



**UNITED STATES
NUCLEAR REGULATORY COMMISSION**
REGION II
245 PEACHTREE CENTER AVENUE NE, SUITE 1200
ATLANTA, GEORGIA 30303-1257

May 31, 2011

Mr. Christopher L. Burton
Vice President
Carolina Power and Light Company
Shearon Harris Nuclear Power Plant
P. O. Box 165, Mail Code: Zone 1
New Hill, North Carolina 27562-0165

**SUBJECT: SHEARON HARRIS NUCLEAR POWER PLANT- NOTIFICATION OF
CONDUCT OF A TRIENNIAL FIRE PROTECTION BASELINE INSPECTION
(NRC INSPECTION REPORT NO. 05000400/2011009)**

Dear Mr. Burton:

The purpose of this letter is to notify you that the U.S. Nuclear Regulatory Commission (NRC) Region II staff will conduct a triennial fire protection baseline inspection at Shearon Harris Nuclear Power Plant in August and September 2011. The inspection team will be led by Mr. Reinaldo Rodriguez, NRC Senior Reactor Inspector, of the Region II Office. The team will be composed of personnel from the NRC Regional and Headquarters Offices. The inspection will be conducted in accordance with Inspection Procedure 71111.05XT, the NRC's baseline fire protection inspection procedure for plants implementing 10 CFR 50.48(c) and National Fire Protection Association Standard (NFPA) 805.

On February 2, 2011, during a telephone conversation between Mr. John Caves (Harris Licensing Supervisor) of your staff, and Mr. Rodriguez, our respective staffs confirmed arrangements for a three-day information gathering onsite visit and a two-week onsite inspection. The schedule for the inspection is as follows:

- Information gathering visit: August 9 - 11, 2011
- Week 1 of onsite inspection: August 29 - September 2, 2011
- Week 2 of onsite inspection: September 19 - 23, 2011

The purposes of the information gathering visit are to obtain information and documentation needed to support the inspection, to become familiar with the Shearon Harris Nuclear Power Plant fire protection programs, fire protection features, and post-fire safe shutdown capabilities and plant layout, mitigating strategies to address Section B.5.b of the Interim Compensatory Measures Order, EA-02-026, of February 25, 2002/10 CFR 50.54(hh)(2). The types of documents the team will be interested in reviewing, and possibly obtaining, are listed in Enclosures 1 and 2. Please contact Mr. Rodriguez prior to preparing copies of the materials listed in the Enclosures. The inspection team will try to minimize your administrative burden by specifically identifying those documents required for inspection preparation.

During the information gathering visit the team will also discuss the following inspection support administrative details: office space; specific documents to be made available to the team in their office space; arrangements for unescorted site access (including radiation protection training, security, safety and fitness for duty requirements); and the availability of knowledgeable plant engineering and licensing organization personnel to serve as points of contact during the inspection.

We request that during the inspection weeks you ensure that copies of analyses, evaluations or documentation regarding the implementation and maintenance of the Harris fire protection program, including post-fire safe shutdown capability, be readily accessible to the team for their review. Of specific interest for the fire protection portion of the inspection are those documents which establish that your fire protection program satisfies NRC regulatory requirements and conforms to applicable NRC and industry fire protection guidance. For the B.5.b portion of the inspection, those documents implementing your mitigating strategies and demonstrating the management of your commitments for the strategies are of specific interest. Also, personnel should be available at the site during the inspection that are knowledgeable regarding those plant systems required to achieve and maintain safe shutdown conditions from inside and outside the control room (including the electrical aspects of the relevant post-fire safe shutdown analyses), reactor plant fire protection systems and features, and the Harris fire protection program and its implementation.

This letter does not contain new or amended information collection requirements subject to the Paperwork Reduction Act of 1995 (44 U.S.C. 3501 et seq.). Existing information collection requirements were approved by the Office of Management and Budget, control number 3150-0011. The NRC may not conduct or sponsor, and a person is not required to respond to, a request for information or an information collection requirement unless the requesting document displays a currently valid Office of Management and Budget control number.

Your cooperation and support during this inspection will be appreciated. If you have questions concerning this inspection, or the inspection team's information or logistical needs, please contact Mr. Rodriguez at (404) 997-4498, or me at (404) 997-4503.

In accordance with 10 CFR 2.390 of the NRC's "Rules of Practice," a copy of this letter and its enclosure will be available electronically for public inspection in the NRC Public Document Room or from the Publicly Available Records (PARS) component of NRC's document system

(ADAMS). ADAMS is accessible from the NRC Web site at <http://www.nrc.gov/reading-rm/adams.html> <http://www.nrc.gov/reading-rm/adams.html> (the Public Electronic Reading Room).

Sincerely,

/RA by Gerald Wiseman Acting For/

Rebecca L. Nease, Chief
Engineering Branch 2
Division of Reactor Safety

Docket No.: 50-400
License Nos.: NPF-63

Enclosures: 1. Triennial Fire Protection Inspection Support Documentation
2. Mitigating Strategies Support Documentation

cc w/encl. (See page 4)

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cc w/encl. (See page 4)

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E-MAIL COPY?	YES NO	YES NO	YES NO	YES NO	YES NO	YES NO	YES NO

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Letter to Christopher L. Burton from Rebecca L. Nease dated May 31, 2011.

SUBJECT: SHEARON HARRIS NUCLEAR POWER PLANT- NOTIFICATION OF
CONDUCT OF A TRIENNIAL FIRE PROTECTION BASELINE INSPECTION
(NRC INSPECTION REPORT NO. 05000400/2011009)

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Triennial Fire Protection Inspection Support Documentation

Note: This is a broad list of the documents the NRC inspection team may be interested in reviewing, and possibly obtaining, during the information gathering site visit. The current version of these documents is expected unless specified otherwise. Electronic media is preferred, if readily available. (The preferred file format on CD-ROM or DVD-ROM is “.pdf”). The CD/DVD-ROM should be indexed, hyperlinked, readable, and searchable to facilitate ease of use. Please provide 5 copies of each CD/DVD-ROM submitted). Information in “lists” should contain enough information to be easily understood by someone who has knowledge of the technology. The lead inspector will discuss specific information needs with the licensee staff and may request additional documents or electronic information.

1. The current version of the fire protection program and fire hazards analysis.
2. Current versions of the fire protection program implementing procedures (e.g., administrative controls, surveillance testing, and fire brigade).
3. Fire brigade training program and pre-fire plans.
4. Post-fire nuclear safety capability systems and separation analysis.
5. Analysis supporting achieving and maintaining nuclear safety performance criteria, including those analyses for areas requiring recovery actions.
6. Piping and instrumentation (flow) diagrams showing the components used to achieve and maintain nuclear safety performance criteria for fires outside the control room and other areas requiring recovery actions at other than primary control stations.
7. Plant layout and equipment drawings that identify the physical plant locations of nuclear safety capability equipment.
8. Plant layout drawings that identify plant fire area delineation, areas protected by automatic fire suppression and detection, and the locations of fire protection equipment.
9. Plant layout drawings which identify the general location of the emergency lighting units required to support achieving and maintaining nuclear safety performance criteria.
10. Plant operating procedures for achieving nuclear safety performance criteria from inside the control room with a postulated fire occurring in any plant area outside the control room, procedures for achieving nuclear safety performance criteria in the event of a fire in either the control or cable spreading room, or any area requiring recovery actions to achieve control from other than the main control room or primary control stations.
11. Maintenance and surveillance testing procedures for nuclear safety capability systems and equipment, and fire barriers, detectors, pumps and suppression systems.
12. Maintenance procedures that routinely verify fuse breaker coordination in accordance with the nuclear safety circuit coordination analysis.

13. A sample of significant design change packages and engineering equivalency evaluations related to fire protection and post-fire nuclear safety capability (e.g., Generic Letter 86-10 evaluations).
14. The reactor plant's probabilistic risk assessment (PRA), results of any PRA reviews, and listings of actions taken or plant modifications conducted in response to PRA information (including NFPA 805, Section 2.4.3, Fire Risk Evaluations, and NFPA 805, Section 2.4.4, Plant Change Evaluations).
15. Temporary modification procedures.
16. Organization charts of site personnel, including fire protection staff personnel.
17. If applicable, layout or arrangement drawings of potential reactor coolant/recirculation pump lube oil system leakage points and associated lube oil collection systems.
18. A listing of the safety evaluations that form the licensing basis for the reactor plant's post-fire nuclear safety capability.
19. Procedures or instructions that control the configuration of the reactor plant's fire protection program, features, and post-fire nuclear safety capability methodology and system design.
20. A list of applicable codes and standards related to the design of plant fire protection features and evaluations of code deviations, including any safety evaluations issued for deviations from NFPA 805 fundamental fire protection program and design elements (NFPA 805, Chapter 3).
21. Procedures or instructions that govern the implementation of plant modifications, maintenance, and special operations, and their impact on fire protection.
22. The three most recent fire protection quality assurance (QA) audits and/or self-assessments.
23. Recent QA surveillances of fire protection activities.
24. A listing of open and closed fire protection condition reports (problem reports / nuclear condition reports (NCRs) / event analysis reports (EARs) / problem identification and resolution reports).
25. Listing of plant fire protection program and licensing basis documents.
26. A listing of the NFPA code versions committed to (NFPA codes of record).
27. A listing of plant deviations from code commitments.
28. Actual copies of engineering equivalency evaluations (e.g., Generic Letter 86-10 evaluations).

Mitigating Strategies Support Documentation

Note: This is a broad list of the documents the NRC inspection team may be interested in reviewing, and possibly obtaining, during the information gathering visit. The current version of these documents is expected unless specified otherwise. Electronic media is preferred, if readily available. (The preferred file format on CD-ROM or DVDROM is ".pdf"). The CD/DVD-ROM should be indexed, hyperlinked, readable, and searchable to facilitate ease of use. Please provide 5 copies of each CD/DVD-ROM submitted). The lead inspector will discuss specific information needs with the licensee staff and may request additional documents or electronic information.

1. A list of all modifications to regulatory commitments made to meet the requirements of Section B.5.b of the ICM Order, EA-02-026, dated February 25, 2002, the subsequently imposed license conditions, and 10 CFR 50.54(hh)(2).
2. Copies of procedures/guidelines that were revised or generated to implement the mitigation strategies. These could be extensive damage mitigation guidelines (EDMGs), severe accident management guidelines (SAMGs), emergency operating procedures (EOPs), abnormal operating procedures (AOPs), etc.
3. A matrix that shows the correlation between the mitigation strategies identified in Nuclear Energy Institute 06-12 and the site-specific procedures or guidelines that are used to implement each strategy.
4. Engineering evaluations/calculations that were used to verify engineering bases for the mitigation strategies.
5. Piping and instrumentation diagram (P&ID) or simplified flow diagrams for systems relied upon in the mitigation strategies. These could be the type used for training.
6. A modification package or simplified drawings/descriptions of modifications that were made to plant systems to implement the mitigation strategies.
7. Copies of procedures used to inventory equipment (hoses, fittings, pumps, etc.) required to be used to implement the mitigation strategies.
8. A list of B.5.b strategies, if any, which have implementing details that differ from that documented in the submittals and the safety evaluation report.
9. A copy of site general arrangement drawing(s) that show the majority of buildings/areas referenced in B.5.b documents.
10. Training records/training matrix/lesson plans related to B.5.b.
11. Copies of Memoranda of Understanding (MOUs) (e.g., with local fire departments) required to implement any mitigating strategies.