

WBN2Public Resource

From: Boyd, Desiree L [dlboyd@tva.gov]
Sent: Monday, August 27, 2012 1:34 PM
To: Epperson, Dan; Wilson, George; Poole, Justin
Cc: Arent, Gordon; Hamill, Carol L; Boyd, Desiree L
Subject: TVA letter to NRC_08-27-2012__TS Table 3.3.3-1 and TSB 3.3.3
Attachments: 08-27-2012__TS Table 3.3.3-1 and TSB 3.3.3_Final.pdf

Please see attached TVA letter that was sent to the NRC today.

Thank You,

~*~*~*~*~*~*~*~*~*~

Desiree L. Boyd

WBN Unit 2 Licensing

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Watts Bar Nuclear Plant, Unit 2
NRC Docket No. 50-391

Subject: **Watts Bar Nuclear Plant (WBN) Unit 2 – Update to Technical Specification (TS) Table 3.3.3-1 and Technical Specification Bases (TSBs) 3.3.3**

- References:
1. TVA Letter to NRC dated June 5, 2012, “Watts Bar Nuclear Plant (WBN) Unit 2 - Submittal of Revised Section 4.3, ‘Fuel Storage,’ Developmental Revision G of the Unit 2 Technical Specification (TS)”
 2. TVA Letter to NRC dated February 28, 2012, “Watts Bar Nuclear Plant (WBN) Unit 2 - Submittal of Developmental Revision G of the Unit 2 Technical Specification (TS) and Technical Specification Bases”

This letter provides an update to Unit 2 TS Table 3.3.3-1, “Post Accident Monitoring Instrumentation,” to delete footnote “(g)” from function 6), “Reactor Vessel Water Level.” In addition, TSB 3.3.3 has been updated to state that function 6), “Reactor Vessel Water Level,” is now a Type A, Category 1 variable. These changes will be incorporated as part of TS, Developmental Revision H at a later date. This condition has been entered into TVA’s corrective action program as Problem Evaluation Report 488815.

Enclosure 1 provides page markups for both of these changes. Enclosure 2 provides the list of regulatory commitments associated with this letter.

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I declare under penalty of perjury that the foregoing is true and correct. Executed on the 27th day of August, 2012.

Respectfully,



Raymond A. Hruby, Jr.
General Manager, Technical Services
Watts Bar Unit 2

Enclosures:

1. WBN Unit 2 TS and TSB Mark-ups
2. List of Regulatory Commitments

cc (Enclosures):

U. S. Nuclear Regulatory Commission
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245 Peachtree Center Ave., NE Suite 1200
Atlanta, Georgia 30303-1257

NRC Resident Inspector Unit 2
Watts Bar Nuclear Plant
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bcc (Enclosures):

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Enclosure 1

WBN Unit 2 TS and TSB Mark-ups

Table 3.3.3-1 (page 1 of 3)
Post Accident Monitoring Instrumentation

FUNCTION	APPLICABLE MODES OR OTHER SPECIFIED CONDITIONS	REQUIRED CHANNELS / TRAINS	CONDITION REFERENCED FROM REQUIRED ACTION D.1
1) Intermediate Range Neutron Flux ^(g)	1 ^(a) , 2 ^(b) , 3	2	E
2) Source Range Neutron Flux	2 ^(c) , 3	2	E
3) Reactor Coolant System (RCS) Hot Leg Temperature (T-Hot)	1, 2, 3	1 per loop	E
4) RCS Cold Leg Temperature (T-Cold)	1, 2, 3	1 per loop	E
5) RCS Pressure (Wide Range)	1, 2, 3	3	E
6) Reactor Vessel Water Level ^(f) (g)	1, 2, 3	2	F
7) Containment Sump Water Level (Wide Range)	1, 2, 3	2	E
8) Containment Lower Comp. Atm. Temperature	1, 2, 3	2	E
9) Containment Pressure (Wide Range) ^(g)	1, 2, 3	2	E
10) Containment Pressure (Narrow Range)	1, 2, 3	4	E
11) Containment Isolation Valve Position ^(g)	1, 2, 3	2 per penetration flow path ^{(d)(i)}	E
12) Containment Radiation (High Range)	1, 2, 3	2 upper containment 2 lower containment	F
13) RCS Pressurizer Level	1, 2, 3	3	E
14) Steam Generator (SG) Water Level (Wide Range) ^(g)	1, 2, 3	1/SG	E

(continued)

BASES

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5. Reactor Coolant System Pressure (Wide Range) (continued)

- to make a determination on the nature of the accident in progress and where to go next in the procedure.

RCS pressure is also related to three decisions about depressurization. They are:

- to determine whether to proceed with primary system depressurization;
- to verify termination of depressurization; and
- to determine whether to close accumulator isolation valves during a controlled cooldown/depressurization.

A final use of RCS pressure is to determine whether to operate the pressurizer heaters.

RCS pressure is a Type A variable because the operator uses this indication to identify events and to monitor the cooldown of the RCS following a steam generator tube rupture (SGTR) or small break LOCA. Operator actions to maintain a controlled cooldown, such as adjusting steam generator (SG) pressure or level, would use this indication.

6. Reactor Vessel Water Level

~~Reactor Vessel Water Level, a non-Type A, Category 1 variable, is provided for verification and long term surveillance of core cooling. It is also used for accident diagnosis and to determine reactor coolant inventory adequacy.~~ **Reactor Vessel Water Level, a Type A, Category 1 variable is provided for:**

- **SI re-initiation criteria,**
- **Pressurizer Level Control,**
- **Criteria for manually re-starting ECCS pumps,**
- **Criteria for closing CLA isolation valves,**
- **RCS Pressure Control,**
- **Verification and long term surveillance of core cooling,**
- **Accident diagnosis,**
- **Determination of reactor coolant inventory adequacy, and**
- **Pressurizer heater control**

(continued)

ENCLOSURE 2
List of Regulatory Commitments

1. This letter provides an update to Technical Specification (TS) Table 3.3.3-1, "Post Accident Monitoring Instrumentation," to delete footnote "(g)" from function 6), "Reactor Vessel Water Level." In addition, Technical Specification Bases 3.3.3 has been updated to state that function 6), "Reactor Vessel Water Level," is now a Type A, Category 1 variable. These changes will be incorporated as part of TS, Developmental Revision H at a later date.