



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

September 28, 2009

Mr. James A. Spina, Vice President  
Calvert Cliffs Nuclear Power Plant, Inc.  
Calvert Cliffs Nuclear Power Plant  
1650 Calvert Cliffs Parkway  
Lusby, MD 20657-4702

SUBJECT: CALVERT CLIFFS NUCLEAR POWER PLANT, UNIT NOS. 1 AND 2 -  
AMENDMENT RE: REVISIONS TO INSERVICE TEST PROGRAM TECHNICAL  
SPECIFICATIONS (TAC NOS. ME1274 AND ME1275)

Dear Mr. Spina:

The Commission has issued the enclosed Amendment No. 294 to Renewed Facility Operating License No. DPR-53 and Amendment No. 270 to Renewed Facility Operating License No. DPR-69 for the Calvert Cliffs Nuclear Power Plant, Unit Nos. 1 and 2. These amendments consist of changes to the Technical Specifications (TSs) in response to your application transmitted by letter dated May 13, 2009.

These amendments revise TS 5.5.8, "Inservice Testing Program," by incorporating TS Task Force Traveler (TSTF) 479, "Changes to Reflect Revision of 10 CFR 50.55a," and TSTF-497, "Limit Inservice Testing Program SR [Surveillance Requirement] 3.0.2 Application to Frequencies of 2 Years or Less." Specifically, the amendments (1) replace references to the American Society of Mechanical Engineers (ASME) Boiler and Pressure Vessel Code, Section XI with the ASME Code for Operation and Maintenance of Nuclear Power Plants for inservice testing activities, and (2) applies the extension allowance of SR 3.0.2 to other normal and accelerated inservice testing frequencies of 2 years or less that were not included in the frequencies of the table listed in TS 5.5.8.a.

A copy of the related Safety Evaluation is enclosed. A Notice of Issuance will be included in the Commission's next regular biweekly *Federal Register* notice.

Sincerely,

A handwritten signature in black ink that reads "Douglas V. Pickett".

Douglas V. Pickett, Senior Project Manager  
Plant Licensing Branch I-1  
Division of Operating Reactor Licensing  
Office of Nuclear Reactor Regulation

Docket Nos. 50-317 and 50-318

Enclosures:

1. Amendment No. 294 to DPR-53
2. Amendment No. 270 to DPR-69
3. Safety Evaluation

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UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

CALVERT CLIFFS NUCLEAR POWER PLANT, INC.

DOCKET NO. 50-317

CALVERT CLIFFS NUCLEAR POWER PLANT, UNIT NO. 1

AMENDMENT TO RENEWED FACILITY OPERATING LICENSE

Amendment No. 294  
Renewed License No. DPR-53

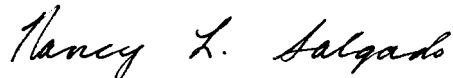
1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Calvert Cliffs Nuclear Power Plant, Inc. (the licensee) dated May 13, 2009, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.2. of Renewed Facility Operating License No. DPR-53 is hereby amended to read as follows:

2. Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 294, are hereby incorporated into the renewed license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance and shall be implemented within 60 days.

FOR THE NUCLEAR REGULATORY COMMISSION



Nancy L. Salgado, Chief  
Plant Licensing Branch I-1  
Division of Operating Reactor Licensing  
Office of Nuclear Reactor Regulation

Attachment:  
Changes to the License and Technical  
Specifications

Date of Issuance: September 28, 2009



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

CALVERT CLIFFS NUCLEAR POWER PLANT, INC.

DOCKET NO. 50-318

CALVERT CLIFFS NUCLEAR POWER PLANT, UNIT NO. 2

AMENDMENT TO RENEWED FACILITY OPERATING LICENSE

Amendment No. 270  
Renewed License No. DPR-69

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Calvert Cliffs Nuclear Power Plant, Inc. (the licensee) dated May 13, 2009, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.2. of Renewed Facility Operating License No. DPR-69 is hereby amended to read as follows:

2. Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 270, are hereby incorporated in the renewed license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance and shall be implemented within 60 days.

FOR THE NUCLEAR REGULATORY COMMISSION



Nancy L. Salgado, Chief  
Plant Licensing Branch I-1  
Division of Operating Reactor Licensing  
Office of Nuclear Reactor Regulation

Attachment:  
Changes to the License and Technical  
Specifications

Date of Issuance: September 28, 2009

ATTACHMENT TO LICENSE AMENDMENTS

AMENDMENT NO. 294 TO RENEWED FACILITY OPERATING LICENSE NO. DPR-53

AMENDMENT NO. 270 TO RENEWED FACILITY OPERATING LICENSE NO. DPR-69

DOCKET NOS. 50-317 AND 50-318

Replace the following page of the Facility Operating License with the attached revised page. The revised page is identified by amendment number and contains marginal lines indicating the areas of change.

Remove Page

3

Insert Page

3

Replace the following page of the Appendix A Technical Specifications with the attached revised page. The revised page is identified by amendment number and contains marginal lines indicating the areas of change.

Remove Page

5.5-7

Insert Page

5.5-7

rules, regulations, and orders of the Commission, now or hereafter applicable; and is subject to the additional conditions specified and incorporated below:

(1) Maximum Power Level

The licensee is authorized to operate the facility at steady-state reactor core power levels not in excess of 2737 megawatts-thermal in accordance with the conditions specified herein.

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 294, are hereby incorporated into this license. The licensee shall operate the facility in accordance with the Technical Specifications.

- (a) For Surveillance Requirements (SRs) that are new, in Amendment 227 to Facility Operating License No. DPR-53, the first performance is due at the end of the first surveillance interval that begins at implementation of Amendment 227. For SRs that existed prior to Amendment 227, including SRs with modified acceptance criteria and SRs whose frequency of performance is being extended, the first performance is due at the end of the first surveillance interval that begins on the date the Surveillance was last performed prior to implementation of Amendment 227.

(3) Additional Conditions

The Additional Conditions contained in Appendix C as revised through Amendment No. 267 are hereby incorporated into this license. Calvert Cliffs Nuclear Power Plant, Inc. shall operate the facility in accordance with the Additional Conditions.

(4) Secondary Water Chemistry Monitoring Program

The Calvert Cliffs Nuclear Power Plant, Inc., shall implement a secondary water chemistry monitoring program to inhibit steam generator tube degradation. This program shall include:

- a. Identification of a sampling schedule for the critical parameters and control points for these parameters;
- b. Identification of the procedures used to quantify parameters that are critical to control points;

C. This license is deemed to contain and is subject to the conditions set forth in 10 CFR Chapter I and is subject to all applicable provisions of the Act, and the rules, regulations, and orders of the Commission, now and hereafter applicable; and is subject to the additional conditions specified and incorporated below:

(1) Maximum Power Level

The licensee is authorized to operate the facility at reactor steady-state core power levels not in excess of 2737 megawatts-thermal in accordance with the conditions specified herein.

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 270 are hereby incorporated into this license. The licensee shall operate the facility in accordance with the Technical Specifications.

(a) For Surveillance Requirements (SRs) that are new, in Amendment 201 to Facility Operating License No. DPR-69, the first performance is due at the end of the first surveillance interval that begins at implementation of Amendment 201. For SRs that existed prior to Amendment 201, including SRs with modified acceptance criteria and SRs whose frequency of performance is being extended, the first performance is due at the end of the first surveillance interval that begins on the date the Surveillance was last performed prior to implementation of Amendment 201.

(3) Less Than Four Pump Operation

The licensee shall not operate the reactor at power levels in excess of five (5) percent of rated thermal power with less than four (4) reactor coolant pumps in operation. This condition shall remain in effect until the licensee has submitted safety analyses for less than four pump operation, and approval for such operation has been granted by the Commission by amendment of this license.

(4) Environmental Monitoring Program

If harmful effects or evidence of irreversible damage are detected by the biological monitoring program, hydrological monitoring program, and the radiological monitoring program specified in the Appendix B Technical Specifications, the licensee will provide to the staff a detailed analysis of the problem and a program of remedial action to be taken to eliminate or significantly reduce the detrimental effects or damage.



5.5 Programs and Manuals

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- a. Testing frequencies applicable to the ASME Code for Operation and Maintenance of Nuclear Power Plants (ASME OM Code) and applicable Addenda as follows:

ASME OM Code and applicable Addenda terminology for inservice testing activities	Required Frequencies for performing inservice testing activities
Weekly	At least once per 7 days
Monthly	At least once per 31 days
Quarterly or every 3 months	At least once per 92 days
Semiannually or every 6 months	At least once per 184 days
Every 9 months	At least once per 276 days
Yearly or annually	At least once per 366 days
Biennially or every 2 years	At least once per 731 days

- b. The provisions of SR 3.0.2 are applicable to the above required Frequencies and to other normal and accelerated Frequencies specified as 2 years or less in the Inservice Testing Program for performing inservice testing activities;
- c. The provisions of SR 3.0.3 are applicable to inservice testing activities; and
- d. Nothing in the ASME OM Code shall be construed to supersede the requirements of any Technical Specification.

5.5.9 Steam Generator (SG) Program

A Steam Generator Program shall be established and implemented to ensure that SG tube integrity is maintained. In addition, the Steam Generator Program shall include the following provisions:



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 294 TO RENEWED

FACILITY OPERATING LICENSE NO. DPR-53

AND AMENDMENT NO. 270 TO RENEWED FACILITY OPERATING LICENSE NO. DPR-69

CALVERT CLIFFS NUCLEAR POWER PLANT, INC.

CALVERT CLIFFS NUCLEAR POWER PLANT, UNIT NOS. 1 AND 2

DOCKET NOS. 50-317 AND 50-318

1.0 INTRODUCTION

By application dated May 13, 2009 (Agencywide Documents Access and Management System Accession No. ML091380325), Calvert Cliffs Nuclear Power Plant, Inc. (the licensee) requested changes to the Technical Specifications (TSs) for the Calvert Cliffs Nuclear Power Plant, Units Nos. 1 and 2 (CCNPP). The changes revise TS 5.5.8, "Inservice Testing Program," by incorporating TS Task Force Traveler (TSTF) 479, "Changes to Reflect Revision of 10 CFR [Title 10 of the *Code of Federal Regulations*] 50.55a," and TSTF-497, "Limit Inservice Testing Program SR [Surveillance Requirement] 3.0.2 Application to Frequencies of 2 Years or Less." Specifically, the amendments (1) replace references to the American Society of Mechanical Engineers (ASME) Boiler and Pressure Vessel Code, Section XI with the ASME Code for Operation and Maintenance of Nuclear Power Plants (OM Code) for inservice testing (IST) activities, and (2) applies the extension allowance of SR 3.0.2 to other normal and accelerated IST frequencies of 2 years or less that were not included in the frequencies of the table listed in TS 5.5.8.a.

2.0 REGULATORY EVALUATION

10 CFR Section 50.55a(f)(5)(ii) requires that, if a revised inservice test program for a facility conflicts with the TSs for that facility, the licensee shall apply to the Nuclear Regulatory Commission (NRC) for amendment of the TSs to conform the TSs to the revised program. The licensee is required to submit the application, as specified in 10 CFR 50.4, at least 6 months before the start of the period during which the provisions become applicable, as determined by 10 CFR 50.55a(f)(4).

In 1990, the ASME published the initial edition of the ASME OM Code, which provides requirements for IST of pumps and valves. The ASME OM Code was developed and is maintained by the ASME Committee on Operation and Maintenance of Nuclear Power Plants. The ASME OM Code was developed in response to the ASME Board on Nuclear Codes and Standards directive that transferred responsibility for development and maintenance of

requirements for the IST of pumps and valves from the ASME Section XI Subcommittee on Nuclear Inservice Inspection to the ASME OM Committee. The ASME intended the ASME OM Code to replace Section XI rules for IST of pumps and valves, and the Section XI requirements for IST of pumps and valves that had been incorporated by reference into NRC regulations were deleted from Section XI in the 2000 Addenda. The CCNPP fourth 10-year interval IST programs were developed to meet the requirements of the 2004 Edition of the ASME OM Code pursuant to 10 CFR 50.55a(f)(4)(ii) as required by 10 CFR 50.55a(f)(4). The TS 5.5.8 reference to Section XI of the ASME Code for IST requirements results in a reference to a deleted portion of the Section XI ASME Code. The licensee submitted this TS amendment to revise the TS to reference the current ASME OM Code requirements. The fourth 10-year IST interval for CCNPP began on July 1, 2008.

Additionally, this TS amendment incorporates TSTF-479 and TSTF-497. NUREG-1432, "Standard Technical Specifications Combustion Engineering Plants", Revision 3.1, incorporated TSTF-479 in December 2005. TSTF-479 addressed changes to Section 5.5.8, IST Program, in Revision 3.1 of the standard TS, to reflect revisions of 10 CFR 50.55a referencing the ASME OM Code and the application of SR 3.0.2 to test frequencies specified in the IST program. NUREG-1432 was again modified via TSTF-497 in October 2006. This traveler updates the standard TS Section 5.5.8.b to specify that the 25% extension applies only to IST frequencies of 2 years or less.

### 3.0 TECHNICAL EVALUATION

#### 3.1 Specific Changes Requested

The licensee has proposed the following changes to CCNPP TS 5.5.8:

- The references to Section XI of the "ASME Boiler and Pressure Vessel Code" for IST requirements would be replaced with "ASME OM Code" in TS Sections: 5.5.8.a and 5.5.8.d.
- TS 5.5.8.b would be revised to apply SR 3.0.2 to other normal and accelerated frequencies specified as 2 years or less in the IST Program.

#### 3.2 Basis for Changes

TS 5.5.8 establishes the SRs for IST of ASME Class 1, 2, and 3 components for CCNPP. TS Section 5.5.8 currently references Section XI of the "ASME Boiler and Pressure Vessel Code" as the source of requirements for the IST of ASME Code Class 1, 2, and 3 pumps and valves.

The regulations in 10 CFR 50.55a(f)(4) establish the effective Code edition and addenda to be used by licensees for performing IST of pumps and valves. The regulations in 10 CFR 50.55a(f)(4)(ii) require licensees to update their IST program to the latest approved edition and addenda of the ASME OM Code incorporated by reference into 10 CFR 50.55a(b). The licensee states that the IST Program for the CCNPP fourth interval was updated to comply with the appropriate revisions of the ASME OM Code and included the 2004 Edition as the new Code of Record for performing IST at CCNPP. As a consequence, the TS 5.5.8 reference to

Section XI of the ASME Boiler and Pressure Vessel Code results in a reference to a deleted portion of the Code.

According to 10 CFR 50.55a(f)(5)(ii), if a revised IST program for a facility conflicts with the TSs for the facility, the licensee is required to apply to the NRC for amendment of the TSs to conform the TSs to the revised program. The licensee must submit the application, as specified in 10 CFR 50.4, at least 6 months before the start of the period during which the provisions become applicable as determined by 10 CFR 50.55a(f)(4). Since TS 5.5.8 references Section XI of the ASME Boiler and Pressure Vessel Code for the IST requirements for pumps and valves, the TSs for CCNPP require revision to change the IST Code references to the ASME OM Code.

The TSTF recognized that IST programs may have frequencies for testing that are based on risk and do not conform to standard testing frequencies specified in the TSs. Traveler TSTF-479 proposed a change to the standard TS contained in NUREG-1432, Revision 3.0, to extend the applicability of SR 3.0.2 to "other normal and accelerated frequencies specified in the IST program." These changes were incorporated in NUREG-1432, Revision 3.1, dated December 2005. The NRC staff expressed concern that applying the 25% extension permitted by SR 3.0.2 to frequencies in excess of 2 years (such as 5 or 10 years as permitted by the ASME OM Code in certain cases) would be inappropriate and requested a change to TSTF-479 to revise the provision for applying SR 3.0.2 to IST test frequencies. TSTF-497, approved in October 2006, updates standard TS 5.5.8.b to allow the 25% extension only for surveillance intervals of 2 years or less. Application of SR 3.0.2 to frequencies of 2 years or less is consistent with the staff position contained in NUREG-1482, "Guidelines for IST at Nuclear Power Plants."

### 3.3 Evaluation

In 1990, the ASME published the initial edition of the ASME OM Code, which provides requirements for IST of pumps and valves. The OM Code was developed and is maintained by the ASME Committee on Operation and Maintenance of Nuclear Power Plants. The ASME OM Code was developed in response to the ASME Board on Nuclear Codes and Standards directive that transferred responsibility for development and maintenance of rules for the IST of pumps and valves from the ASME Section XI Subcommittee on Nuclear Inservice Inspection to the ASME OM Committee. The ASME intended the ASME OM Code to replace Section XI rules for IST of pumps and valves, and the Section XI rules for IST of pumps and valves that had been incorporated by reference into NRC regulations have been deleted from Section XI in the 2000 Addenda of the ASME Code. The ASME publishes a new edition of the ASME OM Code every 3 years, and a new addendum every year. The CCNPP fourth 10-year interval IST program was updated to comply with the 2004 Edition of the ASME OM Code as required by 10 CFR 50.55a(f)(4)(ii).

As a consequence, the TS 5.5.8 reference to Section XI of the ASME Code for IST requirements results in a reference to a deleted portion of the ASME Code. The TS changes do not eliminate any tests and do not relinquish the licensee of its responsibility to seek relief from Code test requirements when they are impractical. The proposed change of the ASME Code from "ASME Section XI" to "ASME OM Code" will eliminate the ASME Code inconsistency between the IST program and the TSs as required by 10 CFR 50.55a(f)(5)(ii); therefore, the NRC staff finds these proposed changes to be acceptable. Additionally, the proposed changes are consistent with the comparable Section 5.5.8 of the standard TS, contained in NUREG-1432, Revision 3.1.

The licensee's proposed change to TS 5.5.8.b applies SR 3.0.2 to the frequencies specified in TS 5.5.8.a and other normal and accelerated frequencies specified as 2 years or less in the IST program. This change recognizes that the IST program may direct that additional tests be performed in accordance with the ASME OM Code that are not at the standard intervals listed in TS 5.5.8.a. This is consistent with the intent of the 25% extension as described in the Bases for SR 3.0.2, in that the extension would provide operational flexibility, but would not significantly degrade the reliability that results from performing the surveillance at the specified frequency. Further, the licensee's proposal to limit application of SR 3.0.2 to frequencies specified as 2 years or less limits the maximum incremental time period between surveillances that could be added by the 25% extension. Without this limitation, some components, such as safety and relief valves which may be tested at surveillance intervals significantly greater than 2 years, could have extensions applied which would be much greater than needed for operational flexibility. These aspects of the proposed change support Code provisions which provide the basis for the IST program and are consistent with guidance contained in NUREG-1482 regarding maximum allowable extensions of test intervals. Therefore, the NRC staff finds this proposed change to be acceptable.

#### 4.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Maryland State official was notified of the proposed issuance of the amendments. The State official had no comments.

#### 5.0 ENVIRONMENTAL CONSIDERATION

The amendments change a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The NRC staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendments involve no significant hazards consideration, and there has been no public comment on such finding ( 74 FR 34046). Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

#### 6.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above that (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: Michael Orenak

Date: September 28, 2009

September 28, 2009

Mr. James A. Spina, Vice President  
Calvert Cliffs Nuclear Power Plant, Inc.  
Calvert Cliffs Nuclear Power Plant  
1650 Calvert Cliffs Parkway  
Lusby, MD 20657-4702

SUBJECT: CALVERT CLIFFS NUCLEAR POWER PLANT, UNIT NOS. 1 AND 2 -  
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Dear Mr. Spina:

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A copy of the related Safety Evaluation is enclosed. A Notice of Issuance will be included in the Commission's next regular biweekly *Federal Register* notice.

Sincerely,  
*/ra/*  
Douglas V. Pickett, Senior Project Manager  
Plant Licensing Branch I-1  
Division of Operating Reactor Licensing  
Office of Nuclear Reactor Regulation

Docket Nos. 50-317 and 50-318

Enclosures:

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NAME	DPickett	SLittle	LSubin	JMcHale as signed on	RElliott (GWaig for)	NSalgado
DATE	09 / / 09	08 / 06 / 09	09 / 09 / 09	06 / 15 / 09	09 / 25 / 09	09 / 28 / 09

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