



Monticello Nuclear Generating Plant
2807 W County Road 75
Monticello, MN 55362

January 17, 2012

L-MT-12-006
10 CFR 50.73

U.S. Nuclear Regulatory Commission
ATTN: Document Control Desk
Washington, DC 20555

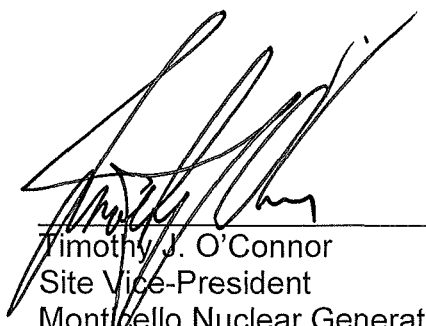
Monticello Nuclear Generating Plant
Docket 50-263
Renewed Facility Operating License No. DPR-22

LER 2011-009 "Automatic Reactor Scram While Performing Turbine – Generator Testing"

A Licensee Event Report (LER) for this occurrence is attached.

Summary of Commitments

This letter contains no new commitments and no revisions to existing commitments.



Timothy J. O'Connor
Site Vice-President
Monticello Nuclear Generating Plant
Northern States Power Company-Minnesota

Enclosure

cc: Regional Administrator, Region III, USNRC
Project Manager, Monticello Nuclear Generating Plant, USNRC
Resident Inspector, Monticello Nuclear Generating Plant, USNRC

1. FACILITY NAME: Monticello Nuclear Generating Plant
 2. DOCKET NUMBER: 05000 - 263
 3. PAGE: 1 OF 3

4. TITLE: Automatic Reactor Scram While Performing Turbine – Generator Testing

5. EVENT DATE			6. LER NUMBER			7. REPORT DATE			8. OTHER FACILITIES INVOLVED	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REV NO	MONTH	DAY	YEAR	FACILITY NAME	DOCKET NUMBER
11	19	2011	2011	009	00	01	17	2012		05000
									FACILITY NAME	DOCKET NUMBER
										05000

9. OPERATING MODE: 1

10. POWER LEVEL: 90%

11. THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check all that apply)

<input type="checkbox"/> 20.2201(b)	<input type="checkbox"/> 20.2203(a)(3)(i)	<input type="checkbox"/> 50.73(a)(2)(i)(C)	<input type="checkbox"/> 50.73(a)(2)(vii)
<input type="checkbox"/> 20.2201(d)	<input type="checkbox"/> 20.2203(a)(3)(ii)	<input type="checkbox"/> 50.73(a)(2)(ii)(A)	<input type="checkbox"/> 50.73(a)(2)(viii)(A)
<input type="checkbox"/> 20.2203(a)(1)	<input type="checkbox"/> 20.2203(a)(4)	<input type="checkbox"/> 50.73(a)(2)(ii)(B)	<input type="checkbox"/> 50.73(a)(2)(viii)(B)
<input type="checkbox"/> 20.2203(a)(2)(i)	<input type="checkbox"/> 50.36(c)(1)(i)(A)	<input type="checkbox"/> 50.73(a)(2)(iii)	<input type="checkbox"/> 50.73(a)(2)(ix)(A)
<input type="checkbox"/> 20.2203(a)(2)(ii)	<input type="checkbox"/> 50.36(c)(1)(ii)(A)	<input checked="" type="checkbox"/> 50.73(a)(2)(iv)(A)	<input type="checkbox"/> 50.73(a)(2)(x)
<input type="checkbox"/> 20.2203(a)(2)(iii)	<input type="checkbox"/> 50.36(c)(2)	<input type="checkbox"/> 50.73(a)(2)(v)(A)	<input type="checkbox"/> 73.71(a)(4)
<input type="checkbox"/> 20.2203(a)(2)(iv)	<input type="checkbox"/> 50.46(a)(3)(ii)	<input type="checkbox"/> 50.73(a)(2)(v)(B)	<input type="checkbox"/> 73.71(a)(5)
<input type="checkbox"/> 20.2203(a)(2)(v)	<input type="checkbox"/> 50.73(a)(2)(i)(A)	<input type="checkbox"/> 50.73(a)(2)(v)(C)	<input type="checkbox"/> OTHER
<input type="checkbox"/> 20.2203(a)(2)(vi)	<input type="checkbox"/> 50.73(a)(2)(i)(B)	<input type="checkbox"/> 50.73(a)(2)(v)(D)	Specify in Abstract below or in NRC Form 366A

12. LICENSEE CONTACT FOR THIS LER
 NAME: Carrie Fosaaen
 TELEPHONE NUMBER (Include Area Code): 763-295-1357

13. COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO EPIX	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO EPIX

14. SUPPLEMENTAL REPORT EXPECTED: YES (If yes, complete 15. EXPECTED SUBMISSION DATE) NO

15. EXPECTED SUBMISSION DATE: MONTH 02, DAY 29, YEAR 2012

ABSTRACT (Limit to 1400 spaces, i.e., approximately 15 single-spaced typewritten lines)

On November 19th, 2011, at approximately 2312 CST, during performance of regularly scheduled Turbine-Generator Quarterly Surveillance, an unplanned reactor scram occurred. Following the reactor scram, reactor water level lowered below the Group II isolation initiation setpoint (+9 in) and an actuation of Primary Containment Isolation System occurred.

The direct cause of the scram was the actuation of the Main Turbine acceleration relay (load rejection) pressure switches. The root cause is under investigation by the site. A supplement to this Licensee Event Report will be submitted following completion of the investigation and will outline corrective actions to address the root cause.

LICENSEE EVENT REPORT (LER)
CONTINUATION SHEET

1. FACILITY NAME	2. DOCKET	6. LER NUMBER			3. PAGE
Monticello Nuclear Generating Plant	05000 -263	YEAR	SEQUENTIAL NUMBER	REV NO.	2 OF 3
		2011	- 009	- 00	

NARRATIVE

Energy industry identification system (EISS) codes are identified in the text within brackets [xx].

EVENT DESCRIPTION

Prior to the event, Monticello Nuclear Generating Plant was in Mode 1 at approximately 90% power.

On November 19, 2011, at approximately 2312 CST, the plant scrammed while performing a Turbine – Generator Quarterly Surveillance Test, which tests the operation of the Speed/Load Changer and Turbine Bypass Valves [V]. The Speed/Load Changer was being lowered to close the Control Valves [V] and concurrently open the Bypass Valves when a Reactor half scram was received followed by a full Reactor scram due to both channels of the Turbine-Generator load reject trip relays [RLY] which receive their signal from oil pressure sensing switches [PIS]. Following the reactor scram, reactor water level lowered below the Group II isolation initiation setpoint (+9 in) and an actuation of Primary Containment Isolation System (PCIS) occurred.

Control Rods fully inserted as expected in response to the Reactor Protection System [JC] (RPS) actuation. Post scram, Reactor Vessel [RPV] water level was controlled using the Feedwater [SJ] and Condensate [SD] systems. No other safety systems actuated or were required to actuate. There was no inoperable equipment at the start of the event that contributed to the event. Off-site power was available and both Emergency Diesel Generators [DG] were operable and available. Crew recognition, response and decision making enabled effective management of the transient.

EVENT ANALYSIS

This event is being reported in accordance with 10 CFR 50.73(a)(2)(iv)(A) – System Actuation. This event is not considered a safety system functional failure.

CAUSE

The direct cause of the scram was the actuation of the Main Turbine acceleration relay (load rejection) pressure switches. The root cause investigation is in progress. A supplement to the Licensee Event Report will be submitted upon completion of the investigation.

SAFETY SIGNIFICANCE

The safety objective of both RPS and PCIS are to provide timely protection at the onset of conditions that could challenge the integrity of the fuel barrier and nuclear system process barriers. The RPS prevents the release of radioactive material from the fuel and nuclear system process barriers by terminating excessive temperature and pressure increases through the initiation of an automatic plant shutdown. PCIS prevents release of radioactive materials by isolating the reactor vessel and closing containment where required. For this event, the RPS, PCIS, and plant safety systems functioned as designed and fuel and nuclear system process barriers remained intact. Consequently, the event did not have an adverse impact on the health and safety of the public.

**LICENSEE EVENT REPORT (LER)
CONTINUATION SHEET**

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NARRATIVE

CORRECTIVE ACTIONS

The root cause investigation is in progress. A supplement to the Licensee Event Report will be submitted upon completion of the investigation and will outline corrective actions to address the root cause.

PREVIOUS SIMILAR EVENTS

There have been no similar licensee event reports in the past three years.