/A SAT UNSAT		
(C)	4.	Determine if Ralph RP's is allowed to make the entry to stop the release.	Determined that Ralph RP is allowed to make the entry for protection of Large populations.	N/A SAT UNSAT		
(C)	5.	Complete 1903.033A	Completed section 1 of 1903.033A, selected box 2 for protection of large populations (dose < 25 Rem)  Completed section 2 of 1903.033A included Joe Mechanic and Ralph RP as exceeding 10CFR20 dose limits.	N/A SAT UNSAT		

Examiner Note: If the applicant fills out the 1903.033A but does not sign the form for the TSC director or for the briefing being complete this is still considered they completed the task correctly.

**END** 

#### PAGE 4 OF 5

#### **EXAMINER'S COPY**

#### JPM INITIAL TASK CONDITIONS:

Given the following Plant conditions:

- The plant is tripped from 100% power.
- 300 gpm LOCA in progress.
- Loss of Offsite power is in progress.
- LOCA EOP is being implemented.
- RDACS indicates an off site release in progress.
- A Site Area Emergency has been declared based on the RDACS dose rate.
- RDACS dose rates are continuing to rise and the trend indicates they will exceed GE
  criteria if the release is not terminated.
- Whole body dose rates in area of work are 7.3 Rem/hr.
- RP estimates that it will take approximately 75 minutes to complete emergency actions and stop the release which will prevent exceeding GE criteria.
- Joe Mechanic and Ralph RP have been selected to make the entry to stop the release but have not volunteered.
- Joe Mechanic and Ralph RP have been briefed on the task and entry requirements.
- The TSC director has authorized 10CFR20 limits can be exceeded.
- Joe Mechanic's ERIMS dose to date is 382 mR and has badge number 20031.
- Ralph RP's ERIMS dose to date 1353 mR and has badge number 20005.

#### **INITIATING CUE:**

The SM directs, use 1903.033 Protective Action Guidelines for Rescue/Repair and Damage control Teams determine Joe Mechanic and Ralph RP's estimated year to date total dose for repair and determine if entry is allowed and complete 1903.033A if entry is allowed.

Joe Mechanic's estimated TEDE dose: 9507 (acceptable band 9307 to 9707) mR
Ralph RP's estimated TEDE dose: 10478 (acceptable band 10278 to 10678) mF
Can Joe Mechanic perform re-entry and complete the task? YE9 / NO
Can Ralph RP perform re-entry and complete the task? YES/ NO

#### PAGE 5 OF 5

#### **EXAMINEE'S COPY**

#### JPM INITIAL TASK CONDITIONS:

Given the following Plant conditions:

- The plant is tripped from 100% power.
- 300 gpm LOCA in progress.
- Loss of Offsite power is in progress.
- LOCA EOP is being implemented.
- RDACS indicates an off site release in progress.
- A Site Area Emergency has been declared based on the RDACS dose rate.
- RDACS dose rates are continuing to rise and the trend indicates they will exceed GE
  criteria if the release is not terminated.
- Whole body dose rates in area of work are 7.3 Rem/hr.
- RP estimates that it will take approximately 75 minutes to complete emergency actions and stop the release which will prevent exceeding GE criteria.
- Joe Mechanic and Ralph RP have been selected to make the entry to stop the release but have not volunteered.
- Joe Mechanic and Ralph RP have been briefed on the task and entry requirements.
- The TSC director has authorized 10CFR20 limits can be exceeded.
- Joe Mechanic's ERIMS dose to date is 382 mR and has badge number 20031.
- Ralph RP's ERIMS dose to date 1353 mR and has badge number 20005.

#### **INITIATING CUE:**

The SM directs, use 1903.033 Protective Action Guidelines for Rescue/Repair and Damage control Teams determine Joe Mechanic and Ralph RP's estimated year to date total dose for repair and determine if entry is allowed and complete 1903.033A if entry is allowed.

Joe Mechanic's estimated TEDE dose:	mR
Ralph RP's estimated TEDE dose:	mR
Can Joe Mechanic perform re-entry and complete the task?	YES / NO
Can Ralph RP perform re-entry and complete the task? YES	S / NO

ANO-2-JPM-NRC-ADMIN-EAL11 REV 02 PAGE 1 of 5 UNIT: REV #: 002 DATE: 2 Emergency Plan A9 SYSTEM/DUTY AREA: TASK: Determine Emergency Action Level/Protective Action Recommendation (Time Critical JPM) JTA#: ANO-SRO-EPLAN-EMERG-278 RO: \_\_\_\_\_\_ SRO: \_\_\_\_\_\_ 4.6 KA REFERENCE: \_\_\_\_\_ 2.4.41 KA VALUE APPROVED FOR ADMINISTRATION TO: RO: SRO: X INSIDE CR: OUTSIDE CR: BOTH: X TASK LOCATION: SUGGESTED TESTING ENVIRONMENT AND METHOD (PERFORM OR SIMULATE): SIMULATOR: Perform CLASSROOM: PLANT SITE: POSITION EVALUATED: RO: \_\_\_\_\_ SRO: ACTUAL TESTING ENVIRONMENT: SIMULATOR: CLASSROOM: PLANT SITE: SIMULATE: PERFORM: TESTING METHOD: APPROXIMATE COMPLETION TIME IN MINUTES: 5 Minutes - Read initial **Conditions** 15 Minutes – EAL Classification REFERENCE(S): 1903.010 **EXAMINEE'S NAME:** KCN: **EVALUATOR'S NAME:** THE EXAMINEE'S PERFORMANCE WAS EVALUATED AGAINST THE STANDARDS CONTAINED IN THIS JPM AND IS DETERMINED TO BE: UNSATISFACTORY: PERFORMANCE CHECKLIST COMMENTS: Start Time Stop Time Total Time DATE:

SIGNATURE INDICATES THIS JPM HAS BEEN COMPARED TO ITS APPLICABLE PROCEDURE BY A QUALIFIED INDIVIDUAL (NOT THE EXAMINEE) AND IS CURRENT WITH THAT REVISION.

#### THE EXAMINER SHALL REVIEW THE FOLLOWING WITH THE EXAMINEE:

The examiner shall ensure that the examinee has been briefed on NUREG 1021 Appendix E.

#### JPM INITIAL TASK CONDITIONS:

- Unit 2 has been operating at 100% for 230 consecutive days
- A plant trip has occurred due to a loss of offsite power with a concurrent Loss of Coolant Accident estimated to be 1500 gpm
- The reactor trip occurred 4 hours ago
- High Range containment radiation monitor, 2RITS-8925-1 reads 74450 R/hr and trending up
- High Range containment radiation monitor 2RITS-8925-2 reads 77730 R/hr and trending up
- RCS average Core Exit Thermocouples are reading 430°F
- PZR pressure is 325 psia
- Containment Pressure is 20 psia and slowly trending down
- RDACs indicates <u>NO</u> offsite release in progress
- All safety systems are responding as designed

# **TASK STANDARD**: Determine the following (Time Critical - 5 minutes to read initial conditions/ 15 minutes to classify event):

- 1. Determine containment radiation is at GE level
- 2. Determine EAL classification GE, 1.5, Containment Radiation readings which indicate LOCA and >50% fuel overheat

TASK PERFORMANCE AIDS: 1903.010, Emergency Action Levels

#### **INITIATING CUE:**

Determine the highest EAL classification from the given initial conditions (list EAL classification event number).

.

		PERFORMANCE CHECKLIST	STANDARD	(Circle One)
	1.	Determined containment radiation readings are not modified due to power history.	Using OP 1903.010 Attachment 6, verified power history does not require modifying containment radiation readings.	N/A SAT UNSAT
(C)	2.	Determined containment radiation is GE level	Using OP 1903.010 Attachment 6, determined containment radiation readings the GE criteria. (critical element verified by classifying EAL as event 1.5)	N/A SAT UNSAT
(C)	3.	Determined GE EAL classification.	Using OP 1903.010, determined GE EAL classification 1.5, Containment Radiation readings which indicate LOCA and >50% fuel overheat	N/A SAT UNSAT
(C)	4.	JPM complete in 15 minutes.	JPM complete in 15 minutes with 5 minutes to read conditions.	N/A SAT UNSAT
			END	_

## **Examiner's Copy**

### JPM is TIME CRITICAL

#### JPM INITIAL TASK CONDITIONS:

- Unit 2 has been operating at 100% for 230 consecutive days
- A plant trip has occurred due to a loss of offsite power with a concurrent Loss of Coolant Accident estimated to be 1500 gpm
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- RCS average Core Exit Thermocouples are reading 430°F
- PZR pressure is 325 psia
- Containment Pressure is 20 psia and slowly trending down
- RDACs indicates NO offsite release in progress
- · All safety systems are responding as designed

#### **INITIATING CUE:**

Determine the highest EAL classification from the given initial conditions (list EAL classification event number).

#### **ANSWER:**

EAL classification and event number:

GE, EAL 1.5, Containment Radiation readings which indicate LOCA and >50%

fuel overheat

# **Examinee's Copy**

#### JPM is TIME CRITICAL

#### JPM INITIAL TASK CONDITIONS:

- Unit 2 has been operating at 100% for 230 consecutive days
- A plant trip has occurred due to a loss of offsite power with a concurrent Loss of Coolant Accident estimated to be 1500 gpm
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- RDACs indicates <u>NO</u> offsite release in progress
- · All safety systems are responding as designed

#### **INITIATING CUE:**

Determine the highest EAL classification from the given initial conditions (list EAL classification event number).

#### **ANSWER:**

EAL classification and event	
number:	

Facility: Arkansas Nuclear One Unit 2	of Examination:			
Exam Level: RO SRO-I X SRO-U Operating Test No.: 2012-1				
Control Room Systems <sup>®</sup> (8 for RO); (7 for SRO-I);	(2 or 3 for SRO-U, i	including 1 ESF)		
System / JPM Title		Type Code*	Safety Function	
S1. ANO-2-JPM-NRC-SIT09 006 A4.02 RO-4.0/SRO-3.8 Isolate SITs following SIAS with SIAS reset		A/D/EN/L/S	2 Inventory Control	
S2. ANO-2-JPM-NRC-ELEC05 062 A4.01 RO3.3/SRO3.1 Transfer Auxiliaries from SU#2 to SU#3 for 2A-1		N/S	6 Electrical	
S3. ANO-2-JPM-NRC-SDBC2 041 A4.05 RO-3.1/SRO-3.3 Manually Operate SDBCS valves		A/L/N/S	4 Heat Removal	
S4. ANO-2-JPM-NRC-CCW01 008 A4.01 RO-3.3/SRO-3.1 Secure CCW system using EOP		D/L/S	8 Plant service systems	
S5. ANO-2-JPM-NRC-CNMT2 103 A3.01 RO-3.9/SRO-4.2 Drain the containment sump		A/D/P/S	5 Containment Integrity	
S6. ANO-2-JPM-NRC-CEA01 001 A4.03 RO-4.0/SRO-3.7 Perform a CEA exercise		D/P/S	1 Reactivity control	
S7. ANO-2-JPM-NRC-CPC06 012 A4.02 RO-3.3/SRO-3.4 Place #1 CEAC to inop on "B" CPC		D/S	7 Instrumentation	
In-Plant Systems <sup>@</sup> (3 for RO); (3 for SRO-I); (3 or 2	2 for SRO-U)			
P1. ANO-2-JPM-NRC-EDDCB 064 A4.01 RO-4.0/SRO-4.3 Start EDG 2K-4B without DC		A/D/E/L	6 Electrical	
P2. ANO-2-JPM-NRC-MGSU 001 A4.08 RO-3.7/SRO-3.4 Start the First MG set		D/L	1 Reactivity control	
P3. ANO-2-JPM-NRC-FPEM1 033 A2.03 RO-3.1/SRO-3.5 Perform emergency SW make up to SFP		D/E/R	8 Plant service systems	
All RO and SRO-I control room (and in-plant) systems must be different and serve different safety functions; all 5 SRO-U systems must serve different safety functions; in-plant systems and functions may overlap those tested in the control room.				
* Type Codes Criteria for RO / SRO-I / SRO-U				
(A)Iternate path $4-6/4-6/2-3$ (C)ontrol room(D)irect from bank $\leq 9/\leq 8/\leq 4$ (E)mergency or abnormal in-plant $\geq 1/\geq 1/\geq 1$ (EN)gineered safety feature $-/-/\geq 1$ (control room s(L)ow-Power / Shutdown $\geq 1/\geq 1/\geq 1$ (N)ew or (M)odified from bank including 1(A) $\geq 2/\geq 2/\geq 1$ (P)revious 2 exams $\leq 3/\leq 3/\leq 2$ (randomly sele(R)CA $\geq 1/\geq 1/\geq 1$				
(N)ew or (M)odified from bank including 1(A) (P)revious 2 exams		≥ 2 / ≥ 2 / ≥ 1 ≤ 3 / ≤ 3 / ≤ 2 (rand	domly selected	

Facility: <u>Arkansas Nuclear One Unit 2</u> Exam Level: RO <b>X</b> SRO-I SRO-U	Date of Examination Operating Test No			
Control Room Systems <sup>@</sup> (8 for RO); (7 for SRO-I);	(2 or 3 for SRO-U, including 1 E	ESF)		
System / JPM Title	Type Cod	e* Safety Function		
S1. ANO-2-JPM-NRC-SIT09 006 A4.02 RO-4.0/SRO-3.8 Isolate SITs following SIAS with SIAS reset	A/D/EN/L/	/S 2		
S2. ANO-2-JPM-NRC-ELEC05 062 A4.01 RO3.3/SRO3.1 Transfer Auxiliaries from SU#2 to SU#3 for 2A-1	N/S	6 Electrical		
S3. ANO-2-JPM-NRC-SDBC2 041 A4.05 RO-3.1/SRO-3.3 Manually Operate SDBCS valves	A/L/N/S	4 Heat Removal		
S4. ANO-2-JPM-NRC-CCW01 008 A4.01 RO-3.3/SRO-3.1 Secure CCW system using EOP	D/L/S	8 Plant service systems		
S5. ANO-2-JPM-NRC-CNMT2 103 A3.01 RO-3.9/SRO-4.2 Drain the containment sump	A/D/P/S	5 Containment Integrity		
S6. ANO-2-JPM-NRC-CEA01 001 A4.03 RO-4.0/SRO-3.7 Perform a CEA exercise	D/P/S	1 Reactivity control		
S7. ANO-2-JPM-NRC-CPC06 012 A4.02 RO-3.3/SRO-3.4 Place #1 CEAC to inop on "B" CPC	D/S	7 Instrumentation		
S8. ANO-2-JPM-NRC-AOP3 010 A2.02 RO-3.9/SRO-3.9 Isolate failed PZR spray valve	A/D/P/S	3 Pressure Control		
In-Plant Systems <sup>®</sup> (3 for RO); (3 for SRO-I); (3 or 2	for SRO-U)			
P1. ANO-2-JPM-NRC-EDDCB 064 A4.01 RO-4.0/SRO-4.3 Start EDG 2K-4B without DC	A/D/E/L	6 Electrical		
P2. ANO-2-JPM-NRC-MGSU 001 A4.08 RO-3.7/SRO-3.4 Start the First MG set	D/L	1 Reactivity control		
P3. ANO-2-JPM-NRC-FPEM1 033 A2.03 RO-3.1/SRO-3.5 Perform emergency SW make up to SFP	D/E/R	8 Plant service systems		
@ All RO and SRO-I control room (and in-plant) systems must be different and serve different safety functions; all 5 SRO-U systems must serve different safety functions; in-plant systems and functions may overlap those tested in the control room.				
* Type Codes Criteria for RO / SRO-I / SRC				
$(A) lternate path \\ (C) ontrol room \\ (D) irect from bank \\ (E) mergency or abnormal in-plant \\ (EN) gineered safety feature \\ (L) ow-Power / Shutdown \\ (N) ew or (M) odified from bank including 1(A) \\ (P) revious 2 exams \\ (R) CA \\ (S) imulator $ $4-6 / 4-6 / 2-3$ $4-6 / 4-6 / 4-6 / 2-3$ $4-6 / 4-6 / 4-6 / 2-3$ $4-6 / 4-6 / 4-6 / 2-3$ $4-6 / 4-6 / 4-6 / 2-3$ $4-6 / 4-6 / 4-6 / 4-6 / 4-6$ $4-6 / 4-6 / 4-6 / 4-6 / 4-6$ $4-6 / 4-6 / 4-6 / 4-6$ $4-6 / 4-6 / 4-6 / 4-6$ $4-6 / 4-6 / 4-6 / 4-6$ $4-6 / 4-6 / 4-6 / 4-6$ $4-6 / 4-6 / 4-6$ $4-6 / 4-6 / 4-6$ $4-6 / 4-6 / 4-6$ $4-6 / 4-6 / 4-6$ $4-6 / 4-6 / 4-6$ $4-6 / 4-6 / 4-6$ $4-6 / 4-6 / 4-6$ $4-6 / 4-6 / 4-6$ $4-6 / 4-6 / 4-6$ $4-6 / 4-6 / 4$		(control room system) (randomly selected)		

Facility: Arkansas Nuclear One Unit 2  Exam Level: RO SRO-I SRO-U X  Date of Examination: 08/27/2012  Operating Test No.: 2012-1				
Control Room Systems <sup>®</sup> (8 for RO); (7 for SRO-I);	(2 or 3 for SRO-U, i	ncluding 1 ESF)		
System / JPM Title		Type Code*	Safety Function	
S1. ANO-2-JPM-NRC-SIT09 006 A4.02 RO-4.0/SRO-3.8 Isolate SITs following SIAS with SIAS reset		A/D/EN/L/S	2 Inventory Control	
S2. ANO-2-JPM-NRC-ELEC05 062 A4.01 RO3.3/SRO3.1 Transfer Auxiliaries from SU#2 to SU#3 for 2A-1		N/S	6 Electrical	
S3. ANO-2-JPM-NRC-SDBC2 041 A4.05 RO-3.1/SRO-3.3 Manually Operate SDBCS valves		A/L/N/S	4 Heat Removal	
In-Plant Systems <sup>®</sup> (3 for RO); (3 for SRO-I); (3 or 2	2 for SRO-U)			
P2. ANO-2-JPM-NRC-MGSU 001 A4.08 RO-3.7/SRO-3.4 Start the First MG set		D/L	1 Reactivity control	
P3. ANO-2-JPM-NRC-FPEM1 033 A2.03 RO-3.1/SRO-3.5 Perform emergency SW make up to SFP		D/E/R	8 Plant service systems	
All RO and SRO-I control room (and in-plant) sometions; all 5 SRO-U systems must serve different overlap those tested in the control room.				
* Type Codes Criteria for RO / SRO-I / SRO-U				
(A)Iternate path (C)ontrol room				
(D)irect from bank (E)mergency or abnormal in-plant	≤9/≤8/≤4 ≥1/≥1/≥1			
(EN)gineered safety feature (L)ow-Power / Shutdown		- / - / ≥1 (con <sup>-</sup> ≥ 1 / ≥ 1 / ≥ 1	trol room system)	
(N)ew or (M)odified from bank including 1(A) (P)revious 2 exams		≥ 2 / ≥ 2 / ≥ 1 ≤ 3 / ≤ 3 / ≤ 2 (rand	domly colocted)	
(R)CA (S)imulator		≥ 1 / ≥ 1 / ≥ 1	donly sciected)	

UNIT: 2	REV #:	004	DATE:				
SYSTEM/DUTY A	REA: Emergency Core	Cooling System	S1				
TASK: Isolate	TASK: Isolate SITs following SIAS Actuation. SIAS has been reset. (Alternate Success Path)						
JTA#: ANO2R	OEOPAOPEMER13						
KA VALUE R	O: <u>4.0</u> SRO:	3.8	KA REFERENCE:	006 A4.02			
APPROVED FOR	ADMINISTRATION TO:	RO: X	SRO: X	-			
TASK LOCATION	I: INSIDE CR:	X OUTSIDE	CR:	ВОТН:			
SUGGESTED TE	STING ENVIRONMENT AN	ND METHOD (PE	RFORM OR SIMULA	TE):			
PLANT SITE:	SIMULA	ATOR: Peri	form LAB:				
POSITION EVAL	UATED: RO:	SRO:					
ACTUAL TESTIN	G ENVIRONMENT: SIN	IULATOR:	PLANT SITE:	LAB:			
TESTING METHO	DD: SIMULATE:	PERFORM:					
APPROXIMATE (	COMPLETION TIME IN MI	NUTES: 15 M	<u>/////////////////////////////////////</u>				
REFERENCE(S):	2202.010 ; OP 2104.001						
EXAMINEE'S NA	ME:		Badge #:				
EVALUATOR'S N	IAME:						
THE EXAMINEE'S PERFORMANCE WAS EVALUATED AGAINST THE STANDARDS CONTAINED IN THIS JPM AND IS DETERMINED TO BE:							
SATISFACTORY	UNSAT	ISFACTORY:					
PERFORMANCE CHECKLIST COMMENTS:							
Start Time	Stop Time	Total Time					
SIGNED:		DATE:					

SIGNATURE INDICATES THIS JPM HAS BEEN COMPARED TO ITS APPLICABLE PROCEDURE BY A QUALIFIED INDIVIDUAL (NOT THE EXAMINEE) AND IS CURRENT WITH THAT REVISION.

# ANO-2-JPM-NRC-SIT09 PAGE 2 OF 7 JOB PERFORMANCE MEASURE

THE EXAMINER SHALL REVIEW THE FOLLOWING WITH THE EXAMINEE: The examiner shall ensure that the examinee has been briefed on NUREG 1021 Appendix E. JPM INITIAL TASK CONDITIONS: A LOCA is in progress. SIAS has been reset. EOP 2202.010 Attachment 36 is complete through Step TASK STANDARD: SIT A, B, and C outlet valves are closed AND the ability to vent the affected SIT (D) is demonstrated. TASK PERFORMANCE AIDS: Examiner Note: Give the Yellow pages to the applicant. When the alternate procedure is required then hand the additional (different color) pages to the applicant. EOP 2202.010 Attachment 36; OP 2104.001 Section 7.0 SIMULATOR SETUP: PZR pressure ~ 650psia. SIT outlet valves have had breakers closed. SIAS and CIAS have been RESET. Safety Injection Tank (2T2D) isolation valve (2CV-5063-2) is set to fail intermediate. Set Event trigger 4 to NE4G0632 (this will trigger T4 when 2CV5063-2 is started closed. T4=CV50632 =.85

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•	41 I	-		10	$\sim$	<i>-</i>

The CRS directs, "Isolate the SIT's using Standard Attachment 36, SIT isolation starting with step 2.

START Time:

	PERF	ORMANCE CHECKLIST	STANDARDS	(Circle One)	
	Examiner Note: Give the Yellow pages to the applicant. When the alternate procedure is required then hand the additional (different color) pages to the applicant.				
	1. (Step 2)	Contact NLO to Locally remove danger tags, unlock, and close the following breakers:	Contacted NLO to Locally remove danger tags, unlock, and close the following breakers:	N/A SAT UNSAT	
		2B51-F2 - 2CV5003-1	2B51-F2 - 2CV5003-1		
		2B51-H1 - 2CV5023-1	2B51-H1 - 2CV5023-1		
		2B61-F2 - 2CV5043-2	2B61-F2 - 2CV5043-2		
		2B61-H1 - 2CV5063-2	2B61-H1 - 2CV5063-2		
		Examiner Cue:			
		When contacted state that the above tags and locks are removed and breakers are closed.			
(C)	2. (Step 3)	Close SIT 2T2A Outlet valve (2CV-5003-1).	On panel 2C17, placed handswitch for 2CV-5003-1 in "CLOSE".	N/A SAT UNSAT	
			Observed green light ON; red light OFF above handswitch.		
(C)	3. (Step 3)	Close SIT 2T2B outlet valve (2CV-5023-1).	On panel 2C17, placed handswitch for 2CV-5023-1 in "CLOSE".	N/A SAT UNSAT	
			Observed green light ON; red light OFF above handswitch.		
(C)	4. (Step	Close SIT 2T2C outlet valve (2CV-5043-2).	On panel 2C16, closed 2CV-5043-2.	N/A SAT UNSAT	
	3)		Observed green light ON; red light OFF above handswitch.		

	PERFORMANCE CHECKLIST		STANDARDS	(Circle One)
(C)	5. (Step 3)	Close SIT 2T2D outlet valve (2CV-5063-2).	On panel 2C16, placed handswitch for 2CV-5063-2 in "CLOSE".	N/A SAT UNSAT
		Examiner Cue: If requested as NLO report that breaker 2B-61 H1 is in the trip free position. If requested to locally check valve position report that CNTMT is not open for access.	Observed green light ON; red light ON above handswitch.  Reported to the SM/CRS that SIT 2T2D outlet valve did NOT close.	

#### **EXAMINER'S NOTE:**

The following step is an Alternate path. Provide the following cue upon receiving the report that 2T2D outlet valve will NOT close:

The SM/CRS directs Continue standard attachment 36, SIT isolation and vent 2T-2D from the control room using the normal vent line.

6. (Step 4)	IF ANY SIT can NOT be isolated, THEN vent or drain affected SIT using 2104.001, Safety Injection Tank Operation.	Transitioned to 2104.001 section 12.	N/A SAT UNSAT
7. (Step 12.1)	IF 18-Month Remote Position Indication (Supplement 4 of Cold Shutdown Valve Testing, 2305.006) required, THEN perform in conjunction with this activity.  Examiner Cue: State that the 18 month remote position indication check is not required.	Asked CRS/SM or determined if the 18-Month Remote Position Indication is required.	N/A SAT UNSAT
8. (Step 12.2.1	IF Containment ventilation available, THEN verify Containment ventilation in service per Containment Atmosphere Control (2104.033).	Asked CRS/SM or determined if the 18-Month Remote Position Indication is required.	N/A SAT UNSAT
	Examiner Cue: State that Containment ventilation is not available.		

# ANO-2-JPM-NRC-SIT09 PAGE 5 OF 7 JOB PERFORMANCE MEASURE

	PERF	ORMANCE CHECKLIST	STANDARDS	(Circle One)	
	9. (Step 12.2.2	IF Equipment Hatch or Temporary Equipment Hatch open, THEN verify air flow into Containment Building.  Examiner Cue: State that	Asked CRS/SM or determined that the Equipment Hatch is closed.	N/A SAT UNSAT	
	10. (Step 12.2.3	IF personnel in CNTMT, THEN perform the following:	Asked CRS/SM or determined that the personnel are not in CNTMT.	N/A SAT UNSAT	
	,	Examiner Cue: State that there are no personnel in CNTMT.			
(C)	11. (Step 12.2.7	IF venting SIT (2T-2D), THEN open ONE of the following:	Opened SIT D vent 2SV-5066. (critical portion)	N/A SAT UNSAT	
	)	SIT D Vent (2SV-5066) SIT D Vent Bypass (2SI-50)	Observed Red light On and Green light OFF above the HS for 2SV-5066.		
			Observed 'D' SIT pressure lowering.		
	EXAMINER'S NOTE:				
This J	This JPM can be stopped once the vent path is established (2SV-5066 open).				
	END				

### **EXAMINER'S COPY**

#### JPM INITIAL TASK CONDITIONS:

You are responsible for any applicable annunciators during the performance of this task. All annunciators that are not applicable to this task will be performed by another operator.

A LOCA is in progress. SIAS has been reset. EOP 2202.010 Attachment 36 is complete through Step 1.

#### **INITIATING CUE:**

The CRS directs, "Isolate the SIT's using Standard Attachment 36, SIT isolation starting with step 2.

#### **EXAMINEE'S COPY**

#### JPM INITIAL TASK CONDITIONS:

You are responsible for any applicable annunciators during the performance of this task. All annunciators that are not applicable to this task will be performed by another operator.

A LOCA is in progress. SIAS has been reset. EOP 2202.010 Attachment 36 is complete through Step 1.

#### **INITIATING CUE:**

The CRS directs, "Isolate the SIT's using Standard Attachment 36, SIT isolation starting with step 2.

UNIT:	2	REV #:	001	DATE:	
SYSTE	M/DUTY AREA	: AC Electrical [	Distribution	S2	
TASK:	Transfer A	Auxiliaries from SU	J#2 to SU#3	for 2A-1.	
JTA#:	ANO2-RO-E	LECD-NORM-30			
KA VAL	.UE RO: _	3.1 SRO:	3.1	_ KA REFERENCE:	062 A4.07
APPRO	VED FOR ADM	MINISTRATION TO:	RO:X	_ SRO:χ	_
TASK L	OCATION:	INSIDE CR:	χ outsi	DE CR:	вотн:
SUGGE	STED TESTIN	G ENVIRONMENT AN	ID METHOD (P	ERFORM OR SIMULA	TE):
PLANT	SITE:	SIMULA	TOR: Perfo	orm CLASSR	OOM:
POSITION	ON EVALUATE	ED: RO:	SRO:		
ACTUA	L TESTING EN	IVIRONMENT: SIM	ULATOR:	PLANT SITE:	CLASSROOM:
TESTIN	IG METHOD:	SIMULATE:	PERFORM	<b>1</b> :	
APPRO	XIMATE COM	PLETION TIME IN MIN	IUTES: 10	Minutes	
REFERI	ENCE(S): OF	P-2107.001 Electrical \$	System Operat	ions	
EXAMIN	NEE'S NAME:			Badge #:	
EVALU	ATOR'S NAME	·			
	(AMINEE'S PEI ID IS DETERM		VALUATED AG	AINST THE STANDAR	RDS CONTAINED IN THIS
SATISF	ACTORY:	UNSAT	ISFACTORY:		
PERFO	RMANCE CHE	CKLIST COMMENTS:			
Start Time		Stop Time	Total Time		
SIGNED	D:		DATE	i:	

SIGNATURE INDICATES THIS JPM HAS BEEN COMPARED TO ITS APPLICABLE PROCEDURE BY A QUALIFIED INDIVIDUAL (NOT THE EXAMINEE) AND IS CURRENT WITH THAT REVISION.

THE EXAMINER SHALL REVIEW THE FOLLOWING WITH THE EXAMINEE: The examiner shall ensure that the examinee has been briefed on NUREG 1021 Appendix E. JPM INITIAL TASK CONDITIONS: Maintenance is complete on 2A-113 SU#2 powering 2A-1 and SU#3 is available to 2A-1. Steps 10.1 and 10.2 are complete. TASK STANDARD: Energized 2A-1 from SU#3 TASK PERFORMANCE AIDS: **OP 2107.001 Electrical System Operations.** Examiner Note: Give the Yellow pages to the applicant. SIMULATOR SETUP:

#### **INITIATING CUE:**

The CRS directs, "Transfer Unit Auxiliaries from SU2 to SU3 on 2A-1 using 2107.001 Section 10.0, starting with step 10.3"

START TIME:	
-------------	--

PERF	ORMANCE CHECKLIST	STANDARDS	(Circle One)	
Examiner Note	Examiner Note: Give the Yellow pages to the applicant.			
BLACK so	ning (Source) and Running (Bus) Vale is Generator Output Breakers. e is 6900V buses.			
1. (Step 10.3.1)	Verify Synchroscope operable as follows:  Select a closed breaker AND insert Synchronize switch.	On panel 2C10, inserted synchronize switch into a closed breaker.  Observed red light ON above handswitch and inserted synchronize switch.	N/A SAT UNSAT	
2. (Step 10.3.2)	Place Synchronize switch to ON for selected closed breaker.	On panel 2C10, placed synchronized switch to on.  Observed the incoming and running volt meters come on scale.	N/A SAT UNSAT	
3. (Step 10.3.3)	Check Incoming (Source) and Running (Bus) volt meters within 200 volts of each other with Running preferably higher than Incoming.	On panel 2C10, checked incoming and running voltages within 200 volts.	N/A SAT UNSAT	
4. (Step 10.3.4)	Place Synchronize switch to OFF AND remove.	On panel 2C10, placed synchronized switch to off and removed from breaker.  Observed the incoming and running volt meters go off scale low.	N/A SAT UNSAT	
5. (Step 10.4.1)	Transfer selected buses from SU2 to SU3 in any order as follows:  Insert Synchronize switch in selected SU3 Feeder Breaker.	On panel 2C10, inserted synchronize switch into breaker 2A-113.	N/A SAT UNSAT	

PERFORMANCE CHECKLIST		ORMANCE CHECKLIST	STANDARDS	(Circle One)
(C)	6. (Step 10.4.2)	Place Synchronize switch to ON.	On panel 2C10, placed synchronized switch to on for breaker 2A-113.	N/A SAT UNSAT
			Observed the incoming and running volt meters come on scale.	
	7. (Step 10.4.3)	Verify Synchroscope between 11 and 1 O'clock position.	On panel 2C10, Checked Synchroscope between 11 and 1 O'clock position.	N/A SAT UNSAT
(C)	8. (Step 10.4.4)	Momentarily place SU3 Feeder Breaker to CLOSE.	On panel 2C10, placed handswitch 152-113CS to the close position and then released the handswitch to the normal after close position.  Observed Red light on, Green light off and white light momentarily off for breaker 2A-113	N/A SAT UNSAT
	9. (Step 10.4.5)	Verify SU3 Feeder Breaker closes.	Checked 2A-113 breaker for SU3 to 2A-1 closed.  Observed Red light on, Green	N/A SAT UNSAT
			light off and white light momentarily off for breaker 2A-113	
	10. (Step 10.4.6)	Check SU2 Feeder Breaker opens.	Recognized that breaker 2A-111 for SU#2 to 2A-1 is opened.	N/A SAT UNSAT
			Observed Red light off, Green light on and white light on for breaker 2A-111	
	11. (Step 10.4.7)	IF SU2 Feeder Breaker fails to open, THEN perform the following:	On panel 2C10, determined that the SU#2 feeder breaker opened.	N/A SAT UNSAT
		Trip SU3 Feeder Breaker.  Examiner Note: Step is N/A	Observed Red light off, Green light on and white light on for breaker 2A-111	
	12. (Step 10.4.8)	Momentarily place SU#2 Feeder Breaker in OPEN to Green Flag.	On panel 2C10, placed 2A-111 hand switch to the open position (152-111 CS) and released to normal after trip.	N/A SAT UNSAT

PERFORMANCE CHECKLIST		ORMANCE CHECKLIST	STANDARDS	(Circle One)
	13. (Step 10.4.9)	Place Synchronize switch to OFF and remove.	On panel 2C10, placed synchronized switch to off and removed from breaker 2A-113.	N/A SAT UNSAT
Examiner note: When the applicant Notifies Unit 1 of SU2 Feeder breaker switch positions the JPM is complete.				
END				

STOP TIME:
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#### **EXAMINER's COPY**

#### JPM INITIAL TASK CONDITIONS:

You are responsible for any applicable annunciators during the performance of this task. All annunciators that are not applicable to this task will be performed by another operator.

SU2 is powering 2A-1 bus due to 2A-113 SU3 feeder breaker maintenance. 2A-113 SU3 feeder Breaker Maintenance has been completed. Steps 10.1 and 10.2 of 2107.001 have been completed.

#### **INITIATING CUE:**

The CRS directs, "Transfer Unit Auxiliaries from SU2 to SU3 on 2A-1 using 2107.001 Section 10.0, starting with step 10.3"

#### **EXAMINEE's COPY**

#### JPM INITIAL TASK CONDITIONS:

You are responsible for any applicable annunciators during the performance of this task. All annunciators that are not applicable to this task will be performed by another operator.

SU2 is powering 2A-1 bus due to 2A-113 SU3 feeder breaker maintenance. 2A-113 SU3 feeder Breaker Maintenance has been completed. Steps 10.1 and 10.2 of 2107.001 have been completed.

#### **INITIATING CUE:**

The CRS directs, "Transfer Unit Auxiliaries from SU2 to SU3 on 2A-1 using 2107.001 Section 10.0, starting with step 10.3"

UNIT: 2 REV #: 002 DATE:					
SYSTEM/DUTY AREA: Steam Dump and Bypass Control System S3					
TASK: Manually operate SDBCS valve. (Alternate Success Path)					
JTA#: ANO2-RO-SDBCS-OFFNORM-508					
KA VALUE RO: 3.1 SRO: 3.3 KA REFERENCE: 041 A4.05					
APPROVED FOR ADMINISTRATION TO: RO: X SRO: X					
TASK LOCATION: INSIDE CR: X OUTSIDE CR: BOTH:					
SUGGESTED TESTING ENVIRONMENT AND METHOD (PERFORM OR SIMULATE):					
PLANT SITE: SIMULATOR: Perform CLASSROOM:					
POSITION EVALUATED: RO: SRO:					
ACTUAL TESTING ENVIRONMENT: SIMULATOR: PLANT SITE: CLASSROOM:					
TESTING METHOD: SIMULATE: PERFORM:					
APPROXIMATE COMPLETION TIME IN MINUTES: 15 Minutes					
REFERENCE(S): OP-2105.008 Steam Dump and Bypass control System Operations, OP-2203.012B 2K-02 ACA					
EXAMINEE'S NAME: Badge #:					
EVALUATOR'S NAME:					
THE EXAMINEE'S PERFORMANCE WAS EVALUATED AGAINST THE STANDARDS CONTAINED IN THIS JPM AND IS DETERMINED TO BE:					
SATISFACTORY: UNSATISFACTORY:					
PERFORMANCE CHECKLIST COMMENTS:					
Start Stop Total Time Time Time					
SIGNED: DATE:					

SIGNATURE INDICATES THIS JPM HAS BEEN COMPARED TO ITS APPLICABLE PROCEDURE BY A QUALIFIED INDIVIDUAL (NOT THE EXAMINEE) AND IS CURRENT WITH THAT REVISION.

THE EXAMINER SHALL REVIEW THE FOLLOWING WITH THE EXAMINEE: The examiner shall ensure that the examinee has been briefed on NUREG 1021 Appendix E. JPM INITIAL TASK CONDITIONS: Reactor tripped and the input to the SDBCS failed low. TASK STANDARD: Steam Generator pressure controlled between 950 psia and 1050 psia. TASK PERFORMANCE AIDS: OP 2203.012B ACA for B-14, and OP 2105.008 SDBCS operations. Examiner Note: Give the Yellow pages to the applicant. When the alternate procedure is required then hand the additional (different color) pages to the applicant. SIMULATOR SETUP: **SDBCS input XMSHDRPRS = 650.** Condenser interlock set on trigger 1 value = true, delete in 5 sec. Event trigger 1 set for 'A' S/G pressure (SGPD(2)) < 1050 psia.

#### **INITIATING CUE:**

The CRS directs, "Manually control both S/G pressures between 950 psia and 1050 psia using turbine bypass valve IAW 2105.008 Exhibit 3, SDBCS emergency operations"

PERF	ORMANCE CHECKLIST	STANDARDS	(Circle One)	
Examiner Note: Give the Yellow pages to the applicant. When the alternate procedure is required then hand the additional (different color) pages to the applicant.				
1. (Step 1)	IF BOTH MSIV's closed, THEN GO TO step 5.0.	On panel 2C16 and 2C17 determined MSIV's 2CV-1010-1 and 2CV-1060-2 open.	N/A SAT UNSAT	
	Examiner note: Step 1 is N/A.	Observed red light ON above hand switches		
2. (Step 2.1)	Perform the following to determine availability of SDBCS valves:  IF the following conditions satisfied, THEN SDBCS Master controller (2PIC-0300) available:  SDBCS controlling S/G pressure at setpoint in automatic  Emergency OFF (2K02-A14) clear  Instrument air available  IF using Turbine Bypass valves, THEN Condenser Interlock (2K02-B14) clear	On panel 2C02, determined that the SDBCS Master controller 2PIC-0300 is not available by the fact it is not controlling at setpoint.  Observed setpoint is ~1040 psig and input is ~ 650 psig with all SDBCS valves closed. Or observed that steam generator pressure is approximately 1100 psia with all SDBCS valves closed with a setpoint of ~1040 psig on 2PIC-0300.	N/A SAT UNSAT	
3. (Step 2.2)	IF the following conditions satisfied, THEN SDBCS Downstream ADV/Turbine Bypass valves available:  Instrument air available Emergency OFF (2K02-A14) clear Power available to selected controllers / valves IF using Turbine Bypass valves, THEN Condenser Interlock (2K02-B14) clear	On panel 2C-14 or PMS verified ~ 100 psig Instrument air pressure.  On 2K02 verified Emergency OFF (2K02-A14) clear. (light out)  On panel 2C-02 verified power available for the selected controllers from the following list: 2HIC-0301, 2HIC-0302, 2HIC-0305.  On 2K02 verified Condenser Interlock (2K02-B14) clear. (light out)	N/A SAT UNSAT	

# ANO-2-JPM-NRC-SDBC2 PAGE 4 OF 7 JOB PERFORMANCE MEASURE

	PERF	ORMANCE CHECKLIST	STANDARDS	(Circle One)	
	Examiner Note:				
	Step # 3 is not applicable.				
	Procedure Note: The SDBCS Master controller can not be set less than 650 psi. Computer points FR1030 and FR1130 can be useful to monitor steam flow.				
(C)	4. (Step 4.1)	IF SDBCS Downstream ADV/Turbine Bypass valves operation in manual desired, THEN perform the following:  Place selected HIC(s) in MANUAL:  2HIC-0301 Downstream Atmospheric Dump valve 2HIC-0305 Downstream Atmospheric Dump valve 2HIC-0302 Turbine Bypass valve 2HIC-0303 Turbine Bypass valve 2HIC-0306 Turbine Bypass valve	On panel 2C02, placed hand indicating controller for any of the following turbine bypass valves in manual.  • 2HIC-0302 Turbine Bypass valve  • 2HIC-0303 Turbine Bypass valve  • 2HIC-0306 Turbine Bypass valve  Observed M on selected controllers placed in manual.	N/A SAT UNSAT	
	5. (Step 4.2)	Adjust output on selected HIC(s) as desired.	Adjusted hand indicating controller output to zero for any of the following controllers.  • 2HIC-0302 Turbine Bypass valve • 2HIC-0303 Turbine Bypass valve • 2HIC-0306 Turbine Bypass valve Observed zero demand on the controller selected for manual operation.	N/A SAT UNSAT	

	PERF	ORMANCE CHECKLIST	STANDARDS	(Circle One)	
(C)	6. (Step 4.3)	Place the selected permissive(s) handswitch to MANUAL:  • 2CV-0301 Downstream Atmospheric Dump valve (2HS-0301)  • 2CV-0305 Downstream Atmospheric Dump valve (2HS-0305)  • 2CV-0302 Turbine Bypass valve (2HS-0302)  • 2CV-0303 Turbine Bypass valve (2HS-0303)  • 2CV-0306 Turbine Bypass valve (2HS-0306)	Placed permissive handswitch for at least one turbine bypass valve(s) to MANUAL that the associated HIC was place in manual in step 4.1.  • 2CV-0302 Turbine Bypass valve (2HS-0302)  • 2CV-0303 Turbine Bypass valve (2HS-0303)  • 2CV-0306 Turbine Bypass valve (2HS-0306)	N/A SAT UNSAT	
(C)	7. (Step 4.4)	*Adjust output on selected HIC as desired.	Adjusted output on selected controllers to lower steam generator pressure into the directed steam generator pressure band of 950 psig to 1050 psig.  Observed selected valve(s) open indication of red light and green light on. Also, observed Steam generator pressure lowering.	N/A SAT UNSAT	
		EXAM	MINER'S NOTE:	I	
	The following is the alternate path portion of this JPM. When S/G pressure reaches 1050 psia an inadvertent Condenser interlock will occur. The following steps are for the Condenser interlock Annunciator Corrective Action				
		Pro	ocedure Note:		
	This in	terlock blocks opening of Turbine E	Bypass valves 2CV-0302, 2CV-0303	, and 2CV-0306.	
(C)	8. (Step 2.1)	Verify SDBCS Atmospheric Dumps opening as required:  • 2CV-0301 Downstream Atmospheric Dump valve • 2CV-0305 Downstream Atmospheric Dump valve  Examiner CUE:  When the applicant is	Used Exhibit 3 of 2105.008 to control S/G pressure and operated selected Atmospheric dump valves.	N/A SAT UNSAT	
		controlling Steam generator pressure between 950 and 1050 psia, THEN end this JPM.			

# ANO-2-JPM-NRC-SDBC2 PAGE 6 OF 7

PERFORMANCE CHECKLIST	STANDARDS	(Circle One)	
	END		

### **EXAMINER's COPY**

### JPM INITIAL TASK CONDITIONS:

You are responsible for any applicable annunciators during the performance of this task. All annunciators that are not applicable to this task will be performed by another operator.

Unit 2 reactor is offline.

The Steam Dump and Bypass control system steam pressure input has failed low.

### **INITIATING CUE:**

The CRS directs, "Manually control both S/G pressures between 950 psia and 1050 psia using turbine bypass valve IAW 2105.008 Exhibit 3, SDBCS emergency operations".

### **EXAMINEE's COPY**

### JPM INITIAL TASK CONDITIONS:

You are responsible for any applicable annunciators during the performance of this task. All annunciators that are not applicable to this task will be performed by another operator.

Unit 2 reactor is offline.

The Steam Dump and Bypass control system steam pressure input has failed low.

### **INITIATING CUE:**

The CRS directs, "Manually control both S/G pressures between 950 psia and 1050 psia using turbine bypass valve IAW 2105.008 Exhibit 3, SDBCS emergency operations".

UNIT:	2			REV #:	009	_		DATE:	·	
SYSTEM	/DUTY	AREA:	Compone	ent Coo	ling W	ater Syste	m ·	S4		
TASK:	Secur	e CCW sy	stem (Usi	ng EOP	)					
JTA#:	ANO-2	2-R-OEOF	AOP-EME	RG-13						
KA VALU	IE I	RO:	3.3	SRO:		3.1	KA REF	ERENCE	:	008 A4.01
APPROV	ED FO	R ADMINI	STRATIO	N TO:	RO:	X	SRO: _	Х		
TASK LO	CATIC	N:	INSIDE C	R:	X	OUTSIDI	E CR:		вотн: _	
SUGGES	SUGGESTED TESTING ENVIRONMENT AND METHOD (PERFORM OR SIMULATE):									
PLANT S	ITE:			SIMULA	ATOR:	Per	form	_ LAB:		
POSITIO	N EVA	LUATED:	RO:			_ SRO:				
ACTUAL	TESTI	NG ENVIF	RONMENT	: SIN	IULAT	OR:	PLA	NT SITE	:	LAB:
TESTING	METH	HOD: S	SIMULATE	:	P	ERFORM:				
APPROX	IMATE	COMPLE	TION TIM	E IN MII	NUTES	i: <u>15 l</u>	Vinutes	<u> </u>		
REFERE	NCE(S)	): 2202.0	010 Standa	ard Atta	chmen	its				
EXAMINE	EE'S N	AME:					ss	6N:		
EVALUA	TOR'S	NAME:								
THE EXAMINEE'S PERFORMANCE WAS EVALUATED AGAINST THE STANDARDS CONTAINED IN THIS JPM AND IS DETERMINED TO BE:										
SATISFA	SATISFACTORY: UNSATISFACTORY:									
PERFOR	MANC	E CHECK	LIST COM	IMENTS	:					
Start Time			op me		To	otal Time				
SIGNED:						DATE:				

SIGNATURE INDICATES THIS JPM HAS BEEN COMPARED TO ITS APPLICABLE PROCEDURE BY A QUALIFIED INDIVIDUAL (NOT THE EXAMINEE) AND IS CURRENT WITH THAT REVISION.

#### THE EXAMINER SHALL REVIEW THE FOLLOWING WITH THE EXAMINEE:

The examiner shall ensure that the examinee has been briefed on NUREG 1021 Appendix E.

JPM INITIAL TASK CONDITIONS: \_The reactor has tripped due to an Loss of Coolant Accident. SIAS has actuated and the Service Water system (suction and discharge) is aligned to the Emergency Cooling Pond. - Actions of OP 2202.003 are complete to Contingency Action step 9.C.4.

TASK STANDARD: CCW loads have been secured and CCW pumps have been stopped.

TASK PERFORMANCE AIDS: EOP 2202.010 Standard Attachment 6

**Examiner Note: Give the Yellow pages to the applicant.** 

SIMULATOR SETUP: SIAS ACTUATED. Align service water suction and discharge to the ECP. (note: The snap needs to be stabilized soon after the trip or RCP high temperature alarms will actuate before RCP's can be secured.)

### **INITIATING CUE:**

The CRS directs: "Secure the CCW system using OP 2202.010 Attachment 6 Step 1."

Start Time:

	PERF	ORMANCE CHECKLIST	STANDARDS	(Circle One)	
Exami	Examiner Note: Give the Yellow pages to the applicant.				
(C)	1. (Step 1.A)	Secure all RCP's.	On panel 2C04, placed the following hand switches in PULL TO LOCK or STOP:  • 2HS-4620 (2P32A),  • 2HS-4621 (2P32B),  • 2HS-4720 (2P32C)  • 2HS-4721 (2P32D)  Observed green lights ON; red lights OFF above RCP hand switches.	N/A SAT UNSAT	
	2. (Step 1.B)	Place both spray valves in MANUAL.	On panel 2C04, placed spray valve man/auto select hand switches 2HS-4651B ("A" Spray Line) and 2HS-4652B ("B" Spray Line) in MANUAL.	N/A SAT UNSAT	
	3. (Step 1.B)	Verify both spray valves closed.	On panel 2C04, observed green light ON above spray valve hand switches for 2CV-4651 and 2CV-4652.	N/A SAT UNSAT	
			OR  Held spray valve hand switches for 2CV-4651 and/or 2CV-4652 in CLOSE until the green light was ON above associated hand switches.		

	PERF	ORMANCE CHECKLIST	STANDARDS	(Circle One)
(C)	4. (Step 1.C)	Verify ALL RCP Controlled Bleedoff valves closed.	On panel 2C17 verified Controlled Bleedoff isolation (2CV-4846-1) Closed.	N/A SAT UNSAT
			On panel 2C16 verified Controlled Bleedoff isolation (2CV-4847-2) Closed.	
			On panel 2C09 closed Controlled Bleedoff relief isolation (2CV-4856). (Only critical portion of this step.)	
			Observed green light ON; red light OFF above all of the above controlled bleedoff isolation valve hand switches.	
		Pr	ocedure note:	
The re	_	steps of this attachment should ha	ave a lower priority than implementa	ition of the controlling
	5. (Step 1.D)	Verify at least ONE Letdown Isolation valve closed:	On panel 2C09, verified Letdown Isolation (2CV-4820-2) Closed.	N/A SAT UNSAT
		2CV-4820-2 2CV-4821-1	<u>OR</u>	
		2CV-4823-2 (least preferred)	On panel 2C09, verified Regen HX Inlet (2CV-4821-1) Closed.	
			<u>OR</u>	
			On panel 2C09, verified Regen HX Outlet (2CV-4823-2) closed.	
			Observed green light ON; red light OFF above at least one of the above letdown isolation valve's hand switches.	
	6. (Step 1.E)	Verify EFW suction aligned to a condensate storage tank.	On panel 2C17 observed green light OFF; red light ON above handswitch for 2CV-0789	N/A SAT UNSAT
		Examiner Cue: If asked, report as a NLO that EFW manual valves are aligned to the 'Q' CST.	On panel 2C16 observed green light OFF; red light ON above handswitch for 2CV-0795.	

PERF	ORMANCE CHECKLIST	STANDARDS	(Circle One)
7. (Step 1.F)	Locally close "SU/BD DI TO EFW PUMP SUCT" valve (2EFW-0706).	Either directed AO to verify 2EFW-0706 closed or verified 2EFW-0706 closed on 2C-33.	N/A SAT UNSAT
	Examiner Cue: If requested report that, 2EFW-0706 has been locally verified locked closed.	On panel 2C33, observed green light ON; red light OFF for SU/BD DI Effluent to EFW 2EFW-0706.	
8. (Step 1.G)	Verify ALL Condensate Pump hand switches in PTL.  Examiner Cue: Due to the location of the other JPM if the applicant attempts to check the condensate pumps, inform them that all condensate pumps are in PTL.	From the initial Cue determined that all Condensate pump HS are in PTL.	N/A SAT UNSAT
9. (Step 1.H)	Locally secure BOTH Waste Gas compressors by placing the following hand switches in OFF:  • "2C75A CONTROL SWITCH" (2HS-2402A) • "2C75B CONTROL SWITCH" (2HS-2402B)  Examiner Cue: Report that, 2C75A and 2C75B Waste Gas compressors have been verified secured.	Directed the WCO to secure both waste gas compressors by placing hand switches 2HS- 2402A and 2HS-2402B in OFF on panels 2C194A/B.	N/A SAT UNSAT

Р	ERFORMAN	CE CHECKLIST	STANDARDS	(	(Circle	One)
(1	unnece the foll 1.l)  2C Co 2C Sai 2C Sai 2C Sai Exami report:	7 5-I	Notified chemistry to secure all unnecessary sample flows to the following panels:  • 2C116 Sample System Control Panel.  • 2C337 SG Secondary Sample Panel.  • 2C145-I Secondary Sample Panel.  • 2C145-II Secondary Sample Panel.	N/A	SAT	UNSAT
(1	Step J.1)   Isolatic Step J.1)   • 2C • 2C	closed ALL RCP CCW on valves: V-5255-1 V-5254-2 V-5236-1	On Panel 2C17/2C16 closed the following valves by placing the hand switches to close:  • 2CV-5255-1 (Throttle valve HS must be maintained) • 2CV-5236-1 • 2CV-5254-2  Observed the Red light OFF and Green light ON.	N/A	SAT	UNSAT
(1		all CCW pump hand es in PULL TO LOCK.	Placed the following hand switches in PULL TO LOCK on panel 2C14:  • 2HS-5225 (2P33A)  • 2HS-5228 (2P33B)  • 2HS-5234 (2P33C)  Observed green lights ON and red lights OFF above hand switches.	N/A	SAT	UNSAT
	Examiner note: Due to the location of the other JPM, this JPM should be stopped when all CCW pumps are placed in PTL.					
			END			

Stop Time:

### **EXAMINER'S COPY**

### JPM INITIAL TASK CONDITIONS

You are responsible for any applicable annunciators during the performance of this task. All annunciators that are not applicable to this task will be performed by another operator.

The reactor has tripped due to a Loss of Coolant Accident.

SIAS has actuated and the Service Water system (suction and discharge) is aligned to the Emergency Cooling Pond due to Lake Dardanelle being unavailable

Actions of OP 2202.003 are complete to Contingency Action step 9.C.4

All condensate pumps are in PTL.

### **INITIATING CUE:**

The CRS directs: "Secure the CCW system using OP 2202.010 Attachment 6 Step 1."

### **EXAMINEE'S COPY**

### JPM INITIAL TASK CONDITIONS

You are responsible for any applicable annunciators during the performance of this task. All annunciators that are not applicable to this task will be performed by another operator.

The reactor has tripped due to a Loss of Coolant Accident.

SIAS has actuated and the Service Water system (suction and discharge) is aligned to the Emergency Cooling Pond due to Lake Dardanelle being unavailable

Actions of OP 2202.003 are complete to Contingency Action step 9.C.4

All condensate pumps are in PTL.

### **INITIATING CUE:**

The CRS directs: "Secure the CCW system using OP 2202.010 Attachment 6 Step 1."

UNIT:2	REV #: <u>001</u>	DATE:			
SYSTEM/DUTY AREA: _C	Containment S5				
TASK: Drain the Cont	ainment Sump. (Alternate	Success Path)			
JTA#: ANO-2-WCO-LRW	/BMS-NORM-7				
KA VALUE RO:3	3.9 SRO: 4.2	KA REFERENCE:	103 A3.01		
APPROVED FOR ADMINIST	TRATION TO: RO: X	SRO:X			
TASK LOCATION:	NSIDE CR: X OUTSIE	DE CR: B0	отн:		
SUGGESTED TESTING EN	VIRONMENT AND METHOD (P	ERFORM OR SIMULATE)	:		
PLANT SITE:	SIMULATOR: Perfo	rm CLASSROOM:			
POSITION EVALUATED:	RO: SRO:				
ACTUAL TESTING ENVIRO	NMENT: SIMULATOR:	PLANT SITE:	CLASSROOM:		
TESTING METHOD: SIN	IULATE: PERFORM	l:			
APPROXIMATE COMPLETI	ON TIME IN MINUTES: 10	<u>Minutes</u>			
REFERENCE(S): OP-2104	I.014 LRW/BMS operations				
EXAMINEE'S NAME:		Badge #:			
EVALUATOR'S NAME:					
THE EXAMINEE'S PERFORMANCE WAS EVALUATED AGAINST THE STANDARDS CONTAINED IN THIS JPM AND IS DETERMINED TO BE:					
SATISFACTORY: UNSATISFACTORY:					
PERFORMANCE CHECKLIS	ST COMMENTS:				
Start Stop					
SIGNED:	DATE	i:			

SIGNATURE INDICATES THIS JPM HAS BEEN COMPARED TO ITS APPLICABLE PROCEDURE BY A QUALIFIED INDIVIDUAL (NOT THE EXAMINEE) AND IS CURRENT WITH THAT REVISION.

THE EXAMINER SHALL REVIEW THE FOLLOWING WITH THE EXAMINEE: The examiner shall ensure that the examinee has been briefed on NUREG 1021 Appendix E. JPM INITIAL TASK CONDITIONS: Containment Sump level ~76% TASK STANDARD: 2CV-2060-1 closed and Aux building sump level did not trend ≥100%. TASK PERFORMANCE AIDS: OP 2104.014 Section 20 **Examiner Note: Give the Yellow pages to the applicant.** SIMULATOR SETUP: Any mode, Containment Sump level ~76% CV20612 value = 1, ramp = 8 sec, set on a Conditional trigger set for rw\_2061o. Using the monitor function the containment sump level can be adjusted using the ctmwtr name and setting the value to 25,000.

### **INITIATING CUE:**

The CRS directs, "Drain the containment sump to 40% using 2104.014 LRW/BMS operations starting with step 20.1.3"

START TIME:\_\_\_\_\_

	PERFO	DRMANCE CHECKLIST	STANDARDS	(Circle One)	
Exa	miner No	te: Give the Yellow pages to the	applicant.		
• li	Cooldown (2102.010).				
		Proc	edure Caution:		
	tainment s capacity.	Sump drainage flow rate is greater	than both Aux Building Sump pump	os (2P-51A and 2P-51B)	
		Pro	ocedure Note:		
		Draining Containment Sump will	I cause rate of change hi alarm (2K0	01-H9).	
(C)	1. (Step 20.1.3)	Open the following valves to drain CNTMT sump: Containment Sump Isolation 2CV-2060-1 (2HS-2060-1) Containment Sump Isolation 2CV-2061-2 (2HS-2061-2)	On panel 2C17 placed CMNT sump isolation valve handswitch 2HS-2060-1 to open.  On panel 2C16 placed CMNT sump isolation valve handswitch 2HS-2061-2 to open.	N/A SAT UNSAT	
			Observed red lights ON above handswitchs		
	2. (Step 20.1.4)	Using level indication or corresponding computer points, monitor the following to prevent overflow:	On panel 2C14, monitored Aux Building sump level (2LIS-2000 or 2LIS-2000B), and 2T-20A (2LIS-2010) waste tank.	N/A SAT UNSAT	
		Aux Building sump (2LIS-2000 or 2LIS-2000B) In service Waste tank: - 2T-20A (2LIS-2010) - 2T-20B (2LIS-2012)	OR  Monitor Aux Building Sump level and 2T-20A waste tank level using PMS computer		

	PERF	ORMANCE CHECKLIST	STANDARDS	(Circle One)
(C)	3. (Step 20.1.5)	Cycle 2CV-2061-2 (2HS-2061-2) as necessary to prevent overflowing Aux Building sump.	On panel 2C16 placed CMNT sump isolation valve handswitch 2HS-2061-2 to close. And Observed did not close.	N/A SAT UNSAT
		Examiner Cue: If candidate reports to the CRS 2CV-2061-2 has failed open, then instruct candidate to take appropriate action.	Critical to maintain Aux building sump level <100%	
	1	EXAM	MINER'S NOTE:	
		The following is the a	Iternate path portion of this JPM	
(C)	4. (Step 20.1.6)	WHEN draining complete, THEN close the following valves:	On panel 2C17 placed CMNT sump isolation valve handswitch 2HS-2060-1 to closed.(Critical)	N/A SAT UNSAT
		2CV-2060-1 (2HS-2060-1)	Observed 2CV-2060-1 closed by the green light on and red light	
		Examiner Cue: If candidate reports to the CRS Tech entry is required due 2CV-2061-2 not closing acknowledge and report the CRS has entered the appropriate Tech Spec.	off. (Not Critical)  Maintained Aux Building sump level <100%. (Critical)	
			END	

STOP	TIME:	
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### **EXAMINER's COPY**

### JPM INITIAL TASK CONDITIONS:

You are responsible for any applicable annunciators during the performance of this task. All annunciators that are not applicable to this task will be performed by another operator.

Containment Sump level is ~76% Steps 20.1.1 and 20.1.2 of 2104.014 LRW/BMS operations are complete

### **INITIATING CUE:**

The CRS directs, "Drain the containment sump to 40% using 2104.014 LRW/BMS operations starting with step 20.1.3".

### **EXAMINEE's COPY**

### JPM INITIAL TASK CONDITIONS:

You are responsible for any applicable annunciators during the performance of this task. All annunciators that are not applicable to this task will be performed by another operator.

Containment Sump level is ~76% Steps 20.1.1 and 20.1.2 of 2104.014 LRW/BMS operations are complete

### **INITIATING CUE:**

The CRS directs, "Drain the containment sump to 40% using 2104.014 LRW/BMS operations starting with step 20.1.3".

UNIT:				
SYSTEM/DUTY AREA: Control Element Drive Mechanism Control System S6				
TASK: Perform control element assembly exercise.				
TASK# ANO2-RO-CEDM-SURV-13 :				
KA VALUE RO: <u>4.0</u> SRO: <u>3.7</u> KA REFERENCE: <u>001 A4.03</u>				
APPROVED FOR ADMINISTRATION TO: RO: X SRO: X				
TASK LOCATION: INSIDE CR: X OUTSIDE CR: BOTH:				
SUGGESTED TESTING ENVIRONMENT AND METHOD (PERFORM OR SIMULATE):				
PLANT SITE: SIMULATOR: Perform LAB:				
POSITION EVALUATED: RO: SRO:				
ACTUAL TESTING ENVIRONMENT: SIMULATOR: PLANT SITE: LAB:				
TESTING METHOD: SIMULATE: PERFORM:				
APPROXIMATE COMPLETION TIME IN MINUTES:15 Minutes				
REFERENCE(S): OP 2105.009 CEDM Control System Operations				
EXAMINEE'S NAME: Badge #:				
EVALUATOR'S NAME:				
THE EXAMINEE'S PERFORMANCE WAS EVALUATED AGAINST THE STANDARDS CONTAINED IN THIS JPM AND IS DETERMINED TO BE:				
SATISFACTORY: UNSATISFACTORY:				
PERFORMANCE CHECKLIST COMMENTS:				
Start         Stop         Total Time           Time				
SIGNED: DATE:				

SIGNATURE INDICATES THIS JPM HAS BEEN COMPARED TO ITS APPLICABLE PROCEDURE BY A QUALIFIED INDIVIDUAL (NOT THE EXAMINEE) AND IS CURRENT WITH THAT REVISION.

THE EXAMINER SHALL REVIEW THE FOLLOWING WITH THE EXAMINEE:
The examiner shall ensure that the examinee has been briefed on NUREG 1021 Appendix E.
JPM INITIAL TASK CONDITIONS:
260 EFPD. OP 2105.009 Supplement 2, "CEA Exercise Test" is completed successfully
through step 2.5.12 for all CEA's except CEA #46. No T-alt installed on either RSPT.
TASK STANDARD:
CEA #46 has been inserted seven (7) steps (5.25 inches) and withdrawn to the programmed insertion
OLA #40 has been inserted seven (1) steps (5.25 menes) and withdrawn to the programmed insertion
limit.
TASK PERFORMANCE AIDS:
OP 2105.009 Supplement 2. Complete Table 2 except for CEA 6-46.
Examiner Note: Give the Yellow pages to the applicant.
SIMULATOR SETUP:
CEA's withdrawn to Programmed insertion limit

### **INITIATING CUE:**

The CRS directs,	"Complete the	<b>CEA Exercise</b>	Test for	CEA #46	using OP	2105.009	Supplement	2
beginning Step 3.	.2.5."				•			

Start	Time:		

	PER	FORMANCE CHECKLIST	STANDARDS	(Circle One)
Exam	iner Note	: Give the Yellow pages to the appl	icant.	
	1. (Step 3.2.5.A)	IF CEA movement delayed for extended period, THEN verify Mode Select switch in OFF.		N/A SAT UNSAT
		Examiner Note: This step is a continuous action that must be completed while moving CEAs		
(C)	2. (Step 3.2.5.B)	Using operable CEA Position indicators, Verify CEA #46 position within 7 inches of CEA's 1, 47, 48 and 49 throughout exercise.	Observed all Group 6 CEA's within seven (7) inches using CEAC #1, CEAC #2, or PMS indications.	N/A SAT UNSAT
		Examiner Note: This step is a continuous action that must be completed while moving CEAs		
	3. (Step 3.2.5.C)	On all available CEAC Operator Module digital displays, Select position indication for CEA # 46.	On panel 2C03, CEA #46 displayed on all available CEAC module displays by one of the following methods:	N/A SAT UNSAT
			- Standard OM,	
			- CEA group plot	
			(Group 6 selected)	
			- CEA values Page 2	
			- CEA subgroup plot	
			(Group 6 selected)	
(C)	4. (Step 3.2.5.D)	Verify Group Select switch selected to group containing individual CEA	On panel 2C03, verified group 6 selected.	N/A SAT UNSAT
(C)	5. (Step 3.2.5.E)	Place the individual CEA selection switches to CEA #46.	On panel 2C03, selected CEA #46.	N/A SAT UNSAT
	0.2.J.L)		Observed the TENS in "4" and the UNITS in "6".	
			OR	
			Observed CEA #46 individual light ON.	

	PERFORMANCE CHECKLIST		PERFORMANCE CHECKLIST STANDARDS		(Circle One)
(C)	6. (Step 3.2.5.F)	Place the mode select switch to MANUAL INDIVIDUAL.	On panel 2C03, placed mode select switch to MI.  Observed MI (MANUAL INDIVIDUAL) light ON.	N/A SAT UNSAT	
(C)	7. (Step 3.2.5.G)	Verify CEA #46 at upper electrical limit (UEL).	On insert 2JC-9058, moved CEA 46 to UEL by going to WITHDRAW on CEA insert until #46 upper electrical limit (UEL) red light on.  Observed UEL red light ON.	N/A SAT UNSAT	

#### Procedure Note:

- CEA insertion is verified to be > 5" by observation of RSPTs. Observing 5.25" change in pulse counter position does NOT mean CEA has moved > 5".
- If all CEAs in the subgroup of CEA being inserted are above upper alarm deadband of 140.83 inches, the CEAC 1 or 2 CEA Deviation annunciators (2K04 J-5/2K04 J-6) will NOT actuate
- Use of CEAC #1 and CEAC #2 standard OM Module to monitor raw input signal is recommended to ensure CEA movement of 5 inches. CEAC group display uses process CEA positions that are rounded off to nearest inch.
- Due to scheduling or component malfunction it is acceptable for CEAs to be inserted to other than Power Operations (2102.004) Attachment D Programmed Insertion Limit.

(C)	8. (Step 3.2.5.H)	Insert CEA #46 seven steps (>5").	Inserted CEA seven steps (>5").  Observed insertion of 5.0 inches or greater on CEAC's #1	N/A SAT UNSAT
			and #2 displays.	
(C)	9. (Step 3.2.5.I)	Return CEA to UEL.	CEA #46 withdrawn to UEL.	N/A SAT UNSAT
	10. (Step 3.2.5.J)	Verify CEA returned to position required by Power Operation (2102.004), Programmed CEA Insertion to Minimize CEA Finger Wear (ATTACHMENT D).	Used Power Operation (2102.004) attachment D to position CEA to minimize Finger wear.	N/A SAT UNSAT
		Exam	iner note:	

#### Examiner note:

The following steps are from Power Operation (2102.004) attachment D.

11. (Step 1.0)	IF a Reactivity Management Brief has NOT been conducted, THEN perform a Reactivity Management Brief per COPD-030 with an SRO.	Determines that a reactivity brief has been conducted per COPD-30	N/A SAT UNSAT
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PER	RFORMANCE CHECKLIST	STANDARDS	(Circle One)	
12. (Step 2.0)	CEAs are positioned at or just below Upper Electrical Limit (UEL) using the following table as a guide.	Determine CEAs should be at UEL minus Two steps.	N/A SAT UNSAT	
	CYCLE POSITION OF BURNUP FULLY (EFPD) WITHDRAWN CEAS			
	> 250 UEL MINUS TWO STEPS			
13. (Step 3.1)	Withdraw CEA to its UEL light in Manual Individual (MI) control using CEDM Control System Operation (2105.009).	Verify CEA #46 is withdrawn to the UEL.	N/A SAT UNSAT	
14. (Step 3.2)	With CEA positioned at UEL, VERIFY Pulse Counter reset to 150 using Plant Computer Operations (1105.010).	Verified PMS computer is indicating 150 inches withdrawn for CEA #46	N/A SAT UNSAT	
(C) 15. (Step 3.3)	Using Pulse Counter indication, insert CEA to comply with Table 1 above using CEDM Control System Operation (2105.009).	CEA #46 inserted to programmed insertion limit (UEL - 2 steps).  Observed CEA #46 at 148.5" on Pulse counter (149" on CEAC's)	N/A SAT UNSAT	
16. (Step 3.2.5.K)	Record results.	Using OP 2105.009 Supplement A Table 1, recorded the results of CEA #46 exercise.	N/A SAT UNSAT	
1	Exami	iner note:		
The CEDMCS control may be left in any mode of operation.				
17. (Step 3.2.5.L)	WHEN all required CEA movements complete, THEN verify Mode Select switch in OFF.	On insert 2JC-9058, selected off mode of operation.	N/A SAT UNSAT	
		END		

#### **EXAMINER'S COPY**

### JPM INITIAL TASK CONDITIONS:

You are responsible for any applicable annunciators during the performance of this task. All annunciators that are not applicable to this task will be performed by another operator.

- 260 EFPD.
- OP 2105.009 Supplement 2, "CEA Exercise Test" is completed successfully through step 3.2.5.K for all CEA's except CEA #46.
- No T-alt installed on either RSPT.
- Voltage Traces are not required.
- A reactivity brief is has been conducted per COPD-30 for this evolution.

### **INITIATING CUE:**

The CRS directs, "Complete the CEA Exercise Test for CEA # 46 using OP 2105.009 Supplement 2, beginning with Step 3.2.5."

### **EXAMINEE'S COPY**

### JPM INITIAL TASK CONDITIONS:

You are responsible for any applicable annunciators during the performance of this task. All annunciators that are not applicable to this task will be performed by another operator.

- 260 EFPD.
- OP 2105.009 Supplement 2, "CEA Exercise Test" is completed successfully through step 3.2.5.K for all CEA's except CEA #46.
- No T-alt installed on either RSPT.
- Voltage Traces are not required.
- A reactivity brief is has been conducted per COPD-30 for this evolution.

### **INITIATING CUE:**

The CRS directs, "Complete the CEA Exercise Test for CEA # 46 using OP 2105.009 Supplement 2, beginning with Step 3.2.5."

UNIT: <u>2</u>	REV #:	013	DATE:	
SYSTEM/DUTY AREA:	CPC/CEAC Sy	stem S7		
TASK: Place #1 CE	EAC to INOP on	"B" CPC		
JTA#: ANO2-RO-C	PC-NORM-7			
KA VALUE RO:	3.3 SRO:	3.4	KA REFERENCE:	012 A4.02
APPROVED FOR ADMII	NISTRATION TO:	RO: X	SRO: X	
TASK LOCATION:	INSIDE CR:	χ OUTSIDE	E CR:	вотн:
SUGGESTED TESTING	ENVIRONMENT AN	ND METHOD (PEI	RFORM OR SIMULA	ΓE):
PLANT SITE:	SIMULA	ATOR: Perfor	CLASSROO	M:
POSITION EVALUATED	): RO:	SRO:		
ACTUAL TESTING ENV	IRONMENT: SIM	IULATOR:	PLANT SITE:	CLASSROOM:
TESTING METHOD:	SIMULATE:	PERFORM:		
APPROXIMATE COMPL	ETION TIME IN MIN	NUTES: 15 M	linutes_	
REFERENCE(S): OP-2	2105.001 CPC/CEA	C Operations		
EXAMINEE'S NAME:			Badge #:	
EVALUATOR'S NAME:				
THE EXAMINEE'S PERI JPM AND IS DETERMIN		VALUATED AGA	INST THE STANDAR	DS CONTAINED IN THIS
SATISFACTORY:	UNSAT	ISFACTORY:		
PERFORMANCE CHEC	KLIST COMMENTS	:		
	Stop Time	Total Time		
SIGNED:		DATE:		

SIGNATURE INDICATES THIS JPM HAS BEEN COMPARED TO ITS APPLICABLE PROCEDURE BY A QUALIFIED INDIVIDUAL (NOT THE EXAMINEE) AND IS CURRENT WITH THAT REVISION.

THE EXAMINER SHALL REVIEW THE FOLLOWING WITH THE EXAMINEE:
The examiner shall ensure that the examinee has been briefed on NUREG 1021 Appendix E.
JPM INITIAL TASK CONDITIONS:
The unit is in Mode 1 (power > 20%). CEAC #1 monthly surveillance authorized.
The excel CPC log is not working use paper form 2105.001A for point required to be changed.
Section 8 of 2105.001 has been completed and all appropriate actions taken for CEAC#1 being set to inop.
TASK STANDARD:
CEAC #1 INOP flag set in CPC Channel B
TASK PERFORMANCE AIDS:
OP 2105.001 Section 17 and 2105.001A form
Examiner Note: Give the Yellow pages to the applicant.
SIMULATOR SETUP:
2105.001 Section 17.0, Form 2105.001A, CPC Function Enable Key.

with the current CPC constants.

Obtain SM permission.

SM permission is given to set CEAC #1 INOP flag.

Obtain the "Function Enable"

Give examinee the "Function

**EVALUATOR CUE:** 

Enable" key.

**EVALUATOR CUE:** 

1.

Step

(17.1)

2.

and may be obtained at any time.

key.

N/A SAT UNSAT

N/A SAT UNSAT

### JOB PERFORMANCE MEASURE

The SM/CRS directs, "Set CEAC #1 INOP Flag in CPC Channel B using OP 2105.001 Section 17.0."

#### **INITIATING CUE:**

Start Time:				
PERFORMANCE CHECKLIST	STANDARDS	(Circle One)		
Examiner Note: Give the Yellow pages to	the applicant.			
NOTE: Give OP2105.001 form 2105.001A to examinee, if not done previously. Do not use Simulator CPC books.				
	Procedure Note:			
Constant values. Values may be to truncation or rounding may differ be	imited in the size of the fields used to d runcated or rounded to fit the size of the etween screens showing the same cons stant value. A plant computer CPC Rep R-ANO-2-2007-1017)	e display fields. The stants or even between		
	stant changes, steps are designated as been performed and are exempt from in el of Use EN-AD-102.			
	nange Log has an EXCEL spreadsheet on COPS2\EXCELFIL\CPC Addressable C			
The form printed out after changes reviews are made.	are made is the OFFICIAL COPY and	is placed in binder after		

If the EXCEL spreadsheet and the OFFICIAL copy do not match, verify EXCEL data is updated

constant.

Examiner note:

The following step is not a procedure step but the Function Enable is required to complete this task

key from the SM.

Obtained SM permission to

Obtained the "Function Enable"

change an addressable

PERF	ORMANCE CHECKLIST	STANDARDS	(Circle One)
3. Step (17.2)	IF EITHER of the following conditions exist:  Proposed changes NOT governed by an approved procedure  Proposed changes outside limits of Attachment C,  THEN obtain OSRC approval before implementing.  Examiner Note:  INOP of CEAC is directly covered by this procedure and is a type I addressable constant and therefore not covered by attachment C.	Evaluated that INOP of CEAC is within the bounds of procedure AND not outside limits of attachment C.	N/A SAT UNSAT
4. Step (17.3)	IF changing Type II Addressable Constant, THEN refer to CPC Software Change Control (1022.002).	Determined that the INOP addressable constant is a type I addressable constant.	N/A SAT UNSAT
5. Step (17.4)	<ul> <li>Change is expected to cause channel trip</li> <li>Effect of change is not known</li> <li>Change is for SAM/BPPCC Type II constants (PID 81 – 89 or 99 – 102)</li> <li>THEN bypass affected CPC as follows:         <ul> <li>IF &lt; 10-3 % power, THEN turn CPC Trip Bypass keyswitch to ON.</li> <li>IF ≥ 10-3 % power, THEN bypass Low DNBR and High LPD inputs using Section 11.0.</li> </ul> </li> <li>Examiner Cue:         <ul> <li>A channel trip is not expected, CPC does not need to be bypassed.</li> </ul> </li> </ul>	Evaluated possibility of causing a channel trip.  Evaluated whether or not affect of bypass operation is known.	N/A SAT UNSAT

PERFORMANCE CHECKLIST		STANDARDS	(Circle One)
6. Step (17.5)	Verify changes NOT being made to any other CPC channel.  Examiner Cue: If requested state, no other	On panel 2C03, verified no changes in progress on any other CPC channel.	N/A SAT UNSAT
7	changes are being made to any other channel.	Determine that neither CEAC is	N/A SAT UNSAT
7. Step (17.6)	IF restoring one or both CEACs, THEN refer to section 8.0 of this procedure.	Determine that neither CEAC is being restored.	N/A SAT UNSAT
8. Step (17.7)	Select CPC by depressing CPC/CEAC Selector pushbutton.	On panel 2C03 at CPC channel "B" operator's module, verified CPC mode selected.	N/A SAT UNSAT
		Observed CPC half of CPC/CEAC SELECTOR pushbutton light ON.	
9. Step (17.8)	Select Channel Checks Display from Main Menu.	On panel 2C03 at CPC channel "B" operator's module, selected CHANNEL CHECKS DISPLAY from Main Menu.	N/A SAT UNSAT
		Observed CHANNEL CHECKS DISPLAY screen displayed.	
10. Step (17.9)	IF > 20% power as indicated by BDT (PID 177), THEN record current value of X1 (PID 406) and LPDKW (PID	On panel 2C03 at CPC channel "B" operator's module, observed power > 20% on BDT (PID 177).	N/A SAT UNSAT
	179) in CPC Addressable Constant Change log.	Recorded current values of PID 406 and PID 179 on form OP 2105.001A in the "As Found" columns of form OP 2105.001A.	
11. Step (17.10)	Select STANDARD OM DISPLAY.	On panel 2C03 at CPC channel "B" operator's module, selected STANDARD OM DISPLAY from Main Menu.	N/A SAT UNSAT
		Observed STANDARD OM DISPLAY screen displayed.	

### Procedure Note:

Type II Addressable Constants are controlled by Form 1022.002C and are therefore NOT required to be logged in CPC Addressable Constant Change log.

	PERF	ORMANCE CHECKLIST	STANDARDS	(Circle One)
	12. Step (17.11)	IF changing Type II Addressable Constant, THEN log AS FOUND value of ADRCHK (PID 046) in CPC Addressable Constant Change log.	Determined that the INOP addressable constant is a type I addressable constant.	N/A SAT UNSAT
		Pr	ocedure Note:	
•	If wrong previous	value. e not completed and executed with	r selection. s not been initiated, pressing 'C' will in 60 seconds, computer will revert	
	13. Step (17.12)	Display addressable constant to be changed.	Displayed PID 062 for CEAC operability.	N/A SAT UNSAT
	14. Step (17.13)	IF changing Type I Addressable Constant, THEN record AS FOUND value in CPC Addressable Constant Change log.	On panel 2C03 at CPC channel "B" operator's module, selected option "P".  Entered PID 062.  Recorded current value for CEANOP (PID 062) in the "As Found Value" column of Form 2105.001A.	N/A SAT UNSAT
	15. Step (17.14)	IF placing ONE or BOTH CEACs in INOP, THEN refer to Section 8.0.	Determined from initial conditions that Section 8 has been completed for placing one CEAC to INOP.	N/A SAT UNSAT
		JE: If Examinee refers to Section d all appropriate actions taken.	n 8.0, Inform Examinee that Section	on 8.0 has been
	16. Step (17.15)	Verify new value is within the allowable values of attachment C	Verified PID 62 value of 1 is allowable.	N/A SAT UNSAT
		Examiners note: 1 is an allowable value for PID 62.		
(C)	17. Step (17.16)	Place FUNCTION ENABLE Switch to ON.	On panel 2C03 at CPC channel "B" operator's module, inserted key in FUNCTION ENABLE Switch.	N/A SAT UNSAT
			Rotated key to ON.  Observed FUNCTION ENABLE ACTIVE light ON.	

	PERFORMANCE CHECKLIST		STANDARDS	(Circle One)
(C)	18. Step (17.17)	Select option C (Change Value).	On panel 2C03 at CPC channel "B" operator's module, selected option C from menu.	N/A SAT UNSAT
			Verified option C Highlights.	

#### Procedure Caution:

- If PID 062 is set to 1 or 2 and operable CEAC fails, then after a 90 minute time delay, a total CEA deviation Penalty Factor of 8.0 will be applied to DNBR and LPD.
- If PID 062 is set to 3 at 100% power, then:
  - DNBR will lower ~ 12%.
  - LPD will rise ~ 10%.
  - Target CEA positions set to 100% withdrawn causing the following:
    - ♦ CPC will NOT provide Subgroup Deviation and Out of Sequence trips is CEA misalignment occurs.
    - ♦ Planar Radial Peaking Factor table lookups and CEA Shadowing corrections will NOT be performed.

(C)	19. Step (17.18)	IF changing ONE or BOTH CEACs operability status, THEN select the appropriate value from the following:  • To make CEAC 1 inoperable, set PID 062 to 1.	Selected 1 for PID 062	N/A SAT UNSAT
(C)	20. Step (17.19)	Enter new value.	On panel 2C03 at CPC channel "B" operator's module, entered numerical value 1 for PID 062 to set CEAC #1 INOP Flag.  Verified PID 062 changes to 1.0.	N/A SAT UNSAT
(C)	21. Step (17.20)	Select option X (Execute).	On panel 2C03 at CPC channel "B" operator's module, selected option X (Execute) from menu.  Verified CEAC INOP light is ON.	N/A SAT UNSAT
	22. Step (17.21)	Verify correct value entered.	On panel 2C03 at CPC channel "B" operator's module, selected option "P".  Entered PID 062.  Verified PID 062 value of 1.0 is displayed.	N/A SAT UNSAT

	PERFORMANCE CHECKLIST		STANDARDS	(Circle One)
(C)	23. Step (17.22)	Place FUNCTION ENABLE Switch to OFF.	On panel 2C03 at CPC channel "B" operator's module, rotated key in FUNCTION ENABLE Switch to OFF position.  Observed FUNCTION ENABLE ACTIVE light OFF.	N/A SAT UNSAT
	24. Step (17.23)	IF changing Type I Addressable Constant, THEN enter in AS LEFT column in CPC Addressable Constant Change log.	Recorded new value of PID 062 in "As Left" column of Form 2105.001A.	N/A SAT UNSAT
	25. Step (17.24)	IF changing Type II Addressable Constant, THEN log AS LEFT value of ADRCHK (PID 046) in CPC Addressable Constant Change log.	Determined that the INOP addressable constant is a type I addressable constant.	N/A SAT UNSAT
	26. Step (17.25)	Select CHANNEL CHECKS DISPLAY from Main Menu.	On panel 2C03 at CPC channel "B" operator's module, selected CHANNEL CHECKS DISPLAY from Main Menu.  Observed CHANNEL CHECKS DISPLAY screen is displayed.	N/A SAT UNSAT
	27. Step (17.26)	IF > 20% power as indicted by BDT (PID 177), THEN record X1 (PID 406) and LPDKW (PID 179) in CPC Addressable Constant Change log and check BOTH affected in expected manner.	On panel 2C03 at CPC channel "B" operator's module, observed power > 20% on BDT (PID 177).  Recorded current values of PID 406 and PID 179 on form OP 2105.001A in the "As Found" columns of form OP 2105.001A.	N/A SAT UNSAT
Examiner Note: When the as left value of PID 179 and 406 is recorded the JPM may be stopped.				I may be stopped.
END				

Stop Time:

#### **EXAMINER'S COPY**

### JPM INITIAL TASK CONDITIONS:

You are responsible for any applicable annunciators during the performance of this task. All annunciators that are not applicable to this task will be performed by another operator.

You are responsible for any applicable annunciators during the performance of this task. All annunciators that are not applicable to this task will be performed by another operator.

The Unit is in Mode 1 (power > 20%).

CEAC # 1 monthly surveillance has been authorized.

2105.001 CPC/CEAC operations Section 8 of has been completed and all appropriate actions have been taken for CEAC#1 being set to inop.

The excel CPC log is not working use paper form CPC addressable constant change log 2105.001A for the point required to be changed.

#### **INITIATING CUE:**

The SM/CRS directs, "Set CEAC # 1 INOP Flag in CPC Channel B using 2105.001 CPC/CEAC operations Section 17.0."

### **EXAMINEE'S COPY**

### JPM INITIAL TASK CONDITIONS:

You are responsible for any applicable annunciators during the performance of this task. All annunciators that are not applicable to this task will be performed by another operator.

The Unit is in Mode 1 (power > 20%).

CEAC # 1 monthly surveillance has been authorized.

2105.001 CPC/CEAC operations Section 8 of has been completed and all appropriate actions have been taken for CEAC#1 being set to inop.

The excel CPC log is not working use paper form CPC addressable constant change log 2105.001A for the point required to be changed.

### **INITIATING CUE:**

The SM/CRS directs, "Set CEAC # 1 INOP Flag in CPC Channel B using 2105.001 CPC/CEAC operations Section 17.0."

UNIT: 2
SYSTEM/DUTY AREA: Pressurizer Spray operation S8
TASK: _Isolate failed open Pressurizer Spray valve. (Alternate Success Path)
JTA#: ANO-2-RO-EOPAOP-OFFNORM-181
KA VALUE RO: 3.9 SRO: 3.9 KA REFERENCE: 010 A2.02
APPROVED FOR ADMINISTRATION TO: RO: X SRO: X
TASK LOCATION: INSIDE CR: X OUTSIDE CR: BOTH:
SUGGESTED TESTING ENVIRONMENT AND METHOD (PERFORM OR SIMULATE):
PLANT SITE: SIMULATOR: Perform CLASSROOM:
POSITION EVALUATED: RO: SRO:
ACTUAL TESTING ENVIRONMENT: SIMULATOR: PLANT SITE: CLASSROOM:
TESTING METHOD: SIMULATE: PERFORM:
APPROXIMATE COMPLETION TIME IN MINUTES: 5 Minutes
REFERENCE(S): OP-2203.028 PZR System Malfunctions
EXAMINEE'S NAME: Badge #:
EVALUATOR'S NAME:
THE EXAMINEE'S PERFORMANCE WAS EVALUATED AGAINST THE STANDARDS CONTAINED IN THIS JPM AND IS DETERMINED TO BE:
SATISFACTORY: UNSATISFACTORY:
PERFORMANCE CHECKLIST COMMENTS:
Start         Stop         Total Time           Time
SIGNED: DATE:

SIGNATURE INDICATES THIS JPM HAS BEEN COMPARED TO ITS APPLICABLE PROCEDURE BY A QUALIFIED INDIVIDUAL (NOT THE EXAMINEE) AND IS CURRENT WITH THAT REVISION.

# ANO-2-JPM-NRC-AOP3 JOB PERFORMANCE MEASURE PAGE 2 OF 6

THE EXAMINER SHALL REVIEW THE FOLLOWING WITH THE EXAMINEE: The examiner shall ensure that the examinee has been briefed on NUREG 1021 Appendix E. JPM INITIAL TASK CONDITIONS: 2CV-4651 PZR spray valve failed open. Mode 1 TASK STANDARD: Reactor tripped and 2P-32A secured. TASK PERFORMANCE AIDS: OP 2203.028 Step 2 **Examiner Note: Give the Yellow pages to the applicant.** SIMULATOR SETUP: All RCPs running. Mode 1 CV4651 value = 1. Trigger1: CV4656 value = .9, DO\_HS\_4656\_G = off, & DO\_HS\_4656\_R = off Conditional trigger 1 set gh4g4656. Have all backup htrs on.

### **INITIATING CUE:**

The CRS directs,	"Isolate PZR spray valve	2CV-4651 using 2203.	028 PZR system	malfunctions step 2'
START TIME:				

Examiner Note: Maintain the simulator in Freeze until the candidate is ready to start.

PERF	ORMANCE CHECKLIST	STANDARDS	(Circle One)			
Examiner Note: Give the Yellow pages to the applicant.						
1. (Step 2.A)	IF ANY PZR Spray valve failed open, THEN perform the following:	On panel 2C04 placed PZR spray valve 2HS-4651B in manual	N/A SAT UNSAT			
	A. Place affected PZR Spray valve in MANUAL and close:	Placed 2CV-4651 handswitch in closed.				
	<ul><li>2CV-4651</li><li>2CV-4652</li></ul>	Observed red and green lights flash above handswitch and the red light remains on.				
2. (Step 2.B.1)	IF affected PZR Spray valve did NOT close, THEN perform the following:	On panel 2C04, placed handswitch for 2CV-4651 in open for 1 sec.	N/A SAT UNSAT			
	Place associated PZR Spray valve handswitch to OPEN for 1 second:	Observed red light ON above handswitch				
	• 2CV-4651 • 2CV-4652					
	•					
3. (Step 2.B.2)	WHEN 5 seconds have elapsed, THEN place handswitch in CLOSE until green indicating light flashes.	On panel 2C04 and 5 sec have elapsed, placed handswitch for 2CV-4651 in close until lights flash.	N/A SAT UNSAT			
		Observed red and green lights flash above handswitch and the red light remains on.				

	PERF	ORMANCE CHECKLIST	STANDARDS	(Circle One)
	4. (Step 2.C)	IF affected PZR Spray valve is 2CV-4651 AND will NOT close, THEN close associated energized Block valves:  2CV-4655  2CV-4656	On panel 2C04, placed 2CV-4655/2CV-4656 handswitch to close.  Observed 2CV-4656 lights out and PZR pressure lowering.	N/A SAT UNSAT
		EXAM	MINER'S NOTE:	
		The following is the a	Iternate path portion of this JPM	
	5. (Step 2.E)	IF both of the following are true, Affected PZR spray valve open. Associated energized PZR Spray Block valve will NOT close, THEN verify ALL PZR heaters ON.	On panel 2C04, observed all pzr heater on as indicated by red lights above the handswitches.	N/A SAT UNSAT
(C)	6. (Step 3.A)	IF RCS pressure lowers to 2000 psia, THEN perform the following:  IF plant in mode 1 or 2, THEN trip Reactor.	When, RCS pressure is observed at 2000 psia or informed by evaluator RCS is 2000 psia, Tripped the reactor.	N/A SAT UNSAT
(C)	7. (Step 3.B)	Stop RCP in loop with failed PZR Spray valve.	Secured 2P-32A Reactor Coolant pump.	N/A SAT UNSAT
	8. (Step 3.C)	IF only one RCP affected AND desired to balance reactor coolant loop temperatures, THEN verify one RCP secured in each loop.	If desired then Secure either 2P-32C or 2P-32D.	N/A SAT UNSAT
			END	

ST	OP	TIME:	
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### **EXAMINER's COPY**

### JPM INITIAL TASK CONDITIONS:

You are responsible for any applicable annunciators during the performance of this task. All annunciators that are not applicable to this task will be performed by another operator.

Mode 1

2CV-4651 PZR spray valve has failed open during boron equalization.

### **INITIATING CUE:**

The CRS directs, "Isolate PZR spray valve 2CV-4651 using 2203.028 PZR system malfunctions starting with step 2".

### **EXAMINEE's COPY**

### JPM INITIAL TASK CONDITIONS:

You are responsible for any applicable annunciators during the performance of this task. All annunciators that are not applicable to this task will be performed by another operator.

Mode 1

2CV-4651 PZR spray valve has failed open during boron equalization.

### **INITIATING CUE:**

The CRS directs, "Isolate PZR spray valve 2CV-4651 using 2203.028 PZR system malfunctions starting with step 2".

UNIT:2	REV #:	07	DATE:				
SYSTEM/DUTY AREA:	Emergency Diesel C	Senerator System					
TASK: Startup a diesel generator without DC control power (2K-4A) (Alternate Success Path)							
JTA#: ANO2AOEDGO	FFNORM5						
KA VALUE RO:	4.0 SRO:	4.3 KA RI	EFERENCE:	064 A4.01			
APPROVED FOR ADMIN	ISTRATION TO: R	O: X SRO:	X				
TASK LOCATION:	INSIDE CR:	OUTSIDE CR:	Х	вотн:			
SUGGESTED TESTING	ENVIRONMENT AND	METHOD (PERFORM	OR SIMULA	ГЕ):			
PLANT SITE: Simulate	e SIMULATO	DR:	LAB:				
POSITION EVALUATED:	RO:	SRO:					
ACTUAL TESTING ENVI	RONMENT: SIMUL	ATOR: PI	LANT SITE:	LAB:			
TESTING METHOD:	SIMULATE:	PERFORM:					
APPROXIMATE COMPLE	ETION TIME IN MINUT	ΓES: 20 Minutes					
REFERENCE(S): OP 2	104.036						
EXAMINEE'S NAME: _			Badge #:				
EVALUATOR'S NAME:							
THE EXAMINEE'S PERFORMANCE WAS EVALUATED AGAINST THE STANDARDS CONTAINED IN THIS JPM AND IS DETERMINED TO BE:							
SATISFACTORY: UNSATISFACTORY:							
PERFORMANCE CHECKLIST COMMENTS:							
	top ime	Total Time					
SIGNED:		DATE:					

SIGNATURE INDICATES THIS JPM HAS BEEN COMPARED TO ITS APPLICABLE PROCEDURE BY A QUALIFIED INDIVIDUAL (NOT THE EXAMINEE) AND IS CURRENT WITH THAT REVISION.

THE EXAMINER SHALL REVIEW THE FOLLOWING WITH THE EXAMINEE:
The examiner shall ensure that the examinee has been briefed on NUREG 1021 Appendix E.
JPM INITIAL TASK CONDITIONS:
The plant is in Mode 3 following an automatic reactor trip. A Station Blackout exists.
There is no red DC power available.
TASK STANDARD:
2DG1 [2K-4A] has been started with normal voltage and frequency.
TASK PERFORMANCE AIDS:
OP 2104.036 Exhibit 1

### **INITIATING CUE:**

The SM/CRS directs, "Perform a loss of DC start of 2DG1 using OP 2104.036, Exhibit 1."

PERFO	DRMANCE CHECKLIST	STANDARDS	(Circle One)
	Pro	cedure Caution:	
Fault	condition that caused loss of DC m	nay still be present and should be rem	noved if known.
	Pr	ocedure Note:	
• Th	nis procedure assumes no AC <u>or</u> D	C power available.	
	ectrical Safety equipment will be rump supply breaker.	equired for closing of the EDG output	t breaker and the SV
1. (Step 1.0)	IF NOT directed to perform this exhibit per Alternate Shutdown (2203.014),	Obtains Arc flash PPE and Electrical PPE on at PPE cabinet.	N/A SAT UNSA
	THEN obtain arc flash PPE (suit, hood, and voltage rated or leather gloves) for Manual Operation of 4160/6900V AC Breaker with Door Open.	Examiner Note: The examinee should show where to obtain the PPE. Do not require the applicant to bring the PPE for simulation of this JPM.	
	Examiner Note:	Simulation of this of W.	
	Arc flash PPE can be obtained from the cabinet across from the elevator on the 372' elevation.		
	Examiner Cue:		
	Arc flash PPE has been obtained.		
2. (Step	Open one of the following sets of breakers:	For the listed breaker, opened panel door and moved breaker	N/A SAT UNSA
2.0)	- 2C107 breaker (2D23-2)	handle to OFF.	
	- 2E11 breaker (2D23-6)		
	- 2C107 breaker (2D23-8)		
	OR		
	- 2E11 breaker (D2E11NA6) (located inside 2E12)		
	- 2C107 breaker (D2C107NA8)		
	- 2C107 breaker (D2C107NA2)		
	Examiner Cue: Show PICTURE #1 pictures for 2D-23 breakers or show PICTURES #7, and #8 for 2C107, 2E-12. Breakers indicate NOT OPEN.		
		NSITION NOTE: 3 switchgear room.	

PERFO	ORMANCE CHECKLIST	STANDARDS	(Circle One)				
EXAMINERS NOTE:  For the 4160v breakers in Steps 2 and 3, do NOT open the breaker cubicle doors. Use the provided pictures. Have the candidate explain the indications he would use to determine the following.  1. Breaker position 2. Closing Spring Status  Both of these indicators are located on the breaker.  Standard: 1. The student must use mechanical breaker position indication. The student must use mechanical closing spring status indication.							
Instructions for (2107.001), Ex	Promanual operation of 4160 VAC B hibits 1 and 2.	ocedure Note: reakers are contained in Electrical Sy aminers Note: respective conditions for the plant co	·				
(C) 3. (Step 3.0)	Verify the following breakers OPEN WITH closing springs charged.  Note: Show PICTURE #2 for 2A-302  SW Pump 2P4A (2A-302)  This breaker will be closed with closing springs charged  Note: Show PICTURE #3 for the remaining breakers of this step.  SW Pump 2P4B (2A-303)  This breaker will be open with closing springs charged  2DG1 Output breaker (2A-308)  This breaker will be open with closing springs charged  2DG1 Output breaker (2A-308)  This breaker will be open with closing springs charged  Examiner Cue: For 2A302 show PICTURE #3 when the mechanical trip button is pushed.	At bus 2A3.  Observed indications for the following:  • Mechanical breaker position  • Mechanical closing spring indication  Operator opened any closed breaker by pushing the mechanical breaker trip button on the breaker	N/A SAT UNSAT				

PERFORMANCE CHECKLIST		STANDARDS	(Circle One)
(C) 4. (Step 4.0)	Verify the following breakers OPEN.  Note: Show PICTURE #3 for the following breakers CS Pump 2P35A (2A-304)  ◆ Breaker is open  LPSI Pump 2P60A (2A-305)  ◆ Breaker is open  HPSI Pump 2P89A (2A-306)  ◆ Breaker is open  HPSI Pump 2P89C (2A-307)  ◆ Breaker is open  Note: Show PICTURE #2 for 2A-309  2A3 Supply breaker (2A-309)  ◆ Breaker is closed  Note: Show PICTURE #3 for 2A-310 and 2A-311  2A3-2A4 Tie (2A-310)  ◆ Breaker is open  2P-7B EFW pumps breaker (2A-311)  ◆ Breaker is open  Examiner Cue: For 2A309 show PICTURE #3 when the mechanical trip button is pushed.	For each listed breaker, verified the breaker is open with the mechanical breaker indication.	N/A SAT UNSAT

- ,	After op	03-1 requires DC power to open	MINERS NOTE: automatically. ened or if he states he is going to c	open 2CV-1503-1,
(C)	5. (Step 5.0)	Verify 2DG1 service water outlet (2CV-1503-1) open using local ratchet.  Examiner Cue:  WCO reports Service Water Valve (2CV-1503-1) is OPEN.	Using radio or telephone communications, instructed WCO to OPEN 2CV-1503-1 locally using manual operator.  OR  Using radio or telephone communications, contacted Control Room personnel to have WCO to OPEN 2DG1 Service	N/A SAT UNSAT
			Water valve (2CV-1503-1) using the manual operator locally.	
<u> </u>	I		NSITION NOTE:	
<del></del> 1		Go	to 2DG1 room.	
	6. (Step 6.0)	Place Exciter Regulator Local/Remote HS in LOCAL.  Examiner Cue: The exciter regulator handswitch indicates	On panel 2E12, rotated Exciter Regulator handswitch to LOCAL.	N/A SAT UNSAT
	_	LOCAL.		
	7. (Step 7.0)	Unlock local Engine Control switch (2HS-2815-1).	On panel 2E11, inserted key in Engine Control Switch.	N/A SAT UNSAT
		Examiner Cue:	Rotated key clockwise to unlock.	
		Engine Control switch is unlocked.		
	8. (Step 8.0)	Place 2HS-2815-1 in START.  Examiner Cue: The Engine Control handswitch indicates START.	Rotated Engine Control handswitch through LOCKOUT to START.	N/A SAT UNSAT
			cedure Caution:	
			OT functional with control power off.  NSITION NOTE:	
		· · · · · · · · · · · · · · · · · · ·	:ker located just outside U2 CR Ext	ension
	9. (Step 9.1)	Obtain FNM 10 amp fuses from CRS bag in Alternate Shutdown file cabinet.	At the Alternate Shutdown file cabinet, obtained at least two FNM 10 amp fuses from the CRS bag.	N/A SAT UNSAT
		Examiner Cue: Fuses in hand.	9-	
			NSITION NOTE:	
		·	to 2DG1 room.	
			caminer Note:	
		onning arc flash PPE is not requ loes not need to be donned.	uired to be performed for this JPM.	intorm the examine

(C)	10. (Step 9.3)	Manually operate EITHER Air Start Solenoid by slowly rotating manual operator stem clockwise: 2SV-2809-1 2SV-2810-1  Examiner Cue: Engine noise indicates start.	Rotated manual operator stem on either Air Start Solenoid clockwise.	N/A SAT UNSAT			
	11. (Step 9.4)	WHEN engine starts, THEN disengage manual operator by rotating stem counter-clockwise.	Upon 2DG1 start, disengaged manual solenoid operator by rotating counter-clockwise.	N/A SAT UNSAT			
Thomas	agativa (		MINER'S NOTE:	Suggest Both			
The ne	egative		QUIRED to implement the Alternate ocedure Note:	Success rain.			
		oltage adjust is NOT available in the egulator maintains 4160 volts as s	nis mode of operation.				
	12. (Step 9.5)	Check generator voltage 3800 to 4400 volts.  Examiner Cue: 2DG1 voltage is ~ 0 volts.	On panel 2E11, checked 2DG1 voltage between 3800 and 4400 volts on V-2DG1-3, 2DG1 local AC voltmeter.	N/A SAT UNSAT			
	Examiner Note:						
Do not	Do not open the local EDG panel.						
Use <b>Pi</b>	cture #4	(Open Panel EDG) picture to ha	ve them describe the location of the	Exciter Field Shutdown			

Use **Picture #4 (Open Panel EDG)** picture to have them describe the location of the Exciter Field Shutdown Contactor (K1). After they locate it show them **Picture #5** (close up picture) to describe operating the slide switch.

Use provided pictures to have the examinee show how they would operate the K1 slide switch and install the fuses for the back up field flash **Picture #6**.

13. (Step 9.6)	IF generator has NO Output Voltage, THEN manually reset Exciter Relay as follows by operating red slide switch on Exciter Field Shutdown Contactor (K1) in 2E12.	Inside panel 2E12, pushed the red slide switch up on the Exciter Field Shutdown Contactor (K1).	N/A SAT UNSAT
	Picture #4 and #5 have the applicants show the location of the K1 relay slide switch.		
	Examiner Cue: Red slide switch moves up. 2DG1 voltage is ~ 0 volts.		

### **Examiner Note:**

Do not open the local EDG panel. Have the examinee describe the location of the fuse holder for the backup field flash fuses (Bottom half of 2E12 on left side) **Picture #6**.

Use provided picture to have the examinee show how they would install the fuses for the back up field flash.

(C)	14. (Step 9.7.1)	IF generator output voltage < 3800 volts, THEN perform the following:  Install FNM 10 amp fuses in fuse block in 2E12 labeled: FUSES SHALL NOT BE INSTALLED DURING NORMAL OPERATION.  Use Picture #6 for installation of the fuses.  Examiner Cue: Fuses installed.	Inside panel 2E12, installed <u>two</u> FNM 10 amp fuses in fuse block labeled "Fuses Shall Not Be Installed During Normal Operation".	N/A SAT UNSAT
(C)	15. (Step 9.7.2)	Place Flashing Power Select Switch (DG1F/SS) to BACKUP. Examiner Cue: Switch indicates BACKUP.	On panel 2E12, placed DG1F/SS to BACKUP.	N/A SAT UNSAT
(C)	16. (Step 9.7.3)	Place Flashing Power Control Switch (DG1F/CS) to BACKUP. Examiner Cue: Voltage indicates ~ 4160 volts.	On panel 2E12, held DG1F/CS in BACKUP until voltage builds up.  On panel 2E11, observed 2DG1 voltage between 3800 and 4400 volts on V-2DG1-3, 2DG1 local AC voltmeter.  Released DG1F/CS.	N/A SAT UNSAT
	17. (Step 9.7.4)	WHEN voltage builds up, THEN release switch.	On panel 2E12, released DG1F/CS when voltage builds up.	N/A SAT UNSAT
	18. (Step 9.8)	Verify generator frequency 59.5 to 60.5 Hz AND maintain as loads are added. Examiner Cue: 2DG1 frequency is ~ 60 Hz.	On panel 2E11, verified 2DG1 frequency between 59.5 and 60.5 Hz and maintained as loads are added.	N/A SAT UNSAT
Examiner Note:  The JPM should be ended after step 9.8 is completed.  END				

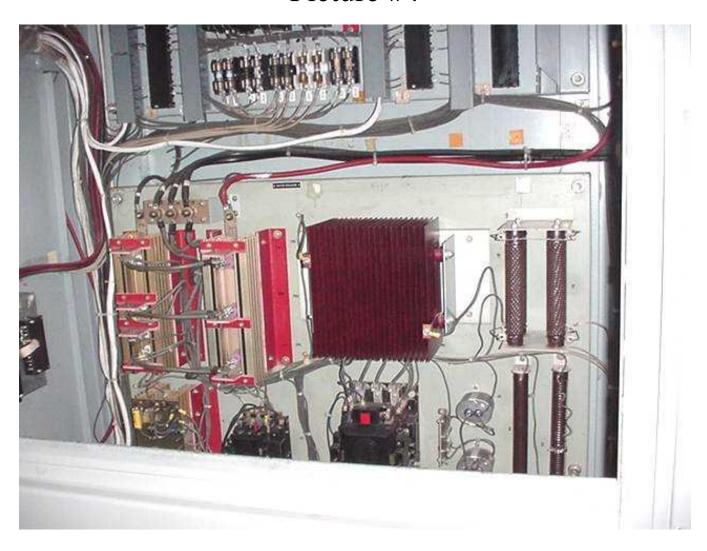


2D-23 continued on the next page.

# 6 10















# Examiner's Copy

### JPM INITIAL TASK CONDITIONS:

The plant is in Mode 3, post reactor trip, no AC or red DC power is available. 2DG2 [2K-4B] is tagged out for maintenance with its oil pump removed.

### **INITIATING CUE:**

The SM/CRS directs, "Perform a loss of DC start of 2DG1 using OP 2104.036, Exhibit 1."

### **Examinee's Copy**

### JPM INITIAL TASK CONDITIONS:

The plant is in Mode 3, post reactor trip, no AC or red DC power is available. 2DG2 [2K-4B] is tagged out for maintenance with its oil pump removed.

### **INITIATING CUE:**

The SM/CRS directs, "Perform a loss of DC start of 2DG1 using OP 2104.036, Exhibit 1."

### ANO-2-JPM-NRC-MGSU PAGE 1 OF 22

### JOB PERFORMANCE MEASURE

UNIT: _	2	_		REV #:	011	_		DATE	:	
SYSTEM	//DUT	Y AREA:	Control I	Element	Drive	Mechanis	m Control	System	1	
TASK:	ASK: Start up a CEA drive motor generator set (first one)									
JTA#:	ANO	2-AO-CEI	OM-NORM2							
KA VAL	UE	RO:	3.7	SRO:		3.4	KA REFI	ERENCE	i:	001 A4.08
APPRO\	VED F	OR ADMII	NISTRATIO	N TO:	RO:	X	SRO:	Х		
TASK L	OCATI	ON:	INSIDE C	R:		OUTSID	E CR:	Х	вотн:	
SUGGES	STED	TESTING	ENVIRONI	IENT AN	ID ME	THOD (PE	RFORM C	OR SIMU	LATE):	
PLANT S	SITE:	Sim	nulate	SIMULA	TOR:			_ LAB:		
POSITIO	ON EV	ALUATED	: RO: _			_ SRO:				
ACTUAL	_ TEST	ING ENV	IRONMENT	: SIM	ULAT	OR:	PLA	NT SITE	:	LAB:
TESTING METHOD: SIMULATE: PERFORM:										
APPROX	XIMAT	E COMPL	ETION TIM	E IN MIN	IUTES	s: <u>15</u>	Minutes	_		
REFERENCE(S): OP 2105.009										
EXAMINEE'S NAME: BADGE #:										
EVALUATOR'S NAME:										
			FORMANCE RMINED TO		VALU	ATED AG	AINST THE	E STANI	DARDS C	ONTAINED IN
SATISF	ACTOF	RY:		UNSAT	ISFAC	TORY:				
PERFORMANCE CHECKLIST COMMENTS:										
Start Time			Stop Fime		Tc	otal Time				
SIGNED	:					DATE	:			

SIGNATURE INDICATES THIS JPM HAS BEEN COMPARED TO ITS APPLICABLE PROCEDURE BY A QUALIFIED INDIVIDUAL (NOT THE EXAMINEE) AND IS CURRENT WITH THAT REVISION.

# ANO-2-JPM-NRC-MGSU JOB PERFORMANCE MEASURE PAGE 2 OF 22

# THE EXAMINER SHALL REVIEW THE FOLLOWING WITH THE EXAMINEE:

The examiner shall ensure that the examinee has been briefed on NUREG 1021 Appendix E.

	JPM INITIAL TASK CONDITIONS:	The following	conditions exist.
--	------------------------------	---------------	-------------------

- Reactor trip circuit breakers 1 through 8 have been verified open.

   Reactor trip circuit breakers 1 through 8 have been verified open.
- 2. Both MG sets are stopped.

TASK STANDARD: The first MG set has been started and placed in service with a normal operating voltage of

TASK PERFORMANCE AIDS: OP 2105.009 Section 7.0; Picture of MG set #1 cabinet.

\_\_\_\_

approximately 240 VAC.

<b>INITIATING</b>	CUE:
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The SM/CRS directs, "Place #1 MG in service using OP 2105.009, Section 7.0."

START	TIME:	

PERF	PERFORMANCE CHECKLIST STANDARDS (Circle One)				
	P	rocedure Note:			
DSS panels (	(2C407/2C408) are powered from 2	2Y26 240V System Output breaker ( 1277)	(CB7). (CR-ANO-2-2005-		
Examiner		3 to have the applicants describe the pictures list in the JPM as des			
1. (Step 7.1.1)	DSS Contactor OR DSS Bypass Breaker closed.  • 29-1/DSS OR 52-1/DSS (MG Set #1)  Examiner Cue: Show picture #4 (The DSS contactor indicates red light on and green light off.)	In the control room and verified DSS contactor or contactor bypass breaker closed by indication on panel 2C14;  OR  At panel 2C407, verified the DSS contact CLOSED by observing the red closed indicating lamp illuminated or verified the contactor bypass breaker closed by observing the red mechanical indicating flag on the breaker indicating closed  OR  At panel 2C-409 just inside the CEDMCS Room, observed the DSS contactor closed by observing the red closed lamp illuminated.	N/A SAT UNSAT		
TRANSITION NOTE:					
	Go to the 2B53 room, elevation 368' TB				
2. (Step 7.1.2)	Check MG set #1 power supply breaker (2B-722) closed.	At MCC 2B7 cubicle 2B-722, observed red "breaker closed" mechanical.	N/A SAT UNSAT		
	Examiner Cue:				
	Either travel to the breaker or show Picture #5 (Breaker status mechanical indicator shows red for "breaker closed").				
	TRANSITION NOTE:				
Go to CEDM MG Set room					

PERFO	ORMANCE CHECKLIST	STANDARDS	(Circle One)		
3. (Step 7.1.3)	Motor Input circuit breaker open (handswitch in OFF)  Examiner Cue:  Show Picture #6 (Show as closed or on but should be open/off) and describe position as Circuit breaker is in OPEN / Control switch is in OFF.	At panel 2C163, verified motor input Breaker OPEN (control switch in OFF).	N/A SAT UNSAT		
4. (Step 7.1.4)	Neutral Interrupter open (handswitch in OFF).  Examiner Cue: Show Picture #7 (Show as closed or on but should be open/off) / Neutral interrupter pointing to OFF.	At panel 2C163, verified neutral interrupter pointing to OFF.	N/A SAT UNSAT		
	EXAMINER'S NOTE:				
The breaker may be inaccessible if motor input/neutral interrupter handle(s) are installed. Therefore, the candidate may be required to explain the indications available to verify generator output breaker OPEN.					
5. (Step 7.1.5)	Generator Output circuit breaker for selected MG set open (inside lower right section of cabinet):	Inside panel 2C163 in the lower right section, verified green flag visible on breaker.  OR	N/A SAT UNSAT		
	Examiner Cue:  If the control room is requested then show picture #8 (Generator Output circuit breaker is OPEN.) or show picture #10 and describe green light on red light off.	Contacted the control room to verify the generator output breaker green OPEN indicating lamp on 2C14 is lit.  OR  At panel 2C163 verified the green "load off" lamp illuminated.			
6. (Step 7.1.6)	Verify motor control selector switch in LOCAL.	At panel 2C163, verified motor control selector switch pointing to LOCAL.	N/A SAT UNSAT		
	Examiner Cue: Show picture #9 (Selector switch in LOCAL).				
1	E	xaminer note:			
Step 7.2.1 is only verifying step 7.1 is complete which the candidate just performed.					

# ANO-2-JPM-NRC-MGSU JOB PERFORMANCE MEASURE PAGE 5 OF 22

PERF	FORMANCE CHECKLIST	STANDARDS	(Circle One)
7. (Step 7.2.2)	Verify Reactor Trip circuit breakers 1 through 8 open.	Verified Reactor trip circuit breakers 1 through 8 are open by any of the following:	
	Examiner Cue:	Initial conditions	
	If the control room is	Observing locally	
	requested then show Picture #8 (Green light on red light off for all reactor trip circuit breakers).	Calling the control room.	
(C) 8. (Step 7.2.3)	Manually close Motor Input circuit breaker by taking handswitch to ON.	At panel 2C163, closed the motor input circuit breaker by rotating the control handle CW from "OFF" to "ON".	N/A SAT UNSAT
	Examiner Cue:  Show picture #6 (Input circuit breaker HS is pointing to on).		
(C) 9. (Step 7.2.4)	Close Neutral Interrupter by taking handswitch to ON.  Examiner Cue:	At panel 2C163, closed the neutral interrupter by rotating the control handle CW from OFF to the ON position.	N/A SAT UNSAT
	Show picture #7 (Neutral interrupter breaker HS is pointing to on).		
10. (Step 7.2.5)	Check Load Off light illuminated.	At panel 2C163, observed the green "load off" light illuminated.	N/A SAT UNSAT
	Examiner Cue:		
	Show Picture #10 (shown as Load on) and describe Green light ON and Red light OFF		
(C) 11. (Step 7.2.6)	Depress Motor On pushbutton to start motor.	At panel 2C163, depressed "MOTOR ON" push-button.	N/A SAT UNSAT
	Examiner Cue:	Observed the red "motor on"	
	Show Picture #11 (the red "motor on" lamp is illuminated.)	lamp lit.	

### Procedure Note:

If voltage band of 240-243 VAC not obtained in step 7.2.7, then Push to Buildup button should be released and substep A. should be performed.

# ANO-2-JPM-NRC-MGSU JOB PERFORMANCE MEASURE PAGE 6 OF 22

PER	FORMANCE CHECKLIST	STANDARDS	(Circle One)		
(C) 12. (Step 7.2.7	light is illuminated,	At panel 2C163 performed the following.  With "push voltage buildup" light on depressed VOLTAGE BUILDUP push-button until voltage is between 240 and 243 VAC.  Released push-button.	N/A SAT UNSAT		
13. (Step 7.2.8	any remaining alarms.	At panel 2C163, cleared any alarms by depressing and releasing RESET push-button.  Observed alarms cleared.	N/A SAT UNSAT		
(C) 14. (Stel 7.2.9	breaker for selected MG set by	At panel 2C163, depressed LOAD ON push-button (2CB) and released.  Observed red "load on" light lit, green "load off" light extinguished.	N/A SAT UNSAT		
	END				

STOP	TIME:	
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## **EXAMINER'S COPY**

### JPM INITIAL TASK CONDITIONS:

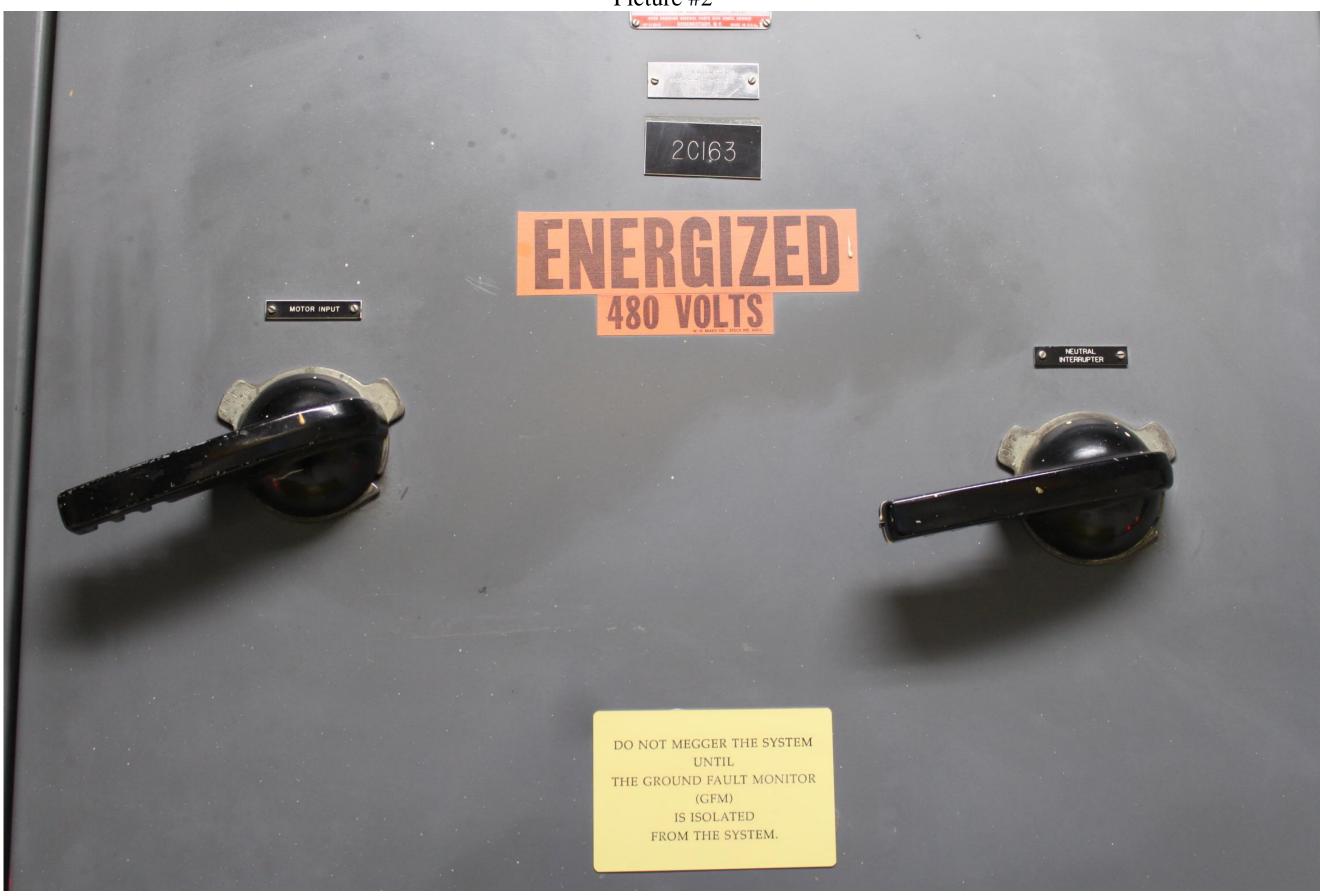
- Reactor trip circuit breakers 1 through 8 have been verified open.
- Both MG sets are stopped.

### **INITIATING CUE:**

The SM/CRS directs, "Place #1 MG in service using OP 2105.009, Section 7.0."



Picture #2



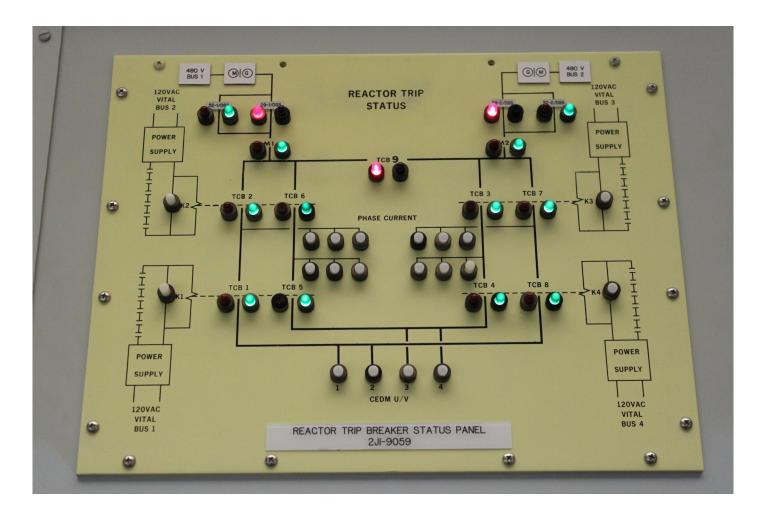














Picture #10



Picture #11



Picture #12





Picture #14



# **EXAMINEE'S COPY**

# JPM INITIAL TASK CONDITIONS:

- Reactor trip circuit breakers 1 through 8 have been verified open.
- Both MG sets are stopped.

### **INITIATING CUE:**

The SM/CRS directs, "Place #1 MG in service using OP 2105.009, Section 7.0."

UNIT: _	2	REV #:	012	DATE:		
SYSTEM	I/DUTY AREA:	Spent Fuel P	ool Cooling S	System		
TASK:	Perform se	ervice water emer	gency makeu	p to the SFP (wi	th Loop 1 s	ervice water)
JTA#:	ANO2-WCO-	SFP-EMER1				
KA VALI	JE RO: _	3.1 SRO:	3.5	KA REFERENCE:	_	033 A2.03
APPROV	/ED FOR ADM	INISTRATION TO:	RO: χ	SRO: X	_	
TASK LO	OCATION:	INSIDE CR:	OUTSID	Ε CR: χ	вотн:	
SUGGES	STED TESTING	S ENVIRONMENT AN	ID METHOD (PE	RFORM OR SIMUL	ATE):	
PLANT S	SITE: Simu	ılate simula	TOR:	CLASSF	ROOM:	
POSITIO	N EVALUATE	D: RO:	SRO:			
ACTUAL	. TESTING EN	VIRONMENT: SIM	ULATOR:	PLANT SITE:	CLAS	SSROOM:
TESTING	G METHOD:	SIMULATE:	PERFORM:			
APPROX	(IMATE COMP	LETION TIME IN MIN	IUTES: 20 I	Vinutes_		
REFERE	NCE(S): OP	-2104.006 Fuel Pool	Systems			
EXAMIN	EE'S NAME:			Badge #:		
EVALUA	TOR'S NAME:					
THE EX		FORMANCE WAS E			RDS CONTAI	NED IN THIS
SATISFA	ACTORY:	UNSAT	ISFACTORY: _			
PERFOR	RMANCE CHEC	CKLIST COMMENTS:				
Start Time		Stop Time	Total Time			
SIGNED	:		DATE:			

SIGNATURE INDICATES THIS JPM HAS BEEN COMPARED TO ITS APPLICABLE PROCEDURE BY A QUALIFIED INDIVIDUAL (NOT THE EXAMINEE) AND IS CURRENT WITH THAT REVISION.

THE EXAMINER SHALL REVIEW THE FOLLOWING WITH THE EXAMINEE:
The examiner shall ensure that the examinee has been briefed on NUREG 1021 Appendix E.
JPM INITIAL TASK CONDITIONS:
(2K11-K5) "FUEL POOL TEMP HI" is in alarm.
Neither fuel pool cooling pump is available.
An operator is stationed at the spent fuel pool to monitor spent fuel pool level.
Fuel pool level is low.
2P66 is aligned to the RWT and running.
TASK STANDARD:
Emergency SFP makeup from Loop 1 service water has been initiated.
TASK PERFORMANCE AIDS:
OP 2104.006 Section 14.3.
Do not use a cloth lanyard in a Radiologically controlled area.
SIMULATOR SETUP:
N/A

The CRS directs, "Align Loop 1 service water to provide emergency makeup to the spent fuel pool starting on step 14.3 of OP 2104.006."

START TIME:\_\_\_\_

	PERF	FORMANCE CHECKLIST	STANDARDS	(Circle One)			
	Examiner Note:						
La	Ladders in the aux building are control by both operations and RP. Operations dept. has keys to the ladders but if access it needed greater than 8' RP must be contacted.						
1. (Step 14.3.1)  IF SFP or RWT aligned for Purification, THEN secure Fuel Pool Purification pump 2P-66 (2HS-5411) to prevent rapid depletion of Fuel Pool DI (2T-5) or addition of Service Water to the RWT.  Examiner CUE:  After the pump is simulated to be stopped Report Green light ON, red light OFF.		Purification, THEN secure Fuel Pool Purification pump 2P-66 (2HS-5411) to prevent rapid depletion of Fuel Pool DI (2T-5) or addition of Service Water to the RWT.  Examiner CUE: After the pump is simulated to be stopped Report Green light	Locally rotated 2P66 handswitch (2HS-5411) to STOP and released.  Observed green light ON, red light OFF.	N/A SAT UNSAT			
	I	<u>TRA</u>	NSITION NOTE:	L			
		Go to elevation 354' RAB, just no	orth of the elevator, to the SFP valv	e gallery.			
	2. (Step 14.3.2)	Verify the following valves closed:  Borated MU or RWT to 2P-66 (2FP-32)	Closed 2FP-32 by turning handwheel clockwise and observing valve stem insertion until resistance felt.	N/A SAT UNSAT			
		Examiner CUE:					
		2FP-32 stem is fully inserted and resistance is felt.					
			NSITION NOTE:				
(C)	3. (Step 14.3.2)	Verify the following valves closed: RWT Recirculation Isolation	Contacted Control Room to close 2CV-5637-1. Or	N/A SAT UNSAT			
		(2CV-5637-1).	Locally at 2CV-5637-1, checked the stem fully down/inserted.				
		Examiner CUE:					
		Control room reports 2CV-5637-1 indicates closed.					
		Or locally at the valve, the stem fully down/inserted.					

	_			
(C)	4. (Step	Verify the following valves closed:	Contacted Control Room to close 2CV-5638-2.	N/A SAT UNSAT
	14.3.2)	RWT Recirculation Isolation		
	(2CV-5638-2) closed.		Locally at 2CV-5637-1, checked the stem fully down/inserted.	
		Examiner CUE:		
		Control room reports 2CV-		
		5638-2 indicates closed.		
		Or locally at the valve, the stem fully down/inserted.		
		TRA	NSITION NOTE:	
		Go to elevation 354' RAB, just no	orth of the elevator, to the SFP valv	e gallery.
(C)	5. (Step 14.3.3)	Open Borated MU, RWT, SW to Fuel Pool (2FP-31).	Opened 2FP-31 by turning handwheel CCW.	N/A SAT UNSAT
	,	Evenines CUE:	Observed value stem fully	
		Examiner CUE:	Observed valve stem fully withdrawn out of the valve.	
(0)		2FP-31 stem is fully extended.	0 1050 401 4 3	NI/A CAT LINICAT
(C)	6.	Open RWT to Fuel Pool Isolation (2FP-46).	Opened 2FP-46 by turning handwheel CCW.	N/A SAT UNSAT
	(Step 14.3.4)		Thanks with the second	
		Examiner CUE:	Observed valve stem fully	
		2FP-46 stem is fully extended.	withdrawn out of the valve.	
		TRA	NSITION NOTE:	
		Go to elevation	335' RAB just off elevator.	
(C)	7. (Step 14.3.5)	Close SW Header 1 Telltale Drain (2SW-8401).	Closed 2SW-8401 by turning handwheel CW.	N/A SAT UNSAT
		Examiner CUE:	Observed valve stem fully	
		2SW-8401 has stem inserted	inserted into the valve.	
		into handwheel.		
	8. (Step 14.3.6,	Record unlocking of SW Hdr 1 Emerg Feed Isol (2SW-57) in Cat E log.	Contacted the Control Room to recorder 2SW-57 in the Cat E log.	N/A SAT UNSAT
	A)	Examiner CUE:		
		2SW-57 has been recorded in the Cat E log.		
(C)	9. (Step 14.3.6,	Unlock and open SW Header 1 Emergency Feed Isolation (2SW- 57).	Unlocked locking device on 2SW-57 using category "E" valve key.	N/A SAT UNSAT
	В)		Opened 2SW-57 by turning	
		Examiner CUE:	handwheel CCW.	
		2SW-57 valve stem is fully extended.	Observed valve stem fully withdrawn out of the valve.	

### **Procedure Caution:**

- If SFP overfilled, water can spill through SFP Cooling/SFP Purification pipe chase (SFP, ~ 402', SW corner) and contaminate SFP valve gallery (AB 354') and hall near 2F-3A/B (AB 335')
- If flow introduced with level > 401' 7", some overflow can occur due to wave motion of water.

#### **Procedure Note:**

If CLP (2LI-5400) used as SFP level indication during SFP drain and fill evolutions, SFP and CLP levels will differ slightly until water levels equalize through seal area.

until water levels equalize through se	ear area.	
IF two Operators available, THEN establish and maintain the following:	Verified that an Operator was stationed to monitor SFP level	N/A SAT UNSAT
Operator stationed at SFP to continuously monitor level:		
SFP (2LI-5401)		
CLP (2LI-5400) if gate removed or deflated		
Examiner CUE:		
Operator is stationed at SFP to monitor level. responds to radio communications.		
Operator stationed at SFP SW Hdr 1 Emerg Feed Isol to SF Pool (2SW-138) for valve adjustment	Examinee positions self at 2SW-138.	N/A SAT UNSAT
Communications between	Examinee establishes	N/A SAT UNSAT
·	stationed at the Spent Fuel Pool	
Operator that stationed at SFP to monitor level responds to radio communications.		
*IF level being maintained without frequent valve adjustment, THEN the following may be performed:	Examinee assesses level being maintained.	N/A SAT UNSAT
Secure Operator at 2SW-138.		
Reassign remaining Operator to continuously monitor SFP level EXCEPT when 2SW-138 adjustment needed.		
Examiner Cue: Operator at SFP will monitor if level can be maintained without frequent adjustments		
	IF two Operators available, THEN establish and maintain the following: Operator stationed at SFP to continuously monitor level: SFP (2LI-5401) CLP (2LI-5400) if gate removed or deflated Examiner CUE: Operator is stationed at SFP to monitor level. responds to radio communications.  Operator stationed at SFP SW Hdr 1 Emerg Feed Isol to SF Pool (2SW-138) for valve adjustment  Communications between stationed Operators. Examiner CUE: Operator that stationed at SFP to monitor level responds to radio communications.  *IF level being maintained without frequent valve adjustment, THEN the following may be performed: Secure Operator at 2SW-138. Reassign remaining Operator to continuously monitor SFP level EXCEPT when 2SW-138 adjustment needed. Examiner Cue: Operator at SFP will monitor if level can be maintained without frequent	IF two Operators available, THEN establish and maintain the following: Operator stationed at SFP to continuously monitor level: SFP (2LI-5401) CLP (2LI-5400) if gate removed or deflated Examiner CUE: Operator is stationed at SFP to monitor level. responds to radio communications.  Operator stationed at SFP SW Hdr 1 Emerg Feed Isol to SF Pool (2SW-138) for valve adjustment  Communications between stationed Operators. Examiner CUE: Operator that stationed at SFP to monitor level responds to radio communications.  *IF level being maintained without frequent valve adjustment, THEN the following may be performed: Secure Operator at 2SW-138. Reassign remaining Operator to continuously monitor SFP level EXCEPT when 2SW-138 adjustment needed.  Examiner Cue: Operator at SFP will monitor if level can be maintained without frequent

### **EXAMINERS NOTE:**

- Final valve position is at discretion of examinee to ensure level is raised/maintained.

# ANO-2-JPM-NRC-FPEM1 PAGE 6 OF 8

### JOB PERFORMANCE MEASURE

	14. (Step 14.3.8)	IF two Operators NOT available, THEN perform the following: Obtain SM/CRS concurrence. Station one Operator to align for fill and continuously monitor level EXCEPT when adjusting valve.	Determined that two operators are available from initial conditions.	N/A SAT UNSAT		
(C)	15. (Step 14.3.9)	Throttle open SW Header 1 Emergency Feed Isolation to SF Pool (2SW-138) between 401'4" and 401'7".  Examiner CUE: Operator stationed at SFP reports level raising slowly.	Throttled open 2SW-138 by turning handwheel CCW.  Observed valve stem withdrawn out of the valve commensurate with valves throttling.	N/A SAT UNSAT		
	END					

STOP		
TIME:		

# **Examiner's Copy**

### JPM INITIAL TASK CONDITIONS:

(2K11-K5) "FUEL POOL TEMP HI" is in alarm. Neither fuel pool cooling pump is available. An operator is stationed at the spent fuel pool to monitor spent fuel pool level. Fuel pool level is low. 2P66 is aligned to the RWT and running.

### **INITIATING CUE:**

The CRS directs, "Align Loop 1 service water to provide emergency makeup to the spent fuel pool starting on step 14.3 of OP 2104.006."

# **Examinee's Copy**

### JPM INITIAL TASK CONDITIONS:

(2K11-K5) "FUEL POOL TEMP HI" is in alarm. Neither fuel pool cooling pump is available. An operator is stationed at the spent fuel pool to monitor spent fuel pool level. Fuel pool level is low. 2P66 is aligned to the RWT and running.

### **INITIATING CUE:**

The CRS directs, "Align Loop 1 service water to provide emergency makeup to the spent fuel pool starting on step 14.3 of OP 2104.006."

Facility: ANO-2	Scenario No.: 1 (New)	Op-Test No.: 2012-1
Examiners:	Operators:	

#### **Initial Conditions:**

~94% power due to Main Steam Line high vibs, MOL, All Engineered Safety Features systems are in standby. #3 and #4 MTG control valve steam lead valves failed closed. Voltage regulator auto tracking circuit is disabled. RED Train Maintenance Week.

#### Turnover:

~94% power due to Main Steam Line high vibs, 260 EFPD. EOOS indicates 'Minimal Risk'. RED Train Maintenance Week. #3 and #4 MTG control valve steam lead valves failed closed. Voltage regulator auto tracking circuit is disabled.

Scheduled Evolution: Swap running Electro Hydraulic (EH) pumps.

Event No.	Malf. No.	Event Type*	Event Description	
1		N (BOP) N (SRO)	Shift Electro Hydraulic (EH) pumps	
2	XSI2PT5602		Containment pressure transmitter fails high. Tech Spec for SRO.	
3	CWS2P3AFLT	C (BOP) R (ATC) C (SRO)	'A' Circulating Water pump trip.	
4	CV48731	C (ATC) C (SRO)	VCT outlet valve fails to close.	
5	XRCCHAPLVL	I (ATC) I (SRO)	'A' Pressurizer Level Channel Fails Low. Tech Spec for SRO	
6	K01-A07 BUS2A1 BUS2A3 A309	M (ALL)	2A-1 bus negative sequence and lockout causing a reactor trip. Also, 2A-3 bus will lockout and the feeder breaker (2A-309) will remain closed.	
7	500LOSE500 500LOSE161 EFW2P7OS	M (ALL)	Loss of Offsite Power and 2P-7A EFW pump overspeed trips causing a complete loss of feedwater.	
8	HPI2P89FAL	C (BOP) C (SRO)	2P-89B High Pressure Safety Injection pump fails to start SIAS	
* (N)ormal, (R)eactivity, (I)nstrument, (C)omponent, (M)ajor				

Total malfunctions. = 6, Malfunctions after EOP entry = 2, Abnormal events = 2, Major transient = 2, EOPs with substantive actions = 2, EOP Contingencies = 1, Critical tasks = 3.

Revision 2 Page 1 of 34

### Scenario #1 Objectives

- 1) Evaluate individual ability to transition running Electro Hydraulic (EH) pumps
- 2) Evaluate individual response to a failure of a Containment pressure transmitter.
- 3) Evaluate individual/crew response to a failure of a Circ Water Pump.
- 4) Evaluate individual response the Volume Control Tank outlet valve failure to operate.
- 5) Evaluate individual ability to perform a reduction in plant power.
- 6) Evaluate individual response to a failure for automatic control of Pressurizer level.
- 7) Evaluate individual ability to respond to Degraded Electrical power.
- 8) Evaluate individual ability to mitigate a Loss of Feedwater.
- 9) Evaluate individual ability to monitor operation of Engineered Safety Features equipment.
- 10) Evaluate individual ability to establish once through cooling.

#### **SCENARIO #1 NARRATIVE**

Simulator session begins with the plant at ~94% power steady state due to High Main Steam line vibrations.

When the crew has completed their control room walk down/brief, they will use 2106.012 Electro Hydraulic system operations procedure to swap lead EH pumps.

When EH pumps have been swapped and cued by the lead examiner, a Containment pressure transmitter will fail high. The Crew should refer to the Annunciator corrective action. The SRO should enter Tech Specs 3.3.1.1 RPS and 3.3.2.1 ESF Actuation system instrumentation. The BOP will be required to bypass the Plant Protection System channel 2 for Containment pressure hi (points 13 & 16) and Containment pressure hi-hi (point 17).

After the BOP has completed bypassing the appropriate points and cued by lead examiner; 'A' Circulating Water pump will trip. The SRO will enter Loss of Condenser Vacuum AOP, 2203.019. The SRO will direct the ATC to commence emergency boration from a Boric Acid Makeup tank to lower reactor power. The VCT outlet valve will fail to reposition and the crew must chose an alternate method to add negative reactivity. Alternate methods include starting a Boric Acid Makeup Pump and opening the emergency borate valve, or inserting Control Element Assemblies. The SRO will direct the BOP to lower turbine load to maintain condenser vacuum within the acceptable region described in the AOP. When condenser vacuum has started to improve and is within the acceptable region of the AOP attachment, emergency boration and CEA insertion will be secured. The crew should then prepare to commence a controlled down power to restore condenser vacuum less than 5.15 inches HG Abs.

After emergency boration is secured and cued by the lead examiner, 'A' Pressurizer level channel will fail low causing letdown to go to minimum, all pressurizer heaters to de-energize, all backup charging pumps to start and will raise actual pressurizer level. The SRO will enter Pressurizer Systems Malfunction AOP, OP 2203.028. The ATC will place letdown in manual to control flow and pressurizer level. The ATC will then select the unaffected pressurizer level channel for control of letdown, charging, and pressurizer heater control. After the unaffected pressurizer level channel is selected the ATC will restore letdown control to automatic. The SRO will enter Tech Spec 3.3.3.6 Post Accident Instrumentation.

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#### **SCENARIO #1 NARRATIVE (continued)**

After letdown has been restored to automatic and cued by the lead examiner, 2A-1 (4160V bus) will have a negative sequence event that causes a bus lockout. When the 2A-1 bus lockout occurs damage will also occur to the 2A-1/2A-3 tie breaker propagating the fault to 2A-3 and a 2A-3 bus lockout. The crew should recognize the 2A-1 bus lock out has reduced the number of available condensate pumps to 2 which will require a reactor trip (Feedwater can not maintain Steam Generator level), then trip the reactor.

The Crew will implement Standard Post Trip Actions (SPTA), OP 2202.001. When the Reactor trips a Loss of Offsite Power will occur and when 2P-7A Emergency Feedwater pump starts, it will overspeed trip causing a complete loss of feedwater. The BOP will close Main Steam Isolation Valves (MSIVs) due to a loss of power causing overcooling. The crew should attempt to open 2A-309 to remove the fault from 2A-3 but due to damage it will not open and can not be racked down. The ATC will align for aux spray to control RCS pressure during the heat up caused by closing the MSIVs. The BOP will isolate Steam Generator (SG) blowdown to conserve SG inventory. The BOP should align upstream atmospheric dump isolation valve to control SG pressure 950 to 1050 psia.

The SRO will diagnose Loss of Feedwater EOP, 2202.006. The SRO will direct energizing the LTOP relief isolation valves and commence trying to restore Feedwater to the Steam Generators. 2P-7A EFW pump will not be able to be reset, 2P-7B EFW pump will be unavailable due to 2A-3 being de-energized, 2P-75 AFW pump is unavailable due to the 2A-1 lockout, Main Feedwater and Condensate pumps are not available due to the loss of offsite power. The Crew will align for once through cooling and transition to the Functional Recovery EOP, 2202.009. When SIAS is actuated 2P-89B HPSI pump will fail to auto start. The BOP should start 2P-89C or 2P-89B HPSI pump. The ATC will align 2P-36C charging pump to green train power and start the pump.

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### **Simulator Instructions for Scenario 1**

Reset simulator to MOL 94% power IC stead state.

Ensure that AACG is secured and annunciators clear.

Place MINIMAL RISK, Green Train Protected and RED Train Maintenance Week signs on 2C11. 2P-89C aligned to green train.

T1, T2, T3, T4, & T5 set to false.

T5 = Reactor trip

Event No.	Malf. No.	Value/ Ramp Time	Event Description
1			Shift Electro Hydraulic (EH) pumps
2	XSI2PT5602 Trigger = T1	27	Containment pressure transmitter fails high. Tech Spec for SRO.
3	CWS2P3AFLT Trigger = T2	True	'A' Circulating Water pump trip.
4	CV48731	1	VCT outlet valve fails to close.
5	XRCCHAPLVL Trigger = T3	0%	'A' Pressurizer Level Channel Fails Low. Tech Spec for SRO
6	K01-A07 BUS2A1 BUS2A3 A309 Trigger = T4	On True/ Delay = 2 min. Locked closed	2A-1 bus negative sequence and lockout causing a reactor trip. Also, 2A-3 bus will lockout and the feeder breaker (2A-309) will remain closed.
7	500LOSE500 500LOSE161 EFW2P7OS Trigger = T5	True True True	Loss of Offsite Power and 2P-7A EFW pump overspeed trips causing a complete loss of feedwater.
8	HPI2P89BFAL	True	2P-89B High Pressure Safety Injection pump fails to start SIAS

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#### **Simulator Operator CUEs** Shift Electro Hydraulic (EH) pumps Cue: If requested as IAO, report that you are standing by at the EH pump ready to raise pressure as required. Cue: When 2P-14B is started wait ~ 15 sec, report that 2P-14B is running sat with discharge pressure is 1650 psi. Cue: When 2P-14A is secured wait ~ 15 sec, report that 2P-14A secured sat. Cue: When requested report that 2PI-9674 is reading 1600 psi. Containment pressure transmitter fails high. Tech Spec for SRO. Trigger T1 Cued by lead examiner Cue: If requested as the WWM, then report that I&C will start planning a package to troubleshoot the failed instrument. Cued by Trigger T2 'A' Circulating Water pump trip. Also the VCT outlet valve fails to lead close. examiner Cue: When contacted as the OAO, then report that 2P-3A has an acrid odor. Cue: When contacted as the IAO, then report the 2P-3A has over current drop flags. Cue: If contacted as the IAO to status B vacuum pump, then report the 2C-5B is running sat. Cue: When contacted as chemistry, then report that chemistry will sample for iodine at the time requested. Cue: If requested to open steam lead valve 2CV-0253 then acknowledge request and have the booth operator open 2CV-0253 after 3 min. Cue: If contacted as the WCO, then report that 2CV-4873-1 motor is turning but the stem does not moving to be moving. Cue: When contacted as the WWM, then report that a planner will begin planning work on 2P-3A. Cued by Trigger T3 'A' Pressurizer Level Channel Fails Low. Tech Spec for SRO lead examiner Cue: If requested as the WWM, then report that I&C will start planning a package to troubleshoot the failed instrument. Cued by Trigger T4 2A-1 bus negative sequence and lockout causing a reactor trip. Also, lead 2A-3 bus will lockout and the feeder breaker (2A-309) will remain closed. examiner Cue: If contacted as the IAO, then wait ~ 2 minutes report the 2A-1 bus lockout relay is tripped and there is an acrid odor but no fire or smoke present. Cue: If contacted as the IAO, then wait ~ 2 minutes report the 2A-3 bus lockout relay is tripped and there is an acrid odor but no fire or smoke present. Cue: If contacted as the IAO to secure #1 EDG, then acknowledge the request and have the booth operator place #1 EDG in local lockout. Cue: If contacted as the STA to report to the control room, acknowledge the request. Cue: If contacted as a NLO to perform Attachment 47 Field Operator Post Trip Actions, acknowledge request. Cue: When contacted as a NLO to close the LTOP relief isol valves, after 2 min have the booth operator close the LTOP breakers, then report the 2B51-E4, and 2B51-K2 are closed. Cue: If contacted as a NLO to assess 2A-309, report that 2A-309 is closed and the breaker has been Cue: If requested as a NLO to locally operate 2A-309, report that it will not open and can not be racked down. Loss of Offsite Power and 2P-7A EFW pump overspeed trips causing Trigger T5 Reactor trip a complete loss of feedwater.

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Cue: If contacted as the WCO, report that 2P-7A has a governor valve casing is cracked, and there is no oil in the turbine casing.

2P-89B High Pressure Safety Injection pump fails to start SIAS

Cue: If requested as IAO, then report that 2P-89B HPSI pump breaker looks normal and has no drop flags.

Cue: If requested as WCO, then report that 2P-89B HPSI pump post start checks are satisfactory.

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Op-Test No.:	2012-1		Scenario N	No.: 1	Event No.: 1		
Event Description: Shift Electro Hydraulic (EH) pumps							
Time	Position		Aţ	oplicant's Acti	ons or Behavior		
			Procedur	e Note:			
could o	• When performing first pump swap after system startup, a larger than normal pressure drop could occur when running pump is secured if pressure compensator for oncoming pump was adjusted with turbine offline.						
• Throttli	ing Pressure C	ompensator C	ontrol valve	e CLOSED wi	ill RAISE pressure.		
• Throttli	ing Pressure C	ompensator C	ontrol valv	e OPEN will I	LOWER pressure.		
	ANY	8.1			hand held radio locally to monitor d to raise pressure if required.		
Cue: If requ pressure as I		report that y			ne EH pump ready to raise		
	ВОР	8.3		perform the fo	d securing 2P-14A, llowing: Pump 2P-14B (2HS-9651).		
Cue: When 2 pressure is 1		ted wait ~ 15	sec, repor	t that 2P-14B	is running sat with discharge		
	ВОР		8.3.2	2P-14B disc psig, THEN	adjustment required to obtain charge pressure (2PI-9679) > 1550 V adjust using 2P-14B Pressure or (2EH-30).		
		Examiner n	ote: Pressu	ıre adjustmer	nt not required.		
	ВОР		8.3.3	Secure EHO	C Pump 2P-14A (2HS-9650).		
Cue: When 2	Cue: When 2P-14A is secured wait ~ 15 sec, report that 2P-14A secured sat.						
	ВОР		8.3.4	Header pres THEN adju	adjustment required to obtain EH ssure (2PI-9674) 1580 to 1620 psig, st using 2P-14B Pressure or (2EH-30).		
Cue: When	requested rep	ort that 2PI-9	9674 is rea	ding 1600 psi			
Termination	criteria: EH	pumps shifte	d or at lea	d examiner's	discretion.		

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Event Description: Containment pressure transmitter fails high. Tech Spec for SRO.

Time	Dooition	Applicantle Actions on Debouien			
Time	Position	Applicant's Actions or Behavior			
Cued by lead examiner	ANY	Announce annunciators:  2K04-A5 CH B RPS/ESF/PRETRIP/TRIP  2K04-B3 PPS Channel TRIP			
	ANY	Report Hi containment pressure and Hi/Hi Containment pressure pretrip/trip on PPS insert.			
		Also, report channel 2 Containment pressure reading 27 psia.			
	BOP	Compare all four channels of Containment pressure.			
	SRO	Implement Annunciator Corrective Action 2203.012D.  2K04-A5 CH B RPS/ESF/PRETRIP/TRIP applicable actions:  2.1 Refer to PPS insert on 2C03 to determine cause.  2.2 Compare all Channels to validate alarm.  2.6 IF channel failed,			
	SRO	Enter Tech 3.3.1.1, and 3.3.2.1.  Examiner Note: SRO must enter Tech Spec 3.3.1.1 action 2 and Tech Spec 3.3.2.1 action 10.			
	SRO	Direct BOP to bypass bistable points for the associated functional units:  Containment Pressure Hi (Bistable 13, 16, and 17).			

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	ВОР	Bypass point 13,	16, and 17 on PPS Channel B:	
		11.1	Refer to Tech Spec 3.3.1.1, 3.3.2.1, 3.3.3.5, 3.3.3.6 and TRM 3.3.1.1.	
		11.2	Circle channel and functional units (points) to be bypassed below:	
			Channel to be bypassed: A B C D	
			Points to be bypassed:	
			1 2 3 4 5 6 7 8 9 10 11 12 (3 16 17)18 19 20	
		11.3	Enter appropriate Tech Spec/TRM actions.	
		11.4	Verify points to be bypassed <u>NOT</u> bypassed in ANY other channel.	
		11.5	Place desired points in BYPASS for selected channel on 2C23.	
		Examiners 1	Note: Key 12 required.	
	ANY	Verify annunciate	or 2K04-C3 PPS CHANNEL BYPASSED	
		Verify correct ch	annels in bypass.	
	SRO	Contact work management.		
Cue: If requested as the WWM, then report that I&C will start planning a package to				

Cue: If requested as the WWM, then report that I&C will start planning a package to troubleshoot the failed instrument.

Termination criteria: Affected channel points placed in bypass or at lead examiner's discretion.

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Event Description: 'A' Circulating Water pump trip. Also the VCT outlet valve fails to close.

Time	Position	Applicant's Actions or Behavior	
Cued by Lead Examiner	ANY	Announce annunciator: 2K12-A9 CIRC WTR SYS POTENTIAL FAILURE.	
	ANY	Report 2P-3A is tripped and the discharge valve is closing / closed.	
	SRO	Enter and implement 2203.019 Loss of Condenser Vacuum AOP.	
	SRO	Open Placekeeping page.	
		2. Notify Control Board Operators to monitor floating steps.	
		3. Check procedure entered due to loss of OR reduced Circulating Water flow.	
	ANY	4. Verify Circ Water pump operation as follows:	
		At least ONE Circ Water pump (2P3A/B) running.	
		ANY stopped Circ Water Pump associated Discharge valve closed:	
		2P-3A 2CV-1224 2P-3B 2CV-1215 the OAO, then report that 2P-3A has an acrid odor. the IAO, then report the 2P-3A has over current drop flags.	
Cuc. When	ANY	■5. Check Condenser Vacuum less than 7 inches HG Abs.	
	ANY	■6. Check condenser Vacuum less than 5.3 inches HG Abs.  Examiner note: Condenser Vacuum will trend > 5.3 inches HG Abs.  (perform contingency)	
	SRO	Direct Step 6 Contingencies.  ■6. IF Condenser pressure greater than 5.3 inches HG Abs, THEN perform the following:	
	ATC	A. Commence Emergency Boration using 2202.010 Exhibit 1.	

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Op-Test No.:	2012-1	Scenario No.:	1	Event No.: 3 & 4		
Event Descri	ption: 'A' Ci	rculating Water pump trip. Als	so the VCT	Γ outlet valve fails to close.		
	ATC	1. Select ONE of the following	owing Em	ergency Boration flowpaths:		
		FLOWPATH		ACTIONS REQUIRED		
		A. Gravity Feed	A.	Verify at least ONE BAM Tank Gravity Feed valve open:  • 2CV-4920-1  • 2CV-4921-1		
		B. BAM pumps pump.	В.	1) Start at least ONE BAM		
				2) Open Emergency Borate valve (2CV-4916-2).		
				3) Verify Boric Acid Makeup Flow Control valve (2CV-4926) closed.		
Emergency		C. RWT to Charging	pumps	C. Open Charging Pump Suction Source From RWT valve (2CV-4950-2).		
Boration		Examiner Note: Gravity Feed is the normal method used for a Circulating water pump trip.				
	ATC	2. Close VCT Outlet valve (2CV-4873-1).				
		Examiner Note: The VCT of perform step 3.	outlet valv	ve will not close and ATC should		
	ATC	Recognize the VCT outlet va				
	ATC	3. <u>IF VCT Outlet valve d</u> <u>THEN</u> verify BAM Pu		close, gency Boration flowpath selected.		
	ATC	4. Verify Reactor Makeu closed.	p Water Fl	low Control valve (2CV-4927)		
		5. Verify at least ONE Cl flow greater than 40 gr		mp running and charging header		
		6. Verify charging header following:	r flow grea	ter than 40 gpm by either of the		
		<ul><li>2FIS-4863 Disch</li><li>Computer Point 1</li></ul>		09) OS, PMS or SPDS)		

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Event Description: 'A' Circulating Water pump trip. Also the VCT outlet valve fails to close.

Cue: If contacted as the WCO, then report that 2CV-4873-1 motor is turning but the stem does not moving to be moving.

Cue: When contacted as the WWM, then report that a planner will begin planning work on 2P-3A.

3A.		, , ,
	ВОР	B. Reduce Turbine Load to maintain Condenser Pressure within acceptable region of Attachment A, Backpressure Limits.
	ВОР	C. Maintain RCS Tc less than 554.7°F, refer to TS 3.2.6 Reactor Coolant System Cold Leg temperature.
	ATC	D. IF CEA insertion necessary to maintain Tc less than 554.7°F, THEN insert Group 6 or Group P CEAs using 2105.009 Exhibit 3 while maintaining CEAs in Acceptable region of COLR.
	ATC	E. WHEN Condenser Vacuum improving and maintained within Acceptable region of Attachment A, Backpressure Limits, THEN secure Emergency Boration.
	ATC	NOTE  Reactor Engineering assistance may be required to determine shutdown margin.
Steps to secure Emergency Boration		<ul> <li>7. WHEN boron concentration greater than minimum required for shutdown margin, THEN perform the following: <ul> <li>A. Verify VCT level ≥ 20%.</li> <li>B. Place VCT Outlet valve (2CV-4873-1) in AUTOMATIC.</li> <li>C. WHEN 2CV-4873-1 open, THEN perform the following: <ul> <li>1) Verify BOTH BAM pumps secured.</li> </ul> </li> <li>2) Verify the following valves closed: <ul> <li>2CV-4920-1</li> <li>2CV-4921-1</li> <li>2CV-4916-2</li> </ul> </li> <li>3) <u>IF</u> 2CV-4950-2 was opened, <u>THEN</u> place handswitch in AUTOMATIC and check that valve closes.</li> </ul> </li> </ul>
	ATC	F. Commence a power reduction using 2102.004 power operations until condenser pressure is less than 5.15 inches HG Abs.
	SRO	*7. Check RCS pressure 2025 to 2275 psia.
	ANY	*8. Check RCS TC 542 to 554.7°F using CPC PID 5, 6, 160, or 161.
<u> </u>		ı

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Event No.: 3 & 4 Op-Test No.: 2012-1 Scenario No.: 1 Event Description: 'A' Circulating Water pump trip. Also the VCT outlet valve fails to close. Cue: If contacted as the IAO to status B vacuum pump, then report the 2C-5B is running sat. ANY **\*9**. Maintain ASI within limits as specified in Core Operating Limits Report (COLR). ANY \*10. Check CEA positions as follows: ALL Regulating Group 6 CEAs greater than 112.5 inches withdrawn. B. ALL Regulating Group 1 through 5 CEAs fully withdrawn. ALL Group P CEAs greater than C. 135 inches withdrawn. **ANY** \*11. Check PZR level within 5% of setpoint. ANY 12. Notify Chemistry to perform the following: Sample for RCS Iodine within A 2 to 6 hours following Reactor power change greater than 15% in any one hour, refer to TS 3.4.8, Specific Activity. Monitor Secondary system chemistry. В Cue: When contacted as chemistry, then report that chemistry will sample for iodine at the time requested. Cue: If requested to open steam lead valve 2CV-0253 then acknowledge request and have the booth operator open 2CV-0253 after 3 min. SRO 13. Perform the following as directed by Operations Management: Maintain Plant conditions using 2102.004, Power Operation. Perform Plant Shutdown using 2102.004, Power Operation. Cue: When contacted as the WWM, then report that a planner will begin planning work on 2P-3A. Termination criteria: When emergency boration has been secured or at lead examiner's discretion.

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Scenario No.: 1

Event No.: 5

Op-Test No.: 2012-1

ATC

**ATC** 

ANY

**ATC** 

**SRO** 

Time	Position	Applicant's Actions or Behavior
Cued by examiner	ANY	Announce alarm 2K10-G6 CNTRL CH 1 LEVEL Lo and 2K10-F6 CNTRL CH 1 LEVEL LO LO.
		Report 2LI-4627-2 and 2LR-4625 indicate normal.
	SRO	Enter and implement AOP 2203.028, PZR Systems Malfunctions.
	SRO	7. Check the following PZR level annunciators clear:
		<ul> <li>"CNTRL CH 1/2 LEVEL LO"         2K10-G6/G7 (Annunciators not clear, implement contingency)</li> <li>"CNTRL CH 1/2 LEVEL HI" 2K10-J6/J7</li> </ul>
	ANY	<ul><li>7. Perform the following:</li><li>A. Compare PZR level instruments to determine the affected channel.</li></ul>
	ATC	C. <u>IF</u> selected control channel failed, <u>THEN</u> perform the following:
		1) Perform the following for Letdown Flow controller (2HIC-4817):
		a) Place controller in MANUAL.
		b) Adjust output to control PZR level within 5% of

2) Place PZR Level Channel Select switch (2HS-4628) to

3) Place PZR Low Level Cutoff select switch (2HS-4642) to

4) Verify PZR heaters and Normal Spray operating to restore

5) WHEN Letdown Flow controller (2HIC-4817) automatic

E. Refer to TS 3.3.3.5, Remote Shutdown Instrumentation and

AND manual signals match, THEN place Letdown Flow

unaffected channel. (Channel 2)

unaffected channel. (Channel B)

RCS pressure 2025 to 2275 psia.

3.3.3.6, Post-Accident Instrumentation.

Examiner Note: SRO must enter Tech Spec 3.3.3.6 action 1 for Post Accident Instrumentation.

SRO Contact work management.

controller in AUTO.

Cue: When contacted as the WWM, then report that I & C planner will begin planning work on failed level instrument.

Termination Criteria: PZR level control selected to channel 2 or at lead examiner's discretion.

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Event Description: 2A-1 bus negative sequence and lockout causing a reactor trip. Also, 2A-3 bus will lockout and the feeder breaker (2A-309) will remain closed. Loss of Offsite Power and 2P-7A EFW pump overspeed trips causing a complete loss of feedwater. 2P-89B High Pressure Safety Injection pump fails to start SIAS

Time	Position	Applicant's Actions or Behavior	
Cued by examiner	ANY	Report 2A-1 bus negative sequence alarm.	
	BOP/SRO	Refer to 2K-01 A7 Annunciator Corrective Action	
	ANY	<ul> <li>2.1 Check all three phase currents for EACH pump on 2A1:</li> <li>A Phase to B Phase</li> <li>A Phase to C Phase</li> <li>B Phase to C Phase</li> </ul>	
	ANY	Report 2A-1 and 2A-3 bus lockout	
		Examiner note: The SRO may discuss Tech Spec 3.8.2.1	

Cue: If contacted as the IAO, then wait ~ 1 minute report the 2A-1 bus lockout relay is tripped and there is an acrid odor but no fire or smoke present.

Cue: If contacted as the IAO, then wait  $\sim 1$  minute report the 2A-3 bus lockout relay is tripped and there is an acrid odor but no fire or smoke present.

	SRO	Direct tripping the reactor due to inability to maintain S/G levels	
	SRO	Enter and implement EOP 2202.001, Standard Post Trip Actions.	
	SRO	Notify Control Board Operators to monitor safety functions using Exhibit 7, CBO Reactor Trip Checklist.	
		2. Open Safety Function Tracking page.	
Reactivity control	ATC	3. Check Reactivity Control established as follows:	
safety		A. Reactor power lowering.	
function		B. ALL CEAs fully inserted by observing ANY of the following:	
		CEA Rod bottom lights illuminated.	
		2) CEAC 1 indicates ALL CEAs fully inserted.	
		3) CEAC 2 indicates ALL CEAs fully inserted.	
	ВОР	4. Check Maintenance of Vital Auxiliaries satisfied:	
Vital		A. Check Main Turbine tripped by BOTH of the following:	
Auxiliaries		ALL Main Stop Valves closed.	
safety function		Generator megawatts indicate zero.	
		B. Generator Output breakers open.	
		C. Exciter Field breaker open.	

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Event Description: 2A-1 bus negative sequence and lockout causing a reactor trip. Also, 2A-3 bus will lockout and the feeder breaker (2A-309) will remain closed. Loss of Offsite Power and 2P-7A EFW pump overspeed trips causing a complete loss of feedwater. 2P-89B High Pressure Safety Injection pump fails to start SIAS

pump fails to start SIAS		
Time	Position	Applicant's Actions or Behavior
Vital Auxiliaries safety function	ВОР	<ul> <li>D. Check the following valves closed:</li> <li>MSR 2E-12A Steam Supply From SG A (2CV-0400) (Not met, perform contingency)</li> <li>MSR 2E-12B Steam Supply From SG B (2CV-0460) (Not met, perform contingency)</li> <li>Step D. contingency actions.</li> </ul>
		<ul><li>D. Close MSIVs:</li><li>2CV-1010-1</li><li>2CV-1060-2</li></ul>
Vital Auxiliaries safety function	ВОР	E. At least ONE 6900v AC bus energized. (Not met)  F. At least ONE 4160v Non-vital AC bus energized. (Not met)  G. BOTH 4160v Vital AC buses energized. (2A-4 Vital bus will be energized but 2A-3 will not be energized due to the feeder breaker failure.)  H. BOTH DGs secured. (Both DGs will be running, Service water will be aligned to #2 EDG and after #1 EDG will not have service water aligned.)  Examiner note: Service Water pressure low alarm will be in for #2 EDG the crew may elect to throttle service water to component cooling water or service water to auxiliary cooling water.  Examiner note: The SRO may discuss Tech Spec 3.8.1.1 for offsite power sources.
Critical Step: Secure #1 EDG prior to completion of the Drill.	BOP/SRO	Step H. contingency actions.  H. IF ANY DG running AND SW NOT aligned, THEN locally stop DG by unlocking and placing "ENGINE CONTROL" handswitch in LOCKOUT:  2E11 2E21  Examiner note #1 EDG must be locally placed in Lockout.

Cue: If contacted as the IAO to secure #1 EDG, then acknowledge the request and have the booth operator place #1 EDG in local lockout.

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Event Description: 2A-1 bus negative sequence and lockout causing a reactor trip. Also, 2A-3 bus will lockout and the feeder breaker (2A-309) will remain closed. Loss of Offsite Power and 2P-7A EFW pump overspeed trips causing a complete loss of feedwater. 2P-89B High Pressure Safety Injection pump fails to start SIAS

pump rans to		T
Time	Position	Applicant's Actions or Behavior
Vital Auxiliaries safety function	ВОР	<ul> <li>I. At least ONE 125v Vital DC bus energized:</li> <li>• 2D01 - SPDS point E2D01</li> <li>• 2D02 - SPDS point E2D02</li> </ul>
RCS Inventory Control Safety function	ATC	<ul> <li>5. Check RCS Inventory Control established as follows:</li> <li>A. PZR level: <ul> <li>10 to 80%.</li> <li>Trending to setpoint.</li> </ul> </li> <li>B. RCS MTS 30°F or greater.</li> </ul>
RCS Pressure Control Safety function	SRO	<ul> <li>6. Check RCS Pressure Control: <ul> <li>• 1800 to 2250 psia.</li> <li>• Trending to setpoint.</li> <li>• Normal PZR Spray and heaters controlling pressure. (Not met, perform contingency)</li> <li>• Valid CNTMT Spray NOT in progress.</li> </ul> </li> <li>Direct the following actions as necessary: <ul> <li>6. Perform as necessary:</li> </ul> </li> <li>D. IF RCP 2P32A or 2P32B stopped, THEN verify associated PZR Spray valve in MANUAL and closed. (Should be performed, RCPs are secured.)</li> <li>G. IF ALL RCPs stopped AND RCS pressure control required, THEN initiate Aux spray using 2202.010 Attachment 48, RCS Pressure Control.</li> </ul>
Core Heat Removal safety function.	ATC	<ul> <li>7. Check Core Heat Removal by forced circulation:</li> <li>A. At least ONE RCP running. (RCPs will be secured due LOOP and CRS should perform the contingency)</li> </ul>
	SRO	Verify the contingency for step 7. A
	ATC	A. IF ALL RCPs stopped, THEN perform the following: 1) Verify BOTH PZR Spray valves in MANUAL and closed. 2) GO TO Step 8.

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Event Description: 2A-1 bus negative sequence and lockout causing a reactor trip. Also, 2A-3 bus will lockout and the feeder breaker (2A-309) will remain closed. Loss of Offsite Power and 2P-7A EFW pump overspeed trips causing a complete loss of feedwater. 2P-89B High Pressure Safety Injection pump fails to start SIAS

Time	Position	Applicant's Actions or Behavior
	ANY	8. Check RCS Heat Removal:
		A. Check SG available by BOTH of the following:
		<ul> <li>At least ONE SG level 10 to 90%.</li> <li>FW maintaining SG level. (Not met, perform contingency)</li> </ul>
		<ul><li>C. Check Feedwater line intact by the following:</li><li>SG level stable or rising.</li></ul>
		<ul><li>NO unexplained step changes or erratic FW flow.</li><li>NO unexplained step changes or erratic Condensate flow.</li></ul>
		D. Check SG pressure 950 to 1050 psia. (Not Met, perform Contingency)
		F. Check RCS TC 540 to 555°F.
RCS Heat Removal Safety	ANY	Perform step 8 contingency actions that are applicable: A. Perform the following:
Function		IF SG level lowering,     THEN verify EFAS actuated.
		5) IF FW NOT available,
		THEN perform the following:
		a) Verify maximum of ONE RCP running in each loop.
		b) Close SG Blowdown Isolation valves:
		<ul><li>2CV-1016-1</li><li>2CV-1066-1</li></ul>
		c) GO TO Step 8.C
		Examiner note: EFAS is actuated but does not function. SRO may discuss Tech Spec 3.0.3 for no EFW trains.

Cue: If contacted as the WCO, report that 2P-7A has a governor valve body is cracked, and there is no oil in the turbine casing.

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Event Description: 2A-1 bus negative sequence and lockout causing a reactor trip. Also, 2A-3 bus will lockout and the feeder breaker (2A-309) will remain closed. Loss of Offsite Power and 2P-7A EFW pump overspeed trips causing a complete loss of feedwater. 2P-89B High Pressure Safety Injection pump fails to start SIAS

Time	Position	Applicant's Actions or Behavior	
RCS Heat Removal Safety Function	ANY	<ul> <li>E. Perform as necessary:</li> <li>2) Verify SDBCS restoring SG pressure 950 psia to 1050 psi using 2105.008 Exhibit 3, SDBCS Emergency Operation</li> </ul>	
		4) IF SG pressure greater than 1050 psia, THEN check MSSVs operating to control SG pressure 1050 psia to 1100 psia.	
		9. Check CNTMT parameters:	
		<ul> <li>A. Temperature and Pressure:</li> <li>• Temperature less than 140°F.</li> <li>• Pressure less than 16 psia.</li> </ul>	
		B. Check CNTMT Spray pumps secured.	
		C. NO CNTMT radiation alarms or unexplained rise in activity:	
Containment Safety Function		<ul> <li>1) CAMS alarms:</li> <li>"CNTMT PART/GAS RAD HI/LO" annunciator (2K10-B6) clear.</li> </ul>	
		<ul> <li>2) RCS leakage alarms:</li> <li>"AREA RADIATION HI/LO" annunciator (2K11-B10) clear.</li> <li>"PROC LIQUID RADIATION HI/LO" annunciator (2K11-C10) clear.</li> </ul>	
		<ul> <li>3) Check the following radiation monitors trend stable:</li> <li>• CNTMT Area</li> <li>• CAMS</li> <li>• Process Liquid</li> </ul>	
		D. NO secondary system radiation alarms or unexplained rise in activity:	
		1) "SEC SYS RADIATION HI" annunciator (2K11-A10) clear.	
		<ul> <li>Secondary Systems Radiation monitors trend stable:</li> <li>Main Steam lines</li> <li>SG Sample</li> <li>Condenser Off Gas</li> </ul>	

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Event Description: 2A-1 bus negative sequence and lockout causing a reactor trip. Also, 2A-3 bus will lockout and the feeder breaker (2A-309) will remain closed. Loss of Offsite Power and 2P-7A EFW pump overspeed trips causing a complete loss of feedwater. 2P-89B High Pressure Safety Injection pump fails to start SIAS

Time	Position	Applicant's Actions or Behavior	
	SRO	10. Notify STA to report to control room.	
		11. Direct NLOs to perform 2202.010 Attachment 47, Field Operator Post Trip Actions.	
		12. Verify Reactor trip announced on Plant page.	
		13. Notify SM to refer to Technical Specifications and 1903.010, Emergency Action Level Classification.	
Cue: If conta	acted as the S	STA to report to the control room, acknowledge the request.	
		O to perform Attachment 47 Field Operator Post Trip Actions,	
acknowledge	e request.		
	SRO	14. Direct control board operators to acknowledge ALL annunciators and announce ALL critical alarms.	
	SRO	15. Check ALL safety function acceptance criteria satisfied. (All safety functions are not satisfied, perform contingency)	
		<ul> <li>15. <u>IF</u> ANY safety function acceptance criteria <u>NOT</u> satisfied, <u>THEN</u> perform the following:</li> <li>A. Notify control room staff of safety functions <u>NOT</u> satisfied.</li> </ul>	
		B. GO TO Exhibit 8, Diagnostic Actions.	
	SRO	Diagnose Loss of Feedwater EOP 2202.006.	
	SRO	Enter and implement Loss of Feedwater EOP 2202.006.	
	SRO	Direct STA to perform the following:  *1. Confirm diagnosis of Loss Of Feedwater by checking SFSC acceptance criteria satisfied every 15 minutes.	
	SRO	*2. Notify SM to refer to Technical Specifications and 1903.010, Emergency Action Level Classification.	
	SRO	3. Open Placekeeping page.	
	SRO	4. Notify Control Board Operators to monitor floating steps.	
	ATC	5. Reduce RCS heat input as follows:	
		A. Stop ALL RCPs.	
		B. Verify BOTH PZR Spray valves in MANUAL and closed.	
		Examiner note: These actions should already be complete.	

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Event Description: 2A-1 bus negative sequence and lockout causing a reactor trip. Also, 2A-3 bus will lockout and the feeder breaker (2A-309) will remain closed. Loss of Offsite Power and 2P-7A EFW pump overspeed trips causing a complete loss of feedwater. 2P-89B High Pressure Safety Injection pump fails to start SIAS

pump rans to	Start SIAS		
Time	Position	Applicant's Actions or Behavior	
	ВОР	6. Conserve SG inventory as follows:	
		A. Verify SG Blowdown Isolation valves closed:	
		• 2CV-1016-1	
		• 2CV-1066-1	
		B. Verify SG Sample valves closed.	
		• 2CV-5850	
		• 2CV-5858	
		• 2CV-5852-2	
		• 2CV-5859-2	
	ВОР	■ 7. Check ALL AC and Vital DC buses energized.	
		Examiner note: not all buses are energized the SRO should go to the	
		contingency.	
	BOP/SRO	■ 7. Perform the following:	
		A. IF BOTH 4160v Non-vital buses 2A1 and 2A2 NOT energized from offsite power, THEN verify MSIVs closed.	
		B. IF ANY AC bus de-energized, THEN commence power restoration using 2202.010 Attachment 11, Degraded Power.	
		C. IF ANY Vital DC bus de-energized, THEN commence power restoration using 2203.037, Loss of 125v DC.	
		Examiner note: The SRO should review attachment 11 but power can not be restored.	
Critical Task:	ATC	*8. Maintain RCS pressure 1800 psia to 2250 psia using 2202.010 Attachment 48, RCS Pressure Control.	
Maintain RCS pressure within PT limit 30 to 200 degrees MTS.		Examiner note: RCS pressure must be maintain within the PT limits 30 to 200 degrees Margin To Sat. (MTS) and less than 2500 PSIA until once through cooling is established.	

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Event Description: 2A-1 bus negative sequence and lockout causing a reactor trip. Also, 2A-3 bus will lockout and the feeder breaker (2A-309) will remain closed. Loss of Offsite Power and 2P-7A EFW pump overspeed trips causing a complete loss of feedwater. 2P-89B High Pressure Safety Injection pump fails to start SIAS

Time	Position	Applicant's Actions or Behavior			
	SRO	The SRO should direct the following action to be completed.			
		9. Locally remove danger tags and close the following breakers:			
		• 2B51-E4 "LTOP RELIEF ISOL 2CV-4730-1"			
		• 2B51-K2 "LTOP RELIEF ISOL			
		2CV-4741-1"  Examinar note: Clasing the above breaker will not aparaize the valves			
		Examiner note: Closing the above breaker will not energize the valves because 2B51 is de-energized from the 2A-3 lockout.			
		a NLO to close the LTOP relief isol valves, after 2 min have the booth			
operator clo	se the LTOP	breakers, then report the 2B51-E4, and 2B51-K2 are closed.			
	SRO	11. IF EFW Pump 2P7B available, THEN establish EFW flow to SGs from 2P7B as follows:			
		Examiner note: 2P-7B is not available due to the 2A-3 lockout.			
		LO to assess 2A-309, report that 2A-309 is closed and the breaker has			
been damag		O to leadly analysis 24, 200, you get that it will not once and one not be			
racked down		O to locally operate 2A-309, report that it will not open and can not be			
	SRO	11. GO TO Step 12.			
	SRO	12. IF EFW Pump 2P7A available, THEN establish EFW flow to SGs from 2P7A as follows:			
		Examiner note: 2P-7B is not available due to the 2A-3 lockout.			
	SRO	12. GO TO Step 13.			
	SRO	13. IF AFW Pump 2P75 available, THEN establish AFW flow to SGs from 2P75 as follows:			
		Examiner note: 2P-75 is not available due to the 2A-1 lockout.			
	SRO	13. IF AFW Pump 2P75 unavailable, THEN perform EITHER of the following:			
		<ul> <li>GO TO Step 13.A if unavailability is due to electrical power supply.         <ul> <li>OR</li> </ul> </li> <li>GO TO Step 14.</li> </ul>			
		Examiner note: The SRO should go to step 14 due to the 2A-1 bus lockout Step 13.A will not restore power during a lockout.			

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Event Description: 2A-1 bus negative sequence and lockout causing a reactor trip. Also, 2A-3 bus will lockout and the feeder breaker (2A-309) will remain closed. Loss of Offsite Power and 2P-7A EFW pump overspeed trips causing a complete loss of feedwater. 2P-89B High Pressure Safety Injection pump fails to start SIAS

pump rans (	to start SIAS		
Time	Position	Applicant's Actions or Behavior	
	ANY	■14. Check IA pressure greater than 65 psig. (Not met, perform contingency)	
	ANY	■14. IF IA NOT available, THEN perform the following: A. Inform Unit 1 of IA cross-connect status.	
		C. IF Unit 1 IA pressure lowers to less than 60 psig, THEN perform the following:	
		1) Close IA Cross-Connect valves.	
		<ul><li> 2CV-3004</li><li> 2CV-3015</li></ul>	
		2) Locally verify the following valves closed:	
		<ul> <li>MANUAL X-CONNECT TO UNIT 1 (2IA-47)</li> <li>MANUAL X-CONNECT TO UNIT 1 (2IA-48)</li> </ul>	
		• 2F-37 INLET FROM DRY HDR (2IA-192)	
	ANY	15. Check MSIVs open. (Not met, perform contingency)	
	ANY	15. Perform the following:	
		A. IF MSIS actuated, THEN reset MSIS using 2202.010 Attachment 14,MSIS Reset (MSIS is not actuated)	
		B. IF CSAS actuated, THEN reset CSAS using 2202.010 Attachment 45, CSAS Reset. (CSAS is not actuated)	
		C. IF MSIS AND CSAS can NOT be reset, THEN GO TO Step 16.	
		D. Open MSIV Bypass valves.	
		E. WHEN SG and Main Steam pressure equalize within 50 psi, THEN open MSIVs.	
		Examiner note: The MSIVs can not be opened due to the LOOP and	
		crew should not attempt to open them.	
	ANY	16. Check at least ONE Condensate pump running. (Not met, perform contingency)	
	I	conduction (	

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Event Description: 2A-1 bus negative sequence and lockout causing a reactor trip. Also, 2A-3 bus will lockout and the feeder breaker (2A-309) will remain closed. Loss of Offsite Power and 2P-7A EFW pump overspeed trips causing a complete loss of feedwater. 2P-89B High Pressure Safety Injection pump fails to start SIAS

Time	Position	Applicant's Actions or Behavior		
	ANY	16. Manually start ONE Condensate pump as follows:		
		Examiner note: Condensate pumps are not available for start due to a		
		loss power.		
	ANY	17. Establish MFW flow to SGs as follows:  Examiner note: Main Feedwater can not be restored to the SG.		
	ANY	18. Establish Condensate flow to SGs as follows:		
		Examiner note: Condensate can not be restored to the SG.		
	SRO	■19. IF level lowers to 70 inches in a SG AND feedwater NOT restored to EITHER SG, THEN perform the following:		
		A. Isolate SG using 2202.010 Attachment 10, SG Isolation.		
		B. Maintain isolated SG pressure 950 psia to 1050 psia using upstream ADV or upstream ADV isolation MOV.		
		C. Continue efforts to restore feed to at least one SG.		
	SRO	■20. IF EITHER of the following conditions exist:		
		BOTH SGs with level less than     70 inches.		
		RCS TC rising in an uncontrolled manner.		
		THEN establish Heat Removal via		
		Once Through Cooling as follows:		
	ANY	A. Close MSIVs from Control Room.		
		Examiner note: MSIVs are already closed		
	BOP/ATC	B. Manually actuate SIAS and CCAS.		
	ВОР	C. Verify ALL HPSI Cold Leg Injection MOVs open.		
	ATC	D. Verify ALL available Charging pumps running.		
	ANY	E. Check 4160v Vital buses 2A3 AND 2A4 energized from offsite power. (Not met, perform contingency)		

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Event Description: 2A-1 bus negative sequence and lockout causing a reactor trip. Also, 2A-3 bus will lockout and the feeder breaker (2A-309) will remain closed. Loss of Offsite Power and 2P-7A EFW pump overspeed trips causing a complete loss of feedwater. 2P-89B High Pressure Safety Injection pump fails to start SIAS

pump fails to	pump fails to start SIAS		
Time	Position	Applicant's Actions or Behavior	
	ВОР	E. Perform the following:	
		1) IF EITHER 4160v Vital bus energized from offsite power, THEN perform the following: ( <b>Not met</b> )	
		a) Commence aligning third     HPSI pump to associated bus.	
		b) WHEN third HPSI pump alignment complete, THEN verify third HPSI pump running.	
		2) IF ANY 4160v Vital bus energized from DG, THEN perform the following:	
		a) Verify ONE HPSI pump running on train supplied by DG. (Not met, BOP should start 2P-89C or 2P-89B)	
		b) GO TO Step 20.G.	
		Examiner note: The SRO may discuss Tech Spec 3.5.2 for HPSI.	
Critical Step:	BOP/ATC		
Establish Once Through cooling		*G. Verify at least ONE HPSI pump running.	
prior to		H. Open ECCS PZR Vent valve (2CV-4698-1).	
S/G dry out with one HPSI pump running		I. Open LTOP/ECCS Relief Isolation valve (2CV-4740-2).	
	ANY	J. Maintain BOTH SG pressures 950 psia to 1050 psia using upstream ADVs or upstream ADV isolation MOVs.	
	ANY	K. GO TO 2202.009, Functional Recovery.	
	SRO	2. Record present time:	
		• Time	
	SRO	* 3. Notify SM to refer to Technical Specifications and 1903.010, Emergency Action Level Classification.	

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Event Description: 2A-1 bus negative sequence and lockout causing a reactor trip. Also, 2A-3 bus will lockout and the feeder breaker (2A-309) will remain closed. Loss of Offsite Power and 2P-7A EFW pump overspeed trips causing a complete loss of feedwater. 2P-89B High Pressure Safety Injection pump fails to start SIAS

Time	Position	Applicant's Actions or Behavior	
	ANY	* 4. Check RCS pressure greater than 1400 psia.	
	ANY	These actions have already been performed.  * 4. Perform the following:	
		B. IF NPSH requirements violated OR RCS MTS less than 30°F, THEN perform the following:	
		1) Stop ALL RCPs.	
		2) Verify BOTH PZR Spray valves in MANUAL and closed.	
		3) GO TO Step 6.	
	ANY	* 6. <u>IF SIAS or MSIS actuated,</u> <u>THEN</u> perform the following:	
		A. Verify at least ONE SW pump running on EACH loop. (Only one SW pump will be running)	
	ANY	B. Check EITHER DG running.	
	ANY	C. Verify running DG SW Outlet valve open:	
		<ul><li>2CV-1503-1</li><li>2CV-1504-2</li></ul>	
	ANY	D. Verify SW pump suction aligned to Lake.	
	ANY	E. Check 4160v Non-vital bus 2A1 OR 2A2 energized from offsite power. (Not met, perform contingency)	
	ANY	E. GO TO Step 7.	

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Event Description: 2A-1 bus negative sequence and lockout causing a reactor trip. Also, 2A-3 bus will lockout and the feeder breaker (2A-309) will remain closed. Loss of Offsite Power and 2P-7A EFW pump overspeed trips causing a complete loss of feedwater. 2P-89B High Pressure Safety Injection pump fails to start SIAS

Time	Docition	Applicantle Actions on Debayion
Time Position ANY		Applicant's Actions or Behavior
		7. IF CCW in service to provide SG Sample Cooler cooling, THEN perform the following:
		A. IF SG "A" has indicated water level, THEN verify the following SG 'A' Sample Valves open:
		<ul><li> 2CV-5850</li><li> 2CV-5852-2</li></ul>
		B. IF SG "B" has indicated water level, THEN verify the following SG 'B' Sample Valves open:
		• 2CV-5858
		• 2CV-5859-2
		C. Notify Chemistry to sample available SGs for activity.
		Examiner note: Neither S/G has water level that should be sampled and CCW is not in service.
	ANY	8. Check ALL available Hydrogen Analyzers in service; refer to 2104.044, Containment Hydrogen Control Operations – Exhibit 1, Hydrogen Analyzer Auto Start. (perform contingency)
	ANY	8. Verify all available Hydrogen Analyzers in service within 70 minutes from start of event.
		• Record time from Entry Section step 2:
		Time
		Examiner note: This step may not be performed due to 70 minute allowance for start.
	ANY	9. Open Functional Recovery Success Path Tracking page.
	SRO	10. Notify Control Board Operators to perform the following:
		<ul> <li>A. Monitor floating steps.</li> <li>B. Verify actuated ESFAS components using 2202.010 Exhibit 9, ESFAS Actuation.</li> </ul>

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Op-Test No.: 2012-1	Scenario No.: 1	Event No.: 6, 7 & 8

Event Description: 2A-1 bus negative sequence and lockout causing a reactor trip. Also, 2A-3 bus will lockout and the feeder breaker (2A-309) will remain closed. Loss of Offsite Power and 2P-7A EFW pump overspeed trips causing a complete loss of feedwater. 2P-89B High Pressure Safety Injection pump fails to start SIAS

pump rums to	Start SIAS		
Time	Position	Applicant's Actions or Behavior	
Exhibit 9 ESFAS actuation.	ВОР	*1.0 IF any abnormalities noted for affected ESFAS actuation, THEN notify CRS.  2.0 IF SIAS, THEN verify the following:  Red Train RWT Outlet (2CV-5630-1) open.  Red Train HPSI Pump in service with proper discharge pressure and flow.  Red Train Service Water Pump in service with proper discharge pressure.  Red Train LPSI Pump (2P60A) in service with proper discharge pressure and flow.  Red Train LPSI Injection MOVs open.  Green Train RWT Outlet (2CV-5631-2) open.  Green Train HPSI Pump in service with proper discharge pressure and flow.  Green Train HPSI Injection MOVs open.  Green Train Service Water Pump in service with proper discharge pressure and flow.  Green Train LPSI Injection MOVs open.  Green Train LPSI Pump (2P60B) in service with proper discharge pressure and flow.  Green Train LPSI Injection MOVs open.  Available Charging Pumps in service with proper discharge pressure and flow.  Service Water Outlet Valves open for #1 and #2 EDGs.  J. IF CCAS, THEN verify the following:  Red Train CNTMT Coolers in service.  Service Water aligned to Red Train CNTMT Coolers.  Red Train Bypass Dampers open.  Green Train Bypass Dampers open.  Green Train Bypass Dampers open.	

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Appenaix D		Scenario 1	Form ES-D-2
Op-Test No.:	2012-1	Scenario No.: 1	Event No.: 6, 7 & 8
lockout and t	he feeder brea eed trips causi	us negative sequence and lockout causing a rker (2A-309) will remain closed. Loss of Ofing a complete loss of feedwater. 2P-89B High	fsite Power and 2P-7A EFW
Time	Position	Applicant's Actions of	r Behavior
	ВОР	4.0 IF CSAS, THEN verify the following	). J.
Exhibit 9 ESFAS actuation.		Red Train CSS Pump (2P35A) in service with proper discharge pressurand flow.  Red Train CSS Header Isolation (2CV-5612-1) open.  Green Train CSS Pump (2P35B) in service with proper discharge pressurand flow.  Green Train CSS Header Isolation (2CV-5613-2) open.  Main Feedwater Block valves closed.  MSIVs closed.  Main Feed pumps tripped.  Condensate pumps secured.  Heater Drain pumps secured.  MSIVs closed.  MSIVs closed.  MSIVs closed.  MSIVs closed.  Main Feedwater Block valves closed.  MSIVs closed.  Main Feedwater Block valves closed.  Green Train Service Water Pump in service with proper discharge pressurant proper main feed pumps tripped.  Condensate pumps tripped.  Condensate pumps secured.  Heater Drain pumps secured.	
	ANY	11. Determine safety function status as	follows:
		A. Check Reactivity Control satisfie	ed by EITHER of the following:

Maximum of ONE CEA NOT fully inserted and Reactor power

Reactor power less than 10-1% and stable or lowering.

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lowering.

**Examiners note: Reactivity will be satisfied.** 

Event Description: 2A-1 bus negative sequence and lockout causing a reactor trip. Also, 2A-3 bus will lockout and the feeder breaker (2A-309) will remain closed. Loss of Offsite Power and 2P-7A EFW pump overspeed trips causing a complete loss of feedwater. 2P-89B High Pressure Safety Injection pump fails to start SIAS

Time	Position	Applicant's Actions or Behavior
	ANY	B. Check Vital DC Auxiliaries satisfied:
		1) At least ONE 125v Vital DC bus energized:
		• 2D01-SPDS point E2D01
		• 2D02-SPDS point E2D02
		2) At least ONE 120v Vital AC bus energized:
		2RS1 - SPDS point E2RS1 or E2RS1RS3
		<ul><li>2RS2 - SPDS point E2RS2 or E2RS2RS4</li><li>2RS3 - SPDS point E2RS3 or E2RS1RS3</li></ul>
		• 2RS4 - SPDS point E2RS4 or E2RS2RS4
		Examiners note: Vital DC will be satisfied.
	ANY	C. Check Vital AC Auxiliaries satisfied:
		1) At least ONE 4160v Vital AC bus (2A3/2A4) energized. <b>Examiners note: Vital AC will be satisfied.</b>
	ANY	D. Check RCS Inventory Control satisfied:
		1) CVCS maintaining PZR level
		10 to 80% [40 to 70%] and level stable or trending to setpoint.
		2) RCS MTS 30°F or greater.
		3) RVLMS LVL 03 or higher elevation indicates WET.
		Examiners note: Inventory Control will not be satisfied due to MTS.
	ANY	E. Check RCS Pressure Control satisfied:
		1) RCS pressure maintained within P-T limits, refer to
		Attachment 1, P-T Limits.
		Examiners note: P-T limits will not be satisfied.

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Event Description: 2A-1 bus negative sequence and lockout causing a reactor trip. Also, 2A-3 bus will lockout and the feeder breaker (2A-309) will remain closed. Loss of Offsite Power and 2P-7A EFW pump overspeed trips causing a complete loss of feedwater. 2P-89B High Pressure Safety Injection pump fails to start SIAS

pump fails to	o start SIAS		
Time	Position	Applicant's Actions or Behavior	
	ANY	F. Check RCS and Core Heat Removal satisfied:	
		At least ONE intact SG available for Heat Removal by EITHER of the following:	
		<ul> <li>Level 10 to 90% [20 to 90%] with FW available.</li> <li>Level being restored with total FW flow of 485 gpm or greater.</li> </ul>	
		2) <u>IF</u> ANY RCP operating, <u>THEN</u> RCS ΔT less than 10°F and <u>NOT</u> rising.	
		3) <u>IF</u> ALL RCPs secured, <u>THEN</u> RCS ΔT less than 50°F and <u>NOT</u> rising.	
		4) RCS T <sub>C</sub> less than 555°F and NOT rising.	
		5) RCS MTS 30°F or greater.	
		6) RVLMS LVL 01 indicates WET.	
		Examiners note: RCS and Core Heat Removal will not be satisfied due	
		to feedwater availability.	
	ANY	G. Check CNTMT Isolation satisfied:	
		1) CNTMT parameters normal:	
		a) "CNTMT RADIATION HI" annunciator (2K10-A6) clear.	
		b) NO unexplained rise in CNTMT radiation.	
		c) CNTMT pressure less than	
		18.3 psia. (Not met, perform contingencies)	
		2) "SEC SYS RADIATION HI" annunciator (2K11-A10) clear.	
		3) NO unexplained rise in Secondary Systems Radiation monitor trends:	
		<ul> <li>Main Steam lines</li> <li>SG Sample</li> <li>Condenser Off Cos</li> </ul>	
		Condenser Off Gas	
		Examiners note: CNTMT Isolation will not be satisfied due to once through cooling.	

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Event Description: 2A-1 bus negative sequence and lockout causing a reactor trip. Also, 2A-3 bus will lockout and the feeder breaker (2A-309) will remain closed. Loss of Offsite Power and 2P-7A EFW pump overspeed trips causing a complete loss of feedwater. 2P-89B High Pressure Safety Injection pump fails to start SIAS

Time	Position	Applicant's Actions or Behavior  1) Verify CNTMT isolation as follows:  • CIAS actuated components are properly aligned, using 2202.010 Attachment 5, CIAS Verification.  • ONE Emergency Penetration Room Vent fan running.  • EACH penetration NOT required to be open has at least ONE isolation valve closed.				
	ANY					
	ANY	COMPONENT DESCRIPTION	1.0 NUMBER	2.0 LOCATION	3.0 POSITION	√
		REGEN HX INLET	2CV-4821-1	2C09	CLOSED	
		REGEN HX OUTLET	2CV-4823-2	2C09	CLOSED	
		CNTMT PURGE SUPPLY V1 INSIDE	2CV-8289-1	2C17	CLOSED	
		CNTMT PURGE SUPPLY V1 OUTSIDE OUTSIDE	2CV-8283-1	2C17	CLOSED	
		CNTMT PURGE EXHAUST V2 INSIDE	2CV-8291-1	2C17	CLOSED	
		CNTMT PURGE EXHAUST V2 OUTSIDE OUTSIDE	2CV-8285-1	2C17	CLOSED	
		SAMPLE ISOLATION VALVE QUENCH TANK LIQ	2SV-5878-1	2C17	CLOSED	
		SAMPLE ISOLATION VALVE RCS	2SV-5833-1*	2C17	CLOSED	
		RCP CCW SUPPLY	2CV-5236-1*	2C17	CLOSED	
Standard		RCP CCW RETURN	2CV-5255-1*	2C17	CLOSED	
Attachment		CNTMT CHILL WATER SUPPLY	2CV-3852-1	2C17	CLOSED	
5		CNTMT CHILL WATER RETURN	2CV-3851-1	2C17	CLOSED	
		RX DRAIN TANK DISCH ISOL	2CV-2202-1	2C17	CLOSED	
		CNTMT VENT HEADER ISOL	2CV-2401-1	2C17	CLOSED (1)	
		CNTMT SUMP DRAIN	2CV-2060-1	2C17	CLOSED	
		RCP BLEEDOFF TO VCT	2CV-4846-1	2C17	CLOSED	
		CNTMT AIR SAMPLE NORTH INSIDE SUPPLY	2SV-8265-1*	2C17	CLOSED	
		CNTMT AIR SAMPLE NORTH INSIDE RETURN	2SV-8259-1*	2C17	CLOSED	
		CNTMT AIR SAMPLE SOUTH INSIDE SUPPLY	2SV-8273-1*	2C17	CLOSED	
		CNTMT AIR SAMPLE SOUTH INSIDE RETURN	2CV-8233-1*	2C17	CLOSED	
		PASS SUMP SAMPLE SUPPLY ISOL	2SV-5634-1*	2C17	CLOSED	
		PASS SUMP SAMPLE RETURN ISOL	2SV-5633-1*	2C17	CLOSED	

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Event Description: 2A-1 bus negative sequence and lockout causing a reactor trip. Also, 2A-3 bus will lockout and the feeder breaker (2A-309) will remain closed. Loss of Offsite Power and 2P-7A EFW pump overspeed trips causing a complete loss of feedwater. 2P-89B High Pressure Safety Injection pump fails to start SIAS

	O Start SIAS	
Time	Position	Applicant's Actions or Behavior
	ANY	H. Check CNTMT Temperature and Pressure Control satisfied:
		1) CNTMT pressure less than 16 psia.
		2) CNTMT temperature less than 140°F.
		Examiners note: CNTMT Temperature and Pressure Control may not be satisfied due to once through cooling.
	ANY	12. Locally remove danger tags and close the following breakers.
		<ul> <li>2B51-E4 "LTOP RELIEF ISOL 2CV-4730-1"</li> <li>2B51-K2 "LTOP RELIEF ISOL 2CV-4741-1"</li> </ul>
	ANY	*13. Check ALL Safety Function acceptance criteria satisfied.
	SRO	*13. Perform the following:
		A. Determine appropriate success paths using Success Path Decision Trees.
		B. Initiate success paths for ALL Safety Functions in the following order:
		1) Jeopardized.
		2) Challenged.
		3) Satisfied.
		C. <u>IF</u> higher priority Safety Function jeopardized <u>AND</u> lower priority safety function success path in progress, <u>THEN</u> <b>GO TO</b> appropriate success path for highest priority safety function in jeopardy.
		D. <u>WHEN</u> success path implemented for EACH Safety Function, <u>THEN</u> RETURN TO Step 14 of Entry procedure.
		O should complete the safety functions in order of hierarchy that are a the satisfied safety functions.
	SRO	Diagnose IC-2 as the first Jeopardized Success path.
	ANY	Verify SIAS and CCAS actuated on PPS inserts.

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Event Description: 2A-1 bus negative sequence and lockout causing a reactor trip. Also, 2A-3 bus will lockout and the feeder breaker (2A-309) will remain closed. Loss of Offsite Power and 2P-7A EFW pump overspeed trips causing a complete loss of feedwater. 2P-89B High Pressure Safety Injection pump fails to start SIAS

Time	Position	Applicant's Actions or Behavior
	ANY	*2. Verify Safety Injection flow to RCS as follows:
		A. Verify at least ONE HPSI pump running.
		B. Verify running HPSI Injection MOVs open.
		C. Verify ALL available Charging pumps running.
		D. Check RCS pressure less than 1390 psia.
		E. Check total HPSI flow acceptable using 2202.010 Exhibit 2, HPSI Flow Curve.
		F. Check total LPSI flow acceptable using 2202.010 Exhibit 3, LPSI Flow Curve.
	ANY	*3. IF CNTMT Spray pumps are running AND ALL of the following are TRUE:
		• CNTMT pressure less than 73.7 psia (59 psig).
		<ul> <li>At least two CNTMT Cooling Fans running.</li> </ul>
		SIAS actuated and flow acceptable per the following:
		- 2202.010 Exhibit 2, HPSI Flow Curve
		- 2202.010 Exhibit 3, LPSI Flow Curve
		THEN perform the following:
		A. Place EITHER CNTMT Spray pump (2P35A/B) in PTL.
		B. Verify CNTMT pressure maintained less than 73.7 psia (59 psig).

Termination criteria: When the appropriate success path identified and HPSI flow is verified using exhibit 2 or at examiner's discretion.

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Facility: ANO-2	Scenario No.: 2 (New)	Op-Test No.: 2012-1
Examiners:	Operators:	

## **Initial Conditions:**

100% MOL; All Engineered Safety Features systems are in standby. #3 and #4 MTG control valve steam lead valves failed closed. 'B' SIT level is high due to inleakage. Voltage regulator auto tracking circuit is disabled. RED Train Maintenance Week.

## Turnover:

100%. 260 EFPD. EOOS indicates 'Minimal Risk'. #3 and #4 MTG control valve steam lead valves failed closed. Voltage regulator auto tracking circuit is disabled. RED Train Maintenance Week. Evolution scheduled: Drain of 'B' Safety Injection Tank to clear the High pressure alarm. OPS-B37 log is not required.

Event No.	Malf. No.	Event Type*	Event Description	
1		N (BOP) N (SRO)	Drain 'B' Safety Injection Tank.	
2	XRC2PT46241		'A' channel wide range pressurizer pressure will fail low. Tech Spec for SRO.	
3	SDS0303DEM DI_HS_0303_1 DI_HS_0303_2	C(BOP) C(SRO)	2CV-0303 Turbine bypass demand fails high.	
4	SCRUNBACK	R (ATC) C (SRO)	Partial Stator water runback. Tech Spec for SRO.	
5	RCP2P32BGRN	M (ALL)	'B' Reactor Coolant Pump (RCP) trip.	
6	RPSRXAUTO	C (ATC) C (SRO)	Reactor Protection System auto trip failure.	
7	MS1003	M (ALL)	Main Steam Safety valve setpoint drifts causing an ESD.	
8	ESF1025 CV1038 MFWPMPBTRP	C (BOP) C (SRO)	EFW valve 2CV-1025-1 fails to auto close, and 2EFW-1038-2 failed open. 'B' Main feedwater pump trips to allow Steam Generator level to lower for Auto EFW actuation.	
9	CV4651	C (ATC) C (SRO)	'A' RCP spray valve failed closed. (Not credited as malfunction after EOP entry due to possibly being discovered prior to EOP entry)	
* (N)or	* (N)ormal, (R)eactivity, (I)nstrument, (C)omponent, (M)ajor			

Total malfunctions. = 7, Malfunctions after EOP entry = 2, Abnormal events = 2, Major transient = 2, EOPs with substantive actions = 1, EOP Contingencies = 0, Critical tasks = 2.

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# Scenario #2 Objectives

- 1) Evaluate individual ability to control Safety Injection Tank level.
- 2) Evaluate individual response to a failure of a Reactor Protection system instrument.
- 3) Evaluate individual response to a Loss of Turbine Load.
- 4) Evaluate individual response to a failure of a Steam Dump Bypass Control System output.
- 5) Evaluate individual response to a Reactor Coolant pump trip.
- 6) Evaluate individual response to a failure of the Reactor Protection system to trip the reactor.
- 7) Evaluate crew ability to mitigate an Excess Steam Demand.
- 8) Evaluate individual response to failed closed Reactor Coolant pump spray valve.
- 9) Evaluate individual ability to monitor operation of Engineered Safety Features equipment and operate Engineered Safety Features equipment.

#### **SCENARIO #2 NARRATIVE**

Simulator session begins with the plant at 100% power steady state #3 and #4 MTG control valve steam lead valves failed closed.

When the crew has completed their control room walk down and brief, the BOP will lower 'B' Safety Injection Tank (SIT) level.

When 'B' SIT level is lowered and cued by lead examiner; Channel A wide range RCS pressure transmitter will fail low. The SRO will use the Annunciator Corrective action and direct the BOP to bypass point 6 on the Plant Protection System. The SRO will also enter Tech Spec 3.3.1.1 for Reactor Protection System and 3.3.2.1 for Engineering Safety Features Actuation system, 3.3.3.5 remote shutdown, and 3.3.3.6 for Post accident instrumentaion. The BOP bypass point #6 on channel 'A' PPS.

After the BOP has bypassed point 6 and cued by lead examiner, 2CV-0303 SDBCS Turbine Bypass valve will fail open. The CRS should enter the overcooling in modes 1 or 2 AOP. The crew should reduce turbine load to maintain reactor power less than 100%. The BOP should take manual control of 2CV-0303 and close the valve limiting the power rise.

When the crew has closed 2CV-0303 and cued by the lead examiner, a partial stator cooling water runback will occur. The SRO will enter the Loss of Turbine Load AOP, OP 2203.024. The SRO should direct emergency boration and/or insert CEAs. The BOP should call NLOs to investigate the loss of Stator Cooling water pump. The SRO will enter Tech Spec 3.2.6 for RCS Tc. After the runback has stopped the crew should verify emergency boration is secured and commence normal boration to match the primary and secondary.

After the crew has secured emergency boration, stabilized the plant, and cued by the lead examiner, 'B' RCP will trip which should cause an automatic reactor trip but the reactor protection system will not auto operate. The ATC should perform the immediate action of manually trip the reactor.

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## **SCENARIO #2 NARRATIVE (continued)**

The Crew will implement Standard Post Trip Actions (SPTA), OP 2202.001. After the reactor trips 2PSV-1003 Main Steam Safety Valve setpoint will drift lower cause an Excess Steam Demand. 'B' Main feedwater pump trips after the reactor trip to allow Steam Generator level to lower for an auto EFW actuation. The EFW valves from 2P-7B EFW pump to 'A' Steam Generator will fail to close and the BOP will manually close 2CV-1025-1 EFW valve or secure 2P-7B EFW pump. The ATC will secure two Reactor coolant Pumps when RCS pressure goes below 1400 psia. The BOP will also override Service Water to Component Cooling water to allow RCP operation.

The SRO will diagnose an Excess Steam Demand event and enter OP 2202.005, Excess Steam Demand. The BOP will steam 'B' S/G using the upstream Atmospheric Dump valve when 'A' S/G blows dry. The ATC should determine that the normal pressurizer spray valve is not functioning properly and transition to using the Auxiliary Spray valve to maintain RCS pressure.

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## **Simulator Instructions for Scenario 2**

Reset simulator to MOL 100% power IC stead state. When Resetting the simulator leave 2CV-0303 HS in auto and override the switch check.

Ensure that AACG is secured and annunciators clear.

Place MINIMAL RISK, Green Train Protected and RED Train Maintenance Week signs on 2C11.

Ensure SIT Hi pressure alarm is in due to 'B' SIT level being elevated.

#3 and #4 MTG control valve steam lead valves failed closed.

T1, T2, T3, T4, T5 set to false.

T5 = Reactor Trip

13 – Reac	tor Trip	1	
Event No.	Malf. No.	Value/ Ramp Time	Event Description
1			Drain 'B' Safety Injection Tank.
2	XRC2PT46241 Trigger = T1	0	'A' channel wide range pressurizer pressure will fail low. Tech Spec for SRO.
3	SDS0303DEM Trigger = T2 DI_HS_0303_1 DI_HS_0303_2	100% / Ramp = 5 min. False True	2CV-0303 Turbine bypass demand fails high.
4	SCRUNBACK Trigger = T3	True/ Delete in = 60 secs.	Partial Stator water runback. Tech Spec for SRO.
5	RCP2P32BGRN Trigger = T4	True	'B' Reactor Coolant Pump (RCP) trip.
6	RPSRXAUTO	True	Reactor Protection System auto trip failure.
7	MS1003 Trigger = T5	0/ Delay = 30 sec. Ramp 15 min.	Main Steam Safety valve setpoint drifts causing an ESD.
8	ESF1025 CV10382 MFWPMPBTRP Trigger 5 for MFWP trip.	True 1 True / Delay = 5 sec.	EFW valve 2CV-1025-1 fails to auto close, and 2EFW-1038-2 failed open. 'B' Main feedwater pump trips to allow Steam Generator level to lower for Auto EFW actuation.
9	CV4651	0	'A' RCP spray valve failed closed. (Not credited as malfunction after EOP entry due to possibly being discovered prior to EOP entry)

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		Simulator Operator CUEs
At T=0		Drain 'B' Safety Injection Tank.
Cued by	Trigger T1	'A' channel wide range pressurizer pressure will fail low.
lead		
examiner		
Cue: If reque	ested as the WWM	, then report that I&C will start planning a package to troubleshoot
the failed inst	trument.	
Cued by	Trigger T2	2CV-0303 Turbine bypass demand fails high.
lead		
examiner		
Cue: If conta	cted as NLO to inv	vestigate 2CV-0303, after approximately 1 min. report that 2CV-
		lly. Also, report current valve position.
Cue: When c	ontacted as the W	WM, then report that I&C maintenance will investigate 2CV-0303.
Cued by	Trigger T3	Partial Stator water runback. Tech Spec for SRO.
lead		
examiner		
Cue: After th	e crew notice the (	Generator runback has stopped, report as the IAO that near the
stator water	skid when you hear	rd 2P-25B Stator water pump trip and 2P-25A auto start. State that
	estigating the skid	when you herd a check valve slam and 2P-25B then stopped
rotating.		
Cue: If conta	cted as Chemistry,	, report that you will sample the RCS for Iodine when requested.
Cued by	Trigger T4	'B' Reactor Coolant Pump (RCP) trip.
lead		
examiner		
Reactor Trip		Main Steam Safety valve setpoint drifts causing an ESD.
		report to the control room, acknowledge the request.
Cue: If conta	cted as a NLO to p	perform Attachment 47 Field Operator Post Trip Actions,
acknowledge	_	
		stry, then report you will sample both S/G for activity and Monitor
RDACS for o	ff site dose release	
		EFW valve 2CV-1025-1 fails to auto close, and 2EFW-1038-2 failed
		open. 'B' Main feedwater pump trips to allow Steam Generator level
		to lower for Auto EFW actuation.
		'A' RCP spray valve failed closed.

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Op-Test No.: 2012-1 Scenario No.: 2 Event No.: 1 Event Description: Drain 'B' Safety Injection Tank. Time Position Applicant's Actions or Behavior **BOP** 7.1 Draining SITs to RDT in Mode 1, 2, 3, or 4 with SIT Outlet valves Open 7.1.1 IF source of SIT inleakage unknown, THEN initiate SIT Level Change (OPS-B37) as desired. 7.1.2 IF OPS-B37 being maintained for SIT to be drained, **BOP** THEN record pre-drain level on OPS-B37. Examiner note: Steps 7.1.1 and 7.1.2 are not applicable. Monitor all RDT AND SIT levels and pressures during \*7.1.3 **BOP** this operation. Open SIT Drain to RDT (2CV-5081). 7.1.4 **BOP** 7.1.5 IF desired level drop < 1/2%, **BOP** THEN close 2CV-5081. Monitor pressure of SIT Injection header to be drained: \*7.1.6 **BOP** • SIS Injection to Loop 2P-32A (2PIS-5000) SIS Injection to Loop 2P-32B (2PIS-5020) • SIS Injection to Loop 2P-32C (2PIS-5040) SIS Injection to Loop 2P-32D (2PIS-5060) 7.1.8 Open desired SIT Drain valve: **BOP** • 2T-2A - SIT A Drain valve (2SV-5001-1) 2T-2B - SIT B Drain valve (2SV-5021-1) • 2T-2C - SIT C Drain valve (2SV-5041-2) 2T-2D - SIT D Drain valve (2SV-5061-2) Procedure Note: If SIT drain header pressurized, then SIT Drain valves may be slow to close due to no DP. (CR-ANO-2-2004-0802)

D1. (CR 11.10 2 200 1 0002)			
	ВОР	7.1.9 WHEN desired SIT level or pressure reached, THEN close SIT Drain valve opened:  • 2T-2A - SIT A Drain valve (2SV-5001-1)  • 2T-2B - SIT B Drain valve (2SV-5021-1)  • 2T-2C - SIT C Drain valve (2SV-5041-2)  • 2T-2D - SIT D Drain valve (2SV-5061-2)	
	ВОР	7.1.12 Verify 2CV-5081 closed.	
II			

Termination criteria: SIT Hi pressure alarm clear or at lead examiner's discretion.

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Event Description: 'A' channel wide range pressurizer pressure will fail low. Tech Spec for SRO.

Time	Position	Applicant's Actions or Behavior
Cued by	ANY	Announce annunciators:
lead examiner		2K04-A4 CH A RPS/ESF/PRETRIP/TRIP 2K04-B3 PPS Channel TRIP
	ANY	Report low pressurizer pressure pretrip/trip on PPS insert.
	ВОР	Compare all four channels and report 2PT-4624-1 indicates zero.
	SRO	Implement Annunciator Corrective Action 2203.012D.
		2K04-A4 CH A RPS/ESF/PRETRIP/TRIP applicable actions:
		2.1 Refer to PPS insert on 2C03 to determine cause.
		2.2 Compare all Channels to validate alarm.
		2.6 <u>IF</u> channel failed, <u>THEN</u> refer to Tech Specs 3.3.1.1, 3.3.2.1, 3.3.3.5, 3.3.3.6, and TRM 3.3.1.1.
		2K04-B3 PPS Channel TRIP applicable actions:
		2.1 Determine which RPS or ESFAS trips have occurred on PPS inserts.
		2.3 <u>IF</u> channel failed, <u>THEN</u> refer to Tech Specs 3.3.1.1 and 3.3.2.1 and TRM 3.3.1.1.
	SRO	Enter Tech Spec 3.3.1.1 action 2, 3.3.2.1 action 10, 3.3.3.5 and 3.3.3.6 action 1.
	SRO	Direct BOP to bypass bistable points for the associated functional units:
		A Pressurizer pressure low (Bistable 6)

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	ВОР	Bypass the point 6 Channel A:	
		11.1 Refer to Tech Spec 3.3.1.1, 3.3.2.1, 3.3.3.5, 3.3.3.6 and TRM 3.3.1.1.	
		Circle channel and functional units (points) to be bypassed below:	
		Channel to be bypassed: (A) B C D	
		Points to be bypassed:	
		1 2 3 4 5 6 7 8 9 10 11 12 13 16 17 18 19 20	
		Enter appropriate Tech Spec/TRM actions.	
		Verify points to be bypassed <u>NOT</u> bypassed in ANY other channel.	
		Place desired points in BYPASS for selected channel on 2C23.	
	ANY	Announce annunciator 2K04-C3 PPS CHANNEL BYPASSED Verify correct channels in bypass.	
	SRO	Contact maintenance/PS liaison.	
Cue: When	Cue: When contacted as the WWM, then report that I&C will start planning a work package to		

Cue: When contacted as the WWM, then report that I&C will start planning a work package to repair the failed instrument.

Termination criteria: Affected channel points placed in bypass or at lead examiner's discretion.

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Op-Test No.: 2012-1 Scenario No.: 2 Event No.: 3 Event Description: 2CV-0303 Turbine bypass demand fails high. Time Position Applicant's Actions or Behavior Cued by ANY Recognize that power is rising and/or that 2CV-0303 shows open lead indications. examiner Cue: If contacted as NLO to investigate 2CV-0303, after approximately 1 min. report that 2CV-0303 so no signs of failures locally. Also, report current valve position. **SRO** Enter and implement Overcooling in Modes 1 or 2 AOP, 2203.050. **BOP** \*1. Check NONE of the following conditions exist: Reactor trip is imminent. Uncontrolled lowering of RCS TAVE (less than 540°F for a critical reactor). RCS pressure less than 2000 psia AND lowering. Unable to control or restore plant parameters. Cooldown causes reactor to go subcritical (due to positive MTC) as indicated by the following: Reactor power less than 10 E-4 Flux level lowering Sustained negative SUR **ANY** 2. Secure any positive reactivity additions by stopping the following evolutions if in progress: CEA withdrawal RCS dilution **BOP** \*3. Control turbine load to maintain the following: Reactor power less than 100% TAVE within 2°F of TREF. IF load reduced by 25 megawatts AND TAVE and TREF NOT matched, THEN notify Shift Manager. **SRO** 4. Notify SM to refer to Technical Specifications and 1903.010, Emergency Action Level Classification.

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	SRO	5. (	Check the following criteria satisfied:
		A	A. Steam leak NOT indicated in CNTMT Building.
		F	3. Steam leak NOT indicated in Auxiliary or Turbine Building.
		C	C. Main Turbine Control Valves properly positioned for present power level.
		I	O. All SDBCS Valves closed. (Not met, perform contingency)
		F	E. FWCS maintaining SG levels at setpoint.
		F	F. MSSV Valve Open (2K04-G9) NOT in alarm.
			G. Gland Seal System pressure 1.5 to 7.0 psig.
	SRO	Ι	O. GO TO Step 9.
	ВОР		F any SDBCS valve affected, THEN close affected SDBCS valves using the following as necessary:
		•	Place individual HIC in manual AND close the affected SDBCS valve.
		•	<ul> <li>Place individual permissive handswitch for affected SDBCS valve in OFF.</li> </ul>
			iner note: The BOP will have to place the HIC in manual and the affected SDBCS valve.
	ATC	13. (	Check PZR level greater than 29% AND trending to setpoint.
	ATC	14.	Check RCS pressure 2025 psia to 2275 psia.
	ANY		Check RCS TC 542 to 554.7°F using CPC PID 5, 6, 160, or 161.
	ВОР		F cause of overcooling event stopped, THEN GO TO 2102.004, Power Operations, as necessary for power adjustment.
	SRO	Contac 2CV-0	ct Work Week Manager to investigate/ repair Steam bypass valve 0303.
Cue: When contacted as the WWM, then report that I&C maintenance will investigate 2CV-			

0303.

Termination criteria: When 2CV-0303 has been closed or at lead examiner's discretion.

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Op-Test No.: 2012-1 Scenario No.: 2 Event No.: Event Description: Partial Stator water runback. Tech Spec for SRO. Time Position Applicant's Actions or Behavior Cued by ANY Announce alarm Generator protection Circuit energized and Stator cooling lead water trouble examiner Cue: After the crew notice the Generator runback has stopped, report as the IAO that near the stator water skid when you heard 2P-25B Stator water pump trip and 2P-25A auto start. State that you were investigating the skid when you heard a check valve slam and 2P-25B then stopped rotating. **SRO** Enter and Implement AOP 2203.024, Loss of Turbine Load. **SRO** Open Placekeeping page. Examiner note: The SRO should implement the floating step for Generator Protection Circuit energized. **SRO ■**3. Check "Generator Protection Circuit Energized" annunciator (2K02-A4) NOT in alarm. ATC 3. Perform the following: A. IF Turbine runback in progress, THEN perform EITHER or BOTH of the following for power reduction: • Emergency Borate per 2202.010, Exhibit 1, Emergency Boration. • Insert Group 6 OR Group P CEAS per 2105.009, CEDM Control System Operation.

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Event Description: Partial Stator water runback. Tech Spec for SRO.

Time	Position	Applicant's Actions or Behavior		
	ATC	Select ONE of the following Emergency Boration flowpaths:		
		FLOWPATH		ACTIONS REQUIRED
		A. Gravity Feed	A.	Verify at least ONE BAM Tank Gravity Feed valve open:  2CV-4920-1 2CV-4921-1
		B. BAM pumps pump.	В.	1) Start at least ONE BAM
				2) Open Emergency Borate valve (2CV-4916-2).
				3) Verify Boric Acid Makeup Flow Control valve (2CV-4926) closed.
Emergency Boration		C. RWT to Charging	pumps	C. Open Charging Pump Suction Source From RWT valve (2CV-4950-2).
	Examiner Note: Gravity Feed is the normal method Water Runback.			· · · · · · · · · · · · · · · · · · ·
	ATC	2. Close VCT Outlet val	ve (2CV-4	4873-1).
	ATC	3. <u>IF VCT Outlet valve does NOT close,</u> <u>THEN</u> verify BAM Pumps Emergency Boration flowpath selected. <b>Examiner Note: This step is N/A</b>		
	ATC	4. Verify Reactor Makeup Water Flow Control valve (2CV-4927)		
	AIC	closed.		,
		5. Verify at least ONE C flow greater than 40 g		ump running and charging header
		6. Verify charging heade following:	r flow gre	eater than 40 gpm by either of the
		<ul><li>2FIS-4863 Discl</li><li>Computer Point</li></ul>		C09) DS, PMS or SPDS)

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Event Description: Partial Stator water runback. Tech Spec for SRO.

Time	Position	Applicant's Actions or Behavior		
	ATC	<ul> <li>NOTE</li> <li>CEA SELECTED indicates selected CEA position.</li> <li>GROUP SELECTED indicates average position of selected group.</li> <li>If ≥ 6 inch deviation occurs in any regulating group, then rod motion is inhibited by PMS CEA sequencing program.</li> </ul>		
Steps for CEA insertion		THEN perform the following at 2C03:  1.1 Place Group Select switch to desired group position.  1.2 IF moving Group P CEAs,		
		P 1.6 Position groups as desired using Manual Control lever.  P 1.7 Place Mode Select switch to OFF.  1.8 Verify Pulse Counter and CEAC indications match.		
	SRO	B. Record time Turbine runback complete:		
	ATC	C. WHEN sufficient boron added, THEN secure Emergency Boration.		
		D. WHEN sufficient CEAS inserted, THEN stop CEA insertion.		

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Event Description: Partial Stator water runback. Tech Spec for SRO.

Time	Position	Applicant's Actions or Behavior	
Steps to secure Emergency Boration	ATC	NOTE Reactor Engineering assistance may be required to determine shutdown margin.  7. WHEN boron concentration greater than minimum required for shutdown margin, THEN perform the following:  A. Verify VCT level ≥ 20%.  B. Place VCT Outlet valve (2CV-4873-1) in AUTOMATIC.  C. WHEN 2CV-4873-1 open, THEN perform the following:  1) Verify BOTH BAM pumps secured.  2) Verify the following valves closed:  • 2CV-4920-1 • 2CV-4921-1 • 2CV-4916-2  3) IF 2CV-4950-2 was opened, THEN place handswitch in AUTOMATIC and check that valve closes.	
	SRO	2. Notify Control Board Operators to monitor floating steps.	
	ATC	*4. Reduce Reactor power to match TAVE within 2°F of TREF using 2104.003 Exhibit 3, normal boration.	
	ANY	*5. Check RCS pressure 2025 to 2275 psia.	
	ANY	*6. Verify SDBCS maintaining SG pressure at setpoint.	
		<ul> <li>7. IF Main Turbine has NOT tripped, THEN perform the following:</li> <li>• Re-establish Turbine control on Load Limit Pot.</li> <li>• Refer to TRM 3.3.4, Turbine Overspeed Protection.</li> </ul>	

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Event Description: Partial Stator water runback. Tech Spec for SRO.

Time	Position	Applicant's Actions or Behavior
	SRO	■9. IF ADVs are open, THEN maintain Hotwell level 30-90% by performing one of the following:
		<ul> <li>Make up using 2106.016 Exhibit 1, Hotwell Makeup Operations.</li> </ul>
		• Slug feed Hotwell by locally throttling open 2CV-0634 Inlet (2CT-0634-1) and 2CV-0634 Outlet (2CT-0634-2).
		Examiner note: ADVs should partially open due to this runback.
	SRO	*10. Check RCS TC 542 to 554.7°F using CPC PID 5, 6, 160, or 161.
		Examiner Note: The SRO must enter TS 3.2.6 for the time when the Tc is/was greater than 554.7°F
	SRO	*11. Check ASI within limits as specified in Core Operating Limits Report (COLR).
	ANY	*12. Check CEA positions as follows:
		<ul> <li>A. ALL Regulating Group 6 CEAs above 112.5 inches.</li> <li>B. ALL Regulating Group 1 through 5 CEAs fully withdrawn.</li> <li>C. ALL Group P CEAs above 135 inches.</li> </ul>
	ANY	*13. Check Reactor NOT tripped.
	ВОР	*14. Verify Feedwater Control System restoring SG levels to setpoint.
	ATC	*15. Check PZR level within 5% of setpoint:
	ВОР	17. Check Condenser pressure less than 5.3 inches Hg Abs.
	SRO	■18. Verify ALL required actions have been completed based on power level, refer to 2102.004, Power Operation.
	SRO	20. IF Reactor power change greater than 15% in 1 Hr, THEN notify Chemistry to sample RCS for Iodine within 2 hours to 6 hours, refer to TS 4.4.8, RCS Activity.

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Event Description: Partial Stator water runback. Tech Spec for SRO.

Time	Position	Applicant's Actions or Behavior	
	SRO	21. Perform the following as directed by Operations Management:	
		<ul> <li>Reload Turbine Generator using 2106.009, Turbine Generator Operations.</li> <li>Maintain Plant conditions using 2102.004, Power Operation.</li> <li>Perform Plant Shutdown.</li> </ul>	

Termination Criteria: When reactivity manipulations are satisfied and Turbine load has been re-established on the load limit pot or at the discretion of the lead examiner.

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Event Description: 'B' Reactor Coolant Pump (RCP) trip, Reactor Protection System auto trip failure, Main Steam Safety valve setpoint drifts causing an ESD, and EFW valve 2CV-1025-1 fails to auto close, and 2EFW-1038-2 failed open. 'B' Main feedwater pump trips to allow Steam Generator level to lower for Auto EFW actuation.

Time	Position	Applicant's Actions or Behavior	
Cued by lead examiner	ANY	Report 2K-11 G1 RCP trip alarm, 'B' RCP is tripped but the reactor is not tripped.	
	SRO	Direct Reactor trip.	
Critical Step	ATC	Trip the Reactor using the RPS push buttons	
Trip the reactor using the RPS pushbuttons within 1 min of 'B' RCP trip.		Examiner note: The SRO may discuss Tech Spec 3.3.1.1 for RPS failure to automatically trip the reactor.	
	SRO	Enter and implement EOP 2202.001, Standard Post Trip Actions.	
	SRO	<ol> <li>Notify Control Board Operators to monitor safety functions using Exhibit 7, CBO Reactor Trip Checklist.</li> <li>Open Safety Function Tracking page.</li> </ol>	
Reactivity control safety function	ATC	3. Check Reactivity Control established as follows: A. Reactor power lowering. B. ALL CEAs fully inserted by observing ANY of the following:  1) CEA Rod bottom lights illuminated. 2) CEAC 1 indicates ALL CEAs fully inserted. 3) CEAC 2 indicates ALL CEAs fully inserted.	

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Event Description: 'B' Reactor Coolant Pump (RCP) trip, Reactor Protection System auto trip failure, Main Steam Safety valve setpoint drifts causing an ESD, and EFW valve 2CV-1025-1 fails to auto close, and 2EFW-1038-2 failed open. 'B' Main feedwater pump trips to allow Steam Generator level to lower for Auto EFW actuation.

to lower for a	Auto EFW act	LUGLIOII.
Time	Position	Applicant's Actions or Behavior
Vital Auxiliaries safety function	ВОР	<ul> <li>4. Check Maintenance of Vital Auxiliaries satisfied: A. Check Main Turbine tripped by BOTH of the following:  <ul> <li>ALL Main Stop Valves closed.</li> <li>Generator megawatts indicate zero.</li> </ul> </li> <li>B. Generator Output breakers open.</li> <li>C. Exciter Field breaker open.</li> <li>D. Check the following valves closed:  <ul> <li>MSR 2E-12A Steam Supply From SG A (2CV-0400)</li> <li>MSR 2E-12B Steam Supply From SG B (2CV-0460)</li> </ul> </li> <li>E. At least ONE 6900v AC bus energized</li> <li>F. At least ONE 4160v Non-vital AC bus energized.</li> <li>G. BOTH 4160v Vital AC buses energized.</li> <li>H. BOTH DGs secured.</li> <li>I. At least ONE 125v Vital DC bus energized:  2D01 - SPDS point E2D01 2D02 - SPDS point E2D02</li> </ul>
RCS Inventory Control Safety function	ATC	<ul> <li>5. Check RCS Inventory Control established as follows:</li> <li>A. PZR level: <ul> <li>10 to 80%.</li> <li>Trending to setpoint. (Will not be met due to ESD Event, perform contingency)</li> <li>B. RCS MTS 30°F or greater.</li> </ul> </li> </ul>

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Event Description: 'B' Reactor Coolant Pump (RCP) trip, Reactor Protection System auto trip failure, Main Steam Safety valve setpoint drifts causing an ESD, and EFW valve 2CV-1025-1 fails to auto close, and 2EFW-1038-2 failed open. 'B' Main feedwater pump trips to allow Steam Generator level to lower for Auto EFW actuation.

Time	Position	Applicant's Actions or Behavior	
RCS Inventory Control Safety function	SRO	Direct the following as necessary:  A. Perform as necessary:  1) IF SIAS actuated on PPS inserts, THEN GO TO Step 6.  2) Verify PZR Level Control system restoring level to setpoint.	
RCS Pressure Control Safety function	ВОР	<ul> <li>6. Check RCS Pressure Control: <ul> <li>1800 to 2250 psia.</li> <li>Trending to setpoint. (Not met due to ESD Event #7, perform contingency)</li> <li>Normal PZR Spray and heaters controlling pressure.</li> <li>Valid CNTMT Spray NOT in progress.</li> </ul> </li> </ul>	
RCS Pressure Control Safety function	SRO	<ul> <li>Direct the following actions as necessary:</li> <li>6. Perform as necessary:</li> <li>A. IF RCS pressure lowers to less than 1400 psia,</li></ul>	

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Event Description: 'B' Reactor Coolant Pump (RCP) trip, Reactor Protection System auto trip failure, Main Steam Safety valve setpoint drifts causing an ESD, and EFW valve 2CV-1025-1 fails to auto close, and 2EFW-1038-2 failed open. 'B' Main feedwater pump trips to allow Steam Generator level to lower for Auto EFW actuation.

Time	Position	Applicant's Actions or Behavior	
	ATC	7. Check Core Heat Removal by forced circulation:	
		A. At least ONE RCP running.	
		B. CCW flow aligned to RCPs.	
		C. Loop delta T less than 10°F.	
		D. RCS MTS 30°F or greater.	
Core Heat Removal safety function.		E. Check SW aligned to CCW. (SW will not be aligned due to SIAS and should perform contingency)	
Tunction.		F. IF SIAS or MSIS actuated, THEN maintain SW header pressure greater than 85 psig.	
	SRO	Direct the contingency for step 7. E	
	ВОР	E. IF CCW available, THEN restore SW to CCW, refer to 2202.010 Exhibit 5, CCW/ACW/SW Alignment.	

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Event Description: 'B' Reactor Coolant Pump (RCP) trip, Reactor Protection System auto trip failure, Main Steam Safety valve setpoint drifts causing an ESD, and EFW valve 2CV-1025-1 fails to auto close, and 2EFW-1038-2 failed open. 'B' Main feedwater pump trips to allow Steam Generator level to lower for Auto EFW actuation.

Time	Position	Applicant's Actions or Behavior				
	ВОР	IF SW suction NOT aligned to lake, THEN RETURN TO procedure in effect.				
		2. IF SW NOT aligned to CCW AND CCW available, THEN perform the following:				
		A. IF RCP seal temperatures less than 180°F, THEN restore SW to CCW by performing the following:				
		Override and open at least ONE SW to CCW/ACW Return valve:				
		• 2CV-1543-1				
		• 2CV-1542-2				
Steps to restore SW		<u>CAUTION</u>				
to CCW		Supplying ACW flow and CCW cooling from a single SW pump may				
		result in low SW header pressure.				
		2). Override and throttle open at least ONE SW to CCW /Main Chillers Supply valve:				
		• 2CV-1530-1				
		• 2CV-1531-2				
		3). Maintain SW header pressure greater than 85 psig.				

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Event Description: 'B' Reactor Coolant Pump (RCP) trip, Reactor Protection System auto trip failure, Main Steam Safety valve setpoint drifts causing an ESD, and EFW valve 2CV-1025-1 fails to auto close, and 2EFW-1038-2 failed open. 'B' Main feedwater pump trips to allow Steam Generator level to lower for Auto EFW actuation.

Time	Position	Applicant's Actions or Behavior					
	BOP/ATC	8. Check RCS Heat Removal:					
		A. Check SG available by BOTH of the following:					
		<ul> <li>At least ONE SG level 10 to 90%.</li> <li>FW maintaining SG level (Not Met, Perform contingency)</li> </ul>					
RCS Heat		B. Check MFW in RTO (Not Met, Perform contingency)					
Removal Safety Function		C. Check Feedwater line intact by the following:					
		SG level stable or rising.					
		<ul> <li>NO unexplained step changes or erratic FW flow.</li> </ul>					
		<ul> <li>NO unexplained step changes or erratic Condensate flow.</li> </ul>					
		D. Check SG pressure 950 to 1050 psia. (Not Met, perform Contingency)					
	ANY	Perform step 8 contingency actions that are applicable:					
		A. Perform the following:					
		1) <u>IF</u> SG level lowering,					
		THEN verify EFAS actuated. (To the 'B' S/G only)					
	ANY	B. Verify EITHER of the following:					
		BOTH MFW pumps tripped.					
		SG levels controlling at setpoint					
	ANY	D. Perform as necessary:					
		1) <u>IF SG pressure 751 psia or less, THEN</u> perform the following:					
		a) Verify MSIS actuated on PPS inserts.					
	ВОР	b) Verify feed secured to the affected SG.					
		Examiner note: The BOP must either close 2CV-1025-1 or secure 2P-7B EFW pump.					
		Examiner note: The SRO may discuss Tech Spec 3.7.2.1 for EFW trains due valve failure.					

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Event Description: 'B' Reactor Coolant Pump (RCP) trip, Reactor Protection System auto trip failure, Main Steam Safety valve setpoint drifts causing an ESD, and EFW valve 2CV-1025-1 fails to auto close, and 2EFW-1038-2 failed open. 'B' Main feedwater pump trips to allow Steam Generator level to lower for Auto EFW actuation.

	T						
Time	Position	Applicant's Actions or Behavior					
	ALL	b) Maintain RCS post-cooldown conditions as follows:					
		<ul> <li>Maintain RCS pressure within P-T limits with P2 heaters and spray using 2202.010 Attachment 48 RCS Pressure Control.</li> </ul>					
	Maintain RCS temperature by steaming using Upstream ADV or Upstream ADV MOV.						
		c) GO TO Step 9.					
		Examiner note: The SRO may discuss Tech Spec 3.7.1.1 for Main					
		Steam Safety valves.					
		1.0 IF BOTH MSIV's closed,					
		THEN GO TO step 5.0.					
		Examiner note: MSIVs are closed.					
		5.0 Perform the following to determine availability of SDBCS valves:					
Steam Dump Exhibit 3		5.1 IF the following conditions satisfied, THEN Upstream ADV are available:					
2		Instrument air available					
		• Emergency OFF (2K02-A14) clear					
		Power available to selected controllers/valves					
		Examiner note: MSIVs are closed.					

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Event Description: 'B' Reactor Coolant Pump (RCP) trip, Reactor Protection System auto trip failure, Main Steam Safety valve setpoint drifts causing an ESD, and EFW valve 2CV-1025-1 fails to auto close, and 2EFW-1038-2 failed open. 'B' Main feedwater pump trips to allow Steam Generator level to lower for Auto EFW actuation.

Time	Position	Applicant's Actions or Behavior			
Time  Steam Dump Exhibit 3	Position	Applicant's Actions or Behavior  6.0 IF operation of Upstream Atmospheric Dump valve from the Control Room desired, THEN perform the following:  6.1 Verify selected HIC in MANUAL with ZERO output demand:  • 2HIC-1001 Upstream Atmospheric Dump valve  • 2HIC-1051 Upstream Atmospheric Dump valve  6.2 Place selected valve(s) permissive handswitch in MANUAL:  • 2CV-1001 Upstream Atmospheric Dump valve (2HS-1001)  • 2CV-1051 Upstream Atmospheric Dump valve (2HS-1051)  6.3 IF MSIS actuated, THEN override actuation for selected MOV isolation:  • 2CV-1002 ADV Upstream Isolation valve  • 2CV-1052 ADV Upstream Isolation valve  * 6.4 Throttle open selected MOV as desired:  • 2CV-1052 ADV Upstream Isolation valve  • 2CV-1052 ADV Upstream Isolation valve			

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Event Description: 'B' Reactor Coolant Pump (RCP) trip, Reactor Protection System auto trip failure, Main Steam Safety valve setpoint drifts causing an ESD, and EFW valve 2CV-1025-1 fails to auto close, and 2EFW-1038-2 failed open. 'B' Main feedwater pump trips to allow Steam Generator level to lower for Auto EFW actuation.

Time	Position	Applicant's Actions or Behavior				
	ANY	9. Check CNTMT parameters:				
		A. Temperature and Pressure:				
		• Temperature less than 140°F.				
		• Pressure less than 16 psia.				
		B. Check CNTMT Spray pumps secured.				
		C. NO CNTMT radiation alarms or unexplained rise in activity:				
		<ul> <li>1) CAMS alarms:         <ul> <li>"CNTMT PART/GAS RAD HI/LO" annunciator</li> <li>(2K10-B6) clear.</li> </ul> </li> </ul>				
Containment Safety Function		<ul> <li>2) RCS leakage alarms:</li> <li>"AREA RADIATION HI/LO" annunciator (2K11-B10) clear.</li> <li>"PROC LIQUID RADIATION HI/LO" annunciator (2K11-C10) clear.</li> </ul>				
		<ul> <li>3) Check the following radiation monitors trend stable:</li> <li>• CNTMT Area</li> <li>• CAMS</li> <li>• Process Liquid</li> </ul>				
		D. NO secondary system radiation alarms or unexplained rise in activity:				
		1) "SEC SYS RADIATION HI" annunciator (2K11-A10) clear.				
		<ul> <li>Secondary Systems Radiation monitors trend stable:</li> <li>Main Steam lines</li> <li>SG Sample</li> <li>Condenser Off Gas</li> </ul>				

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Event Description: 'B' Reactor Coolant Pump (RCP) trip, Reactor Protection System auto trip failure, Main Steam Safety valve setpoint drifts causing an ESD, and EFW valve 2CV-1025-1 fails to auto close, and 2EFW-1038-2 failed open. 'B' Main feedwater pump trips to allow Steam Generator level to lower for Auto EFW actuation.

Time	Position	Applicant's Actions or Behavior			
11110	1	^^			
	SRO	10. Notify STA to report to control room.			
		11. Direct NLOs to perform 2202.010 Attachment 47, Field Operator Post Trip Actions.			
		12. Verify Reactor trip announced on Plant page.			
		13. Notify SM to refer to Technical Specifications and 1903.010, Emergency Action Level Classification.			
Cue: If cont	acted as the S	STA to report to the control room, acknowledge the request.			
Cue: If cont	acted as a NI	O to perform Attachment 47 Field Operator Post Trip Actions,			
acknowledge	e request.				
	SRO	14. Direct control board operators to acknowledge ALL annunciators and announce ALL critical alarms.			
	SRO	15. Check ALL safety function acceptance criteria satisfied. (All safety functions are not satisfied, perform contingency)			
		15. <u>IF ANY safety function acceptance criteria NOT satisfied, THEN perform the following:</u>			
		A. Notify control room staff of safety functions <u>NOT</u> satisfied.			
		B. GO TO Exhibit 8, Diagnostic Actions.			
	SRO	Diagnose Excess Steam Demand EOP 2202.005			
	SRO	Enter and implement Excess Steam Demand EOP 2202.005			

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Event Description: 'B' Reactor Coolant Pump (RCP) trip, Reactor Protection System auto trip failure, Main Steam Safety valve setpoint drifts causing an ESD, and EFW valve 2CV-1025-1 fails to auto close, and 2EFW-1038-2 failed open. 'B' Main feedwater pump trips to allow Steam Generator level to lower for Auto EFW actuation.

Time	Position	Applicant's Actions or Behavior
	SRO	*1. Confirm diagnosis of Excess Steam Demand as follows:
		A. Check SFSC acceptance criteria satisfied every 15 minutes.
		B. IF CCW in service to provide SG Sample Cooler cooling, THEN perform the following:
		<ol> <li>IF SG "A" has indicated water level, THEN verify the following SG 'A' Sample Valves open:</li> </ol>
		<ul><li> 2CV-5850</li><li> 2CV-5852-2</li></ul>
		2) IF SG "B" has indicated water level, THEN verify the following SG 'B' Sample Valves open:
		<ul><li>2CV-5858</li><li>2CV-5859-2</li></ul>
		C. Notify Chemistry to sample available SGs for activity.
		Chemistry, then report you will sample both S/G for activity and site dose releases.
	SRO	2. Notify SM to refer to Technical Specifications and 1903.010, Emergency Action Level Classification.
	SRO	3. Open Placekeeping Page.
	SRO	4. Notify Control Board Operators to monitor floating steps.
	ANY	5. Verify MSIS actuated on PPS inserts.
	ANY	*6. IF SIAS setpoints exceeded by EITHER of the following:
		<ul> <li>RCS pressure 1650 psia or less.</li> <li>CNTMT pressure 18.3 psia or greater.</li> </ul>
		THEN verify SIAS and CCAS actuated on PPS inserts.
	ANY	*7. Verify actuated ESFAS components using 2202.010, Exhibit 9, ESFAS Actuation.

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Event Description: 'B' Reactor Coolant Pump (RCP) trip, Reactor Protection System auto trip failure, Main Steam Safety valve setpoint drifts causing an ESD, and EFW valve 2CV-1025-1 fails to auto close, and 2EFW-1038-2 failed open. 'B' Main feedwater pump trips to allow Steam Generator level to lower for Auto EFW actuation.

Time	Position	Applicant's Actions or Behavior				
Exhibit 9 ESFAS actuation.	BOP	*1.0 IF any abnormalities noted for affected ESFAS actuation, THEN notify CRS.  2.0 IF SIAS, THEN verify the following:  Red Train RWT Outlet (2CV-5630-1) open.  Red Train HPSI Pump in service with proper discharge pressure and flow.  Red Train Service Water Pump in service with proper discharge pressure.  Red Train LPSI Pump (2P60A) in service with proper discharge pressure and flow.  Red Train LPSI Injection MOVs open.  Green Train RWT Outlet (2CV-5631-2) open.  Green Train HPSI Pump in service with proper discharge pressure and flow.  Green Train HPSI Injection MOVs open.  Green Train Service Water Pump in service with proper discharge pressure.  Green Train LPSI Pump (2P60B) in service with proper discharge pressure and flow.  Green Train LPSI Injection MOVs open.  Available Charging Pumps in service with proper discharge pressure and flow.  Service Water Outlet Valves open for #1 and #2 EDGs.  J. IF CCAS, THEN verify the following:  Red Train CNTMT Coolers in service.  Service Water aligned to Red Train CNTMT Coolers.  Red Train Bypass Dampers open.  Green Train Bypass Dampers open.  Green Train Bypass Dampers open.				

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Event Description: 'B' Reactor Coolant Pump (RCP) trip, Reactor Protection System auto trip failure, Main Steam Safety valve setpoint drifts causing an ESD, and EFW valve 2CV-1025-1 fails to auto close, and 2EFW-1038-2 failed open. 'B' Main feedwater pump trips to allow Steam Generator level to lower for Auto EFW actuation.

to lower for A	Auto EFW act	uation.				
Time	Position	Applicant's Actions or Behavior				
	ВОР	4.0 IF CSAS, THEN verify the following:				
Exhibit 9 ESFAS actuation.	ВОР	Red Train CSS Pump (2P35A) in service with proper discharge pressure and flow.  Red Train CSS Header Isolation (2CV-5612-1) open.  Green Train CSS Pump (2P35B) in service with proper discharge pressure and flow.  Green Train CSS Header Isolation (2CV-5613-2) open.  Main Feedwater Block valves closed.  MSIVs closed.  Main Feed pumps tripped.  Condensate pumps secured.  Heater Drain pumps secured.  St.O IF MSIS, THEN verify the following:  MSIVs closed.  Main Feedwater Block valves closed.  Red Train Service Water Pump in service with proper discharge pressure.  Green Train Service Water Pump in service with proper discharge pressure.  Main Feed pumps tripped.  Condensate pumps secured.  Heater Drain pumps secured.				
	SRO	Should elect to pull up the floating step for Maintain Post cooldown				
		conditions. Step 19 if this was not directed in SPTAs.				
Maintain	ATC/BOP	■19. Maintain RCS post-cooldown conditions as follows:				
post- cooldown conditions		A. Maintain RCS temperature by steaming intact SG using EITHER of the following:				
CRITICAL TASK		Upstream ADV.				
		Upstream ADV Isolation MOV.				
		B. Control feedwater flow to intact SG using 2202.010 Attachment 46, Establishing EFW Flow.				
		C. Maintain RCS pressure within P-T limits using 2202.010 Attachment 48, RCS Pressure Control.				
		Critical task: Use attachment 48 to maintain RCS pressure within the Pressure-Temperature limits of 200°F and 30°F Margin to Saturation.				

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Event Description: 'B' Reactor Coolant Pump (RCP) trip, Reactor Protection System auto trip failure, Main Steam Safety valve setpoint drifts causing an ESD, and EFW valve 2CV-1025-1 fails to auto close, and 2EFW-1038-2 failed open. 'B' Main feedwater pump trips to allow Steam Generator level to lower for Auto EFW actuation.

Time	Position	Applicant's Actions or Behavior			
	ANY	■8. Restore ESF/Non-ESF systems post-SIAS as follows:			
		A. Verify at least ONE SW pump running on EACH loop.			
	ANY	B. Verify DG SW Outlet valves open.			
	ANY	C. Verify SW pump suction aligned to Lake.			
	ANY	D. Check 4160v Non-vital bus 2A1 or 2A2 energized from offsite power.			
	ANY	E. Check 4160v Vital buses 2A3 AND 2A4 energized from offsite power.			
	BOP	F. Start SW pumps as needed to maintain SW header pressure.			
	ANY	G. Check SW to CCW restored.			
	ANY	H. Check ACW restored. (Not met, perform contingency)			
	ВОР	H. Restore SW to ACW, refer to 2202.010 Exhibit 5, CCW/ACW/SW Alignment.			
	BOP	I. Maintain SW header pressure greater than 85 psig.			
	ANY	*9. Verify Safety Injection flow to RCS as follows:			
		<ul> <li>A. Check total HPSI flow acceptable using 2202.010 Exhibit 2, HPSI Flow Curve.</li> <li>B. Check total LPSI flow acceptable using 2202.010 Exhibit 3, LPSI Flow Curve.</li> </ul>			
	ANY	■10. Check CCW flow aligned to RCPs.			
	ANY	■11. Check RCS pressure greater than 1400 psia.			
	ANY	■12. WHEN RCS TC less than 510°F, THEN reduce number of running RCPs as follows:			
		A. Verify maximum of ONE RCP running in EACH loop.			
		B. IF RCP 2P32A or 2P32B stopped, THEN verify associated PZR Spray valve in MANUAL and closed.			
	ANY	13. Determine most affected SG by comparing the following:			
		<ul><li>SG steam flows.</li><li>SG pressures.</li><li>SG levels.</li><li>RCS TC.</li></ul>			

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Event Description: 'B' Reactor Coolant Pump (RCP) trip, Reactor Protection System auto trip failure, Main Steam Safety valve setpoint drifts causing an ESD, and EFW valve 2CV-1025-1 fails to auto close, and 2EFW-1038-2 failed open. 'B' Main feedwater pump trips to allow Steam Generator level to lower for Auto EFW actuation.

Time	Position	Applicant's Actions or Behavior				
	ANY	<ul> <li>Isolate most affected SG using 2202.010 Attachment 10, SG Isolation.</li> <li>IF affected SG still pressurized, THEN check MSSVs for affected SG closed by locally checking individual valve tail pipes for leakage.</li> </ul>				

Cue: If contacted as the AO, then report after 5 min that the local portions of Attachment 10 for 'A' S/G are complete.

Termination criteria: Post Cooldown conditions maintained or at the discretion of the lead examiner.

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Op-Test No.: 2012-1		Scenario No.: 2 Event No.: 9			9			
Event Description: 'A' RCP spray valve failed closed.								
Time	Position	Applicant's Actions or Behavior						
	ATC	Recognize that RCS pressure.	Recognize that 'A' RCP spray valve is not opening as required to control RCS pressure.					
PZR Malfund	ctions AOP if	are is recognized during the stator water runback the SRO should refer to needed to open 2CV-4651. Otherwise the guidance is located in Standard uring Post Cooldown control.						
	ATC/SRO	Determine that valve failure to		y is required du	te to 2CV-4651 nor	rmal Spray		
	ATC	THEN p	erform the foll	LIARY Spray, owing: E Charging pun				
	ATC	<ul> <li>B. Close Regen HX to RCP B/C valves:</li> <li>2CV-4827-2</li> <li>2CV-4831-2</li> </ul>						
	ATC	C. Verify PZR Spray (2CV-4651/2CV-4652) or PZR Spray Isolation valves (2HS-4655/2HS-4653) closed.						
	ATC	D. Thr	ottle Aux Spra	y valve (2CV-4	1824-2) as necessar	ry.		
	ATC	G. IF Regen HX to RCS (2TI-4825) AND PZR water phase (2TI-4627) differential temperature greater than 200°F, THEN complete Table 1 of this attachment.						
	ATC			TABLE 1				
		TIME TEMPERATURE						
		SPRAY SPRAY (PZR 2TIS-4607 VALVE VALVE WATER 2TIS-4608 OPENED CLOSED PHASE) 2TI-4825 2TI-4627				DIFFER ENCE		

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Op-Test No.: 2012-1		Scenario No.: 2 Event No.: 9	
Event Descri	ption: 'A' RC	P spray valve failed closed.	
Time	Position	Applicant's Actions or Behavior	
	ВОР	5.0. IF desired to use PZR Heaters, THEN perform the following:	
		A. Cycle available PZR Backup heaters as necessary to control RCS pressure.	
		B. Cycle available PZR Proportional heaters as necessary to control RCS pressure.	
	Termination criteria: When RCS pressure is being controlled with in the PT limits or at examiner's discretion.		

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Facility: ANO-2	Scenario No.: 3 (New)	Op-Test No.: 2012-1
Examiners:	Operators:	

#### **Initial Conditions:**

~4% MOL; All Engineered Safety Features systems are in standby. #3 and #4 MTG control valve steam lead valves failed open. Voltage regulator auto tracking circuit is disabled. RED Train Maintenance Week.

### Turnover:

~4%. 260 EFPD. EOOS indicates 'Minimal Risk'. #3 and #4 MTG control valve steam lead valves failed open. Voltage regulator auto tracking circuit is disabled. RED Train Maintenance Week. 2102.004 power operations section 8 complete up to step 8.8. Steps 8.3 and 8.4 continuous actions. Test Team will roll the turbine and sync to the grid at ~10% power. Steam Bypass valve 2CV-0303 is in auto and 2CV-0302 is in manual balancing condenser heat loads.

Evolution scheduled: Commence Power Escalation @ <15%/Hr.

Event No.	Malf. No.	Event Type*	Event Description
1		R (ATC) N (BOP) N (SRO)	Commence Power escalation to 10% power for turbine sync.
2	DI_HS_4966_1 DI_HS_4966_2 DO_HS_4966_G DO_HS_4966_R	C (ATC) C (SRO)	2P-109B reactor makeup water pump trips
3	NIALINEPWR		'A' Nuclear Instrument (NI) fails high. Tech Spec for SRO.
4	XCV2LT4861	I (ATC) I (SRO)	Volume Control Tank (VCT) level transmitter fails high.
5	DI_C40_S72B ESFCIAS1 K04-C01 K07-C01	C(BOP) C(SRO)	Inadvertent Containment Isolation Actuation Signal (CIAS) on the Green Train. Tech Spec for SRO.
6	RCP2P32DUPP RCP2P32DMID RCP2P32DLOW RCLOCATCD	M (ALL)	'D' RCP seals fail and a 180 gpm RCS leak starts.
7	CEA15STUCK CEA31STUCK	C (ATC) C (SRO)	Control Element Assembly's (CEA) 15 and 31 fail to fully insert during the reactor trip.
8	SIS2P89AX	C (BOP) C (SRO)	2P-89A High Pressure Safety Injection (HPSI) pump has degraded performance.

Total malfunctions. = 6, Malfunctions after EOP entry = 2, Abnormal events = 3, Major transient = 1, EOPs with substantive actions = 1, EOP Contingencies = 0, Critical tasks = 3.

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# Scenario #3 Objectives

- 1) Evaluate individual ability to perform a power escalation.
- 2) Evaluate individual response to a trip of a reactor makeup water pump.
- 3) Evaluate individual response to a failure of Nuclear Instrumentation Channel.
- 4) Evaluate individual response to a failure of a Chemical Volume Control System transmitter failure.
- 5) Evaluate individual response to an Inadvertent Containment Isolation signal.
- 6) Evaluate individual response to Reactor Coolant Pump seal failures and a Reactor Coolant System leak.
- 7) Evaluate individual response to a stuck out CEAs after a Reactor Trip.
- 8) Evaluate individual ability to monitor operation of Engineered Safety Features equipment and respond to a degraded High Pressure Safety Injection pump.
- 9) Evaluate individual ability to mitigate a Loss of Coolant Accident.

#### **SCENARIO #3 NARRATIVE**

Simulator session begins with the plant at ~4% power.

When the crew has completed their control room walk down and brief, they will commence a power escalation using CEAs and dilution of the RCS.

When the required power change has been completed and when cued by lead examiner, 2P-109B reactor make up water pump will trip. This will prevent the crew being able to dilute and raise power. The ATC should coordinate with a NLO to start the other reactor make up water pump 2P-109A.

When 2P-109A reactor makeup water pump has been started and when cued by lead examiner, 'A' NI will fail high. The SRO will enter and implement NI malfunction AOP, 2203.026. The SRO should enter Tech Spec 3.3.1.1 action 2 for RPS. The BOP will be required to bypass the Plant Protection System channel 1 for linear power (point 1), log power (point 2), LPD (point 3), and DNBR (point 4).

When the appropriate Tech spec has been entered and the BOP has bypassed points 1 through 4 and cued by lead examiner; VCT level transmitter 2LT-4861 will fail high. The SRO/ATC should refer/use the Annunciator Corrective action for VCT level low (due to VCT level actually lowering) and the ATC will place the letdown divert valve to the VCT position stopping the RCS mass loss.

After the Crew has place the divert valve to the VCT position and cued by lead examiner, an Inadvertent Containment isolation will occur on the green train causing the CCW to RCP valve and the Main Chilled water to containment valves to close. The SRO will enter Inadvertent CIAS AOP, OP 2203.039. The crew should restore Component Cooling Water (CCW) to RCPs. The SRO will enter Tech Spec 3.6.3.1. for the overridden Containment Isolation valve. The ATC will cycle charging pump to control pressurizer level. The crew should minimize CEA movement due to the loss of cooling. The BOP will start all containment coolers and align Service Water to maintain Containment temperature and pressure in the required band. The SRO should call for maintenance assistance to correct inadvertent green train Containment isolation.

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## **SCENARIO #3 NARRATIVE (continued)**

When the actions of inadvertent CIAS have been completed or at the lead examiners cue the 'D' RCP seals will fail and a RCS leak will start. The SRO should enter the RCP emergencies AOP, 2203.025 due to the first failed seal. The SRO will contact operations management and continue plant operation based on their recommendation. When the other two seals fail the Crew should trip the reactor and secure 'D' RCP. The crew may also secure 'A' or 'B' RCP to balance RCS flows. The SRO may actuate SIAS and CCAS due to the RCS leak.

The Crew will implement Standard Post Trip Actions (SPTA) EOP, 2202.001. When the reactor trips two CEAs will fail to fully insert which requires emergency boration. The ATC will commence emergency boration. The Crew may lower Steam Dump master controller setpoint during SPTAs to aid in maintaining margin to saturation.

The SRO will diagnose either an Excess RCS leakage and enter Excess RCS leakage AOP, 2203.016, or if SIAS is actuated diagnose Loss of Coolant Accident (LOCA). If Excess RCS is diagnosed the SRO should implement the floating step for leakage greater than 44 gpm then actuate SIAS and CCAS and re-diagnose LOCA. The crew should determine that 2P-89A HPSI pump has degraded discharge pressure and start 2P-89C. The ATC will commence cool down of the RCS and control RCS pressure to restore pressurizer level. The BOP will override Service Water to Component Cooling Water and Auxiliary Cooling Water.

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## **Simulator Instructions for Scenario 3**

Reset simulator to MOL ~4 % power IC stead state.

Ensure that AACG is secured and annunciators clear.

Place MINIMAL RISK, Green Train Protected and RED Train Maintenance Week signs on 2C11.

Volume Control tank level ~60%.

Ensure the SDBCS master is in A/L with a setpoint of 1000 psi

Ensure both main feedwater pumps recirc valves are throttled to ~1 gal/rpm

T1, T2, T3, T4, T5, T6 set to false.

T6 = Reactor Trip

$\frac{16 = \text{Read}}{\text{Event}}$	Malf. No.	Value/	Event
No.	Man. 1vo.	Ramp Time	Description
1			Commence Power escalation to 10% power for turbine sync.
2	DI_HS_4966_1	True	2P-109B reactor makeup water pump trips
	DI_HS_4966_2	False	
	DO_HS_4966_G	Off	
	DO_HS_4966_R	Off	
	Trigger = T1		
3	NIALINEPWR Trigger = T2	200%	'A' Nuclear Instrument (NI) fails high. Tech Spec for SRO.
4	XCV2LT4861	100%	Volume Control Tank (VCT) level transmitter fails high.
	Trigger = T3		
5	DI_C40_S72B	True	Inadvertent Containment Isolation Actuation Signal (CIAS) on
	K04-C01	ON	the Green Train. Tech Spec for SRO.
	K07-C01	ON	
	Trigger = T4		
	ESFCIAS1		
6	RCP2P32DUPP	100%	'D' RCP seals fail and a 150 gpm RCS leak starts.
	RCP2P32DMID	100%/ delay = 5 min.	
	RCP2P32DLOW	100%/ delay 5 min.	
	RCSLOCATCD	180	
	Trigger = T5	gpm/ delay 5 min.	
7	CEA15STUCK	0%	Control Element Assembly's (CEA) 15 and 31 fail to fully
	CEA31STUCK	0%	insert during the reactor trip.
	Trigger = T6		
8	SIS2P89AX	35%	2P-89A High Pressure Safety Injection (HPSI) pump has degraded performance.

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		Simulator Operator CUEs
Cued by lead examiner	Trigger T1	2P-109B reactor makeup water pump trips
	cted as NLO to cl	ose 2CVC-61A then wait one minute and report 2CVC-61A is closed.
Cue: If conta	cted as NLO to op	pen 2CVC-61A then wait one minute and report 2CVC-61A is open. ackseat 2CVC-61A then report 2CVC-61A is back seated.
		WM, then report that the planners will start planning a work
	epair the reactor r	
Cued by	Trigger T2	'A' Nuclear Instrument (NI) fails high. Tech Spec for SRO.
lead examiner		
	ontacted as the W	WMM, then report that I&C will start planning a work package to
	led nuclear instru	
Cued by	Trigger T3	Volume Control Tank (VCT) level transmitter fails high.
lead		
examiner		
Cue: If conta	cted as NLO to as	ssess 2LT-4861, then report everything looks normal at the
transmitter.		
Cue: When c	ontacted as the W	WM, then report I&C will start planning a work package to repair
the failed inst	trument.	
Cued by	Trigger T4	Inadvertent Containment Isolation Actuation Signal (CIAS) on the
lead		Green Train. Tech Spec for SRO.
examiner		
	ontacted as the W	WM, then report that I&C will start investigating the inadvertent
CIAS		
Cue: When o	contacted as the V	WWM, then report I&C will monitor CEDM coil temperatures.
Cued by	Trigger T5	'D' RCP seals fail and a 150 gpm RCS leak starts.
lead		
examiner	т. ъ	C . 1El
	Trigger = Rx	Control Element Assembly's (CEA) 15 and 31 fail to fully insert
	trip.	during the reactor trip.
		2P-89A High Pressure Safety Injection (HPSI) pump has degraded
C Te 4	4 1 41 11/00	performance.
		, then report after 2 min that 2P-89A vibrations are high, casing is an abnormal noise.

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Event Description: Plant power escalation from ~ 4%.

Time	Position	Applicant's Actions or Behavior	
T=0	Examiner note: Crew will conduct Reactivity Management Brief per COPD-030 prior to reaching to simulator and may elect to conduct a short re-focus brief in the simulator.		
	SRO	Direct applicable actions of OP, 2102.004 Power Operations.	

### Procedure Note:

- Steps may be performed in any order as plant conditions allow.
- Plant parameters may be monitored using Exhibit 2, Various Parameters Vs Reactor Power.
- Boron adjustment data for power change may be obtained from Reactivity curves located in Plant Data book.
- Applicable steps in this section would be performed following removal of Main Turbine Generator from grid with Reactor maintained critical.

SRO/ATC	* 8.3	During power escalation, use Attachment A of this procedure for ASI control.
SRO/ATC	* 8.4	IF any Group 6 or P CEA RSPT inoperable due to spiking, T-Mod, etc., THEN refer to CPC/CEAC Operations (2105.001) to perform the following as required:
		<ul> <li>IF Group 6 or P must be inserted below 138.58 inches AND all inoperable RSPTs in group input to same CEAC,THEN place affected CEAC in INOP in all operable CPCs.</li> </ul>
		• IF Group 6 or P must be inserted below 138.58 inches AND inoperable RSPTs in group input to BOTH CEACs,THEN perform the following:
		<ul> <li>Place BOTH CEACs in INOP in ALL operable CPCs</li> <li>Restrict usage to Group 6 in accordance with TS 3.1.3.6.</li> </ul>
		• IF Group 6 or P insertion required due to a transient condition before CEACs can be placed in INOP, THEN declare affected CEACs inoperable when group inserted below 138.58 inches.
ATC	8.8	Commence raising power by performing the following as necessary:
		• Dilution using Chemical Addition (2104.003).
		• CEA withdrawal using CEDMCS Control System Operation (2105.009).
	SRO/ATC	SRO/ATC *8.4

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Op-Test No.: 2012-1 Scenario No.: 3 Event No.: 1 Event Description: Plant power escalation from ~ 4%. 2105.009 Procedure note: CEA SELECTED indicates selected CEA position. GROUP SELECTED indicates average position of selected group. If > 6 inch deviation occurs in any regulating group, then rod motion is inhibited by PMS CEA sequencing 1.0 IF moving CEAs in group, THEN perform the following at 2C03: 1.1 Place Group Select switch to desired group position. 1.2 IF moving Group P CEAs, THEN place P Group Select switch to P. Ensure Individual CEA Selection switches aligned to 1.3 CEA in group selected to move. Place Mode Select switch to MANUAL GROUP (MG) or MANUAL SEQUENTIAL (MS). Observe CEAC and Pulse Counter CEA position indications to verify CEA motion and alignment as CEAs are moved. 1.6 Position groups as desired using Manual Control lever. 1.7 Place Mode Select switch to OFF. 1.8 Verify Pulse Counter and CEAC indications match. 2.0 IF moving CEA individually, THEN perform the following: Verify Group Select switch selected to group containing individual CEA. 2.2 Place Individual CEA Selection switches to desired CEA. 2.3 Place Mode Select switch to MANUAL INDIVIDUAL (MI). \*2.4 Observe CEAC and Pulse Counter CEA position indications to verify CEA motion and alignment as CEAs are R moved. A P 2.5 Position CEA as desired using Manual Control lever. 2.6 WHEN individual CEA movement complete, THEN place Mode Select switch to OFF. 2.7 Verify Pulse Counter and CEAC indications match. Procedure Caution: The following section has been determined to have a Reactivity Addition

Potential (RAP) and this activity is classified as a Risk Level R3.

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Op-Test No.: 2012-	-1	Scenario No.: 3	Event No.: 1
Event Description:	Plant power esc	calation from ~ 4%.	
	OP-21	04.003, Chemical Addition Exhib	oit 2 for dilution.
	1.0	Verify EITHER Reactor Makeu running.	p pump (2P-109A OR 2P-109B)
	2.0	Verify Mode Select switch (2HS	S-4928) in DILUTE.
	3.0	Verify Reactor Makeup Water Fas follows:	Flow controller (2FIC-4927) set up
		• In MANUAL OR AUTO.	
		• Demand set to less than Char	ging flow.
	4.0	Verify VCT Makeup Isol valve	(2CV-4941-2) open.
	5.0	Operate Reactor Makeup Water (2FQIS-4927) as follows:	Flow Batch controller
		5.1 Depress Red pushbuttor	n.
		5.2 Verify 2FQIS-4927 set	for desired quantity.
		5.3 Verify 2FIC-4927 indic	ates desired flow rate.
	* 6.0	<ul> <li>Monitor the following paramete</li> <li>RCS Tave</li> <li>Axial Shape Index</li> <li>Reactor power</li> </ul>	ers:
	7.0	IF desired to terminate dilution, THEN reset 2FQIS-4927 to zero	0.
	8.0	<ul> <li>WHEN 2FQIS-4927 at zero,</li> <li>THEN verify the following:</li> <li>RMW Flow Control valve</li> <li>No flow indicated on 2FIC</li> </ul>	
	9.0	Repeat steps 5.0 through 8.0 as	required.
	10.0	Close 2CV-4941-2.	
	11.0	Verify 2FQIS-4927 Batch Volume.	me placard updated to current

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Op-Test No.: 2012-1	Scenario No.: 3	Event No.: 1
Event Description: Plant pow	er escalation from ~ 4%.	
ANY *	8.9 Monitor CPC Raw Powers using F	PQASI screen on PMS.
BOP *	TREF).	
SRO 8	<ul> <li>WHEN Reactor power greater that THEN perform the following: (N/.</li> <li>Verify PMS Mode correct or (PMA).</li> <li>Record entry into Mode 1 in</li> <li>Make open item Station Log to be within its proper controt the next 24 hours</li></ul>	A if Mode 2 was not entered) In Plant Mode Assignment  Station log. In entry; "RCS Lithium required of band within  In plant Mode Assignment  Station log. In plant Mode Assig
ANY 8	<ul> <li>Verify Emergency Feedwater NO</li> <li>2P-7A</li> <li>2P-7B</li> </ul>	T in service:
ANY 8	.16 Verify Auxiliary Feedwater (2P-7	5) NOT in service.
ANY 8	.17 Prior to exceeding 10% power, loc from S/U & Blowdown DI (2EFW • Verify 2EFW-0706 closed us	V-0706).

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Op-Test No.: 2012-1		Scenario No.: 3	Event No.: 1
Event Description: Plan	power escalation	on from ~ 4%.	
		form the following prior to e a average power on any CPC	•
	8.1	8.1 IF PQASI (PID 266) < ± THEN power escalation	± 0.4 on ALL operable CPCs, may continue.
	8.1		± 0.4 on ANY operable CPC, rected by Shift Manager g.
	*8.		greater than + 0.44 OR less than - CPC, THEN DO NOT exceed
Termination criteria:	When reactivity examiner.	y manipulations are satisfie	ed or at the discretion of the lead

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Event Description: 2P-109B reactor makeup water pump trips

Time	Position	Applicant's Actions or Behavior
Cued by lead examiner	ANY	Announce annunciators:  2K12-J4 RMW PUMP TO BLENDER FLOW HI/LO  Examiner note: 2K12-J4 may not annunciate if the crew is not currently diluting the RCS.
	ANY	Crew will recognize that 2P-109B has tripped.
	ATC	2.1 Check flow on 2FIC-4927.
2K12-J4 ACA steps	ANY	<ul> <li>2.3 IF no flow indicated in MANUAL, THEN perform the following:</li> <li>2.3.1 Verify Reactor Makeup Water pump (2P-109A or 2P-109B) running using Chemical Addition (2104.003).</li> </ul>
	ATC	NOTE  2P-109 pumps are normally run in manual with one pump handswitch in START and the other in STOP. 2P-109 Select switch (2HS-4967) position is only applicable if the selected pump handswitch in is AUTO. If in AUTO the pump would be started by the Makeup Mode select switch (2HS-4928)  13.1 IF BOTH Reactor Makeup Water pumps (2P-109A AND 2P-109B) secured,
	ANY	THEN perform the following to start selected pump:  Direct NLO to close valve locally.  13.1.1 Close Discharge Stop Check for pump to be started:
	acted as NLC	• 2P-109A (2CVC-61A) • 2P-109B (2CVC-61B) • to close 2CVC-61A then wait one minute and report 2CVC-61A is
closed.	ATC	13.1.2 Place 2P-109 Pump Select switch (2HS-4967) to pump that will NOT be started.

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ATC	13.1.3 Verify handswitch for pump NOT being started in STOP:				
	• 2P-109A (2HS-4965) • 2P-109B (2HS-4966)				
ATC	13.1.4 Start selected pump by placing handswitch to START:  2P-109A (2HS-4965)  2P-109B (2HS-4966)				
ANY	Direct NLO to close valve locally.  13.1.5 Slowly open pump Discharge Stop Check for pump started:  2P-109A (2CVC-61A)  2P-109B (2CVC-61B)				
ANY	Direct NLO to back seat 2CVC-61A  13.1.6 WHEN header pressurized, THEN fully open AND backseat selected Discharge Stop Check to prevent it from vibrating closed:  2P-109A (2CVC-61A)  • 2P-109B (2CVC-61B)				
open.	Cue: If contacted as NLO to open 2CVC-61A then wait one minute and report 2CVC-61A is open.  Cue: If contacted as NLO to backseat 2CVC-61A then report 2CVC-61A is back seated.				
SRO	8. Notify WWM of Nuclear Instrument malfunctions.				
Cue: When contacted as t package to repair the read	the WWM, then report that the planners will start planning a work ctor make up pump.				
	nen 2P-109A reactor make up water pump has been started or at lead caminer's discretion.				

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Event Description: 'A' Nuclear Instrument (NI) fails high. Tech Spec for SRO.

Time	Position	Applicant's Actions or Behavior
	1	
Cued by lead	ANY	Announce annunciators:  2K04-A4 CH A RPS/ESF/PRETRIP/TRIP
examiner		2K04-A4 CH A RPS/ESF/PRETRIP/TRIP 2K04-B3 PPS Channel TRIP
		2K04 H4 CPC channel sensor failure
	ANY	Crew will recognize that 'A' excore has failed High.
	SRO	Enter and Implement AOP 2203.026, NI Malfunction.
	ANY	Check at least TWO Linear Safety channels operable.
	ANY	2. Check ALL Linear Safety channels operable. (Channel 'A' will not be operable)
		not be operable)
	SRO	2. IF ALL Linear Safety channels NOT operable,
		THEN perform the following:
		A. Refer to TS 3.3.1.1,
		Reactor Protective Instrumentation.
		Examiner note: SRO must enter Tech Spec 3.3.1.1 action 2
	ВОР	C. IF ONLY ONE channel failed,
		THEN verify the following trip functions bypassed:
		• Hi Linear Power {1}
		<ul><li>High LPD {3}</li><li>Low DNBR {4}</li></ul>
		LOW DINDR (4)
	SRO	D. GO TO Step 4.
	ANY	4. Check ALL Log Power channels operable. (Channel 'A' will not be operable)
	SRO	4. IF ALL Log Power channels NOT operable, THEN perform the
		following:
		A. IF Log Power channel #2 failed, THEN refer to TS 3.3.3.5,
		Remote Shutdown Instrumentation.
		D Defen to TS 2.2.1.1
		B. Refer to TS 3.3.1.1, Reactor Protective Instrumentation.
		Examiner note: Tech Spec 3.3.3.5 is not applicable.

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П		
	BOP	D. IF ONLY ONE Log Power channel failed,
		AND associated Linear Safety channel failed,
		THEN verify the following trip functions bypassed:
		Hi Linear Power {1}
		Hi Log Power {2}
		• High LPD {3}
		• Low DNBR {4}
		LOW DIVIN (4)
	BOP	Bypass the points 1, 2, 3, and 4 on Channel A IAW 2105.001 CPC/CEAC
		operations section 11:
		11.1 Refer to Tech Spec 3.3.1.1, 3.3.2.1, 3.3.3.5, 3.3.3.6 and
		TRM 3.3.1.1.
		11.2 Circle showed and functional swite (nainte) to be
		11.2 Circle channel and functional units (points) to be bypassed below:
		Channel to be bypassed: (A) B C D
		Points to be bypassed:
		1 2 3 4 5 6 7 8 9 10 11 12 13 16 17 18 19 20
		Enter appropriate Tech Spec/TRM actions.
		Verify points to be bypassed <u>NOT</u> bypassed in ANY other channel.
		Place desired points in BYPASS for selected channel on 2C23.
		Examiners Note: Key 12 required.
	SRO	8. Notify I&C of Nuclear Instrument malfunctions.

Cue: When contacted as the WWM, then report that I&C will start planning a work package to repair the failed nuclear instrument.

Termination criteria: Affected channel points placed in bypass or at lead examiner's discretion.

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Event Description: Volume Control Tank (VCT) level transmitter fails high.

Time	Position	Applicant's Actions or Behavior				
Cued by lead	ATC	Report alarm 2K12-H5 VCT 2T4 LEVEL HI LO Or				
examiner		Recognize VCT level lowering or 2LT-4861 failed hi.				
		Examiner Note: The crew may notice VCT level lowering or that the transmitter is failed hi prior to receiving VCT level lo alarm				
	ANY	2.1 Check level on 2LIS-4857 and Computer Points (L4857 and L4861).				
	ANY	2.2 IF HIGH Level, THEN perform the following:				
		Examiner Note: The crew should determine that L4861 is failed high and L4857 is reading accurately (trending down)				
	ATC	2.3 IF LOW Level, THEN perform the following:				
2K-12 H5 ACA		2.3.1 Verify VCT Bypass to BMS (2CV-4826) aligned to VCT.				
guidance.	ANY	2.3.2 IF Pressurizer level lowering, THEN GO TO Excess RCS Leakage (2203.016).				
		2.3.3 IF Pressurizer level rising, THEN verify proper letdown flow. Refer to Chemical and Volume Control (2104.002).				
		Examiner Note: PZR level should be stable at setpoint.				
	ATC	2.3.4 Makeup to VCT using Chemical Addition (2104.003).				
	SRO	Contact Work Week Manager to investigate/ repair 2LT-4861 VCT level transmitter.				

Examiner note: 2LT-4861 also controls the VCT LO LO level alarm as well as the re-alignment of Charging pump suction from the VCT to the RWT.

Cue: If contacted as NLO to assess 2LT-4861, then report everything looks normal at the transmitter.

Cue: When contacted as the WWM, then report I&C will start planning a work package to repair the failed instrument.

Termination criteria: When 2CV-4826 placed in the VCT position or at lead examiner's discretion.

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Op-Test No.: 2012-1 Scenario No.: 3 Event No.: 5 Event Description: Inadvertent Containment Isolation Actuation Signal (CIAS) on the Green Train. Tech Spec for SRO. Time Position Applicant's Actions or Behavior Cued by **ANY** Announce alarms for CIAS ACT 2K04-C1 and 2K07-C1 and CCW lead DISCH FLOW LO 2K-11 A1, A3, A5, A7 examiner Examiner note: CCW DISCH FLOW LO alarm is an entry criteria to both RCP emergencies and Inadvertent CIAS. The SRO may enter RCP emergencies to restore CCW to the RCPs but after restoration of CCW to the RCPs, the SRO should complete the actions in Inadvertent CIAS **SRO** Enter and Implement AOP 2203.039, Inadvertent CIAS. **SRO** IF Reactor tripped, THEN verify 2202.001, Standard Post Trip Actions complete. **Examiner note: The Reactor will not be tripped. SRO** 2. Record start time. \_ **ANY** 3. Check CNTMT pressure has NOT exceeded 18.3 psia. **SRO** \*4. IF CCW to RCPs can NOT be restored within 10 minutes, THEN perform the following: Examiner note: The SRO will pass by this step to restore CCW to the RCPs. **ANY** 5. Restore CCW to RCPs as follows: A. Check ALL Controlled Bleedoff temperatures less than 180°F. Critical **BOP** B. Restore CCW to RCPs as follows: Task: Restore 1) Override and open RCP CCW Supply valve (2CV-5236-1). CCW to **RCPs** 2) Override and open RCP CCW Return valve (2CV-5254-2). within 10 minutes. 3) Override and open RCP CCW Return Header Isolation valve (2CV-5255-1). **SRO** C. Refer to Tech Spec 3.6.3.1, Containment Isolation Valves. Examiner note: The SRO must enter 3.6.3.1 for Containment Isolation Valves due to the CCW to RCP valves being overridden which renders the ability to close inoperable.

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Event Description: Inadvertent Containment Isolation Actuation Signal (CIAS) on the Green Train. Tech Spec for SRO.

Time	Position	Applicant's Actions or Behavior		
	ANY	*6. Check ALL RCP bearing and motor temperature annunciators clear:		
		<ul> <li>"UPPER THRUST BRG METAL TEMP HI" (2K11-B1/B3/B5/B7).</li> <li>"LOWER THRUST BRG METAL TEMP HI" (2K11-B2/B4/B6/B8).</li> <li>"STATOR WDG TEMP HI" (2K11-A2/A4/A6/A8).</li> </ul>		
	ANY	*7. Minimize CEA movement.		
	ATC	*8. Maintain PZR Level within 5% of setpoint as follows:		
		A. Cycle Charging pumps as needed.		
		B. Record charging header data using 2202.010, Standard Attachment 44, Charging Header Data.		
	ATC	*9. Maintain VCT level 60 to 75%, refer to 2104.003, Chemical Addition.		
	ANY	*10. Check CNTMT temperature and pressure stable.		
		Examiner note: CNTMT temperature and pressure will be rising due to chilled water isolation.		
	ВОР	*10. IF CNTMT temperature OR pressure rising, THEN perform the following:  A. Verify CNTMT Coolers aligned as follows:		
		<ul> <li>1) ALL available CNTMT Cooling fans running:</li> <li>2VSF-1A</li> <li>2VSF-1B</li> <li>2VSF-1C</li> <li>2VSF-1D</li> </ul>		
		<ul> <li>2) BOTH SW Cooling Inlet valves open:</li> <li>2CV-1511-1</li> <li>2CV-1510-2</li> </ul>		
		<ul> <li>3) BOTH SW Cooling Outlet valves open:</li> <li>2CV-1519-1</li> <li>2CV-1513-2</li> </ul>		
	ANY	B. Refer to TS 3.6.1.4 Internal Pressure, Air Temperature and Relative Humidity.		

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Op-Test No.: 2012-1 Scenario No.: 3 Event No.: 5 Event Description: Inadvertent Containment Isolation Actuation Signal (CIAS) on the Green Train. Tech Spec for SRO. Time Position Applicant's Actions or Behavior **SRO** 11. Reset CIAS on PPS panels as follows: Examiner note: CIAS inadvertently actuated thus can not be reset by the operators, they may attempt to reset CIAS but it will not reset. Examiner note: 2C-39 ESFAS panel in to fully modeled in the simulator. If the applicant tries to assess the status of the panel then give them the following cue. Examiner Cue: If the applicant tries to assess 2C-39 ESFAS panel then inform the applicant that all the light on 2C-39 are on. Cue: If the contacted as the WWM, then report that I&C will start investigating the inadvertent CIAS **SRO** IF CIAS actuation can NOT be reset, THEN GO TO Step 13. **SRO** 13. IF CIAS actuation can NOT be reset, THEN perform the following: A. Check Plant in Mode 1 or 2. Minimize CEA movement. ATC C. Notify I&C to monitor CEDM coil temperatures. D. IF I&C NOT available, THEN refer to 2105.009 Exhibit #2, CEA #01 Upper Gripper Coil Temperature Measurement to determine CEDM coil temperatures. E. Contact System Engineer for assistance.

Termination Criteria: When Service water has been aligned to the CNTMT coolers or at the discretion of the lead examiner.

1) Trip Reactor.

procedure.

F. IF coil temperatures are projected to be >450°F for an extended period AND restoration of CEDM Cooling is NOT imminent,

temperatures exceed 500°F, THEN perform the following:

G. IF CEDM System Engineer NOT available, AND coil

2) GO TO 2202.001, Standard Post Trip Actions.

H. WHEN CIAS has been reset, THEN continue with this

THEN commence a Plant Shutdown.

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Event Description: 'D' RCP seals fail and a 180 gpm RCS leak starts, and Control Element Assembly's (CEA) 15 and 31 fail to fully insert during the reactor trip.

	1	in the reactor trip.			
Time	Position	Applicant's Actions or Behavior			
Cued by lead examiner	ANY	Announce 2K-11 G-3 RCP bleedoff flow hi lo alarm.			
	ANY	Determine that 'D' RCP is the affected RCP and one seal is failed.			
	SRO	Enter and Implement AOP 2203.025, RCP Emergencies.			
	SRO	*1. Check the following criteria for EACH RCP satisfied:			
		D. Seal Stage ΔP greater than 50 psid.			
	SRO	D. GO TO Step 5.			
	ANY	*5. Check ΔP across EACH RCP Seal stage greater than 50 psid.			
	ANY	*5. Perform the following:			
		A. IF ONLY ONE stage failed, THEN perform the following:  1) Monitor RCP Controlled Bleedoff flow and temperature.  2) Notify Operations Management.			
		Examiner note: The other two RCP seals will fail after a 5 minute delay.			
Critical Task: Securing 'D' RCP within 10 minutes of the reactor trip.	ATC	<ul> <li>C. IF THREE or more stages failed on ONE pump, THEN perform the following:</li> <li>1) IF in Mode 1 OR 2, THEN trip Reactor.</li> <li>2) Stop any affected RCP.</li> </ul>			
	ATC	3) IF only one RCP affected, AND desired to balance reactor coolant loop temperatures, THEN verify one RCP secured in each loop.			
	ATC	<ul> <li>4) IF RCP 2P32A OR 2P32B stopped, THEN verify associated PZR Spray valve in MANUAL and closed:</li> <li>• 2CV-4651</li> <li>• 2CV-4652</li> </ul>			
	ALL	5) IF Reactor was manually tripped, THEN GO TO 2202.001, Standard Post Trip Actions.			

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Event Description: 'D' RCP seals fail and a 180 gpm RCS leak starts, and Control Element Assembly's (CEA) 15 and 31 fail to fully insert during the reactor trip.

Time	Position	Applicant's Actions or Behavior						
	SRO	Enter and implement EOP 2202.001	, Standard Post Trip Actions.					
	SRO	Exhibit 7, CBO Reactor Trip						
Dogotivity	ATC	<ol> <li>Open Safety Function Tracking</li> <li>Check Reactivity Control esta</li> </ol>						
Reactivity control safety function	AIC	A. Reactor power lowering.						
Tunction		B.ALL CEAs fully inserted by observing ANY of the following:						
		3) CEAC 2 indicates	ALL CEAs fully inserted. ALL CEAs fully inserted.					
Reactivity	SRO	(Not met due to Event #7, perform contingency)  Direct ATC to perform emergency boration in progress using Exhibit 1,  Emergency Boration.						
safety function		B. Verify emergency box Emergency Boration. Examiner note: The SRO may dis Shutdown Margin.						
	ATC	Select ONE of the following Emergency Boration flowpaths:						
		FLOWPATH	ACTIONS REQUIRED					
Emergency Boration		A. Gravity Feed A.	Verify at least ONE BAM Tank Gravity Feed valve open:  2CV-4920-1 2CV-4921-1					
Critical Task:		B. BAM pumps B.	1) Start at least ONE BAM pump.					
Emergency Boration must be			2) Open Emergency Borate valve					
Boration must be			(2CV-4916-2).					
Boration								

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Event Description: 'D' RCP seals fail and a 180 gpm RCS leak starts, and Control Element Assembly's (CEA) 15 and 31 fail to fully insert during the reactor trip.

Time	Position		Applicant's Actions or Behavior
	ATC	2.	Close VCT Outlet valve (2CV-4873-1).
Emergency Boration	ATC	4.	Verify Reactor Makeup Water Flow Control valve (2CV-4927) closed.
		5.	Verify at least ONE Charging pump running and charging header flow greater than 40 gpm.
	ВОР	4.	Check Maintenance of Vital Auxiliaries satisfied: A. Check Main Turbine tripped by BOTH of the following:
			ALL Main Stop Valves closed.
			<ul> <li>Generator megawatts indicate zero.</li> </ul>
			B. Generator Output breakers open. (Generator output breakers will be closed due to plant alignment, perform contingency to open them)
			C. Exciter Field breaker open.
			D. Check the following valves closed:
Vital Auxiliaries safety			<ul> <li>MSR 2E-12A Steam Supply From SG A (2CV-0400)</li> <li>MSR 2E-12B Steam Supply From SG B (2CV-0460)</li> </ul>
function			E. At least ONE 6900v AC bus energized
			F. At least ONE 4160v Non-vital AC bus energized.
			G. BOTH 4160v Vital AC buses energized.
			H. BOTH DGs secured.
			I. At least ONE 125v Vital DC bus energized:
			2D01 - SPDS point E2D01 2D02 - SPDS point E2D02

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Event Description: 'D' RCP seals fail and a 180 gpm RCS leak starts, and Control Element Assembly's (CEA) 15 and 31 fail to fully insert during the reactor trip.

Time	Position	Applicant's Actions or Behavior
	ATC	5. Check RCS Inventory Control established as follows:
RCS Inventory Control Safety function		<ul> <li>A. PZR level:</li> <li>10 to 80%.</li> <li>Trending to setpoint. (Will not be met due to LOCA Event #6, perform contingency)</li> <li>B. RCS MTS 30°F or greater.</li> </ul>
	SRO	Direct the following as necessary:
RCS Inventory Control		<ul> <li>A. Perform as necessary:</li> <li>1) <u>IF</u> SIAS actuated on PPS inserts, <u>THEN</u> GO TO Step 6.</li> </ul>
Safety function		2) Verify PZR Level Control system restoring level to setpoint.
RCS Pressure Control Safety function	ВОР	<ul> <li>6. Check RCS Pressure Control: <ul> <li>1800 to 2250 psia.</li> </ul> </li> <li>Trending to setpoint. (May not be met due to LOCA Event #6, perform contingency) <ul> <li>Normal PZR Spray and heaters controlling pressure.</li> <li>Valid CNTMT Spray NOT in progress.</li> </ul> </li> </ul>
RCS Pressure Control Safety function	SRO	<ul> <li>Direct the following actions as necessary:</li> <li>6. Perform as necessary:</li> <li>A. <u>IF</u> RCS pressure lowers to less than 1400 psia, <u>THEN</u> trip ONE RCP in EACH loop.</li> <li>D. IF RCP 2P32A or 2P32B stopped, THEN verify associated PZR Spray valve in MANUAL and closed.</li> <li>• RCP A Spray Valve (2CV-4651)</li> <li>• RCP B Spray Valve (2CV-4652)</li> <li>E. <u>IF</u> RCS pressure lowers to 1650 psia or less, THEN perform the following:</li> <li>1) Verify SIAS activated on PDS inserts</li> </ul>
		<ol> <li>Verify SIAS actuated on PPS inserts.</li> <li>GO TO Step 7.</li> <li>Verify PZR Pressure Control system restoring pressure to setpoint.</li> </ol>

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Event Description: 'D' RCP seals fail and a 180 gpm RCS leak starts, and Control Element Assembly's (CEA) 15 and 31 fail to fully insert during the reactor trip.

Time	Position		Applicant's Actions or Behavior
	ATC	7.	Check Core Heat Removal by forced circulation:
Core Heat Removal			A. At least ONE RCP running.
			B. CCW flow aligned to RCPs.
			C. Loop delta T less than 10°F.
safety function.			D. RCS MTS 30°F or greater.
			E. Check SW aligned to CCW.
			F. IF SIAS or MSIS actuated, THEN maintain SW header pressure greater than 85 psig.
	BOP/ATC	8.	Check RCS Heat Removal:
			A. Check SG available by BOTH of the following:
RCS Heat Removal Safety Function			<ul><li>At least ONE SG level 10 to 90%.</li><li>FW maintaining SG level</li></ul>
			B. Check MFW in RTO (MFW may not be in Reactor Trip Override (RTO) due to steam generator levels being at setpoint, and the contingencies are satisfied if steam generator levels are controlling near the setpoint)
			C. Check Feedwater line intact by the following:
			• SG level stable or rising.
			<ul> <li>NO unexplained step changes or erratic FW flow.</li> <li>NO unexplained step changes or erratic Condensate flow.</li> </ul>
			D. Check SG pressure 950 to 1050 psia.
			E. IF desired, THEN perform the following for SDBCS Master Controller using 2105.008 Exhibit 3:
			1) Place controller in Auto/Local.
			2) Set setpoint at 960 psia.
			F. Check RCS TC 540 to 555°F.

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Event Description: 'D' RCP seals fail and a 180 gpm RCS leak starts, and Control Element Assembly's (CEA) 15 and 31 fail to fully insert during the reactor trip.

Time	Position	Applicant's Actions or Behavior
	ANY	9. Check CNTMT parameters:
		A. Temperature and Pressure:
		<ul> <li>Temperature less than 140°F.</li> <li>Pressure less than 16 psia.</li> </ul>
		B. Check CNTMT Spray pumps secured.
		C. NO CNTMT radiation alarms or unexplained rise in activity:
		<ul> <li>1) CAMS alarms:</li> <li>"CNTMT PART/GAS RAD HI/LO" annunciator (2K10-B6) clear.</li> </ul>
Containment Safety Function		<ul> <li>2) RCS leakage alarms:</li> <li>"AREA RADIATION HI/LO" annunciator (2K11-B10) clear. (Not met due to LOCA Event #6)</li> <li>"PROC LIQUID RADIATION HI/LO" annunciator (2K11-C10) clear.</li> </ul>
		<ul> <li>3) Check the following radiation monitors trend stable: (Not met due to LOCA Event #6)</li> <li>• CNTMT Area</li> <li>• CAMS</li> <li>• Process Liquid</li> </ul>
		D. NO secondary system radiation alarms or unexplained rise in activity:
		1) "SEC SYS RADIATION HI" annunciator (2K11-A10) clear.
		<ul> <li>Secondary Systems Radiation monitors trend stable:</li> <li>Main Steam lines</li> <li>SG Sample</li> <li>Condenser Off Gas</li> </ul>

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Event Description: 'D' RCP seals fail and a 180 gpm RCS leak starts, and Control Element Assembly's (CEA) 15 and 31 fail to fully insert during the reactor trip.

(CEA) 13 all		lly insert during the reactor trip.								
Time	Position	Applicant's Actions or Behavior								
	SRO	10. Notify STA to report to control room.								
		11. Direct NLOs to perform 2202.010 Attachment 47, Field Operator Post Trip Actions.								
		12. Verify Reactor trip announced on Plant page.								
	13. Notify SM to refer to Technical Specifications and 19 Emergency Action Level Classification.									
	acted as a NI	STA to report to the control room, acknowledge the request.  LO to perform Attachment 47 Field Operator Post Trip Actions,								
	SRO	14. Direct control board operators to acknowledge ALL annunciators and announce ALL critical alarms.								
	SRO	15. Check ALL safety function acceptance criteria satisfied. (All safety functions are not satisfied, perform contingency)								
		15. <u>IF ANY safety function acceptance criteria NOT satisfied, THEN perform the following:</u>								
		A. Notify control room staff of safety functions <u>NOT</u> satisfied.								
		B. GO TO Exhibit 8, Diagnostic Actions.								
	SRO	Diagnose Loss of Coolant Accident EOP 2202.003 or Excess RCS leakage AOP 2203.016 depending SIAS is actuated.								
	SRO	If Excess RCS leakage is diagnosed, Enter and perform floating step # 9								
	SRO	■ 9. IF RCS leakage greater than 44 gpm AND in Mode 3, 4, OR 5, THEN perform the following:								
	ATC/BOP	B. Actuate SIAS.								
		C. Actuate CCAS.								
	SRO	D. GO TO 2202.010 Exhibit 8, Diagnostic Actions.								
Examiner n	ote: The SRC	may discuss Tech Spec 3.4.6.2 for RCS leakage.								
	SRO	Enter and implement Loss of Coolant Accident EOP 2202.003								

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Event Description: 'D' RCP seals fail and a 180 gpm RCS leak starts, and Control Element Assembly's (CEA) 15 and 31 fail to fully insert during the reactor trip.

Time	Position	Applicant's Actions or Behavior
	SRO	* 1. Confirm diagnosis of LOCA as follows:
		A. Check SFSC acceptance criteria satisfied every 15 minutes.  (Normally performed by the STA)
		B. IF CCW in service to provide SG Sample Cooler cooling, THEN perform the following:
		1) Verify SG Sample valves open.
		<ul> <li>2CV-5852-2</li> <li>2CV-5859-2</li> <li>2CV-5850</li> <li>2CV-5858</li> </ul>
		2) Notify Chemistry to sample SGs for activity.
		C. IF SGs indicate primary to secondary leakage within TS limits, THEN continue with this procedure using SG with lowest leak rate for cooldown.
Cue: When	contacted as	Chemistry, then report you will sample both S/G for activity.
	SRO	* 2. Notify SM to refer to Technical Specifications and 1903.010, Emergency Action Level Classification.
	SRO	3. Open Placekeeping Page.
	SRO	4. Record present time:  • Time
	ANY	5. Verify SIAS and CCAS actuated on PPS inserts.
	SRO	6. Notify Control Board Operators to perform the following:  A. Monitor floating stops
		<ul><li>A. Monitor floating steps.</li><li>B. Verify actuated ESFAS components using 2202.010, Exhibit 9, ESFAS Actuation.</li></ul>
	ANY	*7. Check CCW aligned to RCPs.
	ANY	■8. Check RCS pressure greater than 1400 psia. (May not be met, if not met perform contingency if required)

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Event Description: 'D' RCP seals fail and a 180 gpm RCS leak starts, and Control Element Assembly's (CEA) 15 and 31 fail to fully insert during the reactor trip.

(CEA) 13 al	10 31 1an 10 10	Illy insert during the reactor trip.
Time	Position	Applicant's Actions or Behavior
	ANY	■8. Perform the following:
		A. IF RCS pressure less than 1400 psia, THEN perform the following:
		1) Verify maximum of ONE RCP running in EACH loop.
		2) <u>IF RCP 2P32A or 2P32B stopped, THEN</u> verify associated PZR Spray valve in MANUAL and closed.
		B. IF NPSH requirements violated OR RCS MTS less than 30°F,
		THEN perform the following:
		1) Stop ALL RCPs.
		2) Verify BOTH PZR Spray valves in MANUAL and closed.
	ANY	■9. Restore ESF/Non-ESF systems post-SIAS as follows:
		A. Verify at least ONE SW pump running on EACH loop.
	ANY	B. Verify DG SW Outlet valves open.
	ANY	C. Verify SW pump suction aligned to Lake.
	ANY	D. Check 4160v Non-vital bus 2A1 or 2A2 energized from offsite power.
	ANY	E. Check 4160v Vital buses 2A3 AND 2A4 energized from offsite power.
	ВОР	F. Start SW pumps as needed to maintain SW header pressure.
	ANY	G. Check SW to CCW restored. (Not met, perform contingency)
	ВОР	G. IF CCW available, THEN restore SW to CCW, refer to 2202.010 Exhibit 5, CCW/ACW/SW Alignment.
	ANY	H. Check ACW restored. (Not met, perform contingency)
	ВОР	H. Restore SW to ACW, refer to 2202.010 Exhibit 5, CCW/ACW/SW Alignment.
	ВОР	I. Maintain SW header pressure greater than 85 psig.
	ANY	*10. Verify Safety Injection flow to RCS as follows:
		<ul> <li>A. Check total HPSI flow acceptable using 2202.010 Exhibit 2, HPSI Flow Curve.</li> <li>B. Check total LPSI flow acceptable using 2202.010 Exhibit 3, LPSI Flow Curve.</li> </ul>

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Event Description: 'D' RCP seals fail and a 180 gpm RCS leak starts, and Control Element Assembly's (CEA) 15 and 31 fail to fully insert during the reactor trip.

Time	Position	Applicant's Actions or Behavior
	ANY	■11. Check SG levels greater than 22.2%.
	ВОР	12. Align feedwater as follows:  A. Verify EFW pump 2P7B capable of feeding intact SG using 2202.010 Attachment 46, Establishing EFW Flow.
	ВОР	B. Stop EFW pump 2P7A by overriding and closing (2CV-0340-2).
		C. Verify running MFW pump secured.
		D. Verify ALL MFW Block valves closed.
	ANY	13. Isolate LOCA as follows:
		A. Check for intact CCW system:
		• "PROC LIQUID RADIATION HI/LO" annunciator (2K11-C10) clear.
		CCW Surge Tanks level stable.
	ANY	B. Verify non-actuated valve positions using 2202.010 Attachment 17, LOCA Isolation.
		Examiner note: The SRO may elect to prioritize this attachment to be performed later in the event.
	ANY	14. Check LOCA limited to CNTMT:
		CNTMT Sump level rising.
		<ul> <li>CNTMT temperature, dewpoint, and pressure greater than pre-LOCA values.</li> </ul>
		Aux Building area radiation levels stable.
		<ul> <li>Aux Building Sump level less than 53%.</li> </ul>
		Waste Tanks 2T20A and 2T20B levels stable.
	ANY	■15. Check CNTMT isolation parameters:
		<ul><li>A. CNTMT pressure trend has NOT exceeded 18.3 psia.</li><li>B. "CNTMT RADIATION HI" annunciator (2K10-A6) clear.</li></ul>
	ANY	■16. Check CNTMT pressure trend has NOT exceeded 23.3 psia.
	ANY	*17. IF CNTMT Spray operating, THEN terminate CNTMT Spray as follows:
		Examiner note: CNTMT Spray should not be operating.

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Event Description: 'D' RCP seals fail and a 180 gpm RCS leak starts, and Control Element Assembly's (CEA) 15 and 31 fail to fully insert during the reactor trip.

Time	Position	Applicant's Actions or Behavior
	ВОР	18. Verify ALL available miscellaneous CNTMT ventilation operating:
		A. CNTMT Building Recirculation fans:
		• 2VSF-31A
		• 2VSF-31B
		• 2VSF-31C
		• 2VSF-31D
		B. Reactor Cavity Cooling fans:
		• 2VSF-34A
		• 2VSF-34B
		C. Maximum of 3 CEDM Shroud Cooling units:
		• 2VSF-35A
		• 2VSF-35B
		• 2VSF-35C
		• 2VSF-35D
	ANY	■19. Check ALL AC and Vital DC buses energized.
	ANY	■20. Check IA pressure greater than 65 psig.
	ANY	21. Check for isolated LOCA:
		RCS pressure controlled.
		RCS leakage less than available Charging pump capacity.
		Examiner note: LOCA is not considered isolated.
	SRO	21. IF LOCA NOT isolated, THEN GO TO Section 3, Unisolated LOCA.
	SRO	■1. Perform controlled cooldown to 275°F TC as follows:
	ANY	A. Check RCS TC greater than 275°F.

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Event Description: 'D' RCP seals fail and a 180 gpm RCS leak starts, and Control Element Assembly's (CEA) 15 and 31 fail to fully insert during the reactor trip.

		-
Time	Position	Applicant's Actions or Behavior
	ATC	B. Reset Low PZR Pressure and Low SG Pressure setpoints during cooldown and depressurization.
		C. Verify maximum of ONE RCP running in EACH loop.
		D. IF RCP 2P32A or 2P32B stopped, THEN verify associated PZR Spray valve in MANUAL and closed.
		<ul> <li>E. Monitor cooldown rate as follows:</li> <li>Record RCS TC and PZR temperature using 2202.010         Attachment 8, RCS Cooldown Table. </li> <li>Plot RCS pressure versus RCS TC using 2202.010         Attachment 1, P-T Limits every 15 minutes. </li> </ul>
		F. Initiate RCS cooldown using SDBCS bypass valves or ADVs.
	ВОР	G. Control S/G levels with EFW or AFW using 2202.010 Attachment 46, Establishing EFW Flow.
		H. Secure running MFW pump.
		I. Close ALL MFW Block valves.
		J. Verify maximum of one condensate pump in service.
		K. Maintain condensate header pressure less than 700 psig using condensate pump recircs and MFW pump recircs.

Termination criteria: When the need for a cooldown has been determined or at the discretion of the lead examiner.

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Op-Test No.:	: 2012-1	Scenario No.: 3	Event No.: 8									
Event Descri	iption: 2P-89 <i>E</i>	A High Pressure Safety Injection (HPSI) pu	imp has degraded performance.									
Time	Position	Applicant's Actions	or Behavior									
	ANY	Report that 2P-89A High pressure safety discharge pressure.	injection pump has degraded									
	ANY	Dispatch NLO investigate 2P-89A locally	y									
		WCO, then report after 2 min that 2P-89 aking an abnormal noise.	A vibrations are high, casing is									
	SRO	Direct BOP to secure 2P-89A and start 2P-89C										
		Or										
		Review HPSI termination criteria and if i just secure 2P-89A.	it is met the crew may elected to									
	ВОР	Place 2P-89A in Pull to Lock and remove 2P-89C from Pull to Lock or start 2P-89C										
Examiner N for 2P-89A.		may discuss Tech Spec 3.5.2 if RCS pre	essure is greater than 1700 psia									
Termination	n criteria: W	hen 2P-89A has been started or at exami	ner's discretion.									

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Facility: A	Arkansas	Nuclea	ar One			Date	of Exar	am: 8-27-12 Operating Test No.: 201								012-1			
Α	Е							Sc	enari	os									
P P	V E		1			2		3				ently se as spare		T O	M I N				
I C	N T		CREW SITIO		CREW POSITION				CREV OSITIO			CREW OSITIO		T A L	ı	I M			
A N T	T Y P	S R O	A T C	B O P	S R O	A T C	B O P	S R O	A T C	B O P	S R O	A T C	B O P	_		J M(*) I	U		
RO	E RX					4								1	1	1	0		
	NOR	1						1			1			2	1	1	1		
SRO-I X SRO-U	I/C	2,3,4, 5,8				6,9		2,3,4, 6,7			3,4,5,			12	4	4	2		
	MAJ	6,7				5,7		5			6,7			5	2	2	1		
	TS	2,5						2,4			2,5			4	0	2	2		
RO	RX		3											1	1	1	0		
	NOR				1					1			1	2	1	1	1		
SRO-I	I/C		4,5		2,3,4, 6,8,9					2,4,7			4,8	11	4	4	2		
SRO-U	MAJ		6,7		5,7					5			6,7	5	2	2	1		
	TS				2,4									2	0	2	2		
RO <b>X</b>	RX								1			1		1	1	1	0		
SRO-I	NOR			1			1							2	1	1	1		
	I/C			2,3,8			2,3,8		3,6			3,5		8	4	4	2		
SRO-U	MAJ			6,7			5,7		5			6,7		5	2	2	1		
	TS													0	0	2	2		
RO	RX														1	1	0		
SRO-I	NOR														1	1	1		
	I/C														4	4	2		
SRO-U	MAJ														2	2	1		
	TS														0	2	2		

#### Instructions

- 1. Check the applicant level and enter the operating test number and Form ES-D-1 event numbers for each event type; TS are not applicable for RO applicants. ROs must serve in both the "at-the-controls (ATC)" and "balance-of-plant (BOP)" positions; Instant SROs must serve in both the SRO and the ATC positions, including at least two instrument or component (I/C) malfunctions and one major transient, in the ATC position. If an Instant SRO additionally serves in the BOP position, one I/C malfunction can be credited toward the two I/C malfunctions required for the ATC position.
- Reactivity manipulations may be conducted under normal or controlled abnormal conditions (refer to Section D.5.d) but must be significant per Section C.2.a of Appendix D. (\*) Reactivity and normal evolutions may be replaced with additional instrument or component malfunctions on a 1-for-1 basis.
- 3. Whenever practical, both instrument and component malfunctions should be included; only those that require verifiable actions that provide insight to the applicant's competence count toward the minimum requirements specified for the applicant's license level in the right-hand columns.

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Facility: /	Arkansas	nsas Nuclear One Date of Exam:								m: 8-27-12 Operating Test No.: 2012-1								
Α	E							Sc	enari	os								
P V E		1			2				3(Only used for SROI & 1 RO)			(curren ted as s		T 0		M I N		
L I C	N T		REW		CREW POSITION				CREV OSITIO		PC	T A L	ı	I M				
A N	T Y	S R	A T	ВО	S R	A T	ВО	S R	A T	ВО	S R	A T	ВО		ı	J M(*)	l	
Т	P E	0	С	Р	0	С	P	0	С	Р	0	С	Р		R	ı	U	
RO	RX		3									1		1	1	1	0	
X SRO-I	NOR						1			1				1	1	1	1	
	I/C		4,5				2,3,8			2,4,7		3,5		5	4	4	2	
SRO-U	MAJ		6,7				5,7			5		6,7		4	2	2	1	
	TS													0	0	2	2	
RO	RX					4			1					1	1	1	0	
X SRO-I	NOR			1									1	1	1	1	1	
	I/C			2,3,8		6,9			3,6				4,8	5	4	4	2	
SRO-U	MAJ			6,7		5,7			5				6,7	4	2	2	1	
	TS													0	0	2	2	
RO	RX		i.											0	1	1	0	
SRO-I	NOR	1			1			1			1			2	1	1	1	
SRO-U	I/C	2,3,4, 5,8			2,3,4, 6,8,9			2,3,4, 6,7			3,4,5, 8			11	4	4	2	
X	MAJ	6,7			5,7			5			6,7			4	2	2	1	
	TS	2,5			2,4			2,4			2,5			4	0	2	2	
RO	RX													0	1	1	0	
SRO-I	NOR													0	1	1	1	
	I/C													0	4	4	2	
SRO-U	MAJ													0	2	2	1	
	TS													0	0	2	2	

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Revision 0 Page 1 of 2

Facility: Arkansas Nucle	e of I	Exan	ninati	ion:	8/27/	/12	Оре	eratir	ng Te	st N	lo.:20	012-	1				
		APPLICANTS															
	5	RO BRO- BRO-	u [		S	RO- RO-	ı [		SI	RO-I RO-L	J X		RO SRO-I SRO-U SCENARIO				
Competencies	1 BOP	2 ATC	3 BOP	4 ATC	1 ATC	2 BOP	3 ATC	4 BOP	1	2	3	4	1	2	3	4	
Interpret/Diagnose Events and Conditions	2,3, 6,7, 8	2,4, 5,6, 7,8, 9	2,4, 5,7	3,5, 6,7	2,3, 4,5, 6,7	3,4, 5,7, 8	2,3, 4,5, 6	2,4, 6,7, 8	2,3, 4,5, 6,7, 8	2,3, 4,5, 6,7, 8,9	2,3, 4,5, 6,7	2,3, 4,5, 6,7, 8					
Comply With and Use Procedures (1)	1,2, 6,7	4,5, 6,7, 9	1,2,	1,5, 6,7	3,4, 5,6, 7	1,2, 5,7	1,3, 4,5, 6	2,4, 6,7	2,3, 4,5, 6,7, 8	2,3, 4,5, 6,7, 8	1,2, 3,4, 5,6	1,2, 3,4, 5,6, 7,8					
Operate Control Boards (2)	1,2, 3,6, 7,8	4,5, 6,7, 9	1,2, 4,5, 7	1,3, 5,6, 7	3,4, 5,6, 7	1,2, 3,5, 7,8	1,3, 4,5, 6	1,4, 6,7, 8	N/A	N/A	N/A	N/A					
Communicate and Interact	1,2, 3,5, 6,7, 8	2,3, 4,5, 6,7, 8,9	1,2, 4,5, 7	1,3, 5,6, 7	2,3, 4,5, 6,7, 8	1,2, 3,4, 5,7, 8	1,2, 3,4, 5,6	1,2, 5,6, 7,8	1,2, 3,4, 5,6, 7,8	1,2, 3,4, 5,6, 7,8, 9	1,2, 3,4, 5,6, 7	1,2, 3,4, 5,6, 7,8					
Demonstrate Supervisory Ability (3)	N/A	N/A	N/A	N/A	N/A	N/A	N/A	N/A	1,2, 3,4, 5,6, 7,8	1,2, 3,4, 5,6, 7,8, 9	1,2, 3,4, 5,6, 7	1,2, 3,4, 5,6, 7,8					
Comply With and Use Tech. Specs. (3)	N/A	N/A	N/A	N/A	N/A	N/A	N/A	N/A	2,5	2,4	2,4	2,5					

# Notes:

- Includes Technical Specification compliance for an RO. (1)
- (2) Optional for an SRO-U.
- (3) Only applicable to SROs.

## Instructions:

Check the applicants' license type and enter one or more event numbers that will allow the examiners to evaluate every applicable competency for every applicant.

Facility: Arkansas Nuclea	r On	e	Dat	e of I	Exam	ninati	ion:	8/27/	/12	Оре	eratir	ng Te	est N	lo.:2	012-	1	
		APPLICANTS															
	S	RO SRO-I SRO-I	ı 🖺	X   	S	RO-     RO-	_	\ \ \	_	O RO-I RO-L		] :	RO SRO-I SRO-U				
Competencies	S	CEN	IARI	0	S	CEN	IARIO	<b>O</b>	S	CEN	IARIO	0	ξ	3CE1	NARI	0	
<b>33p3</b>	<b>1</b>	2 BOP	3 ATC	4 ATC	1 sro	2 ATC	3 SRO	4 SRO	1 ATC	2 SRO	<b>3</b>	<b>4</b>	1	2	3	4	
Interpret/Diagnose Events and Conditions	2,3, 6,7, 8	3,4, 5,7, 8	2,3, 4,5, 6	3,5, 6,7	2,3, 4,5, 6,7, 8	2,4, 5,6, 7,8, 9	2,3, 4,5, 6,7	2,3, 4,5, 6,7, 8	2,3, 4,5, 6,7	2,3, 4,5, 6,7, 8,9	2,4, 5,7	2,4, 6,7, 8					
Comply With and Use Procedures (1)	1,2, 6,7	1,2, 5,7	1,3, 4,5, 6	1,5, 6,7	2,3, 4,5, 6,7, 8	4,5, 6,7, 9	1,2, 3,4, 5,6	1,2, 3,4, 5,6, 7,8	3,4, 5,6, 7	2,3, 4,5, 6,7, 8	1,2,	2,4, 6,7					
Operate Control Boards (2)	1,2, 3,6, 7,8	1,2, 3,5, 7,8	1,3, 4,5, 6	1,3, 5,6, 7	N/A	4,5, 6,7, 9	N/A	N/A	3,4, 5,6, 7	N/A	1,2, 4,5, 7	1,4, 6,7, 8					
Communicate and Interact	1,2, 3,5, 6,7, 8	1,2, 3,4, 5,7, 8	1,2, 3,4, 5,6	1,3, 5,6, 7	1,2, 3,4, 5,6, 7,8	2,3, 4,5, 6,7, 8,9	1,2, 3,4, 5,6, 7	1,2, 3,4, 5,6, 7,8	2,3, 4,5, 6,7, 8	1,2, 3,4, 5,6, 7,8, 9	1,2, 4,5, 7	1,2, 5,6, 7,8					
Demonstrate Supervisory Ability (3)	N/A	N/A	N/A	N/A	1,2, 3,4, 5,6, 7,8	N/A	1,2, 3,4, 5,6, 7	1,2, 3,4, 5,6, 7,8	N/A	1,2, 3,4, 5,6, 7,8, 9	N/A	N/A					
Comply With and Use Tech. Specs. (3)	N/A	N/A	N/A	N/A	2,5	N/A	2,4	2,5	N/A	2,4	N/A	N/A					
Notes: (1) Includes Technical Sp		catior	ı com	ıplian	ce fo	r an F	₹O.										

- (2) (3) Optional for an SRO-U.
  - Only applicable to SROs.

## Instructions:

Check the applicants' license type and enter one or more event numbers that will allow the examiners to evaluate every applicable competency for every applicant.