

#### WCNOC-NRC Pre-application Meeting

#### Core Design and Safety Analysis Methodology Transition License Amendment Request

#### September 20, 2012

#### Wolf Creek Nuclear Operating Corporation



#### Meeting Agenda

- Meeting Purpose / Objectives
- Introductions
- Transition to Westinghouse Analysis Methodologies
  - Analysis Methodologies
  - Alternative Source Term
  - Setpoint Uncertainty Analyses
- License Amendment Request (LAR) Content and Schedule
- Questions/Discussion



#### Introductions

- NRC
- WCNOC Team
- Westinghouse Support
- Teleconference Attendees

#### Current State

- WCNOC's Core Design & Safety Analysis organizations performed the USAR Chapter 4 and 15 analyses in-house
  - The USAR Chapter 4 Core Design and Thermal-Hydraulic (T-H) analyses are performed by WCNOC using standard Westinghouse methods through a Technology Transfer agreement
  - The USAR Chapter 15 non-LOCA transient analysis are performed by WCNOC using vendor-independent methods



- Current State (cont.)
  - The USAR Chapter 15 LOCA analyses are performed by Westinghouse
  - Reactor Trip System (RTS) and Engineered Safety Features Actuation System (ESFAS) instrumentation setpoints performed by WCNOC

- Future State
  - The bulk of the safety analyses will be standard Westinghouse analysis methodologies
    - Containment response to be addressed when changes are required to support plant modifications
  - All Westinghouse methods previously approved by the NRC for application at plants like WCGS
  - All analyses to be performed by Westinghouse in accordance with Westinghouse QA processes



- Future State (cont.)
  - Core design and fuel rod design
    - No change for Core Design or Fuel Rod design tools/methods
  - Non-LOCA Transient Analyses
    - W-RETRAN-02 and VIPRE-W
  - Apply Alternative Source Term in radiological dose consequences analyses
    - Provide additional margin for control room/control building envelope in-leakage requirements
  - RTS and ESFAS instrumentation setpoints and supporting uncertainty analyses being performed using current Westinghouse setpoint methodology 7

- Teaming Agreement with Westinghouse
  - Westinghouse is responsible for all Core Design and Safety Analysis (CDSA) activities at WCGS
    - Westinghouse maintains an office at the site
      - Currently 3 Westinghouse CDSA employees are on site
      - Supplemented by Safety Analysis and Licensing staff at HQ and Dallas
    - Activities include:
      - Methodology transition support
      - Full scope core design (including upcoming Cycle 20)
      - Plant support in the CDSA function area

- Summary of Important Changes
  - Safety analyses will be performed to support a future measurement uncertainty recapture (MUR) power uprate application
    - No current plans to pursue the MUR uprate
  - DNB Correlations used with VIPRE-W
    - WRB-2 will continue to be used as the primary DNB correlation to be used for T/H analyses
    - ABB-NV and WLOP DNB correlations will be used for lowpressure conditions or below the first mixing vane grid

- Summary of Important Changes (cont.)
  - Non-LOCA transients and accidents
    - W- RETRAN-02 used in most non-LOCA analyses
    - Continued use of LOFTRAN code in selected analyses
    - Continued use of TWINKLE and FACTRAN 1-D kinetics codes for reactivity insertion transient analyses

- Summary of Important Changes (cont.)
  - Steam Generator Tube Rupture
    - Current WCNOC methodology shows SG overfill
    - Westinghouse methodology is margin to overfill
    - The revised SGTR analysis using Westinghouse methodology shows margin to overfill
      - Supporting reactor operator response time evaluations in progress
      - Dose consequences are calculated assuming overfill does not occur

- Summary of Important Changes (cont.)
  - Instrumentation Setpoints
    - Calculations being performed on RTS Instrumentation, ESFAS Instrumentation, and Loss of Power Diesel Generator (LOP DG) Start Instrumentation Functions using current Westinghouse setpoint methodology
    - Adopt TSTF-493 Rev. 4, Option B

- Summary of Important Changes (cont.)

   Alternative Source Term (per Reg Guide 1.183) will be used in radiological dose consequences calculations
  - Offsite and Control Room Dose calculations only
  - No changes to the EQ dose analyses

- Alternative Source Term (AST) Implementation
  - Consistent with Reg Guide 1.183, "Alternative Radiological Source Terms for Evaluating Design Basis Accidents at Nuclear Power Plants"
  - Any exceptions to the Reg Guide will be identified
  - Full Scope Implementation
    - Reg Guide 1.183 C. 1.2.1- Full implementation addesses all characteristics of the AST, i.e., the composition and magnitude of the radioactive material, its chemical and physical form, and the timing of its release

- AST Implementation (cont.)
  - No changes to the EQ dose analyses
    - Reg Guide 1.183 C. 1.3.5- Licensees may use either the TID 14844 or AST assumptions for performing the EQ analyses
  - New atmospheric dispersion (X/Q) values are being calculated
    - EAB and LPZ (X/Q) values are being calculated in accordance with Reg Guide 1.145
    - Control Room (X/Q) values were calculated in accordance with Reg Guide 1.194

- AST Implementation (cont.)
  - Current licensing basis changes associated with the reanalysis
    - TEDE acceptance criterion
    - Revised analysis assumptions
    - Revised analysis results

- AST Implementation (cont.)
  - Dose consequence analyses reanalyzed with AST
    - Main Steamline Break (USAR Section 15.1.5.3)
    - Loss of Non-Emergency AC Power (USAR Section 15.2.6.3)
    - Locked Rotor (USAR Section 15.3.3.3)
    - Rod Ejection (USAR Section 15.4.8.3)
    - Letdown Line Break (USAR Section 15.6.2.1)
    - Steam Generator Tube Rupture (USAR Section 15.6.3.3)
    - Loss of Coolant Accident (USAR Section 15.6.5.4)
    - Waste Gas Decay Tank Failure (USAR Section 15.7.1)
    - Liquid Waste Tank Failure (USAR Section 15.7.2)
    - Fuel Handling Accident (USAR Section 15.7.4)

- TSTF-493 Rev. 4, Option B
  - Scope:
    - RTS Instrumentation
    - ESFAS Instrumentation
    - LOP DG Start Instrumentation
  - Updated uncertainty analyses and setpoints will be performed using the current Westinghouse setpoint methodology

#### • TSTF-493 Rev. 4, Option B (cont.)

- The Setpoint Control Program (SCP) to be addressed in Section 5.5.19 of TSs
- The Allowable Values will be removed from TS Table 3.3.1-1, Table 3.3.2-1, and SR 3.3.5.3
- SR 3.3.5.3 Trip Setpoint will be relocated to SCP
- The Nominal Trip Setpoints, As Left Tolerance values, As Found Tolerance values and operability determination criteria will be included in the SCP

#### • TSTF-493 Rev. 4, Option B (cont.)

 The WCGS SCP will be based on the Westinghouse SCP Process Flow Diagram (recently discussed in the 6/28/12 NRC-DCPP meeting)

- License Amendment Request
  - Content
    - Attachments Evaluation, Proposed TS Changes (Markups), Revised TS pages, Proposed TS Bases Changes (for information only), Proposed COLR Changes (for information only)
    - Enclosures
      - Plant specific WCAP Licensing Report
        - » Description of safety analyses assumptions
        - » Description of safety analyses results

- License Amendment Request (cont.)
  - Content (cont.)
    - Enclosures
      - Plant specific WCAP containing the setpoint methodology for the uncertainty analyses and setpoints
      - Plant specific WCAP containing the WCGS setpoint calculations
      - WCGS proposed SCP



- License Amendment Request (cont.)
  - Content Enclosures (cont.)
    - Report for full scope implementation of Alternative Source Term
      - Description of the analysis assumptions
      - Description of the analysis results
      - Tables containing the CLB assumptions and results and reanalysis assumptions and results
      - A table identifying the conformance to Reg Guide 1.183 and any exceptions
      - A table that address the issues identified in RIS 2006-04

- License Amendment Request (cont.)
  - Potential Technical Specification Changes
    - TS 3.3.1, RTS Instrumentation
    - TS 3.3.2, ESFAS Instrumentation
    - TS 3.3.5, LOP DG Start Instrumentation
    - New TS 5.5.19, Setpoint Control Program
    - TS 5.6.5, CORE OPERATING LIMITS REPORT (COLR)



#### Schedule

• Submit LAR to NRC

August 2013

- NRC Acceptance Review
- RAI Process
- Start of Refueling Outage 20
- Cycle 21 Startup

January 5, 2015 February 9, 2015



#### **Questions/Discussion**