



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

January 9, 2012

Mr. Thomas D. Gatlin
Vice President, Nuclear Operations
South Carolina Electric & Gas Company
Virgil C. Summer Nuclear Station
Post Office Box 88
Jenkinsville, SC 29065

SUBJECT: VIRGIL C. SUMMER NUCLEAR STATION, UNIT NO. 1 (SUMMER) –
REQUEST FOR ADDITIONAL INFORMATION (TAC NO. ME6879)

Dear Mr. Gatlin:

By letter dated August 16, 2011 (Agencywide Documents Access and Management System Accession No. ML11231A250), South Carolina Electric and Gas Company (SCE&G) submitted a proposed alternative to the American Society of Mechanical Engineers, *Boiler and Pressure Vessel Code* (ASME Code), Section XI inservice inspection (ISI) requirements regarding examination of certain reactor pressure vessel welds for Summer. In accordance with Title 10 of the *Code of Federal Regulations*, Part 50, Section 50.55a(a)(3)(i), and the Nuclear Regulatory Commission's (NRC's) safety evaluation approving the use of WCAP-16168-NP-A, Revision 2, "Risk-Informed Extension of the Reactor Vessel In-Service Inspection Interval," the NRC staff requires the following information to continue its review.

Regarding observed indications from the most recent ISI interval examinations at Summer, as documented in Table 2 of the Proposed Alternative, clearly state the location and size of the one indication that was found in the plate material of the reactor pressure vessel bellline area. Identify the plate material in which the indication was located and identify the inspection methodology used. Was this indication observed in previous ISI interval inspections? Did the size of the indication change during the course of the subsequent ISI inspection? If there was a change in the size of the indication, is that change attributed to improved inspection procedures or an aging mechanism?

We request that a response be provided within forty-five (45) days of the date of this letter.

Sincerely,

A handwritten signature in cursive script that reads "Robert Martin".

Robert E. Martin, Senior Project Manager
Plant Licensing Branch II-1
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket No. 50-395

cc: Distribution via Listserv

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Regarding observed indications from the most recent ISI interval examinations at Summer, as documented in Table 2 of the Proposed Alternative, clearly state the location and size of the one indication that was found in the plate material of the reactor pressure vessel beltline area. Identify the plate material in which the indication was located and identify the inspection methodology used. Was this indication observed in previous ISI interval inspections? Did the size of the indication change during the course of the subsequent ISI inspection? If there was a change in the size of the indication, is that change attributed to improved inspection procedures or an aging mechanism?

We request that a response be provided within forty-five (45) days of the date of this letter.

Sincerely,

/RA/

Robert E. Martin, Senior Project Manager
Plant Licensing Branch II-1
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

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*by email dated 12/30/11

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