



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

November 3, 2009

Mr. Charles G. Pardee
President and Chief Nuclear Officer
Exelon Nuclear
4300 Winfield Road
Warrenville, IL 60555

SUBJECT: THREE MILE ISLAND NUCLEAR STATION, UNIT 1 - ONCE-THROUGH
STEAM GENERATOR TUBE LOADS UNDER CONDITIONS RESULTING
FROM POSTULATED BREAKS IN REACTOR COOLANT SYSTEM UPPER
HOT-LEG LARGE-BORE PIPING (TAC NO. ME1797)

Dear Mr. Pardee:

On June 25, 2009, a public meeting was held between the U.S. Nuclear Regulatory Commission (NRC or Commission) staff and representatives of the Pressurized Water Reactor Owners Group (PWROG) at NRC Headquarters, One White Flint North, 11555 Rockville Pike, Rockville, Maryland, regarding once-through steam generator tube loads under conditions resulting from postulated breaks in reactor coolant system upper hot-leg large-bore piping.

In the meeting, the NRC requested that each Babcock and Wilcox licensee submit a letter summarizing the information discussed at the meeting. The NRC staff followed up this verbal request with a letter dated August 5, 2009 (Agencywide Documents Access and Management System (ADAMS) Accession No. ML092120053).

Exelon Generation Company, LLC (the licensee) provided a response in its letter dated September 4, 2009 (ADAMS Accession No. ML092470426), for Three Mile Island Nuclear Station, Unit 1 (TMI-1). The NRC staff has reviewed the response and provides the following comments:

- The staff has received your response on the reporting requirements of Section 50.46 "Acceptance criteria for emergency core cooling systems for light-water nuclear power reactors," of Title 10 of the *Code of Federal Regulations* (10 CFR). We will contact you if we wish to have further discussions regarding your response.
- Based on its review of this letter, the staff understands that you will be verifying that the design of your replacement steam generators is sufficient to withstand large break loss-of-coolant accident (LBLOCA) loading conditions. In addition, the staff understands that steam generator tube integrity for the replacement steam generators will be maintained for all LBLOCAs (including those in the candy-cane region) as required by Technical Specification (TS) 3.1.1.2, "Steam Generator (SG) Tube Integrity," and TS 6.19, "Steam Generator (SG) Program."

C. Pardee

- 2 -

- In your letter dated September 4, 2009, you committed to submit the results of the analysis of LBLOCA loads on the replacement steam generators by January 31, 2010. Subsequent to this submittal, the TMI-1 Updated Final Safety Analysis Report must be updated to reflect this information. Specifically, 10 CFR 50.71(e) requires that licensees update their Final Safety Analysis Report to reflect; (1) information and analyses submitted to the Commission, and/or, (2) evaluations performed in support of conclusions that changes to the facility do not require a license amendment in accordance with 10 CFR 50.59(c)(2).

Based upon the above discussion, the NRC will be closing TAC No. ME1797. If you have any questions, please contact me at 301-415-2833 or by e-mail at Peter.Bamford@nrc.gov.

Sincerely,



Peter J. Bamford, Project Manager
Plant Licensing Branch I-2
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket No. 50-289

cc: Distribution via Listserv

C. Pardee

- 2 -

- In your letter dated September 4, 2009, you committed to submit the results of the analysis of LBLOCA loads on the replacement steam generators by January 31, 2010. Subsequent to this submittal, the TMI-1 Updated Final Safety Analysis Report must be updated to reflect this information. Specifically, 10 CFR 50.71(e) requires that licensees update their Final Safety Analysis Report to reflect; (1) information and analyses submitted to the Commission, and/or, (2) evaluations performed in support of conclusions that changes to the facility do not require a license amendment in accordance with 10 CFR 50.59(c)(2).

Based upon the above discussion, the NRC will be closing TAC No. ME1797. If you have any questions, please contact me at 301-415-2833 or by e-mail at Peter.Bamford@nrc.gov.

Sincerely,

/ra/

Peter J. Bamford, Project Manager
Plant Licensing Branch I-2
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket No. 50-289

cc: Distribution via Listserv

DISTRIBUTION:

PUBLIC

LPL1-2 Reading

RidsAcrcAcnw_MailCTR Resource

RidsNrrDorlDpr Resource

RidsNrrDorlLpl1-2 Resource

RidsNrrDraAadb Resource

RidsNrrDss Resource

RidsNrrLAABaxter Resource

RidsNrrPMTThreeMileIsland Resource

RidsOgcRp Resource

RidsRgn1MailCenter Resource

KKarwoski, NRR/DCI

ADAMS Accession No. ML093010455

OFFICE	NRR/LPL4/PM	NRR/LPL1-2/PM	NRR/LPL4/LA	NRR/DCI	DRA/AADB/BC	NRR/DSS/D	NRR/LPL1-2/BC	NRR/LPL1-2/PM
NAME	NKalyanam	PBamford	JBurkhardt	KKarwoski	RTaylor	WRuland, SBahadur for	HChernoff	PBamford
DATE	10/29/09	11/02/2009	10/29/09	10/29/09	10/30/09	10/30/2009	11/03/09	11/03/09

OFFICIAL RECORD COPY