

#### UNITED STATES NUCLEAR REGULATORY COMMISSION WASHINGTON, D.C. 20555-0001

January 23, 2012

Mr. Joseph E. Pacher Vice President R.E. Ginna Nuclear Power Plant R.E. Ginna Nuclear Power Plant, LLC 1503 Lake Road Ontario, NY 14519

SUBJECT: REQUEST FOR ADDITIONAL INFORMATION RE: THE 2011 STEAM GENERATOR TUBE INSPECTIONS – R.E. GINNA NUCLEAR POWER PLANT (TAC NO. ME7597)

Dear Mr. Pacher:

By letter dated November 15, 2011, R.E. Ginna Nuclear Power Plant, LLC submitted the results of the 2011 steam generator tube inspections in the report titled, "Steam Generator Tube Inspection Report, End of Cycle 35 Refueling Outage, May 2011." This report was submitted in accordance with R.E. Ginna Technical Specification 5.6.7, "Steam Generator Tube Inspection Report."

The Nuclear Regulatory Commission staff has reviewed the information provided and has determined that additional information is needed to complete its review. Enclosed is the staff's request for additional information (RAI). As discussed with your staff, we understand that you intend to respond to this RAI by February 17, 2012.

Please contact me at 301-415-1364 if you have any questions.

Sincerely,

Dougla V Relat

Douglas V. Pickett, Senior Project Manager Plant Licensing Branch I-1 Division of Operating Reactor Licensing Office of Nuclear Reactor Regulation

Docket No. 50-244

Enclosure: As stated

cc w/encl: Distribution via Listserv

# **REQUEST FOR ADDITIONAL INFORMATION**

# 2011 STEAM GENERATOR TUBE INSPECTION REPORT

## R.E. GINNA NUCLEAR POWER PLANT, LLC

### R.E.GINNA NUCLEAR POWER PLANT

### DOCKET NO. 50-244

By letter dated November 15, 2011 (Agencywide Documents Access and Management System Accession No. ML11325A118), R.E. Ginna Nuclear Power Plant, LLC submitted information summarizing the results of the 2011 steam generator (SG) tube inspections performed at the R.E. Ginna Nuclear Power Plant. In order to complete its review of this document, the staff requests the following additional information:

- 1. During the 2011 inspection, 237 dents were observed in the tubes at the top of the tubesheet on the cold-leg side of the steam generator. During the 2008 inspection, 80 tubes were identified with dents at the top of the tubesheet on the cold-leg side of the steam generator. It was then indicated that there were 156 newly reported dents in the tubes at the top of the tubesheet on the cold-leg side of the steam generator. Please confirm these numbers (e.g., were there 81 dents in the 80 tubes with dents during the 2008 inspection?).
- 2. Regarding the denting at the top of the tubesheet, what is the voltage associated with these dents, what corrective action, if any, was taken to address the denting, and what insights, if any, were obtained from profiling the dents?
- 3. In Table 3.1.1, please clarify what constitutes the "central box area."
- 4. It was indicated that the divider plate in both SGs was inspected. Please clarify the materials of construction of the divider plate, the stub runner, if any, and the associated welds. Please discuss the results of these inspections.
- 5. Please discuss the nature of the restriction of the tube in SG A. For example, is the restriction service-induced, is it located in a low row tube, or has this tube been previously inspected successfully with a 0.610-inch bobbin probe?
- 6. Besides the flow accelerated corrosion in three of the secondary steam separators, was there any other degradation/anomalous conditions identified during the secondary side internals and upper bundle inspections?
- 7. Four indications of wear were identified and attributed to loose parts. Please discuss whether the parts that caused these wear indications were removed from the SGs. In addition, please discuss how these indications have changed with time. Please discuss the number of known loose parts or possible loose parts in the SGs and the source of these loose parts.

8. Please discuss whether there has been any change in the number of tubes in close proximity or the extent to which the affected tubes are in close proximity (e.g., decreased spacing or length of tubing in close proximity increasing).

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Sincerely, /RA/ Douglas V. Pickett, Senior Project Manager Plant Licensing Branch I-1 Division of Operating Reactor Licensing Office of Nuclear Reactor Regulation

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