

**Elizabeth Keighley**

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**From:** Glenn Meyer, *RI*  
**Sent:** Thursday, October 16, 2008 3:37 PM  
**To:** Darrell Roberts; Richard Conte; John Richmond  
**Cc:** Stephen Pindale; Ronald Bellamy; Diane Screnci; Marsha Gamberoni; Marc Ferdas  
**Subject:** Status of Comm Plans for Oyster Creek LR outage inspection  
**Attachments:** A Comm Plan - OC 2008 Outage.doc; Quick look letter.doc

This email addresses the planned comm plans for the upcoming Oyster Creek license renewal inspection. Currently, the comm plan approach is as follows:

Before outage - no comm plan

There was discussion of comm plan on the work scope / inspection scope of the upcoming OC outage. John has determined the main need was a brief listing of areas that the LR inspection plans to address so that possible media questions could be responded to. John sent Diane Screnci such a listing and this appears to have met the need. No further comm plan is being prepared.

Quick Look letter issuance - rough draft attached

Immediately following the LR inspection and prior to restart, the plan is for DRS to issue a quick look letter to address the preliminary drywell evaluation results. I have attached a rough draft of such a quick look letter. The main precedent for such a letter is the 2004 VY engineering inspection summary letter. When the quick look letter is issued, a comm plan will be implemented; a rough draft of one is attached.

IR issuance - to be prepared

A comm plan is likely to be needed when the IR is issued. Given the likely issuance in late December, the comm plan will be developed later.

I have created a folder in G:DRS/EB1/LicRenewal/Oyster Creek/2008 outage, which contains the drafts and prior examples of comm plans, quick look letter, and PN.

Should you have comments on the draft comm plan, the comments would be welcomed by John, the team leader.

Glenn

*G/8*

Received: from R1CLSTR01.nrc.gov ([148.184.99.7]) by R1MS01.nrc.gov  
([148.184.99.10]) with mapi; Thu, 16 Oct 2008 15:37:22 -0400  
Content-Type: application/ms-tnef; name="winmail.dat"  
Content-Transfer-Encoding: binary  
From: Glenn Meyer <Glenn.Meyer@nrc.gov>  
To: Darrell Roberts <Darrell.Roberts@nrc.gov>, Richard Conte  
<Richard.Conte@nrc.gov>, John Richmond <John.Richmond@nrc.gov>  
CC: Stephen Pindale <Stephen.Pindale@nrc.gov>, Ronald Bellamy  
<Ronald.Bellamy@nrc.gov>, Diane Screnci <Diane.Screnci@nrc.gov>, Marsha  
Gamberoni <Marsha.Gamberoni@nrc.gov>, Marc Ferdas <Marc.Ferdas@nrc.gov>  
Date: Thu, 16 Oct 2008 15:37:21 -0400  
Subject: Status of Comm Plans for Oyster Creek LR outage inspection  
Thread-Topic: Status of Comm Plans for Oyster Creek LR outage inspection  
Thread-Index: AckvxptwfGJKHejkQt2urGM3sM81qQ==  
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Accept-Language: en-US  
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X-MS-TNEF-Correlator:  
<2856BC46F6A308418F033D973BB0EE72AA2A369BC0@R1CLSTR01.nrc.gov>  
MIME-Version: 1.0

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**COMMUNICATIONS PLAN**

**Preliminary Results of Oyster Creek Drywell Evaluation**

**Goal**

To effectively communicate the preliminary evaluation results of recent Oyster Creek primary containment (drywell) inspections and the imminent reactor restart by informing stakeholders of the letter addressing preliminary evaluation results ("Quick Look Letter") and addressing the context of the letter within the ongoing license renewal process.

**Key Messages**

- Today the NRC issued a letter with the preliminary results related to the evaluation of the Oyster Creek drywell during a recent NRC outage license renewal inspection.
- The onsite inspection was performed by a team of four NRC inspectors over two weeks, and involved visual inspections of conditions inside and outside of the drywell and review of AmerGen ultrasonic testing (UT) of drywell shell thickness. The inspection determined that X
- The preliminary drywell evaluation results are not unexpected and are congruent with previous AmerGen design evaluations and UT thickness measurements addressed as part of AmerGen's application for a renewed Oyster Creek license.
- It is expected that Oyster Creek will restart the reactor imminently following the recent refueling outage.

*[Handwritten signature]*

**Audience / Stakeholders**

AmerGen (Oyster Creek Nuclear Generating Station)

Senators' DC Offices (Senators Lautenberg & Menendez)

House of Representatives for NJ (Rep. Saxton, Smith, Andrews, Holt, Pallone & Pascrell)

New Jersey Dept. of Environmental Protection

Local officials ?

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**Communication Team**

|                     |                                   |              |
|---------------------|-----------------------------------|--------------|
| Eugene Dacus        | Office of Congressional Affairs   | 301.415.3693 |
| Richard Conte       | Chief, Engineering Branch 1, DRS  | 610.337.5183 |
| Marjorie McLaughlin | State Liaison Officer             | 610.337.5240 |
| Stephen Pindale     | Senior Reactor Inspector (acting) | 610.337.5116 |
| Diane Screnci       | Senior Public Affairs Officer     | 610.337.5330 |
| Neil Sheehan        | Public Affairs Officer            | 610.337.5331 |

Timeline:

| Time Sequence | ACTION   | Responsible Organization/Individual |
|---------------|--|-------------------------------------|
| T= 0 Hour     | Inspection Report Is Signed By Region I Management and emailed to Oyster Creek | RI - Conte                          |
| T = 0.5 Hour  | Call Site VP and Communicate Key Messages                                      | RI - Conte/Collins                  |
| T = 1 Hour    | E-Mail Inspection Report & Key Messages to New Jersey                          | RI - McLaughlin                     |
| T= 1 Hour     | E-Mail Inspection Report & Key Messages to NJ Congressional Offices            | OCA - Dacus                         |
| T= 24 Hour +  | Respond to Media Inquiries   | RI - Screnci/Sheehan                |

Add a line within table for DRP to notify local officials at 1 hour?

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Q: How do the preliminary evaluation results affect the license renewal process?

A:

Q: What are the next steps following the inspection, and what is done with the findings?

A:

Q: Why is the reactor safe to operate with these findings?

A:

File: G:DRS/EB1/LicRenewal/Oyster Creek/2008 Outage/A Comm Plan – OC 2008 outage.doc

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### License Renewal Background

AmerGen submitted a License Renewal Application (LRA) for Oyster Creek on July 22, 2005. The license renewal team inspection occurred in March 2006; Inspection Report 50-219/2006-007 dated May X, 2006, documented the inspection results. The LRA addressed managing the aging of the drywell shell; corrosion of the shell, primarily in the sand bed region, had been identified in the mid-1980s.

ASLB held a hearing on a contention regarding the frequency of planned ultrasonic (UT) inspections of the drywell shell in the sand bed region. On December X, 2007, the ASLB ruled in AmerGen's favor. Citizens (intervenor) appealed this decision to the Commission on January 14, 2008. In May 2008, the Commission requested that the ASLB resolve concerns related to planned 3-D analysis of the drywell shell. The ASLB held oral arguments on September 18, 2008, and anticipates responding to the Commission in October.

Citizens has another appeal to the Commission related to a July 2008 ASLB decision to deny admitting a new contention on metal fatigue. The Commission plans to act by November 10, 2008.

### Subsequent Actions

Region I issue Inspection Report 50-219/2008-007 – Deadline: December 21?, 2008

NRC Commissioners decide on two ASLB appeals related to renewed license - ?

Region I perform non-outage license renewal inspection – planned for March 9 - 27, 2008

The current operating license for Oyster Creek expires on April 9, 2009?.

### Questions and Answers

Q: Why issue a letter with the preliminary results of an inspection?

A: Given the continuing interest in the drywell shell, particularly in the review of the Oyster Creek license renewal application, and the ASLB hearing and oral argument on this subject, the timely public disclosure of the drywell evaluation part of the inspection was determined to be desirable.

Q: When will the inspection results be final?

A: The inspection results will be final when issued in Inspection Report 50-219/2008-007. The planned deadline for report issuance is December 21?, 2008.

Q: What prompted the inspection?

A: This was a scheduled inspection in accordance with the license renewal process under Inspection Procedure 71003. Generally, the inspection addresses license renewal activities which occur during the refueling outage prior to the period of extended operations and which relate to equipment inaccessible (such as the drywell) during reactor operation.

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**UNITED STATES  
NUCLEAR REGULATORY COMMISSION**

REGION I  
475 ALLENDALE ROAD  
KING OF PRUSSIA, PA 19406

November X, 2008

Mr. Charles G. Pardee  
Chief Nuclear Officer (CNO) and Senior Vice President  
Exelon Generation Company, LLC  
Chief Nuclear Officer (CNO)  
AmerGen Energy Company, LLC  
200 Exelon Way  
Kennett Square, PA 19348

**SUBJECT: OYSTER CREEK GENERATING STATION – PRELIMINARY EVALUATION  
RESULTS OF THE PRIMARY CONTAINMENT**

Dear Mr. Pardee:

From October 27 to November 6, 2008, the U. S. Nuclear Regulatory Commission (NRC) performed a team inspection at your Oyster Creek Generating Station related to license renewal activities during the recent refueling outage. A part of the inspection involved review of primary containment (drywell) conditions and ultrasonic testing of the drywell shell thickness. In the interest of timely public disclosure, NRC has decided to issue this letter and its enclosure, which address that part of the inspection.

The enclosure to this letter provides a summary of the inspection scope and preliminary evaluation results related to the drywell. Please note that the final inspection results, including the determinations and characterization of their significance, may change based on additional information and further review. The final inspection results will be documented in NRC Inspection Report 05000219/2008007.

For the drywell the NRC review involved a multi-week inspection of AmerGen's inservice inspection program and included an assessment of license renewal commitments for the current outage. Based on the results of the NRC's inspection activities, the NRC concluded that there were no safety significant conditions with respect to the drywell that would prohibit plant startup, and that there is reasonable assurance that the primary containment is capable of performing its design function throughout the upcoming operating cycle.

In accordance with 10 CFR 2.390 of the NRC's "Rules of Practice," a copy of this letter and its enclosure will be available electronically for public inspection in the NRC Public Document Room or from the Publicly Available Records (PARS) component of NRC's document system (ADAMS). ADAMS is accessible from the NRC Website at <http://www.nrc.gov/reading-rm/adams.html> (the Public Electronic Reading Room).

We appreciate your cooperation. Please contact me at (610) 337-5183 if you have any questions regarding this letter.

Sincerely,

Richard J. Conte, Chief  
Engineering Branch 1  
Division of Reactor Safety

Docket No. 50-219  
License No. DPR-16

Enclosure: Summary of Drywell Inspection Scope and Preliminary Evaluation Results

cc w/encl:

C. Crane, Executive Vice President, Exelon, Chief Operating Officer, Exelon Generation  
M. Pacilio, Chief Operating Officer, Exelon  
T. Rausch, Site Vice President, Oyster Creek Nuclear Generating Station  
J. Randich, Plant Manager, Oyster Creek Generating Station  
J. Kandasamy, Regulatory Assurance Manager, Oyster Creek  
R. DeGregorio, Senior Vice President, Mid-Atlantic Operations  
K. Jury, Vice President, Licensing and Regulatory Affairs  
P. Cowan, Director, Licensing  
B. Fewell, Associate General Counsel, Exelon  
Correspondence Control Desk, AmerGen  
Mayor of Lacey Township  
P. Mulligan, Chief, NJ Dept of Environmental Protection  
R. Shadis, New England Coalition Staff  
E. Gbur, Chairwoman - Jersey Shore Nuclear Watch  
E. Zobian, Coordinator - Jersey Shore Anti Nuclear Alliance  
P. Baldauf, Assistant Director, NJ Radiation Protection Programs





We appreciate your cooperation. Please contact me at (610) 337-5200 if you have any questions regarding this letter.

Sincerely,  
*/RA/*  
Ronald R. Bellamy, Ph.D., Chief  
Projects Branch 6  
Division of Reactor Projects

Docket No. 50-219  
License No. DPR-16

Enclosure: Summary of Drywell Inspection Scope and Preliminary Evaluation Results

Distribution w/encl: (via e-mail)

- S. Collins, RA
- M. Dapas, DRA
- D. Lew, DRP
- J. Clifford, DRP
- R. Bellamy, DRP
- S. Barber, DRP
- A. Rosebrook, DRP
- M. Ferdas, DRP, Senior Resident Inspector
- J. Kulp, DRP, Resident Inspector
- J. DeVries, DRP, Resident OA
- S. Williams, RI OEDO
- H. Chernoff, NRR
- E. Miller PM, NRR
- T. Valentine, Backup PM (Interim), NRR
- ROPreports@nrc.gov (All Inspection Reports)
- Region I Docket Room (with concurrences)

*[Handwritten initials]*

SUNSI Review Complete: \_\_\_\_\_ (Reviewer's Initials)

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|--------|-----------|--|--------|--|------------|--|--|
| OFFICE | RI:DRS    |  | RI:DRS |  | RI:DRS     |  |  |
| NAME   | JRichmond |  | RConte |  | MGamberoni |  |  |
| DATE   | /08       |  | /08    |  | /08        |  |  |

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*[Handwritten signature]*

MEMORANDUM TO: Richard J. Conte, Chief  
Engineering Branch 1, DRS

FROM: John E. Richmond, Team Leader  
Engineering Branch 1, DRS

SUBJECT: SUMMARY OF DRYWELL INSPECTION SCOPE AND PRELIMINARY  
EVALUATION RESULTS

**Something similar to the following from 2006 PN:**

The NRC staff inspection throughout the outage focused on:

- 1) Non-destructive examination results of the drywell shell and torus and related AmerGen evaluations.
- 2) The condition of the inside of the drywell, including trenches in the floor, and the outside of the drywell shell in the sand bed region.
- 3) Structural integrity of the concrete drywell floor and the condition of the embedded portion of the drywell shell.
- 4) The potential impact from various repairs to the containment on the design and licensing bases of the drywell.

The overall results of the staff's observations and review are:

- 1) All UT measurements are greater than the calculated minimum code required thickness for various plates that form the drywell shell.
- 2) There are no adverse conditions of the epoxy coating on the outside of the drywell shell in the former sandbed region.
- 3) Repairs in and around the trough within the reactor vessel pedestal area did not result in any adverse conditions.
- 4) The water discovered in the drywell trenches had no adverse impact on the structural integrity of the concrete floor or the potential for corrosion of the embedded portion of the drywell shell. AmerGen has taken actions to prevent further accumulation of water in this area.
- 5) There are no adverse conditions with respect to the drywell or torus structural integrity that preclude restart.

Based on a review of the technical information, the NRC staff determined that AmerGen has sufficient justification to restart Oyster Creek.

