McCarver, Sammy

From: Polasek, Patrick J [ppolase@entergy.com]
Sent: Friday, September 10, 2010 5:37 AM
To: Cataldo, Paul
Subject: PTRG Report for IP3-2010-02682
Attachments: ipcpr029000.PDF

PTRG report as requested.

Patrick J. Polasek
Manager, Nuclear Information Technology
Entergy Nuclear Operations, Inc.
Indian Point Energy Center
Generation Support Building - 2nd Floor
450 Broadway
Buchanan, New York, 10511-0249
914-271-7238 Office
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Information in this record was deleted in accordance with the Freedom of Information Act, exemptions b6
FOIA- 2011-0272

b7

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Unit No.: 3 Transient (CR) No.: IP3-2010-02682

Transient Description: Unit 3 manually tripped due to water leak in the Exciter housing

Transient Date: 9/9/10 Transient Time: 21:29

A. Attachments:

- [x] Attachment 10.1, POST TRANSIENT EVALUATION COVER SHEET
- [x] Attachment 10.2, GENERAL INFORMATION
- [x] Attachment 10.3, DATA SUMMARY
- [x] Attachment 10.4, EVENT ANALYSIS
- [x] Attachment 10.5, POST TRANSIENT REVIEW GROUP SUMMARY AND RECOMMENDED ACTIONS
- [ ] Attachment 10.6, ITRG TRANSIENT SUMMARY Not required or recommended
- [ ] Attachment 10.7, OSRC REVIEW
  - [ ] Required prior to Restart/ Recovery
  - [ ] IF OSRC review is required THEN ISSUE CA to ensure Post Transient Evaluation is reviewed by OSRC within 14 days of approval date.
- [ ] Attachment 10.8, OM REVIEW AND APPROVAL

B. Reviews and Approvals

- [x] Report final

PTRG Chairperson: Patrick J. Pulaski Date: 9/10/10

- [ ] All recommendations properly resolved for restart/recovery AND all required regulatory approvals have been obtained.

OSRC Review Meeting No.:_____ Date:_____

Assistant Operations Manager:_____ Date:_____

Operations Manager:_____ Date:_____

C. Plant Restart Recovery Authorized ☐ YES ☐ NO ☐ N/R

GMPO:_____ Date: _____

Unit No.: 3 Transient (CR) No.: IP3-2010-02682

Transient Description: Unit 3 manually tripped due to water leak in the Exciter housing

Transient Date: 9/9/10 Transient Time: 21:29

NOTE

- The tracking number for the trip OR transient SHALL use the CA number that required the event investigation.
- The report without the data package SHALL be attached to the CA that required the PTRG.

General Trip/Transient Information

1.0 Reactor Trip/Transient Actuated By: Manual ✓ Auto _____

1.1 Reason for Manual Trip/Transient: Water leak in the Exciter housing (Service Water from the Exciter coolers)

1.2 Reason for the Auto Trip/Transient (include component/system): _____

2.0 Plant Personnel On Shift (print names)

SM ~~Tom Ras~~ @ Nick Lizzo
STA/WE Vin DeClemente
CRS Ron Carpino
FSS John Graham
RO Chris Nilssen
BOP Chee Yun
*NPO
*Other Ralph Orzo

* If involved with the Trip/Transient Event

3.0 *Notifications*

All notifications SHALL be made in accordance with IP-SMM-LI-108/ AP-8.3/ AP-21 and documented in the plant logs.

Comments: (YES) NO LI-108 Attachment 10.2 attached

4.0 Emergency Plan Implemented? YES (NO)

IF yes, THEN give classification and explain:

5.0 Systems Identified with Inadequate Performance

Document in General Information and Data Gathering, the plant parameters immediately prior to and after the trip or transient and identify any abnormal performance of plant systems and components.

34 RCP trip
IR-35, 36 did not trend per expectations

6.0 Off Normal Status of Any Trains/Portions of Safety Systems

32 diesel tagged out for maintenance

7.0 Unusual or Abnormal Status of Other Plant Equipment, Systems

None

8.0 List any AOTs/LCOs in effect

See attached.

9.0 Testing/Surveillance/Maintenance in Progress (pertinent to transient)

None

10.0 Plant Status

☐ Start-Up in progress ☐ Shutdown in progress ☒ Power Operation ☐ Other (Explain)

Other Informative Details:

	Maximum	Minimum
NIS Power Ranges	100	0
CET Temperature	640°F	553°F
RCS Pressure	2234 psig	2007 psig
RCS Temperature	570°F	543°F
VC Pressure	0.5 psig	0.12 psig
VC Temperature	111.5°F	111.5°F

11.0 For reactor trips, obtain the "Sequence of Events" and "Post Trip Logs" per Section 6.2.1.1 **{Reference 3.13}**

Completed: [initials] (Initials) 9/16/10 (Date) 0230 (Time)

Other Informative Details:

1.0 Initial Plant Conditions Just Prior to Trip/Transient Phase 1, Part 1

1.1 Plant Parameters

Reactor Power Level	100	%
Electrical Output	1073	MWe
RCS Temperature (Tave)	569.3	°F
RCS Pressure	2252	psig
Pressurizer Level	46.2	%
Control Bank Position	Bank D Steps 230	
RCS Boron	671	ppm
Condenser Vacuum	27.86	In. Hg

1.2 Source and Status of Offsite Power

2A4|10|P

STA Aux:	(I/S)	OOS	STBY	GT Substation:	(I/S)	OOS	(STBY)
APP R DG:	I/S	OOS	(STBY)	N/A			

480 Volt Buses/EDGs:

2A:	(I/S)	OOS	STBY	EDG 22/(31)	I/S	OOS	(STBY)
3A:	(I/S)	OOS	STBY				
5A:	(I/S)	OOS	STBY	EDG 21/(33)	I/S	OOS	(STBY)
6A:	(I/S)	OOS	STBY	EDG 23/(32)	I/S	(OOS)	STBY

6.9 KV Buses

1:	(I/S)	OOS	STBY	2:	(I/S)	OOS	STBY
3:	(I/S)	OOS	STBY	4:	(I/S)	OOS	STBY
5:	(I/S)	OOS	STBY	6:	(I/S)	OOS	STBY

1.3 Plant Equipment in Operation (circle appropriately)

Reactor Coolant Pumps	21 (31)	22 (32)	23 (33)	24 (34)		
Charging Pumps	21/ 31	22/ 32	23/ 33			
Main Boiler *Feed Pumps*	21 (31)	22 (32)				
Aux. Boiler Feed Pumps	21/ 31	22/ 32	23/ 33 Stby.			
Condensate *Pumps*	21 (31)	22 (32)	23 (33)			
Feed Reg. Valves	Main	Low Flow	Aux. Feed			
Heater Drain *Tank Pumps*	21 (31)	22 (32)				
Condensate Polisher	Partial	Full Flow	(NIS)	N/A		
C.P. Vessels in Service	A	B	C	D	E	F
Circulating Water Pumps	21/ (31)	22/ (32)	23/ (33)	24/ (34)	25/ (35)	26/ (36)
Circulating Water Pump Speed	350	360	345	365	350	355

1.4 Status of Controllers (circle appropriately)

Rod Control	(AUTO)	MANUAL		
Condenser Steam Dumps	(AUTO)	MANUAL	TEMPERATURE	PRESSURE
RCS Makeup	(AUTO)	MANUAL		
Charging Pumps	(AUTO)	MANUA L		
PRZR Pressure Control Channel	1			
PRZR Level Control Channel	2			
Main Feed Reg. Valves in AUTO	21/ (31)	22/ (32)	23/ (33)	24/ (34)
Steam Generator SF/FF:	Steam Flow Channel		Feed Flow Channel	
21/ 31 S/G	A	(B)	A	(B)
22/ 32 S/G	A	(B)	A	(B)
23/ 33 S/G	A	(B)	A	(B)
24/ 34 S/G	A	(B)	A	(B)

1.5 PORV/ Block Valve Status (circle as appropriate)

535 RCS-MOV-535	OPEN	CLOSED	TRIP PULLOUT	DE-ENERGIZED
536 RCS-MOV-536	OPEN	CLOSED	TRIP PULLOUT	DE-ENERGIZED
PCV-455C RCS-MOV-455C	OPEN	CLOSED	AUTO	
PCV-456 RCS-MOV-456	OPEN	CLOSED	AUTO	

1.6 Steam Generator Chemistry Parameters During Transient

8/9 8:33

Chloride (highest)	0.35	ppb	Time	8:33
Cation Conductivity (highest)	1.71	µmho/cm	Time	8:33
Specific Conductivity (highest)	17.3	µmho/cm	Time	8:33
Sodium (highest)	0.32	ppb	Time	8:33
	Maximum		Minimum	
S/G Pressure	1005	psig	740	psig
S/G Level	42	%	0	%

1.7 RCS Parameters During Transient

1.7.1 RCS Pressure remained above setpoint for automatic SI actuation	YES	NO	N/A
1.7.2 RCS Pressure remained below setpoint for PRZR PORV or code safety valve actuation	YES	NO	N/A
1.7.3 RCS temperature decreased less than 100 °F/hr	YES	NO	N/A
1.7.4 Indicated PRZR level remained on scale	YES	NO	N/A

IF any of the above are NO, THEN EXPLAIN: _____

1.8 **ATTACH and CHECK below** sequence of events printout and any other printout, chart, or data which can be used to determine the cause of the reactor transient or to show any abnormal condition during transient or transient recovery. **(Reference 3.13)**

✓ Sequence of Events

_____ Alarm Typewriter Printout (Unit 3)

_____ Trend Typewriter Printout (Unit 3)

_____ Log Typewriter Printout (Unit 3)

_____ Plant Information Computer System (Unit 2)

_____ S/G SF/FF Level Charts/ Paperless Recorders

_____ Reactor Power Charts/ Paperless Recorders

_____ RCS Pressure Chart/ Paperless Recorder

_____ PRZR Pressure Chart/ Paperless Recorder

_____ PRZR Level Chart/ Paperless Recorder

_____ RCS Temp. Chart/ Paperless Recorder

_____ Condenser Vacuum Chart/ Paperless Recorder

✓ Unit Log

_____ NPO Logs

_____ Other Logs or Charts (Chemistry, RP, etc.)

Plant response to Event Phase 1, Part 2 (Reference 3.2)

1.0 All Reactor Trip Breakers OPEN (YES) NO N/A

1.1 All Rod Bottom Lights (YES) NO N/A

1.2 Actuation Time Time Last Breaker Open 21:28:54 N/A

Time of Initial Trip Signal 21:28:54

Difference 40 milliseconds.

1.3 Reactor Trip First Out Annunciator MANUAL TRIP

OR

1.4 Turbine Trip First Out Annunciator _____

1.5 *Did all Control Rods fully insert?*

- **(Unit 2)** A rod is considered fully inserted if rod bottom light is lit and IRPI indicates < 12 steps. YES NO (N/A)
- ***(Unit 3)*** *A rod is considered fully inserted if rod bottom light is lit and IRPI indicates* < 20 steps. (YES) NO N/A

1.6 IF any control Rods did NOT insert, THEN:

- RECORD the rod numbers that did NOT insert: _____
- INITIATE a CR
- CONTACT Reactor Engineering and DIRECT them to assess the operability of the affected rod(s) based on available data and performance trends of the rod(s)
- *NOTIFY Reactor Engineering to evaluate rod drop times and rod recoil and* determine whether any tests need to be conducted.

NOTE

(Unit 3 Only) The opening of the Trip Breakers can be obtained from the Sequence of Events print out. The initial trip time will be equal to zero milliseconds, and Reactor Trip Breaker opening is obtained from address Y0007D and Y0006D

1.7 What was the digital point and its description which initiated the Sequence of Events printout?

21:28:54 sec. Reactor MANUAL Trip

1.8 What was the real time corresponding to elapsed time = 0 on the Sequence of Events printout?

21:28:54 sec.

1.9 *Did either Reactor Trip Breaker open in 150 milliseconds or greater?*

RECORD Time Breaker A 40 milliseconds

Breaker B 34 milliseconds

1.10 IF 150 milliseconds or greater, THEN INITIATE a CR and CONTACT I&C.

1.11 Did both Reactor Trip Breakers open within the following time periods from the time of the trip signal indication?

1.11.1 45 – 92 milliseconds (for auto trip) YES NO (N/A) MANUAL Trip

1.11.2 145 –190 milliseconds (for SI initiated trip – time from SI signal to trip breaker opening)

YES NO (N/A) MANUAL Trip NO SI

2.0 Feedwater Response:

Feedwater Isolation	(YES)	NO	N/A
Aux. Feedwater Auto Start	(YES)	NO	N/A

3.0 SI Initiated On Demand? YES (NO) N/A

3.1 IF yes THEN CHECK safety features actuated

SI Train A		Cont. Spray	
SI Train B		M.S. Isolation	
Cont. Isol. Phase A		IVSWS	
Cont. Isol. Phase B		480V EDG Start	

3.1 Did SI cause actual injection into the core? YES (NO) N/A

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3.2 *Did any EDG fail to perform properly?* YES (NO) N/A

IF yes, THEN EXPLAIN: _____

3.3 Did any system fail to function properly? YES NO N/A

IF yes, THEN EXPLAIN: 34 RCP Trip Stand Pipe high Level NO SCAL Return flow.

4.0 Did Pressurizer pressure and level fail to respond normally? YES (NO) N/A

IF yes, THEN EXPLAIN: _____

5.0 Did any primary system fail to function properly? YES (NO)

IF yes, THEN EXPLAIN: _____

6.0 *Did any secondary system fail to function properly?* YES (NO)

IF yes, THEN EXPLAIN: _____

7.0 Any unusual conditions during Rx transient or transient recovery? YES (NO) N/A

IF yes, THEN EXPLAIN: _____

8.0 Any unusual or unexplained annunciators during transient or transient recovery? (YES) NO

IF yes, THEN EXPLAIN: Source Range Loss of Detector Voltage IN/OUT 34 Stand Pipe ~20" High Level, Low RCP Seal return flow.

9.0 Did Pressurizer PORV and Safetys fail to perform properly? YES (NO) N/A

IF yes, THEN EXPLAIN: _____

10.0 Did any Main Steam Safety Valves lift on demand? YES (NO) N/A

IF yes, THEN EXPLAIN: _____

11.0 Control System Response

11.1 Turbine Runback YES NO (N/A)

IF yes: Runback from _____ MW to _____ MW

Rx Power from _____ % to _____ %

12.0 Does Initiating Event agree with:

12.1 First Out Alarm Panel ? (YES) NO

12.2 Control Room Indications (YES) NO

IF NO, THEN EXPLAIN: _____

13.0 Any radiological release: YES (NO)

IF yes, THEN EXPLAIN: _____

Chronology of Events

TIME	EVENT DESCRIPTION
2105 *	BOP (RO) notified by SM of Service water leak
~ 2115 *	BOP (RO) informed CESO and reduced MVARS to unity
~ 2123 *	Attempts to isolate leak unsuccessful
2129	Manual trip due to Service Water leak in Generator housing

See IP3 Unit Log for subsequent actions / descriptions

* approximate time based on event recollection forms.

Description of Event (ATTACH additional sheets as necessary)

SM notified to BOP (RO) of service water leak into the exciter enclosure. Attempts to isolate leaks were unsuccessful and reports indicated that the leak was getting worse. Manual Reactor trip at 2129

IF more space is required, THEN DUPLICATE Page

Section 1: **Preliminary Safety Assessment**

After reviewing the data NOTE here any indications that are unexpected or abnormal.

34 RCP tripped

34 RCP ~~Seal leak~~ #2 Seal hung open (cocked)

IR-35, 36 did not trend per expectations - off scale low

31 MBFP relief valve lifted, CO2 discharged

Section 2 **Initiating Cause for Reactor Trip/Transient**

manual reactor trip initiated due to SW leakage into exciter enclosure

Section 3 **Plant Parameters Consistent with FSAR, Techincal Specifications, Technical Requirements Manual, Design Basis Documents, and Safety Limits for the Type of Transient**

Yes

Section 4 **Unexpected Aspect of Trip/Transient Behavior**

See Section 1

Section 5: ***Personnel Performance Problems***

None identified

Section 6 **Was conservative decision-making evident during the event?**

Yes - manually tripped reactor when attempts to isolate leak failed was conservative decision making

Section 7 **Procedural Problems**

None observed

Section 8 **Were actions taken outside established procedures? Were they documented and evaluated? Was a CR initiated? (Reference 3.3)**

No.

Operators were OK

Section 9: **Any Detrimental effect on Plant Equipment**

34 RCP seal

Section 10 **Any data unsupported by other diverse indications**

No.

Section 11 **Additional Remarks (explain any degraded trends or equipment responses, unanticipated alarms or indications, repeat transients, any areas of concern, or extent of condition)**

See section none.

Section 12 **Any further reviews/evaluations needed?** 1) Determine why ~~Wanted~~ CO-99-2 lifted

2) Understand 34 RCP fast transient trip

3) Determine why CO2 discharge ~~Wanted~~

4) Determine cause of 34 RCP seal leak

Section 13 **CORRECTIVE ACTIONS**

Section 13A: **For Reactor Trips: Is corrective action recommended prior to returning to power?**

(YES) NO N/A

Specify Recommended Corrective Actions prior to returning to power:

mode 1 • Isolate exciter leak and repair

Mode 2 • ~~Repair~~ Restore 34 RCP seals

mode 2 • Return to normal operations IR-35, 36

• Ensure CO-99-2 is understood and repair as necessary

• Restore or establish compensatory measures as required for CO2

Section 13B:

Specify Recommended Corrective Actions after returning to power: (Include a CA to Simulator Support to benchmark and review the event for simulator and plant differences)

None

Section 14: **Plant Transient Response:** : IF post-transient response is determined to be abnormal, OR there were non-conservative actions/decisions made, OR the PTRG feels the need to have an additional review for any reason, THEN the PTRG can recommend that an ITRG be convened. The basis for this recommendation must be explained below:

NORMAL ABNORMAL

Is the ITRG recommended: NO YES

IF yes, THEN EXPLAIN: _____

Section 15: **Recommended Changes to Plant Status** (e.g., Proceed to Cold Shutdown)

Hold in Mode 3 until recommended actions are completed.

Section 16 **Indicate below any additional information or remarks that could be used to reconstruct the event or identify the cause of the transient.**

None

Section 17: Event Determination

1.0 Is the cause of the trip/ transient positively known (to the extent of a component, e.g., a fuse that requires detailed laboratory analysis for type of failure need NOT be known at the time of Trip/Transient Review) and has been corrected OR appropriate corrective action restraints are in place AND either of the following:

1.1 All safety-related and other important equipment functioned properly during the transient

OR

1.2 IF any safety-related or other important equipment did NOT function properly during the transient, THEN the equipment malfunction has either been corrected OR appropriate corrective action restraints are in place OR redundant equipment is available.

(YES) NO

2.0 IF the answer to question 1.0 (above) is YES, THEN review by an Independent Transient Review Group (ITRG) is NOT required due to the cause of the trip. OSRC review is required within 14 days of SVP signature)

3.0 IF the answer to question 1.0 (above) is NO, THEN review by ITRG and OSRC is required prior to startup. **{Reference 3.14}**

4.0 Was a safety limit violated during this transient?

YES (NO)

5.0 IF the answer to question 4.0 (above) is YES, THEN ITRG, OSRC, and NRC approval is required prior to Startup.

Plant Transient Review Group Members

Patrick J. Polasek	IT	9/10/10 0312
PTRG Chairperson	Department	Date/ Time
Joe Reynolds	CA&A	9/10/10 0312
Member	Department	Date/ Time
Art Singer	Training	9/10/10 0312
Member	Department	Date/ Time
Tom Ras	Operations	9/10/10 0312
Member	Department	Date/ Time
Tat Chan	Eng	9/10/10 0312
Member	Department	Date/ Time
Kevin Curley	Eng	9/10/10 0312

Operations Review and Approval

Reviewed: _____

Manager, Operations — Date/ Time

OM Review and Approval

Is ITRG Required? YES NO

Is ITRG Required prior to restart? YES NO

List approved PTRG Recommended Actions (have PTRG member create and attach CR sheets)

_____

_____

_____

_____

_____ _____

Operations Manager — Date/ Time

1.0 Unit No.: _____ Transient (CR) No.: _____

2.0 Transient Date: _____ Transient Time: _____

3.0 Adequacy and Accuracy of PTRG Evaluation: _____

4.0 Adequacy and Accuracy of PTRG Initiating Cause Determination: _____

5.0 Adequacy and Accuracy of PTRG Evaluation of Plant, Procedure, and Personnel Performance: _____

6.0 Evaluation of Impact of Criteria Requiring the ITRG: _____

7.0 Safety Significance of Event: _____

_____

_____

_____

_____

8.0 Recommended Corrective Actions:

Prior to Recovery Restart: _____

_____

_____

_____

_____

Following Recovery Restart: _____

_____

_____

_____

_____

9.0 ITRG Members:

Name	Signature	Date
RCA Experienced Individual		
Licensing		
System Engineering Manager		

Page 1 of 1

Unit No.: _____ Transient (CR) No.: _____

Transient Date: _____

OSRC Meeting Number: _____ Date: _____

Detail any pertinent OSRC comments below:

_____

_____

_____

_____

_____

_____

NOTE

OSRC Review Required within 14 days of Post Transient Evaluation Approval\Date

Completion of Independent Review:

1. Forward a copy of this independent review to CA&A for the CR file.
2. Attach CR forms for all actions that resulted from the independent review and which will be carried against the CR for this event.

Presented by:

Name Signature Date

_____

Assistant Operations Manager/ designee

Page 1 of 1

All required reviews complete	YES	NO	
The initiating cause of the transient has been identified	YES	NO	
Safety-related and other essential equipment has functioned properly during the transient if required, and if NOT, corrective maintenance has been completed and the equipment satisfactorily tested OR appropriate restraints are in place.	YES	NO	
Provisions for additional monitoring of plant equipment or personnel have been provided if required	YES	NO	
Extent of condition adequately addressed	YES	NO	
The plant is in a condition to restart/recovery and is consistent with the requirements of the FSAR, Technical Specifications, Technical Requirements Manual, and station procedures	YES	NO	
If a technical specification safety limit exceeded, NRC approval obtained	YES	NO	N/A
Plant Restart/ Recovery Recommended	YES	NO	

Name Signature Date/Time

Operations Manager

Entergy	CONDITION REPORT	CR-IP3-2010-02682

Originator: Lewis,Matthew W
Originator Phone: 8281
Originator Site Group: IP3 Operations Mgmt IP3
Operability Required: Y
Supervisor Name: Dinelli,John
Reportability Required: Y
Discovered Date: 09/09/2010 21:54
Initiated Date: 09/09/2010 22:06

Condition Description:

Unit 3 was manually tripped at 21:29 due to water leak in the Exciter housing (Service Water, from the Exciter Coolers). Anomalies noted on the plant trip: 34 RCP tripped during the 6.9KV Bus transfer (6.9KV Bus 2; Bus 2 did re-energize via UT2/ST5 tie breaker and Bus 2 did not receive an undervoltage), 31 MBFP suction relief valve lifted causing a CO2 discharge on 31 MBFP (subsequently reseated and CO2 secured).

Immediate Action Description:

Entered E-0 and ES-0.1.

Suggested Action Description:

Additional parameters noted on 34 RCP following the trip: Standpipe high level alarm, loss of seal return flow on both indicators, RCDT trending up abnormally fast (frequent pump-downs); possible #2 seal failure.

Entergy	CORRECTIVE ACTION	CR-IP3-2010-02682

CA Number: 1

	Site	Group	Name
Assigned By:	IP3	Info Technology Mgmt IP3	Polasek,Patrick J
Assigned To:	IP3	Info Technology Mgmt IP3	Polasek,Patrick J

Subassigned To :

Originated By: Polasek,Patrick J 9/9/2010 23:27:37

Performed By:

Subperformed By:

Approved By:

Closed By:

Current Due Date: 09/13/2010 **Initial Due Date:** 09/13/2010

CA Type: ACTION

Plant Constraint: MODE 1/PWR OPERATION

CA Description:

Perform a PTRG on the September 9, 2010 manual reactor trip.

Response:

Subresponse :

Closure Comments:

Entergy	CORRECTIVE ACTION	CR-IP3-2010-02682

CA Number: 2

	Site	Group	Name
Assigned By:	IP3	Info Technology Mgmt IP3	Polasek,Patrick J
Assigned To:	IP3	Info Technology Mgmt IP3	Polasek,Patrick J
Subassigned To :			

Originated By: Polasek,Patrick J 9/9/2010 23:36:26

Performed By:

Subperformed By:

Approved By:

Closed By:

Current Due Date: 09/23/2010 **Initial Due Date:** 09/24/2010

CA Type: ACTION

Plant Constraint: #NONE

CA Description:

In accordance with IP-SMM-OP-105, section 6.5.7, ensure OSRC review is completed within 14 days of the PTRG approval date.

Response:

Subresponse :

Closure Comments:

Entergy	CORRECTIVE ACTION	CR-IP3-2010-02682

CA Number: 3

	Site	Group	Name
Assigned By:	IP3	Info Technology Mgmt IP3	Polasek,Patrick J
Assigned To:	IP3	Training Mgmt IP3	Robenstein,Richard

Subassigned To :

Originated By: Polasek,Patrick J 9/9/2010 23:45:44

Performed By:

Subperformed By:

Approved By:

Closed By:

Current Due Date: 11/29/2010 **Initial Due Date:** 11/30/2010

CA Type: ACTION

Plant Constraint: #NONE

CA Description:

In accordance with procedure IP-SMM-OP-105, benchmark the event for simulator and plant differences.

Response:

Subresponse :

Closure Comments:

Entergy	CORRECTIVE ACTION	CR-IP3-2010-02682

CA Number: 4

	Site	Group	Name
Assigned By:	IP3	Info Technology Mgmt IP3	Polasek,Patrick J
Assigned To:	IP3	Operations Watch Mgmt IP3	Dinelli,John
Subassigned To :			

Originated By: Polasek,Patrick J 9/10/2010 00:06:03
Performed By:
Subperformed By:
Approved By:
Closed By:

Current Due Date: 10/21/2010 **Initial Due Date:** 10/22/2010
CA Type: ACTION
Plant Constraint: #NONE

CA Description:
In accordance with procedure, IP-SMM-OP-105, section 6.7.3, Operations to assemble an operations performance review (OPRC) to evaluate crew performance during the transient.

Response:

Subresponse :

Closure Comments:

Entergy	CORRECTIVE ACTION	CR-IP3-2010-02682

CA Number: 5

	Site	Group	Name
Assigned By:	IP3	Info Technology Mgmt IP3	Polasek,Patrick J
Assigned To:	IP3	Maint Mech Mgmt IP3	Bouderau,Gregory

Subassigned To :

Originated By: Polasek,Patrick J 9/10/2010 03:36:13

Performed By:

Subperformed By:

Approved By:

Closed By:

Current Due Date: 09/13/2010 **Initial Due Date:** 09/13/2010

CA Type: ACTION

Plant Constraint: MODE 1/PWR OPERATION

CA Description:

** MODE 1 ** Isolate exciter leak and repair.

Response:

Subresponse :

Closure Comments:

Entergy	CORRECTIVE ACTION	CR-IP3-2010-02682

CA Number: 6

	Site	Group	Name
Assigned By:	IP3	Info Technology Mgmt IP3	Polasek,Patrick J
Assigned To:	IP3	System Eng Primary System Staff IP3	Chan,Tat
Subassigned To :			

Originated By: Polasek,Patrick J 9/10/2010 03:39:20

Performed By:

Subperformed By:

Approved By:

Closed By:

Current Due Date: 09/11/2010 **Initial Due Date:** 09/11/2010

CA Type: ACTION

Plant Constraint: MODE 2/STARTUP-CRITI

CA Description:

** MODE 2 ** Restore 34 RCP seals.

Response:

Subresponse :

Closure Comments:

Entergy	CORRECTIVE ACTION	CR-IP3-2010-02682

CA Number: 7

	Site	Group	Name
Assigned By:	IP3	Info Technology Mgmt IP3	Polasek,Patrick J
Assigned To:	IP3	System Eng Elec/I&C Mgmt IP3	Andreozzi,Vincent Jos
Subassigned To :			

Originated By: Polasek,Patrick J 9/10/2010 03:42:18

Performed By:

Subperformed By:

Approved By:

Closed By:

Current Due Date: 09/11/2010 **Initial Due Date:** 09/11/2010

CA Type: ACTION

Plant Constraint: #NONE

CA Description:

Determine the cause of 34 RCP trip.

Response:

Subresponse :

Closure Comments:

Entergy	CORRECTIVE ACTION	CR-IP3-2010-02682

CA Number: 8

	Site	Group	Name
Assigned By:	IP3	Info Technology Mgmt IP3	Polasek,Patrick J
Assigned To:	IP3	Maint Elect Mgmt IP3	Lijoi,John J
Subassigned To :			

Originated By: Polasek,Patrick J 9/10/2010 03:44:42

Performed By:

Subperformed By:

Approved By:

Closed By:

Current Due Date: 09/11/2010 **Initial Due Date:** 09/11/2010

CA Type: ACTION

Plant Constraint: MODE 2/STARTUP-CRITI

CA Description:

*** MODE 2 *** Return to normal operations IR-35, 36.

Response:

Subresponse :

Closure Comments:

Entergy	CORRECTIVE ACTION	CR-IP3-2010-02682

CA Number: 9

	Site	Group	Name
Assigned By:	IP3	Info Technology Mgmt IP3	Polasek,Patrick J
Assigned To:	IP3	System Eng Secondary System Mgmt IP3	Vasely,Michael J

Subassigned To :

Originated By: Polasek,Patrick J 9/10/2010 03:47:19

Performed By:

Subperformed By:

Approved By:

Closed By:

Current Due Date: 09/13/2010 **Initial Due Date:** 09/13/2010

CA Type: ACTION

Plant Constraint: #NONE

CA Description:

Ensure CD-99-2 is understood and repair as necessary.

Response:

Subresponse :

Closure Comments:

Entergy	CORRECTIVE ACTION	CR-IP3-2010-02682

CA Number: 10

	Site	Group	Name
Assigned By:	IP3	Info Technology Mgmt IP3	Polasek,Patrick J
Assigned To:	IP3	P&C Eng Codes Mgmt IP3	Troy,Michael J

Subassigned To :

Originated By: Polasek,Patrick J 9/10/2010 03:49:11

Performed By:

Subperformed By:

Approved By:

Closed By:

Current Due Date: 09/13/2010 **Initial Due Date:** 09/13/2010

CA Type: ACTION

Plant Constraint: #NONE

CA Description:

Restore or establish compensatory measures as required for CO2.

Response:

Subresponse :

Closure Comments:

Entergy	ADMIN	CR-IP3-2010-02682

Remarks Description:

Closure Description:

ATTACHMENT 10.2 **NOTIFICATIONS**

GENERAL INSTRUCTIONS

Do not delay in making a required NRC immediate notification. If unable to contact an Entergy individual, proceed to the next appropriate step and try to contact the individual again after the attachment is completed.

INITIAL NOTIFICATIONS

1. IF an Emergency Event, enter the Emergency Plan IP-EP-120, "Emergency Plan Classifications" and make required notifications (1-Hour NRC hotline notification is required).

2. IF a Non-Emergency Event (1-hr, 4-hr, 8-hr, or 24-hour NRC hotline notification is required), THEN NOTIFY the NRC Operation Center by the first of the following which ever is successful within the time limit specified in Attachment 10.1:

Telephone System	Telephone Number
a. Emergency Notification System (Control Room Telephone)	Lift Receiver
b. NRC Operations Center Commercial Telephone	
1. Primary	(301) 816-5100/ 800-532-3469
2. Backup (1st)	301-951-0550/800-449-3694
3. Backup (2nd) & 3rd	301-415-0550/301-415-0553

Maintain a continuous open communications channel with the NRC when requested.

NOTE

AS AN AID TO THE NOTIFICATION PROCESS, INFORMATION MAY BE FAXED TO THE NRC OPERATIONS CENTER AT **(301) 816-5151 AFTER THE PHONE NOTIFICATION IS COMPLETED.** A FAX SHALL NOT BE USED IN LIEU OF TELEPHONE NOTIFICATION.

3. For contaminated spills/leaks or release of hazardous substances notify as follows:

   a. ~~For radiological contaminated spills and leaks notify radiation protection and if reporting is required~~ ensure IPEC Communications and the Corporate Duty Manager is notified with direction to notify ENN Government Affairs and ENN Corporate Communications with the need for them to inform local, state and municipality stakeholders of the contaminated spill/leak. Notification of State, County, Town should be within one business day (See EN-RP-113).

   b. For releases of a hazardous substance in transportation accidents involving radioactive material that meet the criteria of Title 40 CFR 302.6, EPA Notification Requirements and Title 40 CFR 355.40, Emergency Planning and Notification, refer to IPEC procedure OAP-039 (Transportation Incidents Involving Radioactive Material). Reporting requirements are also contained in 6 NYCRR Part 597.2, Table 1 and Title 49 CFR, DOT, Subpart C, Hazardous Material, Part 171.15 and 171.16.

4. If NRC Region I is specified for a Non-Emergency event on Attachment 10.1, THEN notify the NRC Region Office by the first of the following means which is successful within the time limit specified in Attachment 10.1:

USNRC Regional Office	(610) 337-5000
Via NRC Operations Center	(301) 816-5100

5. For ALL notifications:

   a. NOTIFY the first of one of the following:

Name	Work	Home	Beeper/Mobile
Joseph E. Pollock (Site VP -IPEC)	734-6700, 734-6701		845-803-5785
Anthony Vitale (GMPO)	734-5221	(b)(6)	(b)(6)
Anthony Williams (OM)	598-5162		

EXEMPTION 6

OCC

   b. Notify the Corporate Duty Manager, WPO in accordance with corporate policy EN-OM-128.

Name	Work	Home	Beeper/Mobile
Corporate Duty Manager	N/A	N/A	(b)(6)

EXEMPTION 6

2158

NOTE

The individual contacted shall ensure the remaining personnel in step 5.a are notified as required.

   c. IF unable to contact one of the individuals in Step 5.a, contact the following "designated manager":

Name	Work	Home	Beeper/Mobile
~~Pat Conroy~~	~~734-6668~~	(b)(6)	(b)(6)

N/A

EXEMPTION 6

The designated manager contacted shall ensure the personnel in Step 4.a are notified.

   d. Notify Communications by the first of one of the following:

Name	Work	Home	Beeper/Mobile
Jerry Nappi	271-7132	(b)(6)	(b)(6)
Andrea Blizard	271-7081		

EXEMPTION 6

Voice mail

e. Notify Emergency Planning by the first of one of the following:

Name	Work	Home	Beeper/Mobile
Brian Sullivan	271-7479	(b)(6)	
Frank Phillips	271-7170		

EXEMPTION 6

f. Ensure Notification of the other Indian Point Unit Control Rooms

Unit 2 numbers - (b)(6)
Unit 3 numbers -

EXEMPTION 6

NOTE

For items of interest or of management discretion - the individual contacted in step 5.a or step 5.b shall review the event circumstances and explicitly determine the need for further reporting and advise directly the SM as to the reporting required.

6. For reportable events notify an NRC Resident Inspector.

a. For an Item of Interest, notify an NRC Resident Inspector ONLY if told to do so by the individual contacted in Step 5.a or Step 5.c.

b. For Management Discretion, notify an NRC Resident Inspector if Item 80 is being reported.

c. IF the Senior Resident Inspector CANNOT be notified, call the alternate inspector:

Name	Work	Home	Beeper/Mobile
Unit 2 – Resident Inspector ~~Odunayo Ayegbusi~~	Int; 734-5347 ~~Ext: 914-739-8585~~		(b)(6)
Unit 3 – Resident Inspector Paul Cataldo	Int: 734-5347 Ext: 914-739-8585		

EXEMPTION 6

2204

e. IF the NRC Resident Inspectors cannot be contacted at work or at home, THEN Notify the NRC Resident Inspector by dialing 739-8565, 739-9360 and leave a recorded message.

IP3 Unit Log
Friday, September 10, 2010

Midnight
OAP-5

9/9/2010 08:00:00	32 EDG CO2 system placed in manual IAW SOP-FP-3 to support 32 EDG outage [Dignam, John, FSS]
9/9/2010 18:01:00	The following actions have been completed for STA turnover as per OAP-002: Reviewed the following: Shift Orders, Unit Log (since last watch or 5 days), AOT/ODCM Tracking Log, Standing Orders, Policy book and a Control Board walkdown with the off-going STA. [DeClemente, Vinnie, Shift Technical Advisor]
9/9/2010 19:00:00	PLANT STATUS: Unit on line @ 100%, 3-POP-2.1 in effect, 1-2-3 Essential SW Header, Load Limit 1 in control, CPF bypassed via Post Filter Bypass. Aux steam inservice to Unit 2. EQUIPMENT OUT OF SERVICE: R-5, R-56C OOS. CET's R-10 & K-13 (Temp. Alt. R-10 removed from alarm ckt) OOS. Diesel Fire Pump PTO'd. 32 PZR BU Htr. Group Ckt #1 PTO'd. RCP Seal Water Return Filter PTO'd & bypassed. SFP Filter PTO'd & Demin bypassed for filter replacement. CO2 Tank 3-1 (North Tank) PTO'd & vented. CO2 tank 3-2 (SouthTank) Low level alarm is up. FSB Fan PTO'd w/T. Alt removing fan from alarm ckt. 31 Primary Water Pump PTO'd. OTHER: Rx Vessel Outer O-ring I/S. IA/AA x-tied. TCV-1103 bypass open to lower VC temp. LCV-1129 in "Man" @ 10% open for CST temp control. Temp Alt. 31 EDG DF-LCV-1207B (disables limit switch for valve). SGBD I/S to SGBDR @ ~20 gpm each. GE Mobile I/S to CST @ 20 gpm. PROTECTED EQUIPMENT: Electric Fire Pump, Feeders 95891, 96951, 96952, 95331, 95332, 13W92 and 13W93, 32 Primary Water Pump, 31 & 33 EDG's. [Yun, Cheehun, BOP Operator and key 211]
9/9/2010 19:16:00	Performed 50 gallon dilution in accordance with 3-SOP-CVCS-003, Reactor Coolant System Boron Concentration Control, to offset core burn up and maintain Reactor Power optimized near 100 %. Total anticipated Tave effect is less than 0.1 degree F. [Nilsson, Chris, Operator at the Controls]
9/9/2010 20:40:00	Dilution reactivity addition effect complete. Less than a 0.1 degree F Tave change. [Nilsson, Chris, Operator at the Controls]
9/9/2010 20:08:52	STA Vital Area Tours completed as per OAP-115, Operations Commitments and Policy Details. [DeClemente, Vinnie, Shift Technical Advisor]
9/9/2010 21:00:00	Placed 32 Monitor Tank on Recirc @ 83 %. [Nilsson, Chris, Operator at the Controls]
9/9/2010 21:10:00	Start 3-PT-Q028, Containment Isolation Valves PCV-1190, PCV-1191, and 1192 Pressure Relief System test. [Martin, Marc, Nuclear NPO and key 215]
9/9/2010 21:20:00	3-PT-Q028, Containment Isolation Valves PCV-1190, PCV-1191, and 1192 Pressure Relief System test completed - Sat. [Martin, Marc, Nuclear NPO and key 215]
9/9/2010 21:29:00	Manual Reactor Trip due to Service Water Leak in Generator Housing. 34 Reactor Coolant Pump Tripped immediately following Reactor Trip. [Hedges, Luke, MISC]

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IP3 Unit Log
Friday, September 10, 2010

Midnight
OAP-5

	9/9/2010 21:30:00	34 RCP tripped when Reactor was tripped. [Yun, Cheehun, BOP Operator and key 211]
	9/9/2010 21:31:00	34 Spray valve isolated. [Yun, Cheehun, BOP Operator and key 211]
	9/10/2010 02:16:24	No entry text specified. [Hedges, Luke, MISC]
	9/9/2010 21:33:00	CO2 Discharge at 31 MBFP. Cause determined to be high temperature resulting from lifting of the 31 MBFP suction relief valve. [Hedges, Luke, MISC]
	9/9/2010 21:35:00	Transitioned to ES 0.1 Reactor Trip Response [Hedges, Luke, MISC]
	9/9/2010 21:38:00	Entered 3-TS -10-2845 due to CO2 inoperable due to low level in South CO2 tank (57%), Security providing continuous fire watch in 15' & 33" Control Building & Roving Fire watch in EDG cells Technical Specifications Unit: 3 Type: 3TRM Section: 3.7.A.7 Action Statement: 3TRM_3_3.7.A.7_A (2), Unit: 3 Type: 3TRM Section: 3.7.A.7 Action Statement: TRM_3_3.7.A.7_B (1) [DeClemente, Vinnie, MISC]
	9/9/2010 21:39:00	345 KV Motor Operated Disconnect Switch F1-3 Open [Hedges, Luke, MISC]
	9/9/2010 21:42:00	Both MBFPs verified tripped [Hedges, Luke, MISC]
	9/9/2010 21:48:00	Condenser Steam Dumps transferred to Pressure Control Mode. [Hedges, Luke, MISC]
	9/9/2010 21:50:00	Observed no Seal Return Flow from 34 RCP. [Hedges, Luke, MISC]
	9/9/2010 21:52:00	31 and 32 Source Range Detectors are energized with Source Range Loss Of Detector Voltage Alarm still in alarm. 31 and 32 Intermediate Range Detectors observed to be pegged low. Reference CR-IP3-2010-02686 [Hedges, Luke, MISC]
	9/9/2010 21:53:00	Source Range Loss Of Detector Voltage Clear. [Hedges, Luke, MISC]
	9/9/2010 21:57:00	Received Source Range Loss Of Detector Voltage Alarms. 31 and 32 Source Range Instruments observed to be energized. [Hedges, Luke, MISC]
	9/9/2010 21:55:00	Secured 32 and 33 Condensate Pumps [Hedges, Luke, MISC]
	9/9/2010 21:59:00	Opened turbine drain valves [Hedges, Luke, MISC]
	9/9/2010 22:00:00	MSR Drain Tank Non-Return Check Valve 33A did not close as expected. [Hedges, Luke, MISC]

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IP3 Unit Log
Friday, September 10, 2010

Midnight
OAP-5

	9/9/2010 22:00:00	Reheat Steam Block Valves Closed. [Hedges, Luke, MISC]
	9/9/2010 22:18:00	33A is closed [Hedges, Luke, MISC]
	9/9/2010 22:00:00	Performed 3-PT-W019, Electrical Verification - Offsite Power Sources and AC Distribution - Sat. Performed due to the inoperability of 32 EDG due to being PTO'd for Maintenance. [Yun, Cheehun, BOP Operator and key 211]
	9/9/2010 22:50:35	Completed 1 hour notifications IAW SMM-LI-108 to the following stakeholders: NRC resident Paul Cataldo PSC Paul Eddy CDM Sam Davis Power Marketing Joe Zirella [Dewey, Donald, MISC]
	9/9/2010 23:58:00	Placed the Main Turbine Generator on the Turning Gear in accordance with 3-SOP-TG-TG-001, Main Turbine Turning Gear Operation. [Nilsson, Chris, Operator at the Controls]
	9/9/2010 23:58:00	Main Turbine on the Turning Gear. [Hedges, Luke, MISC]
	9/10/2010 00:05:00	Start 3-PT-V1, Source Range Analog Channel Functional Test. [Nilsson, Chris, Operator at the Controls]
	9/10/2010 00:47:00	3-PT-V1, Source Range Analog Channel Functional Test completed - Sat. [Nilsson, Chris, Operator at the Controls]
	9/10/2010 00:07:00	Received "VC Sump Pump Running" alarm due to VC Sump Pumpout. Per 3-ARP-009 actions: Watch Chemist notified, from Unit Log last VC pumpout occurred on 09/05/10 @ 16:35 hrs. R11, R12, VC Humidity, and FCU Weir levels all normal. [Nilsson, Chris, Operator at the Controls]
	9/10/2010 00:10:00	"VC Sump Pump Running" alarm clear. VC Sump Pumpout Secured. [Nilsson, Chris, Operator at the Controls]
	9/10/2010 00:14:00	Informed by Unit 2 CCR that 138kV and 13.8kV voltages are Sat. [Nilsson, Chris, Operator at the Controls]
	9/10/2010 00:20:00	Secured 32 Rod Drive MG set. [Yun, Cheehun, BOP Operator and key 211]
	9/10/2010 00:29:16	Initiated 4 & 8 hour verbal report to the NRC (Howie Crouch) for today's reactor trip and aux boiler feed pump auto start. NRC issued the event notification #46241 and recorded the notification time as 00:29. [Dewey, Donald, MISC]
	9/10/2010 00:52:00	Energized VC Lighting and completed PAB prerequisites for VC Entry per OAP-007. [Banse, John, MISC]

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IP3 Unit Log
Friday, September 10, 2010

Midnight
OAP-5

	9/10/2010 01:36:00	VC lighting secured due to VC entry completed. 46' inner crane wall cages verified secured upon VC exit. [Banse, John, MISC]
	9/10/2010 01:30:00	Secured 32 and 35 Service Water pumps. [Yun, Cheehun, BOP Operator and key 211]

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Allowed Outage Time for Indian Point 3

LCO Number	Section Number	Start Date	Required Date	Status	Required Action	Initiating Items
3-ODCM-06-0094	Radiation Monitor 56C being taken out of service due to spiking causing spurious auto diversions. (PLT Report Submitted 11/20/2006). The Monitor is currently disconnected to prevent alarming. On 7/26/09 Channel failures on R-56A, B, &C also rendered R-56 A & B non-functional per ONOP-RM-001. Ref WR 00170944 & CR IP3-2009-03201. R-56A and B are returned to service operable on 7/30/09 at 16:21. Refer to WO 00164743-01.					
	3ODCM R-56A/B.	10/17/2007 1:44:00 AM	10/18/2007 2:40:49 AM	Exited 10/17/2007 10:47:00 AM	direct the chemist to take grab samples daily from the combined system for gross activity.	3-PRM -IXMITR-RD-56C
	3ODCM R-56A/B.	10/24/2006 8:30:00 PM	11/23/2006 8:30:00 PM	Exited 11/23/2006 12:00:00 PM	explain why this could not be done to the Plant Leadership Team (PLT)	
	3ODCM R-56A/B.	10/24/2006 8:30:00 PM	11/23/2006 8:30:00 PM	Exited 11/20/2006 3:59:00 PM	explain why this could not be done to the Plant Leadership Team (PLT)	
	3ODCM R-56A/B.			Potential Future Action	direct the chemist to take grab samples daily from the combined system for gross activity.	
	3ODCM R-56A/B.	7/26/2009 7:19:00 PM	7/31/2009 10:45:48 AM	Exited 7/30/2009 4:21:00 PM	direct the chemist to take grab samples daily from the combined system for gross activity	
3-PTS -08-0414	R-5 did not perform all of its required function ***** R-5 also has a Part 21 issue against it. Refer to CR-IP3-2009-3590 ***** Unit 3FSB Exhaust fan has been rebuilt, however vibrations have been trending up steadily, on 8/31/10 Performance recommended no longer running this fan, the Exhaust fan is inoperable. FSAT32 inlet damper replacement					
	3TS 3.3.8.A.1			Potential Future Action	A.1 Place FSBEVS in operation.	3-ARM -COMPTR-R-5 3-HVFS -DAMPER-FSAT32 INLET
	3TS 3.3.8.A.2			Potential Future Action	A.2 Suspend movement of recently irradiated fuel in the fuel storage building.	3-HVFS -FAN -FSBEF-FAN
	3TS 3.7.13.A.1			Potential Future Action	A.1 Suspend movement of recently irradiated fuel assemblies in the fuel storage building.	
	3TRM 3.7.F.B.1			Potential Future Action	B.1 Restore Operabiltiy	
	3TRM 3.7.F.B.2			Potential Future Action	B.2 Isolate the FSBEVS when moving fuel in the FSB	
	3TRM 3.7.F.C.1			Potential Future Action	C.1 Suspend movement of irradiated fuel in the associated building.	
	3TRM 3.7.F.D.1			Potential Future Action	Present a report to OSRC on planned corrective action.	
3-PTS -08-0416	Triennial Fire Inspection revealed CCR prompt evacuation or fire in CCR may prevent securing PORVs by tripping 31/32 125VDC distribution panel breakers, and no RNO is provided. Hourly fire watch for all levels Control bldg, Electrical tunnels and EDG's.					
	3TRM 3.7.B.OTF_	4/17/2009 3:42:00 AM	9/13/2010 7:03:21 AM	Open 83.6 hrs left	I.1 Verify Fire Watches are being conducted as prescribed	
	3TRM 3.7.B.B.1	4/9/2009 7:25:00 AM	4/9/2009 8:25:00 AM	~~Exited 4/9/2009~~ 7:26:00 AM	B.1 Establish an hourly fire watch in the Fire Watch Area(s) designated in Table 3.7.B-1 for the inoperable Function.	
	3TRM 3.7.B.A.1	2/8/2008 4:46:00 PM	2/8/2008 4:46:00 PM	Exited 2/8/2008 4:46:00 PM	A.1 Enter the Condition(s) referenced in Table 3.7.B-1,	

Allowed Outage Time for Indian Point 3

LCO Number	Section Number	Start Date	Required Date	Status	Required Action	Initiating Items
	3TS 3.2.2.A.1.2.1			Potential Future Action	A.1.2.1 Reduce THERMAL POWER to < 50% RTP.	
3-PTS -09-0526	31 Source range Unable to set High Flux @ shutdown					
	3TS 3.9.2.A.1			Potential Future Action	A.1 Suspend CORE ALTERATIONS.	3-RPC -IBISSW-NC-31-101
	3TS 3.9.2.A.2			Potential Future Action	A.2 Suspend positive reactivity additions.	
	3TS 3.9.2.B.1			Potential Future Action	B.1 Initiate action to restore one source range neutron flux monitor to OPERABLE status.	
	3TS 3.9.2.B.2			Potential Future Action	B.2 Perform SR 3.9.1.1.	
3-PTS -10-0572	Fire detector FP-DET-2-11 (CS-2-11) - Cable Spreading Room (FDCP ZONE 2) - Failed as per PT-SA13					
	3TRM 3.7.A.4.A.1			Potential Future Action	A.1 Establish an hourly fire watch patrol, where accessibility permits, in the affected location(s).	3-FP -XMITR-FP-DET-2-11
	3TRM 3.7.A.4.A.2			Potential Future Action	A.2 Restore the required fire detectors to OPERABLE status.	
3-PTS -10-0582	MET TOWER DIESEL GENERATOR out of service due to trip on high temperature during 3PT-M47					
	3TRM 3.3.B.A.1			Potential Future Action	A.1 DEMONSTRATE the ability to obtain meteorological data, using IP-EP-510.	3-MTDG - -MET-DIESEL GEN
	3TRM 3.3.B.A.2			Potential Future Action	A.2 Notify IP2 of system inoperability.	
	3TRM 3.3.B.A.3			Potential Future Action	A.3 Restore the inoperable Meteorological Instrument Channel to OPERABLE status.	
	3TRM 3.3.B.B.1			Potential Future Action	B.1 Prepare and submit a Special Report to the On-Site Safety Review Committee outlining the actions taken, the cause of the inoperability and the plans for restoring the meteorological monitoring instrumentation channel(s) to OPERABLE status.	
3-PTS -10-0583	Flux drive 'D' inoperable. The drive is not producing reliable traces for Flux Calculations. Refer to CR-IP3-2010-2015 and WR 00206100.					
	3TS 3.2.1.A.1			Potential Future Action	A.1 Reduce THERMAL POWER ? 1% RTP for each 1% FQ(Z) exceeds limit.	3-INCO -TOOL -NDDM-F
	3TS 3.2.2.A.1.1			Potential Future Action	A.1.1 Restore FNDH to within limit.	
	3TS 3.2.2.A.1.2.1			Potential Future Action	A.1.2.1 Reduce THERMAL POWER to < 50% RTP.	
	3TS 3.9.2.A.1			Potential Future Action	A.1 Suspend CORE ALTERATIONS.	
	3TS 3.9.2.A.2			Potential Future Action	A.2 Suspend positive reactivity additions.	

Allowed Outage Time for Indian Point 3

LCO Number	Section Number	Start Date	Required Date	Status	Required Action	Initiating Items
	3TRM 3.3.E.B.1			Potential Future Action	B.1 Prepare and submit a Special Report, "Inoperable Seismic Monitoring Instrumentation," PORC, outlining the cause of the malfunction and the plans for restoring the instrument(s) to OPERABLE status.	
	3TRM 3.3.E.B.1	9/9/2009 9:00:00 AM	9/19/2009 9:00:00 AM	Exited 9/19/2009 9:00:00 AM	B.1 Prepare and submit a Special Report, "Inoperable Seismic Monitoring Instrumentation," PORC, outlining the cause of the malfunction and the plans for restoring the instrument(s) to OPERABLE status.	
3-TS -10-2579	32 EDG inop for Work Week 1037					
	3TS 3.8.1.B.1	9/9/2010 7:23:00 AM	9/9/2010 10:00:29 PM	Open 2.6 hrs left	B.1 Perform SR 3.8.1.1 for the offsite circuits.	3-EDG -ENGINE-DE-32
	3TS 3.8.1.B.2	9/9/2010 7:23:00 AM	9/9/2010 11:23:00 AM	Exited 9/9/2010 7:27:00 AM	B.2 Declare inoperable the required features supported by the inoperable DG when its required redundant feature is inoperable.	
	3TS 3.8.1.B.3.1	9/9/2010 7:23:00 AM	9/10/2010 7:23:00 AM	Exited 9/9/2010 7:27:00 AM	B.3.1 Determine OPERABLE DG(s) are not inoperable due to common cause failure.	
	3TS 3.8.1.B.3.2			Potential Future Action	B.3.2 Perform SR 3.8.1.2 for OPERABLE DGs.	
	3TS 3.8.1.B.4	9/9/2010 7:23:00 AM	9/12/2010 7:23:00 AM	Open 60 hrs left	B.4 Restore DG to OPERABLE status.	
	3TS 3.8.1.F.1			Potential Future Action	F.1 Be in MODE 3.	
	3TS 3.8.1.F.2			Potential Future Action	F.2 Be in MODE 5.	
3-TS -10-2676	Zone 117 for 32 RCP Smoke Detector (BSD-2) failed during the performance of 3PT-A39. Report was submitted to OSRC and action completed 5/16/10 @ 1209 as per CR-IP3-2010-01243.					
	3TRM 3.7.A.4.A.1	4/30/2010 9:00:00 AM	4/30/2010 10:00:00 AM	Exited 4/30/2010 9:00:00 AM	A.1 Establish an hourly fire watch patrol, where accessibility permits, in the affected location(s).	3-FP -XMITR-BSD-2
	3TRM 3.7.A.4.A.2	4/30/2010 9:00:00 AM	5/14/2010 9:00:00 AM	Open -2842.4 hrs left	A.2 Restore the required fire detectors to OPERABLE status.	
	3TRM 3.7.A.4.C.1	5/14/2010 9:00:00 AM	6/13/2010 9:00:00 AM	Exited 5/26/2010 12:09:00 PM	C.1 Submit a Special Report to the PORC in accordance with specification 5.4.B.	
3-TS -10-2713	'OAP-007 Operations Actions to support VC Entry (Weld Channel/Nitrogen isolated to 80' & 95' Air Locks -AOT and Incore Flux Drives off - PAOT)					
	3TS 3.2.1.A.1			Potential Future Action	A.1 Reduce THERMAL POWER 1% RTP for each 1% FQ(Z) exceeds limit.	3-INCO - -INCORE CONTROL PAN
	3TS 3.6.10.A.1	6/5/2010 1:15:00 AM	6/5/2010 5:15:00 AM	Exited 6/5/2010 1:35:00 AM	A.1 Isolate the WC&PPS supply to the affected components by use of at least one closed and de?activated automatic valve, closed manual valve, blind flange, or check valve with flow through the valve secured.	3-WCPS -VALVE -PS-24 3-WCPS -VALVE -PS-25 3-WCPS -VALVE -PS-26 3-WCPS -VALVE -PS-27
	3TS 3.6.2.A.1			Potential Future Action	A.1 Verify the OPERABLE door is closed in the affected air lock.	

Allowed Outage Time for Indian Point 3

LCO Number	Section Number	Start Date	Required Date	Status	Required Action	Initiating Items
	3TRM 3.7.A.1.F.1	9/1/2010 1:51:00 PM	9/1/2010 2:51:00 PM	Exited 9/1/2010 2:00:00 PM	F.1 Establish an hourly fire watch patrol in the Turbine Building (15' elevation south loading well), Control Building (15' elevation) and the Administration Service Building (15' elevation near the fire brigade room).	
	3TRM 3.7.A.1.F.1			Potential Future Action	F.1 Establish an hourly fire watch patrol in the Turbine Building (15' elevation south loading well), Control Building (15' elevation) and the Administration Service Building (15' elevation near the fire brigade room).	
	3TRM 3.7.A.1.G.1	9/8/2010 1:51:00 PM	10/8/2010 1:51:00 PM	Open 690.4 hrs left	G.1 Submit a Special Report to the PORC in accordance with specification 3.7.A.8.C.	
	3TRM 3.7.A.1.C.1			Potential Future Action	C.1 For the diesel driven fire pump, enter Condition F.	

Entergy	CONDITION REPORT	CR-IP3-2010-02682

Originator: Lewis,Matthew W
Originator Phone: 8281
Originator Site Group: IP3 Operations Mgmt IP3
Operability Required: Y
Supervisor Name: Dinelli,John
Reportability Required: Y
Discovered Date: 09/09/2010 21:54
Initiated Date: 09/09/2010 22:06

Condition Description:

Unit 3 was manually tripped at 21:29 due to water leak in the Exciter housing (Service Water, from the Exciter Coolers). Anomalies noted on the plant trip: 34 RCP tripped during the 6.9KV Bus transfer (6.9KV Bus 2; Bus 2 did re-energize via UT2/ST5 tie breaker and Bus 2 did not receive an undervoltage), 31 MBFP suction relief valve lifted causing a CO2 discharge on 31 MBFP (subsequently reseated and CO2 secured).

Immediate Action Description:

Entered E-0 and ES-0.1.

Suggested Action Description:

Additional parameters noted on 34 RCP following the trip: Standpipe high level alarm, loss of seal return flow on both indicators, RCDT trending up abnormally fast (frequent pump-downs); possible #2 seal failure.

Entergy	CONDITION REPORT	CR-IP3-2010-02683

Originator: Martin,Dustin
Originator Phone: 5298
Originator Site Group: IP3 Operations Watch Staff IP3
Operability Required: Y
Supervisor Name: Buchal,Timothy J
Reportability Required: Y
Discovered Date: 09/09/2010 22:01
Initiated Date: 09/09/2010 23:07

Condition Description:

During unit 3 plant trip 34 RCP tripped during fast transfere of inside busses to outside power. Subsequently 34 RCP seal return flow both high and low indicators are currently reading zero and 34 RCP stand pipe high level alarm is locked in.

Immediate Action Description:

WR# 212295 for RCP trip during fast transfere
WR# 212296 for RCP #1 seal return flow

Suggested Action Description:

Trouble shoot and repair

EQUIPMENT:

Tag Name	Tag Suffix Name	Component Code	Process System Code
RCPCPC4	NC/SR/MR	PUMP	RCS

Entergy	CONDITION REPORT	CR-IP3-2010-02684

Originator: Martin,Dustin
Originator Phone: 5298
Originator Site Group: IP3 Operations Watch Staff IP3
Operability Required: N
Supervisor Name: Buchal,Timothy J
Reportability Required: N
Discovered Date: 09/09/2010 23:11
Initiated Date: 09/09/2010 23:14

Condition Description:
During Unit 3 trip CD-99-2 31 MAIN BOILER FEED PUMP SUCTION RELIEF VALVE relieved.

Immediate Action Description:
WR# 212301

Suggested Action Description:
Verify relief valve setpoint correct.

EQUIPMENT:

Tag Name	Tag Suffix Name	Component Code	Process System Code
CD-99-2	HCLM/NS	VALVE	COND

REFERENCE ITEMS:

Type Code	Item Desc
WON	212301

Entergy	CONDITION REPORT	CR-IP3-2010-02685

Originator: Martin,Dustin
Originator Phone: 5298
Originator Site Group: IP2 Operations Watch Staff IP2
Operability Required: Y
Supervisor Name: Buchal,Timothy J
Reportability Required: Y
Discovered Date: 09/09/2010 23:16
Initiated Date: 09/09/2010 23:24

Condition Description:
Due to CD-99-2 31 MBFP suction relief valve lifting during unit 3 plant trip CO2 was discharged on 31 MBFP.

Immediate Action Description:
Posted 5' with danger tape and set up blowers to ventilate 5'.
Entered TRM action statement 3.7.A.7 and implement required actions for having less than required amount of CO2

Suggested Action Description:
none

Entergy	CONDITION REPORT	CR-IP3-2010-02686

Originator: Martin,Dustin
Originator Phone: 5298
Originator Site Group: IP2 Operations Watch Staff IP2
Operability Required: Y
Supervisor Name: Buchal,Timothy J
Reportability Required: Y
Discovered Date: 09/09/2010 23:30
Initiated Date: 09/09/2010 23:33

Condition Description:
During Unit 3 trip IR 35 and IR 36 did trend as expected and are now pegged low.

Immediate Action Description:
Verified SR energized as required.
WO # 212303,212304

Suggested Action Description:
Trouble shoot and repair if necessary N35, and N36

EQUIPMENT:

Tag Name	Tag Suffix Name	Component Code	Process System Code
NIS		SYSTEM	NIS

REFERENCE ITEMS:

Type Code	Item Desc
WON	212303, 212304

EVENT RECOLLECTION

Event Description: Rx trip following service water leak into exciter

Event Occurrence: Date 9/9/10 Time 2129 Statement: Date/Time 9/9/10 2327

Name: Chee Yun Position BOP (RO) Dept: OPS

PERSONAL STATEMENTS

Use as many sheets as you need. Try to address as many of these questions as may be appropriate. For those questions you cannot answer write "did not observe" or "do not know" in the question. Please enter times when things happen so an accurate chronological time line can be reconstructed.

1. What happened? Concentrate first on what you saw or know first hand, but also include "what you heard". Don't be concerned if there are "holes" or inconsistencies in your understanding of the event.

Got report of service water leak into the excite enclosure. Attempted to isolate leak was unsuccessful and the leak got worse. Directed by CRS to trip the Rx at that point.

2. What caused you to be aware of the event?

Notified by SM of the service water leak.

3. What conditions existed just prior to the event (note any abnormal or unusual lineups)?

Normal full power steady state.

4. Did you notice any specific PARAMETER VALUES you think may be particularly important?

☐ YES ☑ NO If you answered "YES", then explain:

5. Did you note any relays, annunciators, computer alarms, that changed state during the event?

☐ YES ☑ NO If you answered "YES", then explain:

EVENT RECOLLECTION (cont'd)

6. WHEN did various events occur? Any times you remember may help us re-construct the incident.

Notified by Sm of service water leak @ 2105. Informed CERO and reduced MWE's to unity by 2115. Attempts to isolate leak unsuccessful at 0f 2123, Tripped the Rx @ 2129

7. What happened after the event?

Rx trip. 34 RCP tripped.

8. Did you notice ANY UNUSUAL SENSATIONS? Noises - smells - heat - moisture or mugginess?

☐ YES ☑ NO If you answered "YES", then explain:

9. WAS HELP AVAILABLE when you needed it?

☑ YES ☐ NO If you answered "NO", then explain:

10. Were COMMUNICATIONS audible and clear? Did they help you understand what was going on?

☑ YES ☐ NO If you answered "NO", then explain:

11. Have you ever seen or known of this type of event before?

☑ YES ☐ NO If you answered "YES", then explain:

Previous unit trip (transformer failure ?) 34 RCP tripped as well.

EVENT RECOLLECTION (cont'd)

12. Were the procedures adequate?

☑ YES ☐ NO If you answered "NO", then explain:

13. Was it necessary to take any actions outside established procedures? If so, document those actions here.

No

14. Do you know of any lessons learned from this event?

☐ YES ☑ NO If you answered "YES", then identify lessons:

15. Do you know WHO ELSE was on watch that might have information on what occurred?

☑ YES ☐ NO If you answered "YES", then identify individual(s):

Team 3B

16. *Is there any other information that you believe is relevant from your personal observations and experience with this event?*

☐ YES ☑ NO If you answered "YES", then provide details:

Chee Yun — 9/9/10

Print Name and Sign — Date

8282

Phone Extension

EVENT RECOLLECTION

Event Description: _____

Event Occurrence: Date 9/9/2010 Time ~2100 hr 21:29 Statement: Date/Time 9/10/2010 0015

Name: N.S. Lizzo Position SM Dept: OPERATIONS

PERSONAL STATEMENTS

Use as many sheets as you need. Try to address as many of these questions as may be appropriate. For those questions you cannot answer write "did not observe" or "do not know" in the question. Please enter times when things happen so an accurate chronological time line can be reconstructed.

1. What happened? Concentrate first on what you saw or know first hand, but also include "what you heard". Don't be concerned if there are "holes" or inconsistencies in your understanding of the event.

CONV WATER (SAGDTK) RECORDED WATER LEAKING INTO EXCITER HOUSING. WATER WAS OBSERVED TRICKLING INTO SOUTHEAST CORNER FOR POINTS ABOVE AND "RAINING" ONTO WINDOW/WALL ON NORTHEAST SIDE. ATTEMPT TO ISOLATE "B" COOLER BY CLOSING SWT-25-2 WAS NOT SUCCESSFUL. WATER WAS OBSERVED POOLING IN EXCITER HOUSING (≈4.5"). I ENTERED THE CCR AND ORDERED RX TRIP RX TRIP AT 21:29

2. What caused you to be aware of the event?

NPO CALL TO SM OFFICE.

3. What conditions existed just prior to the event (note any abnormal or unusual lineups)?

32 EDG OOS FOR PM. INCREASED MONITORING OF EXCITER HOUSING DUE TO PREVIOUSLY IDENTIFIED LEAKAGE INDICATION.

4. Did you notice any specific PARAMETER VALUES you think may be particularly important?

☐ YES ☐ NO If you answered "YES", then explain:

SEE ABOVE

5. Did you note any relays, annunciators, computer alarms, that changed state during the event?

☐ YES ☑ NO If you answered "YES", then explain:

EVENT RECOLLECTION (cont'd)

6. WHEN did various events occur? Any times you remember may help us re-construct the incident.

~ 2100 ; INITIAL NOTIFICATION 21:29 RX TRIP

7. What happened after the event?

ALL ABNORMAL CONDITIONS LISTED, IDENTIFIED + DOCUMENTED.

8. Did you notice ANY UNUSUAL SENSATIONS? Noises - smells - heat - moisture or mugginess?

☐ YES ☑ NO If you answered "YES", then explain:

NONE OTHER THAN THOSE NOTED ON CRS

9. WAS HELP AVAILABLE when you needed it?

☑ YES ☐ NO If you answered "NO", then explain:

10. Were COMMUNICATIONS audible and clear? Did they help you understand what was going on?

☑ YES ☐ NO If you answered "NO", then explain:

11. Have you ever seen or known of this type of event before?

☑ YES ☐ NO If you answered "YES", then explain:

OCT 2006 + 0450 2005

EVENT RECOLLECTION (cont'd)

12. Were the procedures adequate?

☑ YES ☐ NO If you answered "NO", then explain:

13. Was it necessary to take any actions outside established procedures? If so, document those actions here.

ISOLATED FLOW TO EXCITER COOLERS

14. Do you know of any lessons learned from this event?

☐ YES ☑ NO If you answered "YES", then identify lessons:

15. Do you know WHO ELSE was on watch that might have information on what occurred?

☑ YES ☐ NO If you answered "YES", then identify individual(s):

TEAM 3B, TEAM 3D PERSONNEL & AOM MATT LEWIS

16. *Is there any other information that you believe is relevant from your personal observations and* experience with this event?

☐ YES ☑ NO If you answered "YES", then provide details:

NICK LIZZO

Print Name and Sign

9/10/2013

Date

8281

Phone Extension

EVENT RECOLLECTION

Event Description: Unit 3 Trip due to SW leak in Exciter

Event Occurrence: Date 9/9/10 Time 2129 Statement: Date/Time 9/9/10 2253

Name: Vin DeClemente Position STA Dept: Ops

PERSONAL STATEMENTS

Use as many sheets as you need. Try to address as many of these questions as may be appropriate. For those questions you cannot answer write "did not observe" or "do not know" in the question. Please enter times when things happen so an accurate chronological time line can be reconstructed.

1. What happened? Concentrate first on what you saw or know first hand, but also include "what you heard". Don't be concerned if there are "holes" or inconsistencies in your understanding of the event.

SM reported a leak was I'd in Exciter housing. Actions were taken to isolate the leak but were unsuccessfull, SM & AOM discussed that if the leak was not isolable that a plant Trip would be needed. Manual Trip was performed

2. What caused you to be aware of the event? at 2129

SM reported that the NPO identified a SW leak in the Exciter

3. What conditions existed just prior to the event (note any abnormal or unusual lineups)?

Service water leak in Exciter housing. Isolations was unsuccessful

4. Did you notice any specific PARAMETER VALUES you think may be particularly important?

☐ YES ☑ NO If you answered "YES", then explain:

5. Did you note any relays, annunciators, computer alarms, that changed state during the event?

☐ YES ☑ NO If you answered "YES", then explain:

EVENT RECOLLECTION (cont'd)

6. WHEN did various events occur? Any times you remember may help us re-construct the incident.

Manual Rx Trip at 2129. 34 RCP Trip at 2130.

7. What happened after the event?

34 RCP Tripped. 31 MBFP Relief Valve (Suction) CD-99-2 lifted causing a CO^2 discharge; Approx 15 min later the relief valve stated.

8. Did you notice ANY UNUSUAL SENSATIONS? Noises - smells - heat - moisture or mugginess?

☐ YES ☑ NO If you answered "YES", then explain:

9. WAS HELP AVAILABLE when you needed it?

☑ YES ☐ NO If you answered "NO", then explain:

10. Were COMMUNICATIONS audible and clear? Did they help you understand what was going on?

☑ YES ☐ NO If you answered "NO", then explain:

11. Have you ever seen or known of this type of event before?

☑ YES ☐ NO If you answered "YES", then explain:

I remember OE on subject of RCP tripping on a Rx Trip.

EVENT RECOLLECTION (cont'd)

12. Were the procedures adequate?

☑ YES ☐ NO If you answered "NO", then explain:

13. Was it necessary to take any actions outside established procedures? If so, document those actions here.

No

14. Do you know of any lessons learned from this event?

☐ YES ☑ NO If you answered "YES", then identify lessons:

15. Do you know WHO ELSE was on watch that might have information on what occurred?

☑ YES ☐ NO If you answered "YES", then identify individual(s):

Team 3B - Ron Carpino, Chris Nilssen, Chee Yun, Ralph Orzo, Nick Lizzo, Matt Lewis.

16. Is there any other information that you believe is relevant from your personal observations and experience with this event?

☐ YES ☑ NO If you answered "YES", then provide details:

Vin DeClemente Vincent F. DeClem 9/9/10

Print Name and Sign Date

5660.

Phone Extension

w 9/6/10 Excitr

EVENT RECOLLECTION

Event Description: ~~H2~~ cooler leak / manual / reactor Trip

Event Occurrence: Date 9/9/10 Time 2129 Statement: Date/Time 9/9/10 2307

Name: Chris N. Lesog Position RO (ATC) Dept: OPS

PERSONAL STATEMENTS

Use as many sheets as you need. Try to address as many of these questions as may be appropriate. For those questions you cannot answer write "did not observe" or "do not know" in the question. Please enter times when things happen so an accurate chronological time line can be reconstructed.

1. What happened? Concentrate first on what you saw or know first hand, but also include "what you heard". Don't be concerned if there are "holes" or inconsistencies in your understanding of the event.

Instructed to perform manual RX Trip due to SW Leak - RX Tripped, Turbine Tripped - 2 IAPI initials stuck until manually actuated 34 RCP Tripped No Seal return Low or Hi range

2. What caused you to be aware of the event?

Was RO In CCR.

3. What conditions existed just prior to the event (note any abnormal or unusual lineups)?

ATTemps to Isolate SW leak in progress - report back was Leak was getting worse.

4. Did you notice any specific PARAMETER VALUES you think may be particularly important?

☐ YES ☐ NO If you answered "YES", then explain:

Source Range loss detector in/out a number of times. 34 Standpipe level, 34 seal return hi/lo LOW. 2 IAPI stuck.

5. Did you note any relays, annunciators, computer alarms, that changed state during the event?

☑ YES ☐ NO If you answered "YES", then explain:

34 RCP Lo Flow / Source Range loss det. Voltage

EVENT RECOLLECTION (cont'd)

6. WHEN did various events occur? Any times you remember may help us re-construct the incident.

2129 Trip, 2135 into ES-0.1, 2148 STM dumps in pressure control, 2300 BRK 1+3 closed

7. What happened after the event?

monitored plant.

8. Did you notice ANY UNUSUAL SENSATIONS? Noises - smells - heat - moisture or mugginess?

☐ YES ☑ NO If you answered "YES", then explain:

9. WAS HELP AVAILABLE when you needed it?

☑ YES ☐ NO If you answered "NO", then explain:

10. Were COMMUNICATIONS audible and clear? Did they help you understand what was going on?

☑ YES ☐ NO If you answered "NO", then explain:

11. Have you ever seen or known of this type of event before?

☑ YES ☐ NO If you answered "YES", then explain:

previous manual trip due to SW in Exciter.
Last Recent Trip has 34 RCP trips.

CW 9/1/10

EVENT RECOLLECTION (cont'd)

12. Were the procedures adequate?

☑ YES ☐ NO If you answered "NO", then explain:

13. Was it necessary to take any actions outside established procedures? If so, document those actions here.

NO

14. Do you know of any lessons learned from this event?

☐ YES ☑ NO If you answered "YES", then identify lessons:

15. Do you know WHO ELSE was on watch that might have information on what occurred?

☑ YES ☐ NO If you answered "YES", then identify individual(s):

Unit 3 watch staff for 9/9/10

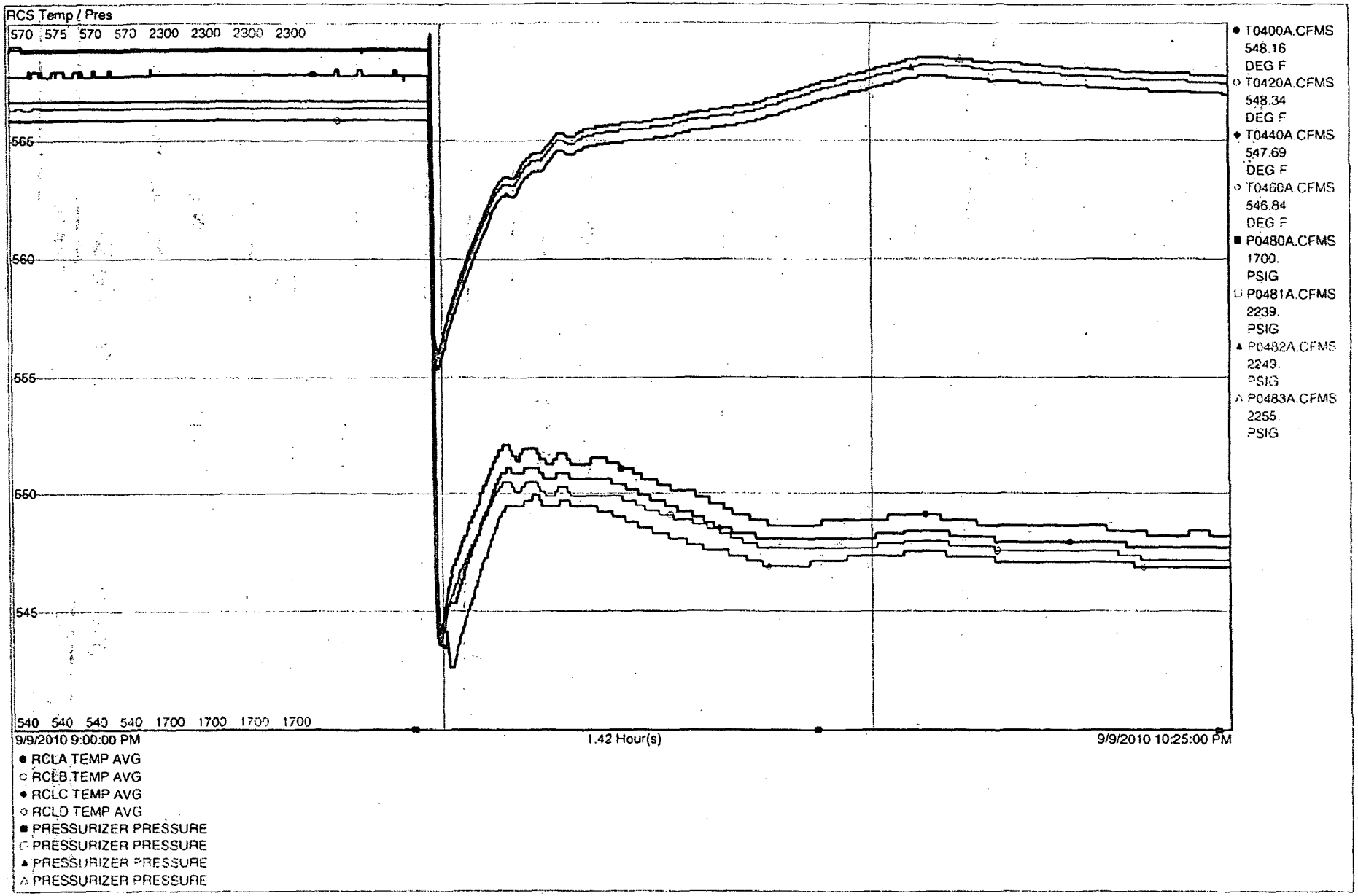
16. Is there any other information that you believe is relevant from your personal observations and experience with this event?

☐ YES ☑ NO If you answered "YES", then provide details:

Chris N. Lyon — Print Name and Sign

9/9/10 — Date

9282 — Phone Extension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545
540 540 540 540 1700 1700 1700 1700
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1.42 Hour(s)
9/9/2010 10:25:00 PM
● RCLA TEMP AVG
○ RCLB TEMP AVG
◆ RCLC TEMP AVG
◇ RCLD TEMP AVG
■ PRESSURIZER PRESSURE
□ PRESSURIZER PRESSURE
▲ PRESSURIZER PRESSURE
△ PRESSURIZER PRESSURE
● T0400A.CFMS
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DEG F
○ T0420A.CFMS
548.34
DEG F
◆ T0440A.CFMS
547.69
DEG F
◇ T0460A.CFMS
546.84
DEG F
■ P0480A.CFMS
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PSIG
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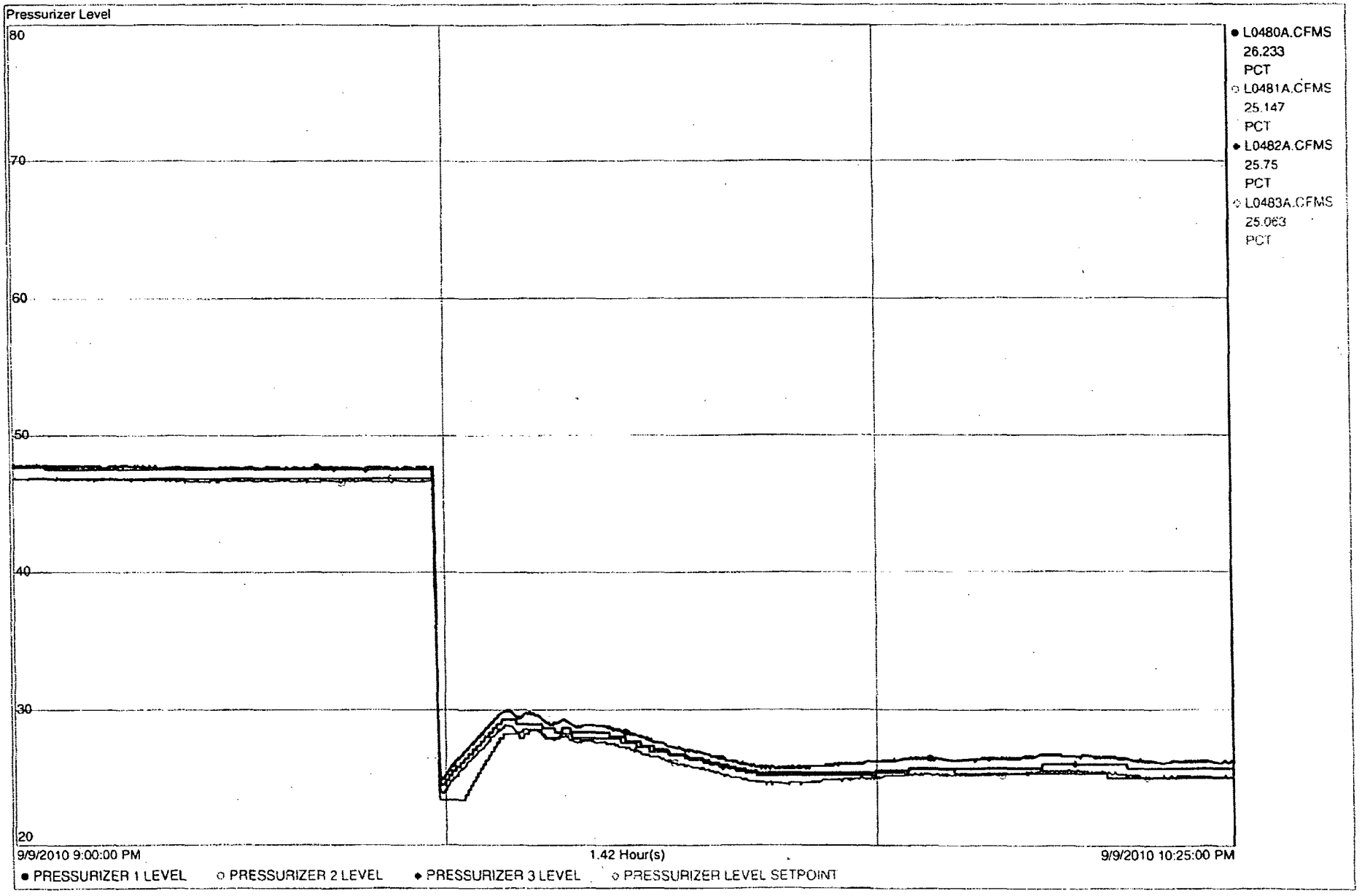
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9/9/2010 10:25:00 PM
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PCT
L0481A.CFMS
25.147
PCT
L0482A.CFMS
25.75
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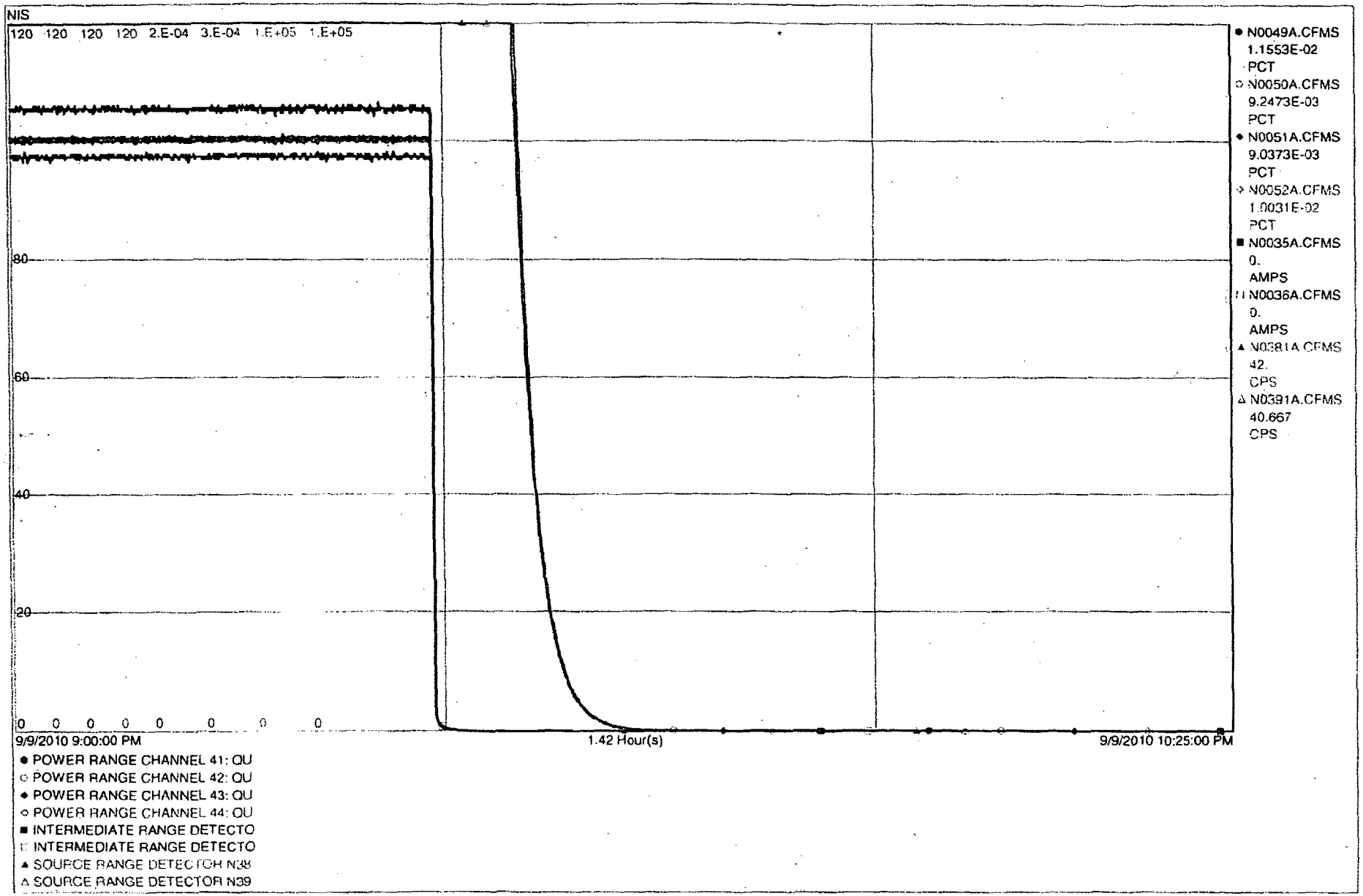
PRESSURIZER 2 LEVEL
PRESSURIZER 3 LEVEL
PRESSURIZER LEVEL SETPOINT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60
40
20
0 0 0 0 0 0 0 0
9/9/2010 9:00:00 PM
1.42 Hour(s)
9/9/2010 10:25:00 PM
● POWER RANGE CHANNEL 41: QU
○ POWER RANGE CHANNEL 42: QU
◆ POWER RANGE CHANNEL 43: QU
◇ POWER RANGE CHANNEL 44: QU
■ INTERMEDIATE RANGE DETECTO
□ INTERMEDIATE RANGE DETECTO
▲ SOURCE RANGE DETECTOR N38
△ SOURCE RANGE DETECTOR N39
● N0049A.CFMS
1.1553E-02
PCT
○ N0050A.CFMS
9.2473E-03
PCT
◆ N0051A.CFMS
9.0373E-03
PCT
◇ N0052A.CFMS
1.0031E-02
PCT
■ N0035A.CFMS
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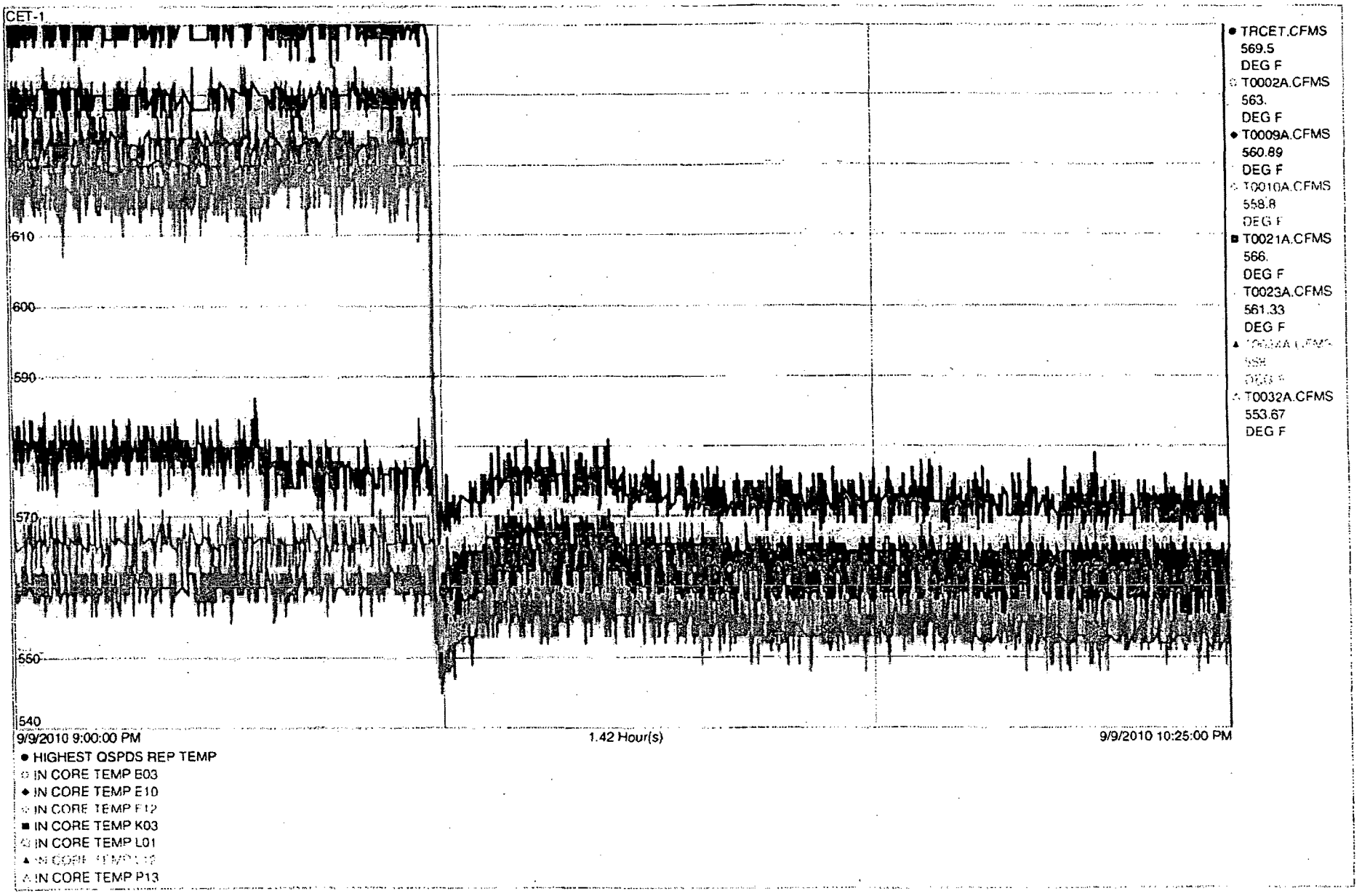
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570
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540
9/9/2010 9:00:00 PM
1.42 Hour(s)
9/9/2010 10:25:00 PM
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○ IN CORE TEMP B03
◆ IN CORE TEMP E10
◇ IN CORE TEMP F12
■ IN CORE TEMP K03
△ IN CORE TEMP L01
▲ IN CORE TEMP L12
△ IN CORE TEMP P13
● TRCET.CFMS
569.5
DEG F
○ T0002A.CFMS
563.
DEG F
◆ T0009A.CFMS
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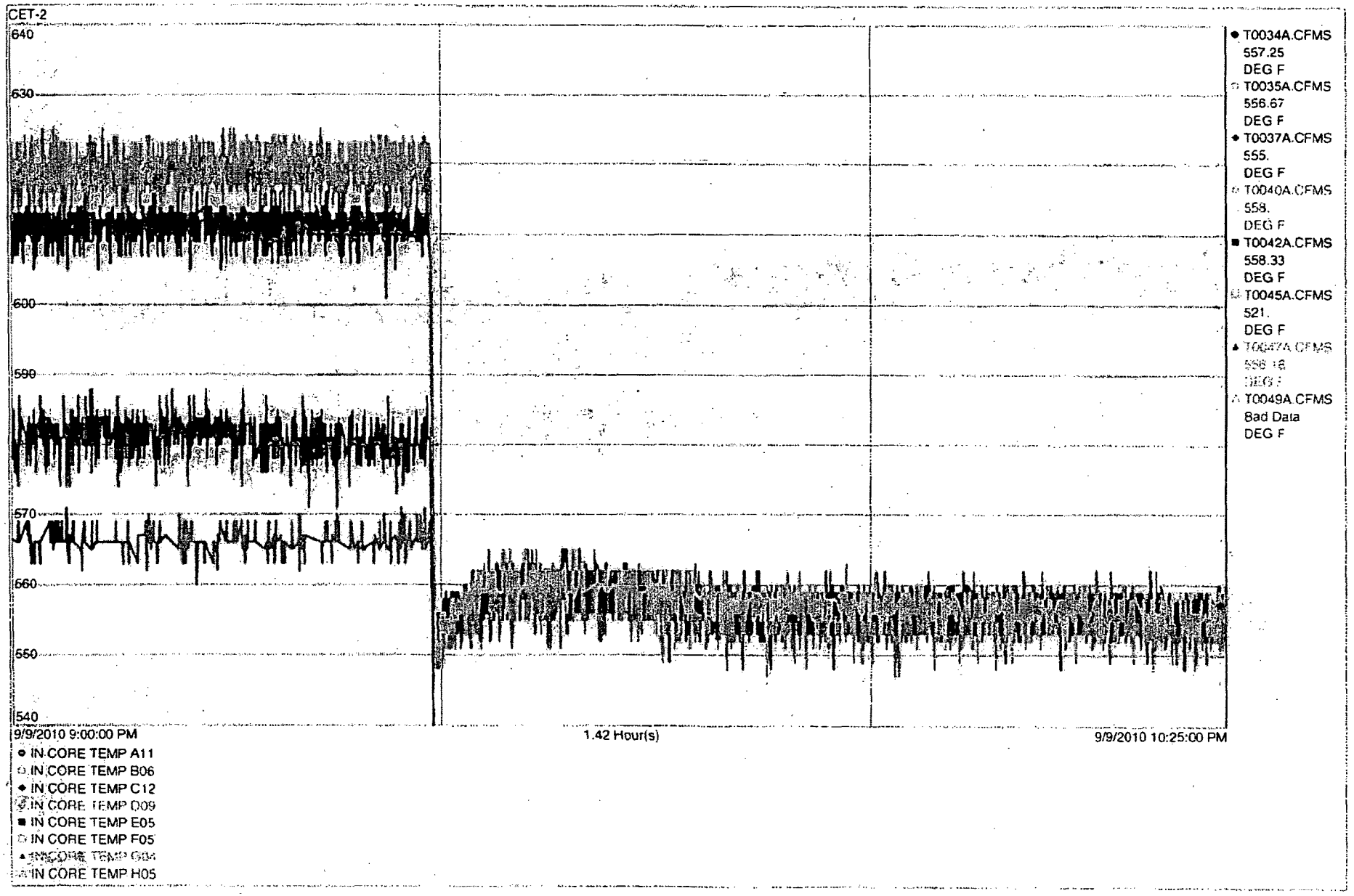
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9/9/2010 9:00:00 PM
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9/9/2010 10:25:00 PM
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DEG F
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DEG F
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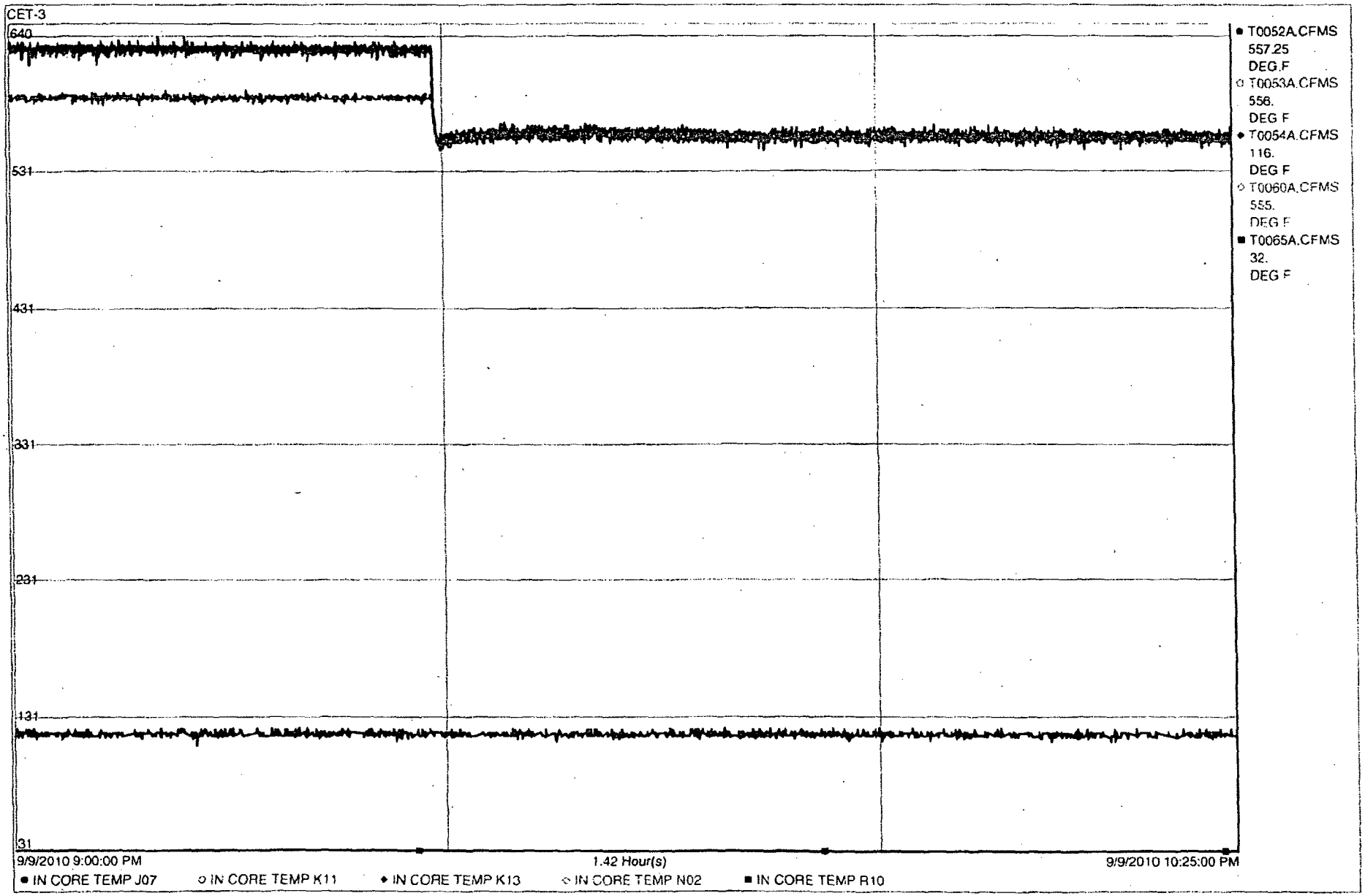
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9/9/2010 10:25:00 PM
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○ IN CORE TEMP K11
◆ IN CORE TEMP K13
◇ IN CORE TEMP N02
■ IN CORE TEMP R10
● T0052A.CFMS
557.25
DEG.F
○ T0053A.CFMS
556.
DEG F
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DEG F
◇ T0060A.CFMS
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DEG F
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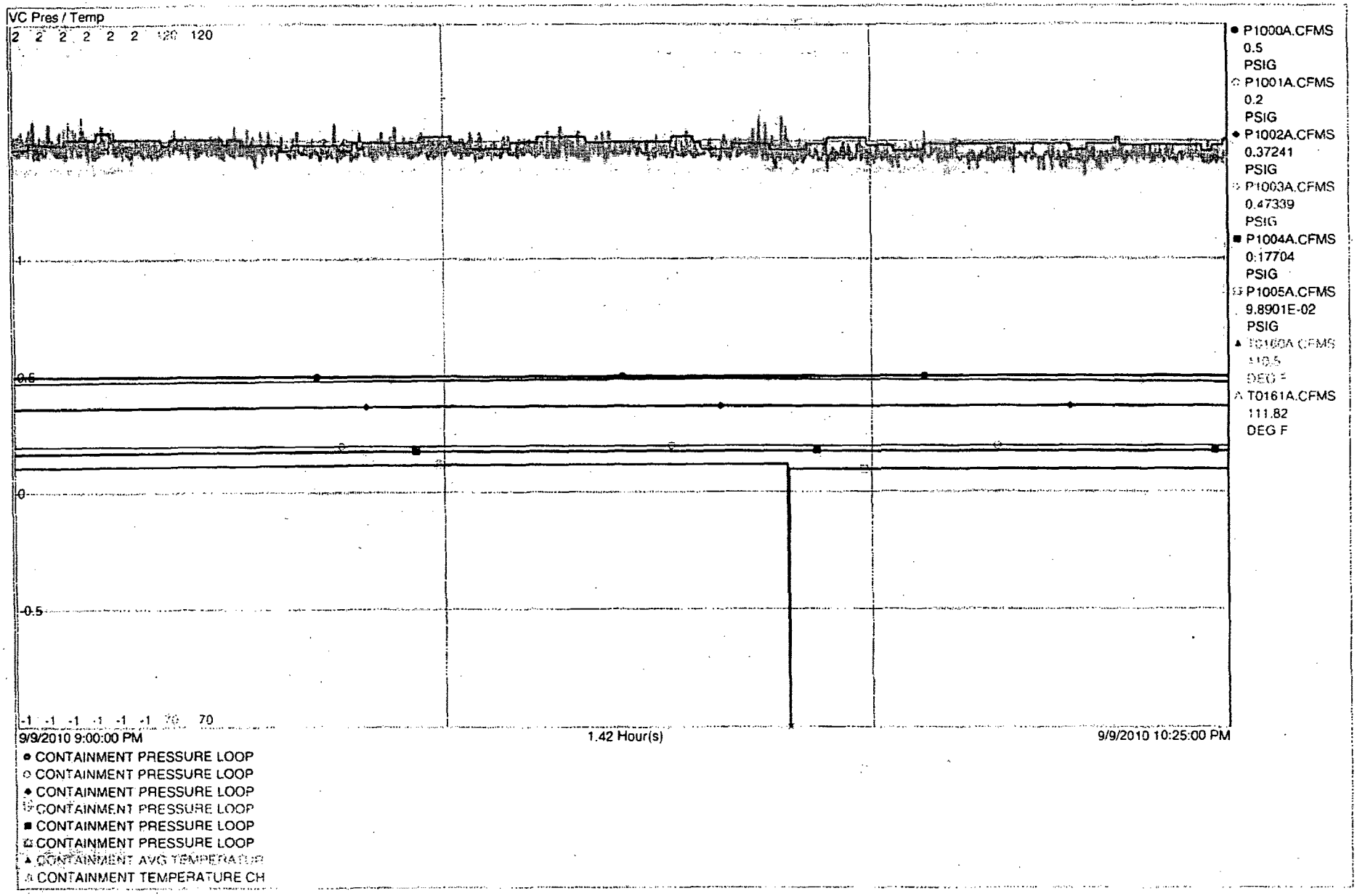
DEG F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9/9/2010 9:00:00 PM
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9/9/2010 10:25:00 PM
P1000A.CFMS
0.5
PSIG
P1001A.CFMS
0.2
PSIG
P1002A.CFMS
0.37241
PSIG
P1003A.CFMS
0.47339
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PSIG
P1005A.CFMS
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T0161A.CFMS
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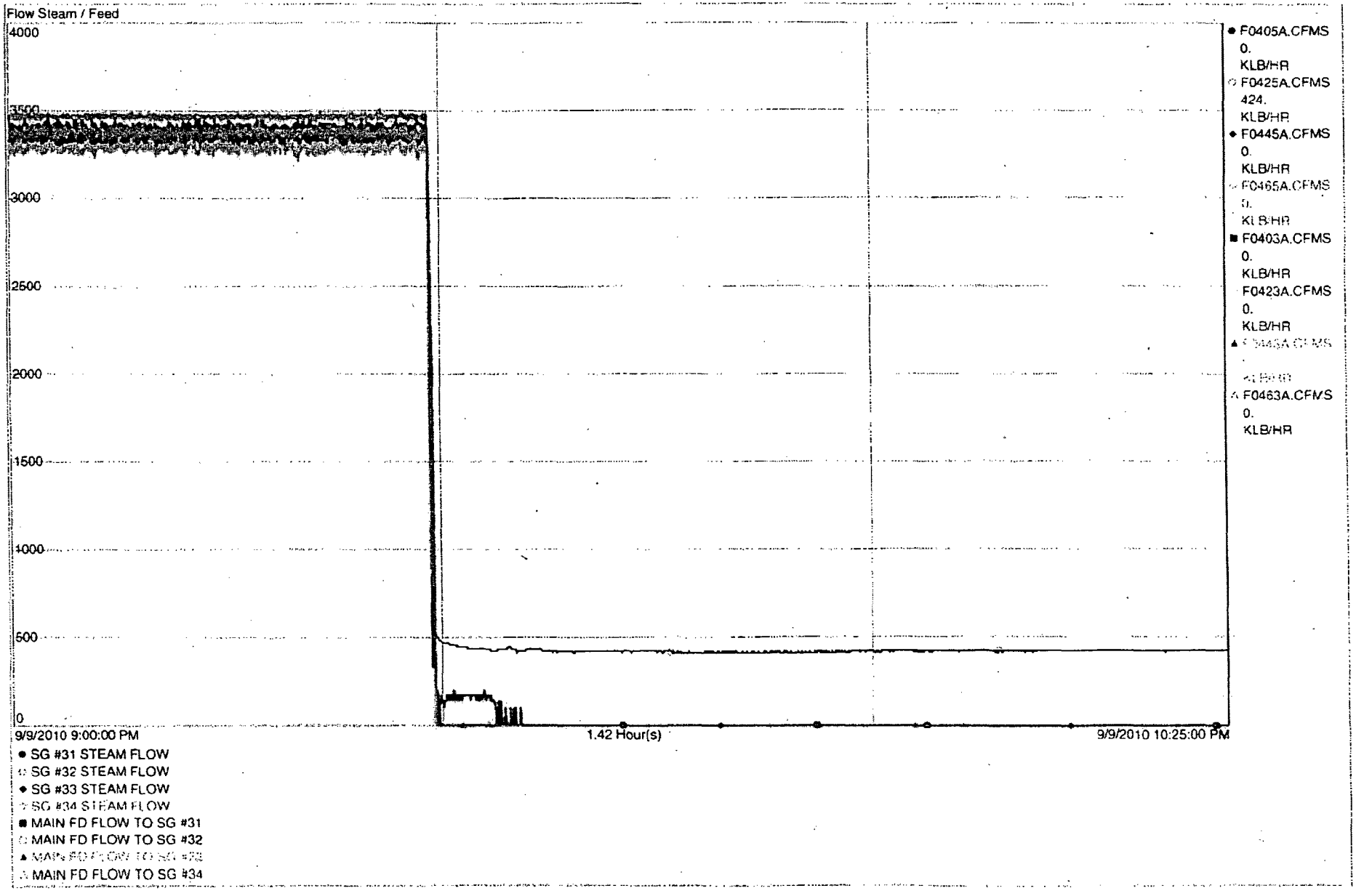
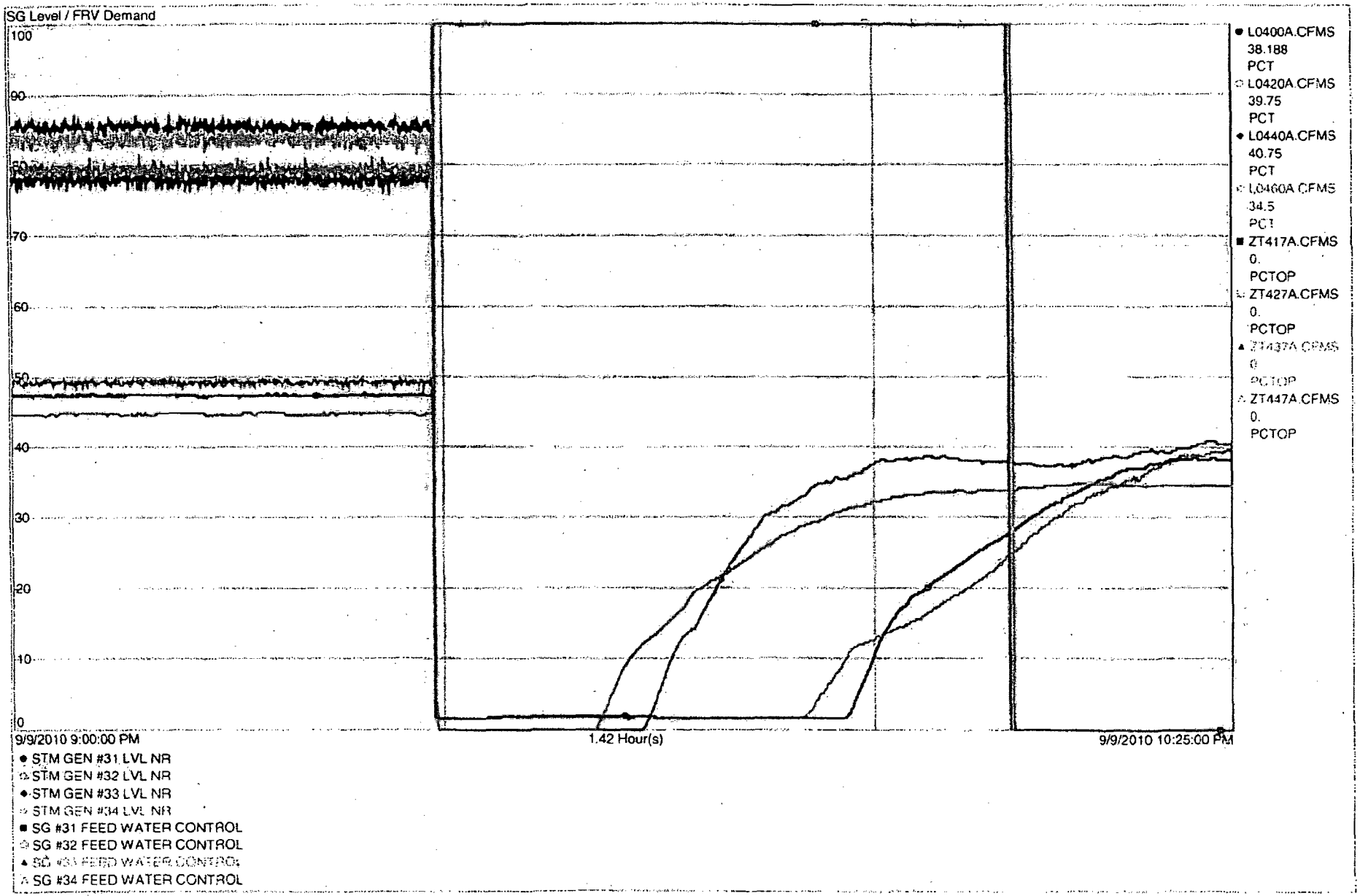
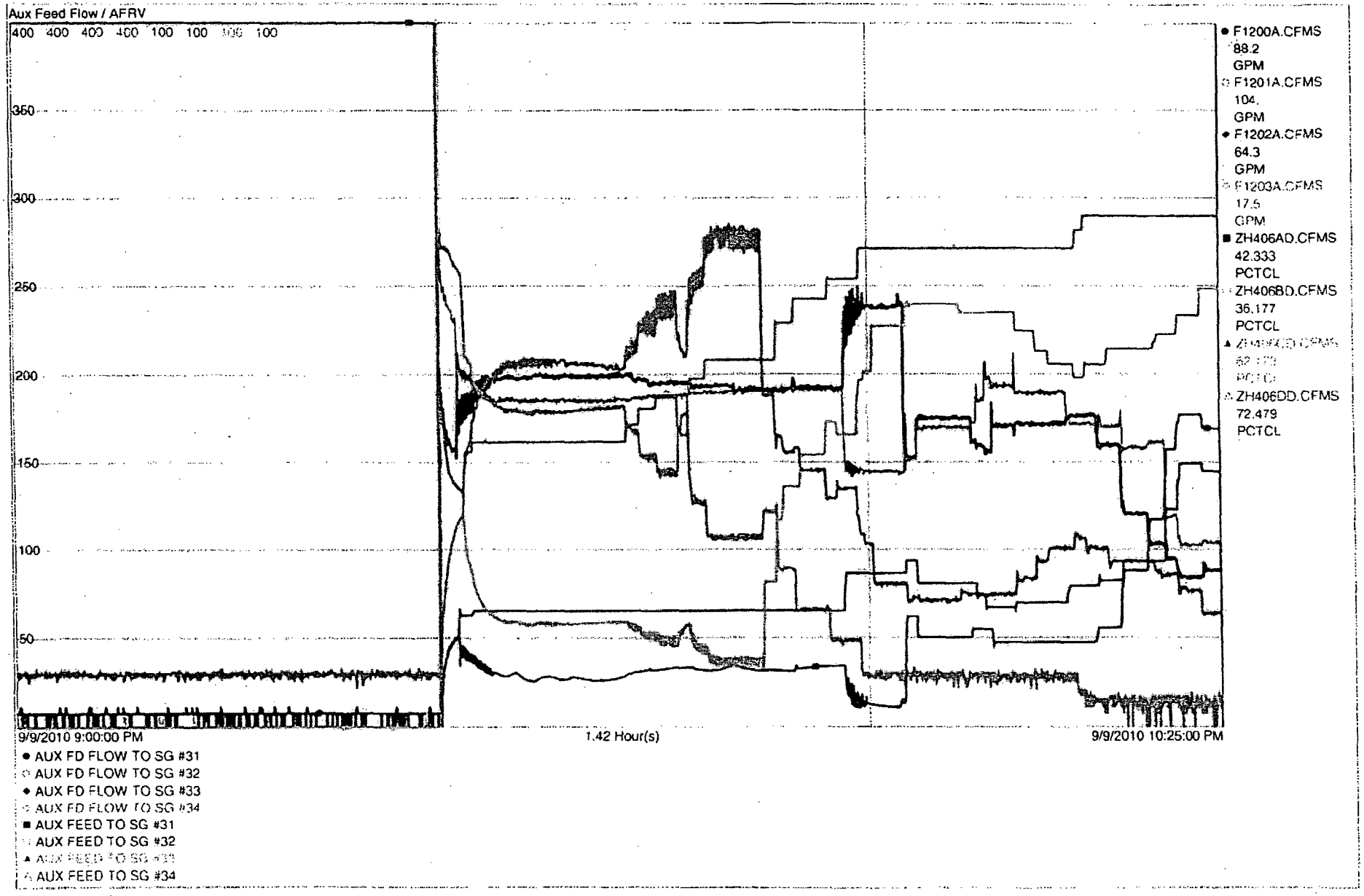
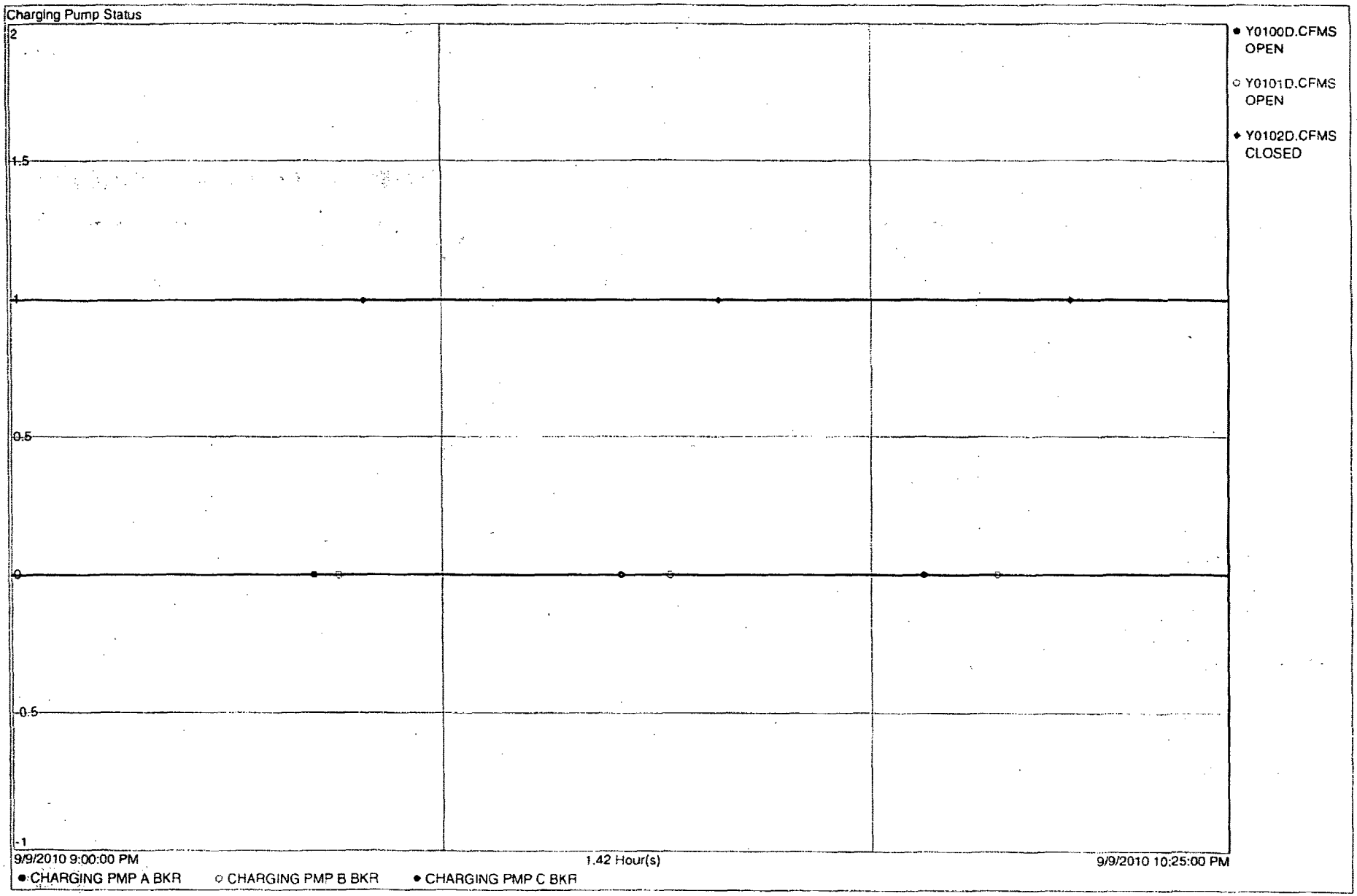
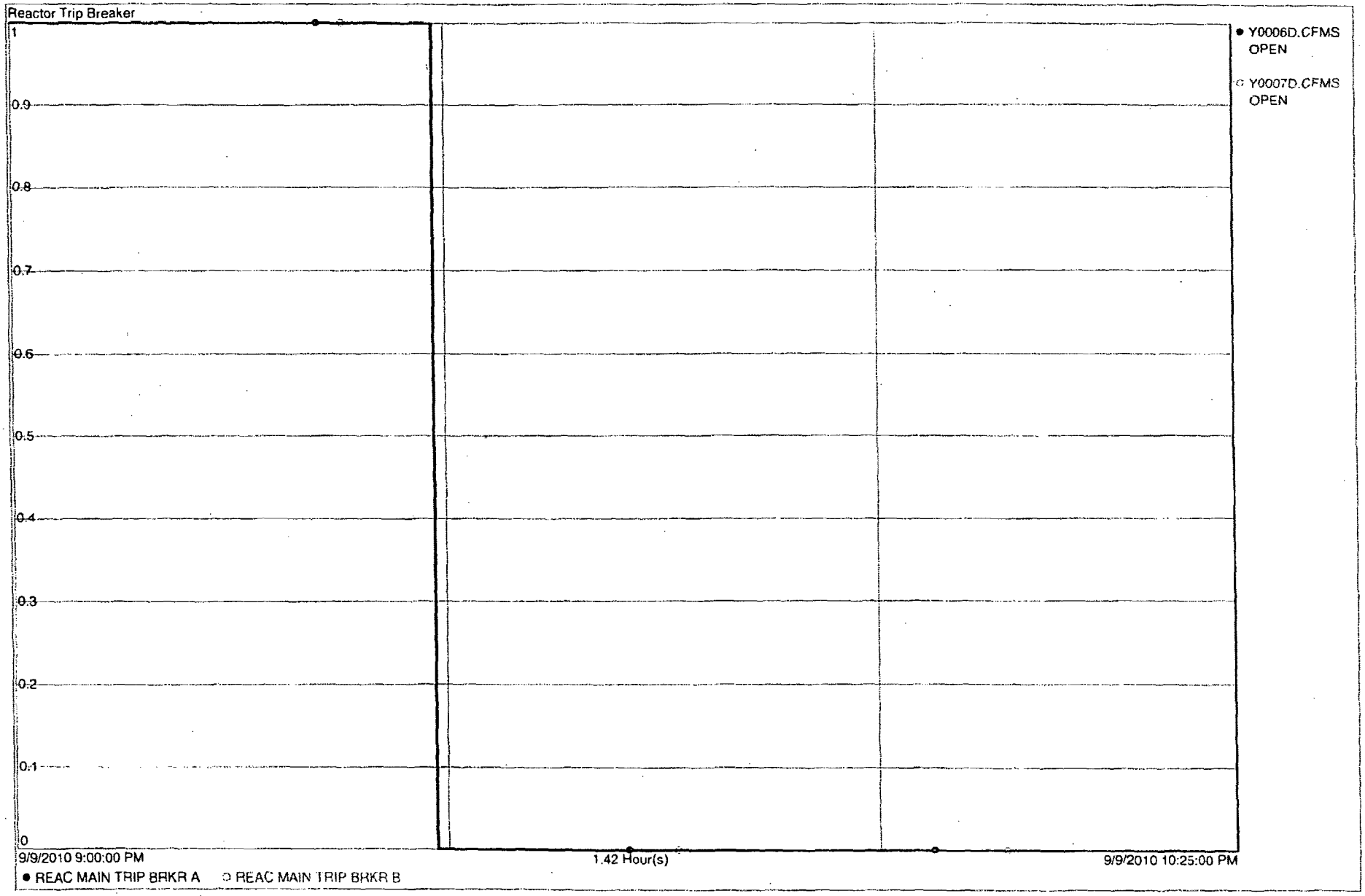
CONTAINMENT AVG TEMPERATUR
CONTAINMENT TEMPERATURE CH


Flow Steam / Feed
4000
3500
3000
2500
2000
1500
1000
500
0
9/9/2010 9:00:00 PM
1.42 Hour(s)
9/9/2010 10:25:00 PM
F0405A.CFMS
0.
KLB/HR
F0425A.CFMS
424.
KLB/HR
F0445A.CFMS
0.
KLB/HR
F0465A.CFMS
0.
KLB/HR
F0403A.CFMS
0.
KLB/HR
F0423A.CFMS
0.
KLB/HR
F0463A.CFMS
0.
KLB/HR
SG #31 STEAM FLOW
SG #32 STEAM FLOW
SG #33 STEAM FLOW
SG #34 STEAM FLOW
MAIN FD FLOW TO SG #31
MAIN FD FLOW TO SG #32
MAIN FD FLOW TO SG #34


SG Level / FRV Demand
100
90
80
70
60
50
40
30
20
10
0
9/9/2010 9:00:00 PM
1.42 Hour(s)
9/9/2010 10:25:00 PM
L0400A.CFMS
38.188
PCT
L0420A.CFMS
39.75
PCT
L0440A.CFMS
40.75
PCT
L0460A.CFMS
34.5
PCT
ZT417A.CFMS
0.
PCTOP
ZT427A.CFMS
0.
PCTOP
ZT437A.CFMS
0
PCTOP
ZT447A.CFMS
0.
PCTOP
STM GEN #31 LVL NR
STM GEN #32 LVL NR
STM GEN #33 LVL NR
STM GEN #34 LVL NR
SG #31 FEED WATER CONTROL
SG #32 FEED WATER CONTROL
SG #33 FEED WATER CONTROL
SG #34 FEED WATER CONTROL


Aux Feed Flow / AFRV
400 400 400 400 100 100 100 100
350
300
250
200
150
100
50
F1200A.CFMS
88.2
GPM
F1201A.CFMS
104.
GPM
F1202A.CFMS
64.3
GPM
F1203A.CFMS
17.5
GPM
ZH406AD.CFMS
42.333
PCTCL
ZH406BD.CFMS
36.177
PCTCL
ZH406DD.CFMS
72.479
PCTCL
9/9/2010 9:00:00 PM
1.42 Hour(s)
9/9/2010 10:25:00 PM
AUX FD FLOW TO SG #31
AUX FD FLOW TO SG #32
AUX FD FLOW TO SG #33
AUX FD FLOW TO SG #34
AUX FEED TO SG #31
AUX FEED TO SG #32
AUX FEED TO SG #34


Charging Pump Status
2
1.5
1
0.5
0
-0.5
-1
9/9/2010 9:00:00 PM
1.42 Hour(s)
9/9/2010 10:25:00 PM
● CHARGING PMP A BKR
○ CHARGING PMP B BKR
◆ CHARGING PMP C BKR
● Y0100D.CFMS OPEN
○ Y0101D.CFMS OPEN
◆ Y0102D.CFMS CLOSED


Reactor Trip Breaker
1
0.9
0.8
0.7
0.6
0.5
0.4
0.3
0.2
0.1
0
Y0006D.CFMS OPEN
Y0007D.CFMS OPEN
9/9/2010 9:00:00 PM
1.42 Hour(s)
9/9/2010 10:25:00 PM
REAC MAIN TRIP BRKR A
REAC MAIN TRIP BRKR B