

UNITED STATES NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

January 19, 2012

LICENSEE:

Exelon Generation Company, LLC

FACILITY:

LaSalle County Station, Units 1 and 2

SUBJECT:

SUMMARY OF DECEMBER 7, 2011, PUBLIC MEETING WITH EXELON GENERATION COMPANY, LLC REGARDING THE PROPOSED EXTENDED POWER UPRATE LICENSE AMENDMENT REQUEST FOR LASALLE COUNTY STATION, UNITS 1 AND 2 (TAC NOS. ME7495 AND ME7496)

On December 7, 2011, a Category 1 public meeting was held between the U.S. Nuclear Regulatory Commission (NRC) and representatives of Exelon Generation Company, LLC (EGC, the licensee) at the NRC Headquarters, Two White Flint North, 11545 Rockville Pike, Rockville, Maryland. The purpose of the meeting was to discuss a planned Extended Power Uprate (EPU) licensee amendment request (LAR) for LaSalle County Station, Units 1 and 2 (LaSalle). A list of attendees is provided as Enclosure 1.

During the meeting, EGC informed the NRR staff of their plans to submit a LAR for a 12.5 percent increase in licensed thermal power for LaSalle in the 3rd Quarter of 2012. The meeting discussion focused on review standards for annulus pressurization loads and emergency core cooling system (ECCS) net positive suction head (NPSH) analysis. The licensee presented slides contained in Enclosure 2 and discussed a pre-application checklist enclosed in the public meeting notice (Agencywide Documents Access and Management System (ADAMS) Accession No. ML113200075).

The meeting piloted the use of the Nuclear Energy Institute (NEI) developed pre-application checklist. The purpose of the pre-application meeting using the NEI checklist is for NRC staff and the licensees to reach common understanding of the regulatory criteria and standards to be applied in the review of significant licensing actions with a goal of enhancing the effectiveness and efficiency of the review process. A series of LaSalle EPU pre-application meetings are being planned for additional technical area topics. During each subsequent pre-application meeting, the NEI checklist will be used to support discussions. Additional information about the NEI checklist pilot process may be found in public meeting summary from November 2, 2011 (ADAMS Accession No. ML113210594).

The following actions were agreed to between the NRC and the EGC prior to submission of the LaSalle EPU LAR.

Summary of Follow-up Actions

- EGC and the NRC achieved an understanding of the methodology being proposed for use for determination of break Mass and Energy release and annulus pressurization calculations. The NRC acknowledged that the use of the Transient Reactor Analysis Code – G (TRACG) methodology was an acceptable approach.
- The TRACG evaluation for use in break Mass and Energy release and annulus
 pressurization calculations should also have a description identifying the differences in
 application of TRACG at LaSalle compared to its application in the economic simplified
 boiling-water reactor safety analysis and Grand Gulf's EPU application. EGC agreed to

- provide a summary of the differences along with a justification for use of the methodology in the EPU application.
- Based on recent Advisory Committee on Reactor Safeguards comments associated with the use of American National Standards Institute and American Nuclear Society (ANSI/ANS) 58.2-1988 for evaluating jet impingement loads, the NRC staff noted that the use of this standard as it applies to the proposed EPU may be inappropriate. The NRC staff also noted that ANSI/ANS 58.2-1988 is a withdrawn standard.
- EGC and the NRC achieved an understanding regarding the proposed methodology used to determine the ECCS NPSH. EGC's approach and use of current guidance was considered to be acceptable.
- Agreement was achieved regarding the use of 21 percent uncertainty when performing EPU ECCS NPSH calculations. It was noted that the uncertainty value may be impacted by Boiling-Water Reactor Owner's Group (BWROG) evaluations currently underway for non-containment accident pressure plants. Also, there may be an elimination of the "Maximum Erosion Zone" requirement. EGC agreed to address any final results of the BWROG evaluation prior to submittal in the application.
- EGC and the NRC achieved an understanding that the EPU application would address mixed core analysis or provide justification for excluding.

The meeting notice and agenda are available under ADAMS Accession No. ML113200075. The public was invited to observe the meeting. No members of the public were in attendance. Public Meeting Feedback forms were not received.

Please direct any inquiries to me at 301-415-1115, or Nicholas.DiFrancesco@nrc.gov.

Sincerely,

Nicholas DiFrancesco, Project Manager Plant Licensing Branch III-2 Division of Operating Reactor Licensing Office of Nuclear Reactor Regulation

Docket Nos. 50-373 and 50-374

Enclosures:

1. List of Attendees

2. Licensee Handout

cc w/encl: Distribution via ListServ

LIST OF ATTENDEES

DECEMBER 7, 2011, PUBLIC MEETING WITH EXELON GENERATION COMPANY, LLC REGARDING THE PROPOSED EXTENDED POWER UPRATE LICENSE AMENDMENT

REQUEST FOR LASALLE COUNTY STATION, UNITS 1 AND 2

NRC

Jake Zimmerman Nicholas DiFrancesco Araceli T. Billoch Colòn William Jessup John Huang Ogbonna Hopkins John Tsao Tom Alexion Sheldon Stuchell **Greg Casto** Rick Plasse Rick Stattel David Rahn Tai Huang Ahsan Sallman Tony Ulses Garry Armstrong

Muhammad Razzaque Nageswara Karipineni

Exelon Kenneth Ainger Kevin Borton John Rommel Harold Vinyard Tim Brom

Tim Byam Vikram Shah Terry Simpkin Faramarz Pournia Stevie Du Pont

GE-Hitachi

Curt Robert Bruce Hagemeier Sara Ruddy



LaSalle County Station

Pre-Application Meeting Extended Power Uprate

December 7, 2011

Introductions

Exelon Team

- Kenneth Ainger Project Management Director, EPU
- Kevin Borton Power Uprate Licensing Manager
- John Rommel Power Uprate Engineering Director
- Harold Vinyard LaSalle Engineering Director
- Tim Byam Power Uprate Lead Licensing Engineer
- Vikram Shah Power Uprate Senior Engineering Manager
- Terry Simpkin LaSalle Regulatory Assurance Manager
- Faramarz Pournia Power Uprate Project Manager
- Stevie Du Pont Power Uprate Licensing Engineer



Agenda and Meeting Purpose

- Introduce LaSalle EPU Staff
- Describe NEI Pre-Submittal Meeting Pilot
- Present LaSalle Extended Power Uprate Schedule and Approach
- Describe Key Aspects of Technical Evaluations and Obtain Feedback
 - Annulus Pressurization Loads
 - ECCS NPSH Analysis
- Discuss Potential Topics for Future Meetings



NEI Pilot - Pre-submittal Meetings

- Purpose is to enhance License Amendment Request pre-submittal meetings
 - Reach a common understanding on the regulatory criteria and standards to be applied during the NRC review of the proposed changes
 - Identify potential application issues that can be addressed during the application conceptual phase that will reduce acceptance review time, requests for additional information, and application review time

Process

- Pilot Checklist is used to focus on applicable review criteria, codes, standards, justification required for use of a new analytical method, applicability of a precedent, or feasibility of a desired schedule in order to reach alignment with the NRC
- NRC meeting notice and meeting summary will docket the expectations and outcomes of the alignment in order to greatly reduce the risk and uncertainty associated with future application acceptance and NRC review
- LaSalle specific checklist focus
 - Verify methodology and approach used for analyses of Annulus Pressurization Load
 - Verify current NRC expectations and approach regarding ECCS NPSH calculations



Background

- LaSalle Original License
 - Unit 1 licensed 1982 / Unit 2 licensed 1983
 - Original Licensed Thermal Power (OLTP) of 3323 MWt per unit
- LaSalle Previous Uprates
 - Stretch Power Uprate of 5% in 2000 to 3489 MWt
 - MUR Uprate of 1.6% in 2010 to 3546 MWt
 - Current Licensed Thermal Power (CLTP) of 3546 MWt
- EPU
 - Projected Power Uprate level of 3988 MWt (increase ~12.5% of current licensed power or 120% of original licensed power)



Schedule Overview

NRC Communication Schedule

2nd Pre-Submittal Meeting:

Target February 2012

3rd Pre-Submittal Meeting:

Target April 2012

Final Pre-Submittal Meeting:

Target June 2012

EPU Implementation Schedule

Submit LAR:

Target 3rd QTR 2012

LAR Approval:

Target 1st QTR 2014 (20 months)

• Unit 2 Implementation:

2nd QTR 2015 (Outage L2R15)

• Unit 1 Implementation:

1st QTR 2016 (Outage L1R16)

Steam Dryer Evaluation could impact above schedule



EPU LAR Approach

- LAR will meet criteria in NRC RS-001, "Review Standard for Extended Power Uprates"
- Evaluations supporting the LAR were performed using Constant Pressure Power Uprate (CPPU) Licensing Topical Report (NEDC – 33004P-A) (commonly called CLTR)
 - Fuel related evaluations were performed to the guidance in NRC-approved NEDC-32424P-A (commonly called ELTR1)
 - Safety issues identified in ELTR1 that should be addressed in a plant-specific EPU license amendment request are addressed in the LaSalle Specific Power Uprate Safety Analysis Report (PUSAR) (NEDC-33603P)
 - For generically evaluated issues the PUSAR references the NRC-approved generic evaluations in either ELTR1 or ELTR2 (NEDC-32523P-A)
- No Submittals Linked to Proposed EPU Submittal
- Incorporated Past RAIs



EPU LAR Approach

Exelon's submittal will include

- Cover Letter and Amendment Request
- Attachments
 - Description/Evaluation of Proposed Changes including No Significant Hazards Consideration
 - Markup of Operating License and Technical Specifications
 - Markup of Technical Specifications Bases and Technical Requirements Manual (Information Only)
 - Power Uprate Safety Analysis Report (PUSAR) (non- proprietary, proprietary, and affidavit)
 - Regulatory Commitments
 - Supplemental Environmental Report
 - · List of Modifications
 - EPU Startup Test Plan
 - Grid Stability Study
 - PRA Report
 - Flow Induced Vibration (FIV) Piping and Component Evaluation
 - Instrument Setpoint Calculations Affected by EPU
 - Steam Dryer Evaluation (High Cycle Fatigue Report) (non-proprietary, proprietary, and affidavit)



Exelon Nuclear

Annulus Pressurization Loads Analysis

Vikram Shah

Annulus Pressurization (AP) Loads Analysis

Purpose

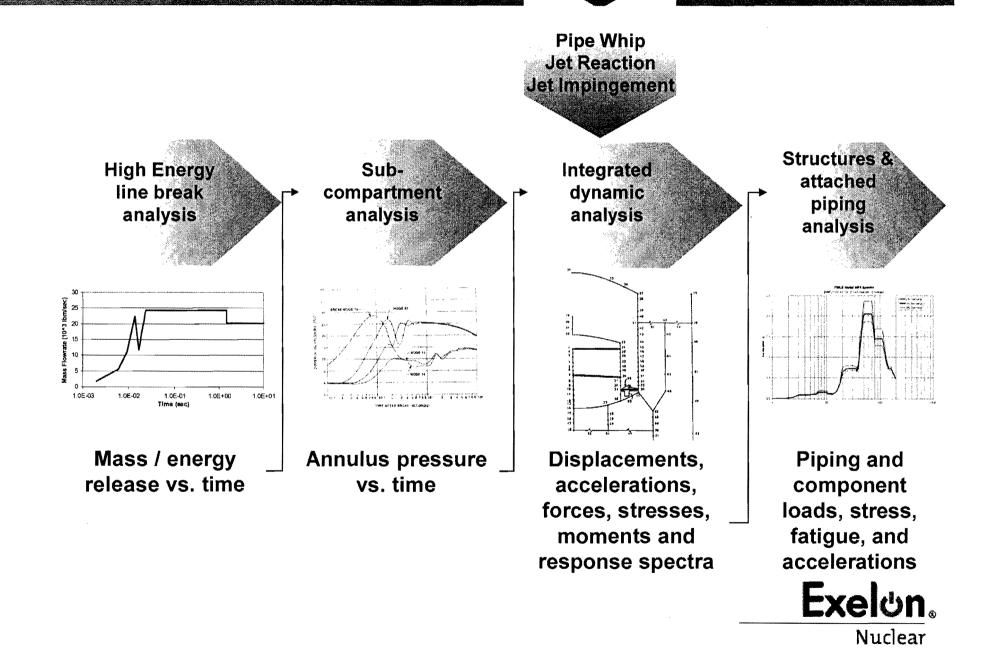
- Verify methodology and approach used for analyses of Annulus Pressurization Load
 - Perform the EPU break mass and energy release (M&E) and pressurization calculations for the annulus pressurization using the GEH TRACG computer code
 - TRACG based M&E release methodology for the AP loads will address GEH corrective action of the Safety Communication SC09-01

Methodology

- Current CLTP Methodology
 - NEDO-24548, Annulus Pressurization Load Adequacy Evaluation
 - Computer Code: RELAP4/ MOD 3
- EPU Methodology
 - NEDE-32176P, Revision 4, TRACG Model Description
 - NEDE-32177P, Revision 3, TRACG Qualification
 - NEDE-33083P-A, TRACG Application for ESBWR, October 2005
 - NEDE-33440P, Revision 2, TRACG ESBWR Safety Analysis Additional Information, March 2010
 - Computer Code: TRACG V.04
- Same computer code was used in the Grand Gulf EPU Submittal to address AP loads



Annulus Pressurization (AP) Loads Analysis Overview



Annulus Pressurization Loads Analysis Methods Comparison

Evaluation	CLTP Method	EPU Method	Comparison / Comments
Mass and Energy Release	Generic NEDO- 24548	TRACG 04	Note 1
Annulus Pressurization Loads	RELAP4/MOD 3	TRACG 04	Note 2
Jet Loads	ANSI 176 (draft)	ANSI/ANS 58.2- 1988	Same method, Old standard superseded.
Pipe Whip Restraint	PDA	PDA	Same
Dynamic Structural Analysis	SAP4G07	SAP4G07	Same

- Note 1 The TRACG 04 allows the calculation of mass and energy (M&E) release rates to include the physical attributes of the piping system for both rated and off-rated conditions. TRACG eliminates unphysical and artificially imposed jumps in mass and energy. Providing estimates of M&E at off-rated and rated conditions addresses the concerns identified in GEH Safety Communication SC 09-01, Annulus Pressurization Loads Evaluation, dated June 8, 2009
- Note 2 The use of the TRACG 04 vessel component together with a fine mesh model (336 nodes) of the LaSalle annulus provides a more detailed annulus pressurization response than the analysis of record, which uses a coarse node (35 node) RELAP 4/MOD 3 model.



Annulus Pressurization (AP) Loads Analysis

Summary

- The application of TRACG for both the mass and energy release analysis and the annulus pressurization analysis is appropriate to provide a response frequency used in all downstream load analyses
- SC 09-01 will be addressed by analyzing the pipe breaks considered in the LaSalle design and licensing basis at various rated and off-rated operating conditions with bounding conditions being used in the downstream analysis
- Confirm NRC agreement that the above approach to calculation of AP Loads is acceptable



Exelon_®

ECCS NPSH Analysis

Vikram Shah

Purpose:

Verify methodology and Exelon's approach to perform NPSH analysis for a non-CAP credit plant is in accordance with NRC's expectations, and draft regulatory guidance

Methodology

- SECY 11-0014, Use of Containment Accident Pressure in Analyzing Emergency Core Cooling System and Containment Heat Removal System Pump Performance in Postulated Accidents
 - Event-specific analyses determine containment response
 - Suppression Pool Temperature
 - Event-specific NPSH_a determined for each applicable event/pump
 - NPSH_a compared to NPSH_{reff} for each applicable pump
 - Time in Maximum Erosion Zone



Key Assumptions

- No Containment Accident Pressure
- Deterministic analysis with conservative inputs for DBA-LOCA, ASDC and Small Break LOCA.
- Nominal inputs for non-design basis events (SBO, Appendix R, ATWS) analyses
- Vendor supplied NPSHr 3% curves
- 21% uncertainty for NPSHr_{eff} for DBA-LOCA, ASDC and Small Break LOCA events
- 0% uncertainty for non-design basis events (ATWS, SBO, Appendix R)
- Assumes minimum Suppression Pool Inventory (level) for all events
- All events are evaluated at 102% of EPU power



Preliminary Results

Event	Pump	NPSHa (Feet)	NPSHr _{eff} (Feet)	Margin (Feet)
DBA-LOCA	RHR	18.8	16.9	1.9
DBA-LOCA	HPCS	19.7	6.1	13.6
DBA-LOCA	LPCS	19.0	2.4	16.6
ASDC	RHR	17.8	16.9	0.9
ASDC	HPCS	17.5	6.1	11.4
ASDC	LPCS	18.0	2.4	15.6
SBO	RHR	19.2	14.0	5.2
SBO	RCIC	Analysis in Progress		
SBO	HPCS	18.9	5.0	13.9
ATWS	RHR	32.4	14.0	18.4
App R	RHR	25.9	14.0	11.9
App R	HPCS	25.7	5.0	20.7
App R	LPCS	26.1	2.0	24.1

NPSHr_{eff} values consistent with draft guidance (NPSHr_{eff} = NPSHr3% + uncertainties) including 21% uncertainty for Design Basis Events and 0% uncertainty for non-Design Basis Events

No modifications are required to achieve the above results



Summary

- Exelon's approach to perform NPSH analysis is in accordance with NRC expectations and draft guidance
 - Demonstrates that adequate positive margin exist for ECCS/RCIC pumps
 - Demonstrates that ECCS/RCIC pumps will perform their safety functions
- Confirm NRC agreement that the above approach to determine ECCS NPSH is acceptable



Future EPU Meeting Topics and Schedule

- Follow-up EPU Meetings
 - Proposed Topics
 - Steam Dryer Strategy and FIV Analysis
 - Impact and Changes to Human Factors
 - Impact on Primary Containment internal pressure (P_a)
 - Ultimate Heat Sink Analysis
 - Setpoint Calculations
 - Alternate Source Term Analysis
- Next meeting target February 2012



Meeting Summary

- Pilot Alignment and Outcome
 - Discussion
 - Checklist Mark-up

Critique



Acronym List

- ASDC Alternate Shut Down Cooling
- ATWS Anticipated Transient Without Scram
- CAP Containment Accident Pressure
- CLTP Current Licensed Thermal Power
- DBA Design Basis Accident
- ECCS Emergency Core Cooling System
- EPU Extended Power Uprate
- ESBWR Economic Simplified Boiling Water Reactor
- FIV Flow Induced Vibration
- GEH General Electric Hitachi
- HPCS High Pressure Core Spray
- LAR License Amendment Request
- LOCA Loss of Coolant Accident
- LPCS Low Pressure Core Spray
- LTR Licensing Topical Report
- MUR Measurement Uncertainty Recapture power uprates
- MWt Mega Watts thermal
- NEI Nuclear Energy Institute
- NPSH Net Positive Suction Head
- NPSHa Net Positive Suction Head available
- NPSHr Net Positive Suction Head required
- NPSHreff Effective Net Positive Suction Head required
- PRA Probabilistic Risk Assessment
- PUR Power Uprate
- RAI Request for Additional Information
- RHR Residual Heat Removal
- SBO Station Black Out
- SC GEH Safety Communication
- SECY Commission Papers (Written issues papers the NRC staff submits to the Commission to inform them about policy, rulemaking, and adjudicatory matters)



- provide a summary of the differences along with a justification for use of the methodology in the EPU application.
- Based on recent Advisory Committee on Reactor Safeguards comments associated with the use of American National Standards Institute and American Nuclear Society (ANSI/ANS) 58.2-1988 for evaluating jet impingement loads, the NRC staff noted that the use of this standard as it applies to the proposed EPU may be inappropriate. The NRC staff also noted that ANSI/ANS 58.2-1988 is a withdrawn standard.
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Sincerely,

/RA/

Nicholas DiFrancesco, Project Manager Plant Licensing Branch III-2 Division of Operating Reactor Licensing Office of Nuclear Reactor Regulation

Docket Nos. 50-373 and 50-374

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