



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

June 10, 2011

Mr. R. M. Krich
Vice President, Nuclear Licensing
Tennessee Valley Authority
3R Lookout Place
1101 Market Street
Chattanooga, TN 37402-2801

SUBJECT: BROWNS FERRY NUCLEAR PLANT, UNIT 3 - REQUEST FOR ADDITIONAL
INFORMATION REGARDING RELIEF FROM WELD EXAMINATION
COVERAGE REQUIREMENTS, REQUEST FOR RELIEF 3-ISI-26
(TAC NO. ME5914)

Dear Mr. Krich:

By letter dated March 21, 2011, Tennessee Valley Authority submitted a request relief from weld examination coverage requirements specified in the American Society of Mechanical Engineers Boiler and Pressure Vessel Code, Section XI, 2001 Edition, 2003 Addenda, as amended by Title 10 of the *Code of Federal Regulations*, Paragraph 50.55a(b)(2)(xv)(A)(2), for one full penetration weld due to access limitations caused by design. Based on our review of your submittal, the U. S. Nuclear Regulatory Commission staff finds that a response to the enclosed request for additional information is needed before we can complete the review.

This request was discussed with Mr. Tom Matthews of your staff on June 6, 2011, and it was agreed that a response would be provided by July 15, 2011.

If you have any questions, please contact me at (301) 415-1055.

Sincerely,

A handwritten signature in black ink, appearing to read "Chris Gratton".

Christopher Gratton, Senior Project Manager
Plant Licensing Branch II-2
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket No. 50-296

Enclosure: Request for Additional Information

cc w/encl: Distribution via Listserv

DRAFT REQUEST FOR ADDITIONAL INFORMATION

REGARDING RELIEF REQUEST 3-ISI-26

TENNESSEE VALLEY AUTHORITY

BROWNS FERRY NUCLEAR PLANT, UNIT 3

DOCKET NO. 50-296

By letter dated March 21, 2011 (Agencywide Documents Access and Management System Accession No. ML110830037), Tennessee Valley Authority, the licensee for Browns Ferry Nuclear Plant, Unit 3 (BFN-3), submitted a request for relief (RR) from American Society of Mechanical Engineers (ASME Code), Boiler and Pressure Vessel Code Section XI requirements, in accordance with the provisions of Title 10 of the *Code of Federal Regulations*, Part 50, Section 50.55a, Paragraph (a)(3)(iii). This RR pertains to the third inservice inspection interval at BFN-3, which ends on November 18, 2015. The subject weld is a reactor pressure vessel nozzle-to-vessel (head) full penetration weld (N6A-NV).

In order to complete its review of the licensee's submittal, the U.S. Nuclear Regulatory Commission staff requires a response to the following questions:

1. Have any visual examinations been conducted on the associated nozzle inner radius area, such as those allowed by ASME Code Case N-648-1, "Alternative Requirements for Inner Radius Examinations of Class 1 Reactor Vessel Nozzles, Section XI, Division 1"?
2. If any such inspections have been completed, were any indications found in the associated N6A-NV nozzle-to-vessel head weld? If so, please describe each indication in detail.

Enclosure

June 10, 2011

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