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UNITED STATES
NUCLEAR REGULATORY COMMISSION
REGION I
475 ALLENDALE ROAD
KING OF PRUSSIA, PA 19406

November X, 2008

Mr. Charles G. Pardee
Chief Nuclear Officer (CNO) and Senior Vice President
Exelon Generation Company, LLC
Chief Nuclear Officer (CNO)
AmerGen Energy Company, LLC
200 Exelon Way
Kennett Square, PA 19348

SUBJECT: OYSTER CREEK GENERATING STATION – PRELIMINARY EVALUATION RESULTS OF THE PRIMARY CONTAINMENT

Dear Mr. Pardee:

From October 27 to November 6, 2008, the U. S. Nuclear Regulatory Commission (NRC) performed a team inspection at your Oyster Creek Generating Station related to license renewal activities during the recent refueling outage. A part of the inspection involved review of primary containment (drywell) conditions and ultrasonic testing of the drywell shell thickness. In the interest of timely public disclosure, NRC has decided to issue this letter and its enclosure, which address that part of the inspection.

The enclosure to this letter provides a summary of the inspection scope and preliminary evaluation results related to the drywell. Please note that the final inspection results, including the determinations and characterization of their significance, may change based on additional information and further review. The final inspection results will be documented in NRC Inspection Report 05000219/2008007.

For the drywell the NRC review involved a multi-week inspection of AmerGen's inservice inspection program and included an assessment of license renewal commitments for the current outage. Based on the results of the NRC's inspection activities, the NRC concluded that there were no safety significant conditions with respect to the drywell that would prohibit plant startup, and that there is reasonable assurance that the primary containment is capable of performing its design function throughout the upcoming operating cycle.

In accordance with 10 CFR 2.390 of the NRC's "Rules of Practice," a copy of this letter and its enclosure will be available electronically for public inspection in the NRC Public Document Room or from the Publicly Available Records (PARS) component of NRC's document system (ADAMS). ADAMS is accessible from the NRC Website at http://www.nrc.gov/reading-rm/adams.html (the Public Electronic Reading Room).

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We appreciate your cooperation. Please contact me at (610) 337-5183 if you have any questions regarding this letter.

Sincerely,

Richard J. Conte, Chief
Engineering Branch 1
Division of Reactor Safety

Docket No. 50-219
License No. DPR-16

Enclosure: Summary of Drywell Inspection Scope and Preliminary Evaluation Results

cc w/encl:
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M. Pacilio, Chief Operating Officer, Exelon
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Correspondence Control Desk, AmerGen
Mayor of Lacey Township
P. Mulligan, Chief, NJ Dept of Environmental Protection
R. Shadis, New England Coalition Staff
E. Gbur, Chairwoman - Jersey Shore Nuclear Watch
E. Zobian, Coordinator - Jersey Shore Anti Nuclear Alliance
P. Baldauf, Assistant Director, NJ Radiation Protection Programs

We appreciate your cooperation. Please contact me at (610) 337-5200 if you have any questions regarding this letter.

Sincerely,
/RA/
Ronald R. Bellamy, Ph.D., Chief
Projects Branch 6
Division of Reactor Projects

Docket No. 50-219
License No. DPR-16

Enclosure: Summary of Drywell Inspection Scope and Preliminary Evaluation Results

Distribution w/encl: (via e-mail)
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DATE	/08		/08		/08			

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MEMORANDUM TO: Richard J. Conte, Chief
Engineering Branch 1, DRS

FROM: John E. Richmond, Team Leader
Engineering Branch 1, DRS

SUBJECT: SUMMARY OF DRYWELL INSPECTION SCOPE AND PRELIMINARY EVALUATION RESULTS

Something similar to the following from 2006 PN:

The NRC staff inspection throughout the outage focused on:

1) Non-destructive examination results of the drywell shell and torus and related AmerGen evaluations.
2) The condition of the inside of the drywell, including trenches in the floor, and the outside of the drywell shell in the sand bed region.
3) Structural integrity of the concrete drywell floor and the condition of the embedded portion of the drywell shell.
4) The potential impact from various repairs to the containment on the design and licensing bases of the drywell.

The overall results of the staff's observations and review are:

1) All UT measurements are greater than the calculated minimum code required thickness for various plates that form the drywell shell.
2) There are no adverse conditions of the epoxy coating on the outside of the drywell shell in the former sandbed region.
3) Repairs in and around the trough within the reactor vessel pedestal area did not result in any adverse conditions.
4) The water discovered in the drywell trenches had no adverse impact on the structural integrity of the concrete floor or the potential for corrosion of the embedded portion of the drywell shell. AmerGen has taken actions to prevent further accumulation of water in this area.
5) There are no adverse conditions with respect to the drywell or torus structural integrity that preclude restart.

Based on a review of the technical information, the NRC staff determined that AmerGen has sufficient justification to restart Oyster Creek.