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August 10, 2009

UN#09-336

ATTN: Document Control Desk  
U.S. Nuclear Regulatory Commission  
Washington, DC 20555-0001

Subject: UniStar Nuclear Energy, NRC Docket No. 52-016  
Response to Request for Additional Information for the  
Calvert Cliffs Nuclear Power Plant, Unit 3,  
RAI No. 124, ASME Code Class 1, 2, and 3 Components

Reference: John Rycyna (NRC) to Robert Poche (UniStar Nuclear Energy), "RAI No 124  
EMB1 2472.doc" email dated July 14, 2009

The purpose of this letter is to respond to the request for additional information (RAI) identified in the NRC e-mail correspondence to UniStar Nuclear Energy, dated July 14, 2009 (Reference). This RAI addresses ASME Code Class 1, 2, and 3 Components, as discussed in Section 3.9.3 of the Final Safety Analysis Report (FSAR), as submitted in Part 2 of the Calvert Cliffs Nuclear Power Plant (CCNPP) Unit 3 Combined License Application (COLA), Revision 5.

The enclosure provides our response to RAI No. 124, Question 03.09.03-1. Our response to Question 03.09.03-1 does not include any new regulatory commitments and does not impact COLA content.

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If there are any questions regarding this transmittal, please contact me at (410) 470-4205, or Mr. Michael J. Yox at (410) 495-2436.

*I declare under penalty of perjury that the foregoing is true and correct.*

Executed on August 10, 2009



Greg Gibson

Enclosure: Response to NRC Request for Additional Information RAI No. 124, Question 03.09.03-1, ASME Code Class 1, 2, and 3 Components, Calvert Cliffs Nuclear Power Plant, Unit 3

cc: John Rycyna, NRC Project Manager, U.S. EPR COL Application  
Laura Quinn, NRC Environmental Project Manager, U.S. EPR COL Application  
Getachew Tesfaye, NRC Project Manager, U.S. EPR DC Application (w/o enclosure)  
Loren Plisco, Deputy Regional Administrator, NRC Region II (w/o enclosure)  
Silas Kennedy, U.S. NRC Resident Inspector, CCNPP, Units 1 and 2  
U.S. NRC Region I Office

GTG/SFW/kat

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**Enclosure**

**Response to NRC Request for Additional Information RAI No. 124  
Question 03.09.03-1, ASME Code Class 1, 2, and 3 Components  
Calvert Cliffs Nuclear Power Plant, Unit 3**

**RAI No. 124**

**Question 03.09.03-1**

SRP 3.9.3 Appendix A, Section 7.A. (iv) states that the staff "may request the submission of the Code-required Design Documents (such as Design Specifications, Design Reports, Load Capacity Data Sheets, or other related material or portions thereof), in order to establish that the design criteria, the analytical methods, and functional capability satisfy the guidance provided by [SRP 3.9.3 Appendix A]. This may include information provided to, and received from, component and support manufacturers. As an alternative to the applicant submitting these documents, the staff may request them to be made available for review at the applicant's or vendor's office."

The Calvert Cliffs Nuclear Power Plant (CCNPP) Unit 3 Final Safety Analysis Report (FSAR), Rev. 3, Section 3.9.3 states:

"{Constellation Generation Group and UniStar Nuclear Operating Services} shall prepare the design specifications and design reports for ASME Class 1, 2, and 3 components that comply with and are certified to the requirements of Section III of the ASME Code (ASME 2004). The design specifications shall be prepared prior to procurement of the components while the ASME code reports shall be prepared during as-built reconciliation of the systems and components conducted before fuel load."

In view of the guidance provided in SRP 3.9.3, as stated above, the staff requests the applicant to provide a specific schedule when the staff can audit the design specifications of risk significant (in the D-RAP List) ASME class 1, 2, and 3 components.

Additionally, the names of the applicants should be corrected in FSAR Section 3.9.3 and throughout the FSAR.

**Response**

In response to U.S. EPR Design Certification Application RAI No. 178 Question 03.09.03-18<sup>a</sup>, dates were provided for availability of design specifications for audit. The following table provides design specifications and typical design specification audit availability dates for U.S. EPR ASME Code Class 1, 2 and 3 Components.

U.S. EPR ASME Code Class 1, 2 and 3 Component Audit Availability Dates	
Design specifications for safety-related ASME Code Class 1 reactor coolant system (RCS) heavy components (e.g., reactor pressure vessel, steam generator, pressurizer), piping and supports	04/2009
Typical design specification for safety-related ASME Code Class 2 and 3 pumps	04/2009
Typical design specification for ASME Code Class 2 and 3 heat exchangers and ASME Code Class 2 and 3	06/2009

<sup>a</sup> R. Wells (AREVA NP INC) to G. Tesfaye (NRC), "Response to U.S. EPR Design Certification Application RAI No. 178, FSAR Ch 3" email dated February 24, 2009 (ML090550453)

While design specifications have not been prepared for each U.S. EPR reliability assurance program (RAP) item, typical design specifications are available to audit. This will provide NRC sufficient information to perform a review of the design certification application, since design specifications for each type of safety-related component have consistent design requirements.

The anticipated availability date of the design specifications for each of the site specific components in the RAP is presented in the table below. This represents the best date currently available and is subject to modification as the detailed design and engineering schedule is finalized.

System	Component General ID	Component Description	Component Type	Specification Schedule Activity ID	Forecast Completion Date <sup>b</sup>
ESWS	30PEBX0AA004	ESWS Pump Discharge Check Valves	Check Valves	PV021PD1000	08/2012
ESWS	30PEBX0AA005	ESWS Pump Discharge Isolation MOVs	MOV	PV021PD1000	08/2012
ESWS	30PEBX0AP001	ESWS Motor Driven Pumps	Pump	PEOME1300	03/2010
ESWS	30PEDX0AN001	UHS Cooling Tower Cooling Fans	Fan	PEOME1076	03/2010
ESWS	30PEDX0AN002	UHS Cooling Tower Cooling Fans	Fan	PEOME1076	03/2010
MFWS	30LAB3XAA001	FWS HP Heater Bypass Pneumatic Valve	Pneumatic Valve	JVO7IC1050	10/2010
MFWS	30LAB3XAA002	FWS HP Heater Bypass Pneumatic Valve	Pneumatic Valve	JVO7IC1050	10/2010

The applicant names were corrected in Calvert Cliffs Nuclear Power Plant Unit 3 COLA Revision 5.

**COLA Impact**

The COLA FSAR will not be revised as a result of this response.

<sup>b</sup> This represents the best date currently available and is subject to modification.