

August 20, 2009

Mr. Jon Franke, Vice President
Crystal River Nuclear Plant (NA1B)
ATTN: Supervisor, Licensing & Regulatory Programs
15760 W. Power Line Street
Crystal River, FL 34428-6708

SUBJECT: REQUEST FOR ADDITIONAL INFORMATION FOR THE REVIEW OF THE
CRYSTAL RIVER UNIT 3 NUCLEAR GENERATING PLANT, LICENSE
RENEWAL APPLICATION (TAC NO. ME0274)

Dear Mr. Franke:

By letter dated December 16, 2008, Florida Power Corporation submitted an application pursuant to Title 10 of the *Code of Federal Regulations Part 54*, to renew the operating licenses Crystal River Unit 3 Nuclear Generating Plant (CR-3), for review by the U.S. Nuclear Regulatory Commission (NRC or the staff). The staff is reviewing the information contained in the license renewal application and has identified, in the enclosure, areas where additional information is needed to complete the review. Further requests for additional information may be issued in the future.

Items in the enclosure were discussed with Mr. Michael Heath, and a mutually agreeable date for the response is within 30 days from the date of this letter. If you have any questions, please contact me at 301-415-3733 or by e-mail robert.kuntz@nrc.gov.

Sincerely,

/RA/

Robert F. Kuntz, Sr. Project Manager
Projects Branch 2
Division of License Renewal
Office of Nuclear Reactor Regulation

Docket No. 50-302

Enclosure:
As stated

cc w/encl: See next page

August 20, 2009

Mr. Jon Franke, Vice President
Crystal River Nuclear Plant (NA1B)
ATTN: Supervisor, Licensing & Regulatory Programs
15760 W. Power Line Street
Crystal River, FL 34428-6708

SUBJECT: REQUEST FOR ADDITIONAL INFORMATION FOR THE REVIEW OF THE
CRYSTAL RIVER UNIT 3 NUCLEAR GENERATING PLANT, LICENSE
RENEWAL APPLICATION (TAC NO. ME0274)

Dear Mr. Franke:

By letter dated December 16, 2008, Florida Power Corporation submitted an application pursuant to Title 10 of the *Code of Federal Regulations Part 54*, to renew the operating licenses Crystal River Unit 3 Nuclear Generating Plant (CR-3), for review by the U.S. Nuclear Regulatory Commission (NRC or the staff). The staff is reviewing the information contained in the license renewal application and has identified, in the enclosure, areas where additional information is needed to complete the review. Further requests for additional information may be issued in the future.

Items in the enclosure were discussed with Mr. Michael Heath, and a mutually agreeable date for the response is within 30 days from the date of this letter. If you have any questions, please contact me at 301-415-3733 or by e-mail robert.kuntz@nrc.gov.

Sincerely,

/RA/

Robert F. Kuntz, Sr. Project Manager
Projects Branch 2
Division of License Renewal
Office of Nuclear Reactor Regulation

Docket No. 50-302

Enclosure:
As stated

cc w/encl: See next page

DISTRIBUTION: See next page
ADAMS Accession No.:ML092250701

OFFICE	PM:RPB2:DLR	LA:DLR	BC:RER2:DLR	BC:RPB2:DLR	PM:RPB2:DLR
NAME	RKuntz	SFiguroa	RAuluck	DWrona	RKuntz
DATE	08/18/09	08/18/09	08/19/09	08/20/09	08/20/09

OFFICIAL RECORD COPY

REQUEST FOR ADDITIONAL INFORMATION
LICENSE RENEWAL APPLICATION FOR
CRYSTAL RIVER UNIT 3
DOCKET NO: 50-302
(TAC NO. ME0274)

RAI 2.1-1

Background

10 CFR 54.4(a)(1) requires that safety-related systems, structures, and components required to be within the scope of license renewal are those which are relied upon to remain functional during and following design basis events to ensure (i) the integrity of the reactor coolant pressure boundary; (ii) the capability to shut down the reactor and maintain it in a safe shutdown condition; or (iii) the capability to prevent or mitigate the consequences of accidents which could result in potential offsite exposures comparable to those referred to in 10 CFR 50.34(a)(1), 50.67(b)(2), or 100.11. During the scoping and screening methodology audit, performed June 23-25, 2009, the staff reviewed the applicant's implementing documents and reports relative to the evaluation of safety-related SSC's for inclusion within the scope of license renewal.

Issue

The staff determined that there were components identified as safety-related in the equipment data base which were not included within the scope for license renewal in accordance with 10 CFR 54.4(a)(1) as follows:

- As discussed in implementing documents and reports, the applicant had identified SSC's located in the turbine building that were designated as safety-related in the equipment database but were evaluated and determined to not meet the criteria of 10 CFR 54.4(a)(1). In addition, certain SSC's are connected to safety-related cables attached to safety-related buses.
- As discussed in implementing documents and reports, safety-related supports are attached to a common wall between the turbine building and the intermediate building, and located within the turbine building space. However, although the supports are physically located within the turbine building, they have been evaluated as a portion of the intermediate building.

Request

The staff requests that the applicant provide the basis for not including the safety-related SSC's and supports, located within the turbine building, in accordance with 10 CFR 54.4(a)(1), and, as applicable, for not performing an evaluation of nonsafety-related SSC's, located within the same space, for inclusion within the scope of license in accordance with 10 CFR 54.4(a)(2).

List any additional SSCs which were included within the scope of license renewal as a result of your review performed in response to this RAI and list those structures and components for which aging management reviews were conducted. For each additional structure and

component, describe the aging management programs, as applicable, to be credited for managing the identified aging effects.

RAI 2.1-2

Background

10 CFR 54.4(a)(2) requires that nonsafety-related SSC's whose failure could prevent satisfactory accomplishment of functions identified in 10 CFR 54.4(a)(1) be included within the scope of license renewal. During the scoping and screening methodology audit, performed June 23-25, 2009, the staff reviewed the applicant's implementing documents and reports relative to the evaluation of nonsafety-related SSC's for inclusion within the scope of license renewal.

Issue

The staff was not able to determine that nonsafety-related pipe, attached to safety-related SSC's and extending beyond the space or structure containing the nonsafety-related/safety-related interface, was included within the scope of license renewal up to, and including, an anchor, equivalent anchor or bounding condition.

Request

The staff requests that the applicant provide a discussion on the methodology used to determine the portion of nonsafety-related pipe, attached to safety-related SSC's, where the nonsafety-related pipe extends beyond the space or structure containing the nonsafety-related/safety-related interface, to be included within the scope of license renewal in accordance with 10 CFR 54.4(a)(2). In addition, provide a discussion on the use of anchors, equivalent anchors and bounding conditions used as the boundary for nonsafety-related pipe, attached to safety-related SSC's, included within the scope of license renewal.

Issue

The staff determined that the applicant had performed an evaluation to not include fluid filled nonsafety-related SSC's located in specific portions of structures, which also contained safety-related SSC's, within the scope of license renewal. This evaluation identified approximately five areas located in the auxiliary building which contained low or moderate energy fluid-filled nonsafety-related SSCs in addition to safety-related pipe or cables and connectors (which had been included within the scope of license renewal in accordance with 10 CFR 54.4(a)(1)). However, the low or moderate energy fluid-filled nonsafety-related SSC's had not been included within the scope of license renewal in accordance with 10 CFR 54.4(a)(2).

Request

The staff requests that the applicant provide the basis for not including fluid-filled nonsafety-related SSC's, located in the same space as safety-related SSC's, within the scope of license renewal in accordance with 10 CFR 54.4(a)(2).

Issue

The staff identified several open penetrations in a wall connecting the turbine building to the intermediate building. The staff was not able to determine if the applicant had performed an evaluation to identify the potential effect of the failure of nonsafety-related SSC's located in the turbine building on safety-related SSC's located in the intermediate building.

Request

The staff requests that the applicant provide the basis for not including fluid-filled nonsafety-related SSC's, located in the turbine building, which have the potential to interact with safety-related SSC's located within the intermediate building, within the scope of license renewal in accordance with 10 CFR 54.4(a)(2).

List any additional SSCs which were included within the scope of license renewal as a result of your review performed in response to this RAI and list those structures and components for which aging management reviews were conducted. For each additional structure and component, describe the aging management programs, as applicable, to be credited for managing the identified aging effects.

Letter to J. Franke from R. Kuntz, dated August 20, 2009

DISTRIBUTION:

SUBJECT: REQUEST FOR ADDITIONAL INFORMATION FOR THE REVIEW OF THE
CRYSTAL RIVER UNIT 3 NUCLEAR GENERATING PLANT, LICENSE
RENEWAL APPLICATION (TAC NO. ME0274)

HARD COPY:

DLR RF

E-MAIL:

PUBLIC

RidsNrrDlr Resource

RidsNrrDlrRpb1 Resource

RidsNrrDlrRpb2 Resource

RidsNrrDlrRer1 Resource

RidsNrrDlrRer2 Resource

RidsNrrDlrRerb Resource

RidsNrrDlrRpob Resource

RidsNrrDciCvib Resource

RidsNrrDciCpnb Resource

RidsNrrDciCsgb Resource

RidsNrrDraAfpb Resource

RidsNrrDraApla Resource

RidsNrrDeEmcb Resource

RidsNrrDeEeeb Resource

RidsNrrDssSrxb Resource

RidsNrrDssSbpb Resource

RidsNrrDssScvb Resource

RidsOgcMailCenter

RKuntz

DBrittner

AJones, OGC

LLake, RII

MSykes, RII

TMorrissey, RII

RReyes, RI

Crystal River Nuclear Generating
Plant, Unit 3

cc:

Mr. R. Alexander Glenn
Associate General Counsel (MAC-BT15A)
Florida Power Corporation
P.O. Box 14042
St. Petersburg, FL 33733-4042

Mr. James W. Holt
Plant General Manager
Crystal River Nuclear Plant (NA2C)
15760 W. Power Line Street
Crystal River, FL 34428-6708

Mr. William A. Passetti, Chief
Department of Health
Bureau of Radiation Control
2020 Capital Circle, SE, Bin #C21
Tallahassee, FL 32399-1741

Attorney General
Department of Legal Affairs
The Capitol
Tallahassee, FL 32304

Mr. Ruben D. Almaluer, Director
Division of Emergency Preparedness
Department of Community Affairs
2740 Centerview Drive
Tallahassee, FL 32399-2100

Chairman
Board of County Commissioners
Citrus County
110 North Apopka Avenue
Inverness, FL 34450-4245

Mr. Stephen J. Cahill
Engineering Manager
Crystal River Nuclear Plant (NA2C)
15760 W. Power Line Street
Crystal River, FL 34428-6708

Mr. Daniel R. Westcott
Supervisor, Licensing & Regulatory
Programs
Crystal River Nuclear Plant
15760 W. Power Line Street
Crystal River, FL 34428-6708

Mr. Jon A. Franke, Vice President
Crystal River Nuclear Plant (NA1B)
ATTN: Supervisor, Licensing & Regulator
Programs
15760 W. Power Line Street
Crystal River, FL 34428-6708

Senior Resident Inspector
Crystal River Unit 3
U.S. Nuclear Regulatory Commission
6745 N. Tallahassee Road
Crystal River, FL 34428

Ms. Phyllis Dixon
Manager, Nuclear Assessment
Crystal River Nuclear Plant (NA2C)
15760 W. Power Line Street
Crystal River, FL 34428-6708

Mr. David T. Conley
Associate General Counsel II - Legal Dept.
Progress Energy Service Company, LLC
Post Office Box 1551
Raleigh, NC 27602-1551

Mr. Daniel L. Roderick
Vice President, Nuclear Projects &
Construction
Crystal River Nuclear Plant (SA2C)
15760 W. Power Line Street
Crystal River, FL 34428-6708

Mr. Mark Rigsby
Manager, Support Services - Nuclear
Crystal River Nuclear Plant (SA2C)
15760 W. Power Line Street
Crystal River, FL 34428-6708

Mr. Robert J. Duncan II
Vice President, Nuclear Operations
Progress Energy
Post Office Box 1551
Raleigh, NC 27602-1551

Mr. Brian C. McCabe
Manager, Nuclear Regulatory Affairs
Progress Energy
Post Office Box 1551
Raleigh, NC 27602-1551