

**UNITED STATES
NUCLEAR REGULATORY COMMISSION
ATOMIC SAFETY AND LICENSING BOARD**

Before Administrative Judges:

**Alex S. Karlin, Chairman
Dr. Richard E. Wardwell
Dr. William H. Reed**

In the Matter of)

**ENTERGY NUCLEAR VERMONT YANKEE, LLC)
and ENTERGY NUCLEAR OPERATIONS, INC.)**

(Vermont Yankee Nuclear Power Station))

**Docket No. 50-271-LR
ASLBP No. 06-849-03-LR**

NEW ENGLAND COALITION, INC.

CONTENTIONS 2A and 2B

PREFILED EXHIBITS

NEC-JH_03 – NEC-JH_24

April 28, 2008

Volume 1

**Review of Entergy Nuclear Vermont Yankee, LLC and
Entergy Nuclear Operations, Inc. (“Entergy”) Analyses of
the Effects of Reactor Water Environment on Fatigue Life of
Risk-significant Components During the Period of Extended
Operation**

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I. BACKGROUND

A. Basic Technical Principles

Fatigue is an age-related degradation mechanism caused by cyclic stressing of a component by either mechanical or thermal stresses that eventually cause the component to crack. Under such cyclic loading, a crack will be initiated and the component will fail under stresses that are substantially lower than those that cause failure under static loadings. During each loading cycle, some fraction of the component's fatigue life is exhausted, its size depending on the magnitude of the applied stress. Eventually, after N cycles, the component's allowable fatigue life is fully expended. The number of cycles n at any given stress amplitude divided by the corresponding N is called the usage fatigue factor. The cumulative usage fatigue factor, CUF, is simply a summation of the individual usage factors. ASME Code Section III requires that CUF must not exceed unity. The CUF is expressed as

$$CUF = \sum n_k / N_k$$

The basic equation that describes the crack growth rate for a given stress intensity includes two empirical constants, C and x. A large data base exists on the empirical constants C and x, which was derived from laboratory tests mostly in air under controlled conditions. This equation can predict crack growth reliably as long as it is used under the conditions that were used to calibrate C and x. This principle is very important in assessing how Entergy used laboratory data to calculate fatigue life of selected components at the VY plant.

To account for the fact that crack propagation in water is different than in air, the individual usage factor in air is multiplied by a corresponding correction factor F_{en} . F_{en} is simply the ratio of the fatigue life in air at room temperature to the fatigue life in water at the local temperature. The environmentally corrected CUF is defined as,

$$CUF_{en} = F_{en} (CUF)$$

Fen is derived from laboratory data on the effect of strain on fatigue life, i.e. the number of cycles to failure. NUREG/CR-6909 describes such laboratory tests in detail.

The procedures to analyze components for fatigue are specified in Section III of the ASME Code. The Code provides fatigue curves for various materials, which specify the allowable number of cycles for a given stress intensity. The code requires that the CUF at any given location be maintained below one. Since the Code used data from laboratory tests with smooth specimens, the code made allowances (2 on stress and 20 on cycles) in recognition that a test specimen in air may have a longer fatigue life than actual components in a reactor. The most current ASME code also provides a simplified set of rules in Subparagraph NB-3600, and a more rigorous rule in Subparagraph NB- 3200, which is based on using a finite element analysis to calculate CUF values. Replacing the simplified analysis with a more detailed analysis has the advantage of removing unwanted conservatism from the results of the simplified analysis. Since the detailed analysis may require a larger data base than the simplified analysis, the user must ascertain that the necessary data base exists. When such information is not available, and the user instead makes arbitrary assumptions, the benefit of the detailed analysis is completely negated.

B. Regulatory Requirements

NRC regulation 10 CFR § 54.21(c) requires that each license renewal application must include “an evaluation of time-limited aging analyses” (“TLAA”) for components covered by the license renewal regulations.¹ If

¹ TLAAAs are defined as:

Those licensee calculations and analyses that:

- (1) Involve systems, structures, and components within the scope of license renewal, as delineated in § 54.4(a);
- (2) Consider the effects of aging;
- (3) Involve time-limited assumptions defined by the current operating term, for example, 40 years;
- (4) Were determined to be relevant by the licensee in making a safety determination;
- (5) Involve conclusions or provide the basis for conclusions related to the capability of the system, structure and component to perform its intended functions, as delineated in § 54.4(b); and

the applicant is unable to demonstrate that TLAAs “remain valid for the period of extended operation” or that they “have been projected to the end of the period of extended operation,” it must demonstrate that “the effects of aging on the intended function(s) will be adequately managed for the period of extended operation.” 10 C.F.R. 54.21(c)(1)(i)-(iii).

NUREG-1801, Rev. 1, Generic Aging Lessons Learned (GALL) Report (2005) (“NUREG-1801”) also provides guidance for the preparation of TLAAs.² NUREG-1801 advises that a license renewal applicant may address “the effects of the coolant environment on component fatigue life by assessing the impacts of the reactor coolant environment on a sample of critical components for the plant.” *Id.*, Vol. 2 at X M-1. Examples of critical components are identified in NUREG/CR-6260, Application of NUREG/CR-5999 Interim Fatigue Curves to Selected Nuclear Power Plant Components (1995). The sample of critical components “can be evaluated by applying environmental life correction factors to the existing ASME Code fatigue analyses.” NUREG-1801, Vol. 2 at X M-1. If these components are found not to comply with the acceptance criteria (i.e., CUF less than one), “corrective actions” must be taken that “include a review of additional affected reactor coolant pressure boundary locations.” *Id.* at X M-2. As explained further in industry guidance document MRP-47:

The locations evaluated in NUREG/CR-6260 [2] for the appropriate vendor/vintage plant should be evaluated on a plant-unique basis. For cases where acceptable fatigue results are demonstrated for these locations for 60 years of plant operation including environmental effects, additional evaluation or locations need not be considered. However, plant-unique evaluations may show that some of the NUREG/CR-6260 [2] locations do not remain within allowable limits for 60 years of plant operation when environmental effects are considered. In this situation, plant specific evaluations should expand

(6) Are contained or incorporated by reference in the CLB [current licensing basis].

² NUREG-1801 is referenced with approval in Regulatory Guide 1.188, Rev. 1, *Standard Format and Content for Applications to Renew Nuclear Power Plant Operating Licenses* (2005) (“Reg. Guide 1.188”).

the sampling of locations accordingly to include other locations where high usage factors might be a concern.³

II. ENTERGY'S CUFen ANALYSES

A. Brief History

The VYNPS License Renewal Application (LRA) Table 4.3-3 summarizes Entergy's evaluation of effects of reactor water environment on the fatigue life of nine components for the period of extended operations. The components selected correspond to the limiting locations identified in NUREG/CR-6260.⁴ LRA Table 4.3-3 states that the environmentally corrected Cumulative Usage Factor (CUFen) of the following risk-significant reactor components will exceed unity: feedwater nozzle, RR inlet nozzle, RR outlet nozzle, RR piping tee, core spray nozzles, core spray safe end, and feedwater piping.

To address this problem, Entergy chose to "refin[e] the fatigue analyses to lower the predicted CUFs to less than 1.0."⁵ Entergy's refinement of its CUFen analysis proceeded in two steps: (1) an initial reanalysis involving, in part, the use of a simplified Green's function method to calculate stress loads during plant transient operations; and (2) a "confirmatory" reanalysis of only the feedwater nozzle that did not involve use of the simplified Green's function method. I have reviewed the reports of both Entergy's initial CUFen reanalysis, and its "confirmatory" reanalysis of the feedwater nozzle that Entergy produced to NEC.⁶

The five elements of Entergy's initial reanalysis included:

³ MRP-47, Revision 1, Electric Power Research Institute, *Materials Reliability Program: Guidelines for Addressing Fatigue Environmental Effects in a License Renewal Application* at 3-4 (2005).

⁴ Safety Evaluation Report Related to the License Renewal of Vermont Yankee Nuclear Power Station (February 2008)("FSER"), NRC Staff_Exh_01 at 4-32.

⁵ LRA at 4.3-7.

⁶ These reports are submitted in this proceeding as Exhibits NEC-JH_04 – NEC-JH_21.

1. Development of a finite element model
2. Development of heat transfer coefficients
3. Development of Green Functions
4. Development of thermal transient definitions
5. Performance of Stress and Fatigue Analysis.

Entergy reported the results of its initial reanalysis in the Table 1, reproduced below:

TABLE 1
VYNPS Cumulative Usage Factors for
NUREG/CR-6260 Limiting Locations⁷

	NUREG-6260 Location	Material	Overall* Environmental Multiplier (F _{en})	Environmentally Adjusted CUF
1	RPV vessel shell/ bottom head	Low alloy steel	9.51	0.08
2	RPV shell at shroud support	Low alloy steel	9.51	0.74
3	Feedwater nozzle forging blend radius	Low alloy steel	10.05	0.64
4	RR Class 1 piping (return tee)	Stainless steel	12.62	0.74
5	RR inlet nozzle forging	Low alloy steel	7.74	0.50
6	RR inlet nozzle safe end	Stainless steel	11.64	0.02
7	RR outlet nozzle forging	Low alloy steel	7.74	0.08
8	Core spray nozzle forging blend radius	Low alloy steel	10.05	0.0432 0.1668
9	Feedwater piping riser to RPV nozzle	Carbon steel	1.74	0.29

* Effective multiplier for past and projected operating history, power level, and water chemistry.

The NRC Staff rejected Entergy's initial CUF_{en} reanalysis. As reported in the FSER, Entergy and the NRC Staff "were unable to resolve the issues raised [with respect to Entergy's use of Green's functions to calculate stress loads]."⁸ The NRC Staff therefore requested that Entergy perform, and Entergy did perform, the additional "confirmatory" CUF_{en} analysis of the feedwater nozzle, using the ASME Code Section III, Subsection NB-3200 methodology to calculate the stress intensities "without referencing Green's function."⁹

⁷ Exhibit NEC-JH_35 at Attachment 2.

⁸ FSER, NRC Staff Exhibit 01 at 4-40.

⁹ FSER, NRC Staff Exhibit 01 at 4-41; *See also*, Exhibit NEC-JH_22 (Summary of Meeting Held on January 8, 2008, Between the U.S. Nuclear Regulatory Commission Staff and Entergy Nuclear Operations, Inc. Representatives to Discuss the Response to a Request for Additional Information Pertaining to the Vermont Yankee Nuclear Power Station License Renewal Application).

At the February 7, 2008 meeting of the ACRS, which I attended, the NRC Staff informed the ACRS that it was satisfied with the CUFen calculations based on Entergy's then-reported "confirmatory" results for the feedwater nozzle. As reported in the FSER, however, during a subsequent February 14, 2008 audit of Entergy's confirmatory analysis, the NRC Staff requested that Entergy recalculate the feedwater nozzle CUFen yet again, substituting a different Fen value. Specifically, NRC Staff requested use of "the maximum Fen value used in [Entergy's] previous analyses," rather than "different, but appropriate" Fen values Entergy had used in its "confirmatory" analysis.¹⁰

The following Table 2 summarizes how Entergy's reported CUFen values for the feedwater nozzle have changed with each iteration of its analysis.

Table 2- CUFen Calculations For the Feedwater Nozzle

REFERENCE	CUF	Fen	CUFen
License Renewal Application Table 4.3-3	0.750	3.81	2.86
Entergy Initial CUFen Reanalysis Using Simplified Green's Function. NEC Exhibit JH_18 at 3-18, Table 3-10.	0.0636	10.05	0.6392
Entergy "Confirmatory" CUFen Reanalysis. NEC Exhibit JH_21 at 7, Table 1.	0.0889	3.97	0.3531
Adjusted "Confirmatory" Reanalysis result verbally provided during February 14, 2008 NRC Staff audit of Entergy's "Confirmatory" Reanalysis. FSER, NRC Staff Exhibit 1 at 4-42.			0.8930

A comparison of Entergy's result using the simplified Green's function method, 0.639, with its "confirmatory" result, ultimately 0.8930 as recalculated February 14, 2008, demonstrates that the simplified Green's

¹⁰ FSER, NRC Staff Exhibit 01 at 4-42.

function method underestimates CUF by about 40%. As reported in the FSER, the NRC Staff therefore concluded that “the results of the Green’s function application using the specific software could underestimate CUF, and therefore cannot be the analysis of record.”¹¹

The NRC Staff has designated Entergy’s “confirmatory” analysis the “analysis of record” for the feedwater nozzle.¹² The NRC Staff has also recommended a license condition that would require Entergy to perform the “confirmatory” analysis for the spray (CS) and recirculation (RR) nozzles no later than two years before the start of the life extension period.¹³

The NRC Staff is now revisiting the sufficiency of environmentally-assisted fatigue analyses based on the simplified Green’s function method, which the NRC had previously accepted in support of license renewal for plants other than Vermont Yankee. On April 18, 2008, the NRC Staff issued a Regulatory Issue Summary (“RIS”), requesting that “license renewal applicants that have used this simplified Green’s function methodology perform confirmatory analyses to demonstrate that the simplified Green’s function analyses provide acceptable results.”¹⁴ This RIS also states: “For plants with renewed licenses, the staff is considering additional regulatory actions if the simplified Green’s function methodology was used.”¹⁵ On April 3, 2008, the NRC Staff issued a Notification of Information in Docket No. 50-219-LR (License Renewal for Oyster Creek Nuclear Generating Station), stating that it will require “confirmatory” fatigue analyses due to Oyster Creek’s reliance on the simplified Green’s function method.¹⁶

¹¹ Id. at 4-43.

¹² Id. at 4-43.

¹³ Id.

¹⁴ Exhibit NEC-JH-23 at 2.

¹⁵ Id.

¹⁶ Exhibit NEC-JH_24.

III. ASSESSMENT OF ENTERGY'S CUFen REANALYSES

The following discussion explains my assessment of both Entergy's initial and "confirmatory" CUFen reanalyses. Part A explains that Entergy failed to produce information necessary to validate both analyses. Part B lists key assumptions underlying both analyses. Part C explains why, as a results of Entergy's key assumptions, both analyses underestimated CUFen, and overestimated expected fatigue life. Part D discusses the significance of Entergy's failure to perform an error analysis. Part E explains why the "confirmatory" analysis of the feedwater nozzle does not bound the analysis for other components.

A. Incomplete Information

The materials Entergy has produced to NEC in the ASLB proceeding do not include all the information necessary to establish the validity of Entergy's CUFen reanalyses, initial or "confirmatory." Specifically, Entergy has not provided:

1. Adequate layout drawings of the plant piping. Based on the information provided, I cannot determine how the connecting pipes are oriented with respect to the nozzles; how many diameters the pipe is straight upstream of each nozzle; or whether there are any discontinuities, such as welds, upstream of the nozzle.¹⁷ This information is necessary to validate the assumption of uniform heat transfer distribution.

2. A complete description of the methods or models used to determine velocities and temperatures during transients. For example, the following discussion appears in the Structural Integrity Associates, Inc. ("SIA") report of Entergy's initial CUFen reanalysis, VY-16Q-307:

The internal heat transfer coefficient h for the transients with flow occurring in the pipe is calculated based on the following relation for forced convection:

¹⁷ Exhibit NEC-JH_25 is illustrative of the layout drawings Entergy produced to NEC.

$$h = 0.023 \text{ Re}^{0.8} \text{ Pr}^{0.4} k/D$$

Where Re = Reynolds' number
 Pr = Prandtl number
 k = Thermal conductivity
 D = Pipe diameter

The heat transfer coefficients were calculated by PIPESTRESS using the above relation. The flow rates described for each transient in Section 3 were used. For the transients where flow is stopped, the natural convection heat transfer coefficient was used. The formula for h is:

$$h=0.55 (\text{Gr Pr})^{0.25} k/L$$

Where Gr = Grashof Number
 L = Pipe diameter

PIPESTRESS only has the forced convection heat transfer formula built in, so an equivalent flow rate was determined that would give the same heat transfer coefficient as the free convection coefficient.¹⁸

I cannot determine, based on this discussion, how this was done when the flow goes to zero. I discuss this issue in more detail in Part III(C)(2) of this report.

B. Entergy's Assumptions

Both Entergy's Initial and "Confirmatory" CUFen Reanalyses incorporated the following assumptions:

1. The environmental correction factor, Fen, depends only on the temperature, the dissolved oxygen, the sulphur content and the strain rate.

¹⁸ Exhibit NEC-JH_10 at 12-13 (emphasis added).

2. With respect to determination of the heat transfer coefficients in all three nozzles:
 - a. Nozzle entrance and exit effects can be neglected
 - b. Water properties do not change with temperature
 - c. Uniform circumferentially.
3. The base metal under the cladding at the feedwater blend radius has no cracks.
4. The number of transients will increase linearly with time during the life extension period.¹⁹ It was assumed that the 40-year CUFs can be multiplied by 1.5 to project those values to the end of the 60 year extended period.
5. The oxygen at the surface of any component can be evaluated based on plant records, using the EPRI –BWRVIA computer code.

Entergy's Initial CUFen Reanalysis also included the following additional assumption:

6. Green's functions can be used as a substitute for the ASME Code Section III, Subsection NB-3200 method.

C. Assessment of Assumptions

Entergy's above-stated assumptions resulted in the underestimation of CUFen, and the overestimation of expected fatigue life, for the following reasons.

1. Environmental Correction Factor, Fen

Entergy calculated the Fen parameters based on outdated Argonne National Laboratory (ANL) statistical equations stated in NUREG/CR 6583 and NUREG/CR 5704 ("the NUREG equations"), which were derived more

¹⁹ Exhibit NEC-JH_18 at 3-18, note 2 (CUF results based on "actual cycles accumulated to-date and projected to 60 years.").

than nine years ago.²⁰ In February 2007, ANL updated the previous data and published its results in NUREG/CR-6909.²¹ The revised ANL equations are based on a much larger database and the limits of their applicability is more clearly stated.

The developer of the revised ANL equations, O. Chopra, stated to the ACRS:

To apply the laboratory data to actual reactor components, we need to adjust these results to account for parameters or variables which we know affect fatigue life but are not included in this data. And these variables are **mean stress, surface finish, size, and loading history.**²²

This same caveat is repeated in NUREG/CR-6909. To account for uncertainties, the NUREG report states:

“Under certain environmental and loading conditions, fatigue lives in water relative to those in air can be a factor of ≈ 12 lower for austenitic stainless steels, ≈ 3 lower for Ni-Cr-Fe alloys, and ≈ 17 lower for carbon and low-alloy steels.”

NUREG/CR-6909 at 62.²³

Entergy did not provide any data on the surface roughness of the components it evaluated. The ANL equations were developed using a crack free, smooth specimen. In comparison to a smooth surface, a rough surface would reduce the fatigue life by a factor of 3.²⁴ Since most of the components Entergy evaluated were fabricated from carbon or low alloy steel, they are susceptible to flow accelerated corrosion, FAC, which characteristically increases surface roughness. In the case of the VY

²⁰ Exhibit NEC-JH_18 at 3-1.

²¹ Exhibit NEC-JH_26.

²² Exhibit NEC-JH_27 at 22.

²³ Exhibit NEC-JH_26 at 62.

²⁴ Exhibit NEC-JH_26 at 14.

feedwater nozzle, the existence of surface cracks at the blend radius both in the clad and the base metal is another factor that must be considered (see Comment 3 below).

Because of the above uncertainties, I believe that it is appropriate to use a factor of 17, at a minimum, to correct the CUFs for environmental effects.

At the February 7, 2008 ACRS meeting, which I attended, in response to an ACRS member question as to why Entergy is allowed to use old fatigue data, the NRC staff stated only that it has traditionally used the old data in approving LRAs and did not want to change the procedures at this time.²⁵ The Staff stated that the new data will apply to new reactor applications.²⁶ It would appear that it would be equally important, if not more important, to apply the new data to a 40 year reactor.

2. Heat Transfer

Entergy used the following heat transfer equations to calculate the thermal stress for each transient:

1. $h = 0.023 (Re)^{.8} (Pr)^{.4} k/D^{.27}$
2. $h = 0.55 (GrPr)^{.25} k/L^{.28}$
3. $h = 0.555 (R (R-R_s) g k^3 h_{fg} / (u d \Delta T))^{.25} (R = \rho, u = \mu)^{.29}$

Equation 1 is applicable only to a fully developed turbulent flow, constant fluid properties in pipes. The flow in all three nozzles is not the same as in a straight pipe because the nozzle is relatively short and it

²⁵ Exhibit NEC-JH_28 at 96-97.

²⁶ Id.

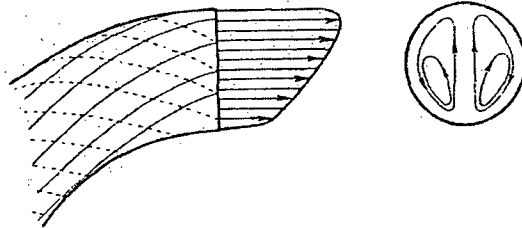
²⁷ Exhibit NEC-JH_04 at 11, Table 4.

²⁸ Exhibit NEC-JH_14 at 14.

²⁹ Exhibit NEC-JH_19 at 7.

contains discontinuities. It is difficult to see how the flow could be fully developed, especially at the exit from the nozzle at the blend radius area (Region 6).³⁰ Nevertheless, depending on the Reynolds number and the distance from the inlet to the nozzle, the heat transfer can be either above or below the value specified by Equation 1. Plots for calculating the heat transfer at the entrance section of pipes can be found on page 212 of Reference 2.³¹ Equation 1 also must be corrected by the ratio of the viscosities evaluated at the bulk and wall temperatures during each transient. Page 212 of Reference 2 also provides such a correction.³²

To justify the use of the axisymmetrical model, Entergy must first show that the flow upstream of each nozzle is fully developed at the entrance to the nozzle and its main axis coincides with the axis of



the nozzle. As shown in Reference 3 and the above sketch, the velocity distribution in the nozzle will vary circumferentially.³³ Such flow distribution would lead to circumferentially varying wall temperature and different stress distribution than would be predicted by an axisymmetrical model.

To my knowledge, Entergy has not provided to NEC the complete piping layout as it exists now in the plant. Unless special precautions were

³⁰ See, Exhibit NEC-JH_04 at 16.

³¹ Exhibit NEC-JH_29.

³² Id.

³³ Exhibit NEC-JH_30.

taken during installation, one must assume that the connecting pipe is at some angle with respect to the nozzle and therefore the axisymmetrical assumption is not valid.

Equation 2 is used to calculate average heat transfer coefficients when the flow is driven by gravitational forces. This equation is not appropriate for applications where one is required to determine local stress distributions along the pipe and not average stress distributions.

Equation 2 does not apply because, for some transients, the forced convection internal flow in pipes stops, and the flow becomes driven by gravity forces.³⁴ Based on physical considerations, the flow does not just suddenly go from forced convection to natural convection, but it rather goes through a mixed forced/free convection region. In the free convection region, the flow is driven by gravity forces and its fundamental characteristic is commonly described by a flow down a vertical plate where both the velocity and the heat transfer coefficient vary with the height of the plate. The natural convection flow inside a pipe is more complex and is based on empirical correlations of the average heat transfer coefficient such as given in Equation 2 for laminar flow. This equation does not describe the variation in the heat transfer coefficient, and the stresses, along the pipe.

The following statement quoted from one report of Entergy's initial CUFen reanalysis demonstrates that Entergy ignored the inherently local feature of natural convection:

PIPESTRESS only has the forced convection heat transfer formula built in, so an equivalent flow rate was determined that would give the same forced convection heat transfer coefficient as the free convection heat transfer coefficient.³⁵

Such a procedure is appropriate for the determination of overall heat balances but not for the determination of stress distributions. In my opinion, the stress analysis should not be dictated by what is available in a given computer program; it should be driven by the nature of the problem.

³⁴ Exhibit NEC-JH_14 at 14.

³⁵ Id.

Equation 3 is an empirical equation for the average heat transfer coefficient during condensation of refrigerants at low laminar velocities. For higher flow rates, a different equation must be used. Entergy did not specify that the flow in the nozzle was laminar. More importantly, to calculate the temperature distribution in the nozzle, one must use local heat transfer coefficients, not average values. Average heat transfer coefficients can only be used to calculate overall heat balances, not local temperatures.

Entergy's CUFen results are based on the assumption that the stresses are axisymmetric in all nozzles. As shown on page 26 of SIA report VY-16Q-310, the stress in a given nozzle is very sensitive to the heat transfer coefficient.³⁶ Throughout its analyses, Entergy used location-independent heat transfer coefficients, which is inappropriate, as I have explained in the above discussion.

3. Base Metal Cracks

In the late 1970s, the feedwater nozzles of most BWR plants developed cracks due to high cycle fatigue because of differences in the thermal properties of the cladding and the base metal. The cladding was removed from most BWR plants, with the exception of Vermont Yankee and a few others. NUREG-O609. In the Millstone 1 plant, some cracks penetrated to 1/3' at the blend radius area. Because the cladding is 5/16" thick and high cycle fatigue cracks propagate to depths of about 1/4" or more, the base metal may contain cracks, especially after 40 years of service. Id. In RAI 4.3-H-02, VY admitted that the cladding may contain cracks,³⁷ but has not provided any data to indicate that these cracks did not penetrate the base metal. They did, however, admit to the possibility that such cracks will penetrate the base metal. The 2001 inspection of the feedwater nozzles only indicates that the results were "acceptable".³⁸ Since Ultrasonic Inspection, UT, measures only the total length of a crack and, based on the VY drawings Entergy has produced, the exact thickness of the clad is not known,³⁹

³⁶ Exhibit NEC-JH_13 at 26.

³⁷ Exhibit NEC-JH_32.

³⁸ Exhibit NEC-JH_33 at 4.

³⁹ Exhibit NEC-JH_25.

Entergy has not provided any proof that the base metal is not cracked. One therefore must assume that the base metal is cracked and account for these cracks in the ASME Code analysis. The ASME Section III, NB 3122.3 does not require Entergy to include the cladding in the structural analysis because the cladding is less than 10% of wall thickness. When, however, subsurface cracks are known to exist, they can not be ignored in the ASME Code analysis, and must be included together with the cladding.

4. Number of Transients

Entergy's apparent assumption that the number of transients the plant would experience varies linearly with time must be challenged. The failure frequency of pressure vessels (and mechanical and electrical components) is statistically very high later in life due to aging of the plant. The recent VYNPS 20% power uprate introduced new stresses on already aging components, and will likely increase the number of unanticipated transients, as demonstrated by the August, 2007 collapse of the VYNPS cooling tower and plant shutdown due to a steam valve failure. VYNPS experienced two unanticipated transients within 10 days in late August 2007. Based on this experience and the assumption of linearity, one could predict 912 transients during the next 25 years. The above extreme case illustrates that Entergy must consider a more conservative number of transients than predicted by the linear formula to project the number of transients during the extended period of operation.

Entergy provided no justification for selecting a non-conservative factor for projecting the number of transients. In my opinion, the number of transients proposed by Entergy should be at a minimum multiplied by 1.2 to account for the probability of an increase in unanticipated failures due to the 20% power uprate.

5. Oxygen

Even though the F_{en} varies exponentially with oxygen concentration, Entergy did not discuss the reasons for not including unanticipated changes in water chemistry (oxygen excursions) during the extended period. Nor did they explain how the chemistry data from the feedwater line or the

electrochemical potential measurements relate to the oxygen concentration at the component surface during transients.

Only in February 2008, in response to an NRC Staff request for information concerning how Entergy's CUFen analysis accounted for water chemistry effects, Entergy stated for the first time that the EPRI –BWRVIA computer code was used at VY to assess the oxygen concentration at the surface of a given component.⁴⁰

NRC requires that analytical codes be assessed and benchmarked against measured plant data. Safety Evaluation by the Office of Nuclear Reactor Regulation Related to Amendment No. 229 to Facility Operating License No. DPR-28, Entergy Nuclear Vermont Yankee, LLC and Entergy Nuclear Operations, Inc., Vermont Yankee Nuclear Power Station, Docket No. 50-271 § 2.8.7.1.⁴¹ A code is only considered valid within the range in which the data was provided.⁴² Entergy did not describe how the BWRVIA code was benchmarked.

The oxygen concentration at the surface of any given component can only be estimated by considering the kinetics of oxide buildup and dissolution throughout the plant. Since Entergy has not described the algorithm in the BWRVIA code, one must assume that the oxygen concentrations that were used by Entergy to calculate the Fens contain unknown errors.

6. Green's Function

In its initial analysis, Entergy applied a simplified Green's function method to calculate stresses for each transient, instead of using the ASME Code, Section III, Subsection NB-3200 approach.⁴³ The Green's function is a powerful tool that, when properly applied, can considerably reduce the cost of the ASME code analysis, especially when the number of transients is

⁴⁰ Exhibit NEC-JH_34 at Attachment 2.

⁴¹ Exhibit NEC-JH_35.

⁴² Id.

⁴³ See, e.g., Exhibit NEC-JH_04.

very large. The Green's function is also, however, an approximate technique in comparison to the NB-3200 methodology, which may introduce errors in the final calculations of the CUF.

As discussed in Part II(A) of this report, a comparison of Entergy's results using the simplified Green's function method with the results of its "confirmatory" analysis for the feedwater nozzle demonstrate that the Green's function method underestimated CUF by about forty percent. For this reason, also as discussed in Part II(A) of this report, the NRC Staff rejected Entergy's initial CUFen analysis.

D. Lack of Error Analysis

To validate its analytical techniques, Entergy should have performed an error analysis to show the admissible range for each variable. Based on the reports of Entergy's CUFen reanalyses produced to NEC,⁴⁴ it has not done so. The lack of error analysis is troubling. For example, Entergy reported a CUFen of 0.74 for the RHR Class 1 piping (Table 1, above). In light of the fact that data scatter in fatigue studies often exceeds an order of magnitude, the value of 0.74 without an error band has little significance and imparts little confidence that fatigue failure will not occur.

E. "Confirmatory" Analysis of Feedwater Nozzle

I have reviewed the reports produced to NEC of the additional "confirmatory" CUFen analysis of the feedwater nozzle that Entergy conducted at the request of the NRC Staff.⁴⁵ This analysis contains all of the errors in calculation of both CUF and Fen values that I have discussed in Part III(C) above, except that the simplified Green's function method was not used.

Even if it were valid, I do not agree that the "confirmatory" analysis would bound the analysis for components other than the feedwater nozzle. There are considerable differences in geometry, heat transfer characteristics, and loadings between the feedwater and the other two nozzles. These differences could result in different stress distributions which would affect

⁴⁴ Exhibits NEC-JH_04 – NEC-JH_21.

⁴⁵ Exhibits NEC-JH_19 – NEC-JH_21.

the CUFs. Entergy did not discuss these differences; instead it only provided the following vague and unscientific statement:

The analysis of the feedwater nozzle is bounding for the core spray and recirculation outlet nozzles since the calculated usage factors are at least 70% less than those for the feedwater nozzle and the number and severity of thermal transients are less.⁴⁶

The statement that the feedwater nozzle results are bounding could only be justified if Entergy had demonstrated an understanding of the reasons for the differences in the CUFs obtained by the simplified Green's function analysis and those that were obtained by the more exact classical ASME analysis. **Entergy was not able to do so.**

IV. HOPENFELD CUF_{en} RECALCULATION

The CUF_{en}s calculated by Entergy, with and without the simplified Green's function method, contain error and they are unreliable. An alternative to these calculations is to use the conservative CUFs as were originally provided in LRA and multiply them by the bounding values given in NUREG/CR-6909. The results of this procedure are given below in Table 3.

⁴⁶ Exhibit NEC-JH_35 at Attachment 1.

TABLE 3 - Recalculated Cumulative Usage Factors for Sample Locations at VYNPS

No.	NUREG/CR-6260 Sample Location (License Renewal Application, Table 4.3-3)	CUF (VYNPS License Renewal Application, Table 4.3-3)	Fen (Ref. 1)	Recalculated CUFen
1	Vessel shell & bottom head	0.400	17	6.80
2	Core spray safe end	0.182	12	2.18
3	Feed water nozzle	0.750	17	12.75
4	RHR return Piping	0.032	12	0.38
5	RR inlet nozzle	0.610	17	10.37
6	RR piping tee	0.397	12	4.76
7	RR outlet nozzle	0.810	17	13.77
8	Core spray nozzle	0.625	17	10.62
9	Feed water piping	0.427	17	7.26

V. SUMMARY

By introducing five key assumptions, excluding those connected with use of the Green's function methodology, Entergy purports to show that the CUFens for all NUREG/CR-6260 limiting locations are less than one. My assessment demonstrates that Entergy ignored critical factors in making its assumptions. When these assumptions are lifted and more appropriate and conservative assumptions are introduced, the CUFen for all but one of the components exceeds unity.

Entergy has not demonstrated that the predicted fatigue life of risk-significant components at VY will meet the ASME criteria for safe operation for the extended period of operation. Neither Entergy's initial analysis nor its "confirmatory" analysis demonstrate that CUFens for the components listed in License Renewal Application 4.3-3 or NUREG/CR-6260 limiting locations are less than one. It is my opinion that acceptance of Entergy's results will lead to an unjustified reduction in the scope of fatigue monitoring at the Vermont Yankee plant.

Energy should be required to develop a valid methodology for calculating CUFen; expand its fatigue analysis to components in addition to the NUREG/CR-6260 locations if a valid CUFen analysis indicates that CUFen for any NUREG/CR-6260 location will exceed unity; and formulate a meaningful plan to properly inspect and maintain all components which are susceptible to fatigue.

VI. REFERENCES

1. J. P. Holman, Heat Transfer, 1981 Ed.
2. E. R. G. Eckert and R. Drake, Heat and Mass Transfer 2nd, Ed 1959.
3. H. Schlichting, Boundary Layer Theory, 4th Ed. 1960.
4. NUREG/CR-6909, "Effect of LWR coolant Environment on Fatigue Life of Reactor Materials" (Final Report), ANL -06/08 U.S. NRC, Wash., D.C. Feb. 2007.
5. NUREG/CR-6583, "Effect of LWR Coolant Environment on Fatigue-Design Curves of Carbon and Low Alloy Steels," March 1998.
6. NUREG/CR 5704, "Effect of LWR Coolant Environments on Fatigue Design Curves of Austenitic Stainless Steel," April 1999.
7. NUREG/CR-6936, "Probability of Failure and Uncertainty Estimate for Passive Components – A Literature Survey," May 2007.

VII. GLOSSARY OF TERMS

Cumulative Usage Factor (CUF) – A summation of usage fatigue factors.

Fatigue -- An age-related degradation mechanism caused by cyclic stressing of a component by either mechanical or thermal stresses that eventually cause the component to crack

Feedwater Nozzle- A short pipe welded to the reactor vessel through which feedwater enters the vessel.

Fen – An environmental correction factor used to account for differences between fatigue in water and fatigue in air, defined as the ratio of the fatigue life in air at room temperature to that in water at the service temperature.

Green's Function – A simplified numerical technique for thermal stress calculations.

Laminar Flow – Sometimes known as streamline flow, it occurs when a fluid flows in parallel layers, with no disruption between layers.

Recirculation Nozzle - A short pipe welded to the reactor vessel through which water flow either in or out of the jet pump.

Spray Nozzle – A nozzle on top of the vessel used to cool the core in case of an accident.

Transient – Plant response to a change in power level.

Turbulent Flow – Fluid (gas or liquid) flow in which the fluid undergoes irregular fluctuations or mixing, in contrast to laminar flow, in which the fluid moves in smooth paths or layers. In turbulent flow, the speed of the fluid at a point is continuously undergoing changes in both magnitude and direction.

Usage Fatigue Factor -- The number of cycles n at any given stress amplitude divided by the corresponding number of cycles to end of life, N .



Structural Integrity Associates, Inc.

File No.: VY-16Q-301 **NEC-JH_04**

CALCULATION PACKAGE

Project No.: VY-16Q

PROJECT NAME:

Environmental Fatigue Analysis of VYNPS

CONTRACT NO.:

10150394

CLIENT:

Entergy Nuclear Operations, Inc

PLANT:

Vermont Yankee Nuclear Power Station

CALCULATION TITLE:

Feedwater Nozzle Stress History Development for Green Functions

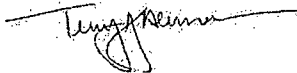

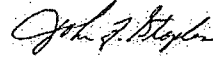
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1.0 OBJECTIVE

The objective of this calculation is to compute the pressure stresses, thermal stresses, and the Green's Functions for high (100%), mid (40%), and low (25%) flow thermal loading of the Vermont Yankee Nuclear Power Station feedwater nozzle.

2.0 FEEDWATER NOZZLE MODEL

An axisymmetric finite element model of the feedwater nozzle was developed in Reference [1] using ANSYS [2]. The geometry used in Reference [1] was utilized in this calculation. The material properties are taken at an average temperature of 300°F. This average temperature is based on a thermal shock of 500°F to 100°F which will be applied to the FE model for Green's Function development. Table 1 listed the material properties at 300°F. The meshed model is shown in Figure 1.

3.0 APPLIED LOADS

Both pressure and thermal loads will be applied to the finite element model.

3.1 Pressure Load

A uniform pressure of 1000 psi was applied along the inside surface of the feedwater nozzle and the vessel wall. A pressure load of 1000 psi was used because it is easily scaled up or down to account for different pressures that occur during transients. In addition, a cap load was applied to the piping at the end of the nozzle. Since only nodes were modeled, the nodal forces shown in Table 2 are defined by the following equation:

$$F_{element} = \pi(IR)^2 P \cdot \frac{\pi(R_o^2 - R_i^2)}{\pi(OR^2 - IR^2)}$$

where:

P = Pressure = 1,000 psi

IR = Inner Radius = 4.8345 in

OR = Outer Radius = 5.42 in

R_i = Inside Radius of element that node is attached to

R_o = Outside Radius of element that node is attached to

F_{node} = The average of the element forces on either side of the node.

Note: The force on the innermost and outermost nodes is calculated as one half of the force on the element that they are attached to.

The calculated nodal forces were applied as positive values so they would exert tension on the end of the model. The ANSYS input file FWP_VY.INP, in the computer files, contains the feedwater



nozzle geometry as well as the pressure loading. Figures 2, 3, and 4 show the internal pressure distribution, cap load, and symmetry condition applied to the vessel end of the model, respectively.

3.2 Thermal Load

Thermal loads are applied to the feedwater nozzle model. The heat transfer coefficients after power uprate were determined from Reference [1]. These values were determined for various regions of the finite element model for 100% (4,590 GPM), and 25% (1,148 GPM) [1]. The annulus leakage flow rate is assumed to be 25 GPM for non-EPU conditions and 31 GPM for EPU conditions. The 25 GPM value is calculated by scaling the 23 GPM [Page 6, 4] value up by approximately 9%. The 23 GPM value is scaled up to provide some conservatism and allow for inaccuracies in the determination of leakage flow. The 31 GPM value is calculated by multiplying the 25 GPM value by 1.25 [Page 6, 4]. Based on this, the annulus leakage flow rate is assumed to be 8 GPM for EPU conditions with 25% flow rate. The temperatures used are based upon a thermal shock from 500°F to 100°F. An additional 40% flow rate (1836 GPM and 13 GPM) was added in this calculation.

3.2.1 Heat Transfer Coefficients

Referring to Figure 5, heat transfer coefficients were applied as following:

Region 1

The heat transfer coefficient, h , for 100% flow is 3705 BTU/hr-ft²-°F at 300°F. [1, Table 5]

The heat transfer coefficient, h , for 40% flow is 1780 BTU/hr-ft²-°F at 300°F. [Table 4]

The heat transfer coefficient, h , for 25% flow is 1222.2 BTU/hr-ft²-°F at 300°F. [1, Table 4]

Region 2

Per Reference [1], the heat transfer coefficient for Region 2 (safe end-to-thermal sleeve contact region) should be linearly transitioned from the value of the heat transfer coefficient used in Region 1 to the value used in Region 3.

Region 3

The heat transfer coefficient, h , for 100% flow is 1489 BTU/hr-ft²-°F at 300°F. [1, Table 9]

The heat transfer coefficient, h , for 40% flow is 743 BTU/hr-ft²-°F at 300°F. [1, Table 9]

The heat transfer coefficient, h , for 25% flow is 504 BTU/hr-ft²-°F at 300°F. [1, Table 9]

Region 4

Per Reference [1], the heat transfer coefficient for Region 4 (thermal sleeve transition in diameter) should be linearly transitioned from the value of the heat transfer coefficient used in Region 3 to the value used in Region 5.

Region 5

The heat transfer coefficient, h , for 100% flow is 177.4 BTU/hr-ft²-°F at 300°F. [1, Table 16]

The heat transfer coefficient, h , for 40% flow is 88.5 BTU/hr-ft²-°F at 300°F. [1, Table 16]

The heat transfer coefficient, h , for 25% flow is 60 BTU/hr-ft²-°F at 300°F. [1, Table 16]

Region 6

Per Reference [1], the heat transfer coefficient for Region 6 (nozzle inner blend radius) should be linearly transitioned from the value of the heat transfer coefficient used in Region 5 to the value used in Region 7.

Region 7

Per Reference [1], the heat transfer coefficient for Region 7 (reactor vessel inside wall) is a constant of 864 BTU/hr-ft²-°F. This value is consistent with the feedwater nozzle work performed in the past for VY and should be used for all reactor conditions.

Region 8

The heat transfer coefficient, h , is 0.2 BTU/hr-ft²-°F [1].

3.2.2 Boundary Fluid Temperatures

For the Green's Functions, a 500°F – 100°F thermal shock is run to determine the stress response to a one-degree change in temperature. The following temperatures are valid when there is water flow. Values between defined points are linearly interpolated. For the 100%, 40%, and 25% flow cases, the thermal shock is run as follows:

Regions 1 to 5

$$T = 500^{\circ}\text{F} - 100^{\circ}\text{F}$$

Region 6

Linearly transitioned from the value of the temperature used in Region 5 to the value used in Region 7

Region 7

$$T = 500^{\circ}\text{F}$$

Region 8

$$T = 120^{\circ}\text{F}$$



4.0 THERMAL AND PRESSURE LOAD RESULTS

The three flow dependent thermal load cases outlined in Section 3.0 were run on the finite element model. Appendix A contains the thermal transient input files FWT_VY_100.INP, FWT_VY_40.INP, and FWT_VY_25.INP for 100%, 40%, and 25% full flow rate, respectively. The three flow dependent input files for the stress runs are also included in Appendix A. The stress filenames are FWS_VY_100.INP, FWS_VY_40.INP, and FWS_VY_25.INP for 100%, 40%, and 25% full flow rate, respectively.

The critical safe end location was chosen as node 192, which has the highest stress intensity due to thermal loading under high flow conditions. As shown in Figures 6 and 7, Node 192 is located on the inside diameter of the nozzle safe end of the model and the maximum stress occurs at 1.4 seconds.

The critical blend radius location was chosen, based upon the highest pressure stress. Conservatively assuming the cladding has cracked, the critical location is selected as node 657 at base metal of the nozzle, as shown in Figures 8 and 9.

The stress intensity for use in the Green's functions are calculated from the component stresses (X, Y, and Z) and compared to the stress intensity reported by ANSYS. As seen in Figure 10, the Z-X calculated total stress intensity best matches the ANSYS reported stress intensity for 100% flow at the safe end. Therefore, the Z-X stress will be used for the total and membrane plus bending Green's functions for all flow rates for the safe end. As seen in Figure 11, the Z-X calculated total stress intensity best matches the ANSYS reported stress intensity for 100% flow at the blend radius in very beginning. Therefore, the Z-X stress will be used for the total and membrane plus bending Green's functions for all flow rates for the blend radius.

The stress time history for the critical paths was extracted during the stress run for 100% flow rate. This produced two files, HFSE.OUT and HFBLEND.OUT, which contain the thermal stress history. The membrane plus bending stresses and total stresses for the Green's Functions were extracted from these files to produce the files HFSE_Inside.RED and HFBLEND_Inside.RED, where SE and BLEND corresponded to the safe end and blend radius locations, respectively.

The stress time history for the critical paths was extracted during the stress run for 40% flow rate. This produced two files, MFSE.OUT and MFBLEND.OUT, which contain the thermal stress history. The membrane plus bending stresses and total stresses for the Green's Functions were extracted from these files to produce the files MFSE_Inside.RED and MFBLEND_Inside.RED, where SE and BLEND corresponded to the safe end and blend radius locations, respectively.

The stress time history for the critical paths was extracted during the stress run for 25% flow rate. This produced two files, LFSE.OUT and LFBLEND.OUT, which contain the thermal stress history. The membrane plus bending stresses and total stresses for the Green's Functions were extracted from these files to produce the files LFSE_Inside.RED and LFBLEND_Inside.RED, where SE and BLEND corresponded to the safe end and blend radius locations, respectively.



As the models were run with a 400°F step change in temperature, and the Green's Functions are for a 1°F step change in temperature, all data values were divided by 400. The governing Green's Functions for the feedwater nozzle during 100% flow, 40% flow, and 25% flow are shown in Figures 12 to 23. The data for the Green's Functions is included in the files HFBR_M+B-Green.xls, HFBR_T-Green.xls, HFSE_M+B-Green.xls, HFSE_T-Green.xls, MFBR_M+B-Green.xls, MFBR_T-Green.xls, MFSE_M+B-Green.xls, MFSE_T-Green.xls, LFBR_M+B-Green.xls, LFBR_T-Green.xls, LFSE_M+B-Green.xls, and LFSE_T-Green.xls in the project Files. Where HF, MF, and LF corresponded to 100% flow, 40% flow, and 25% flow rate, respectively. M+B and T corresponded to membrane plus bending stress and total stress, respectively.

The pressure stress intensities for the path were extracted during the pressure run. The pressure stresses were extracted along the nodal paths as shown in Figures 7 and 9. This produced two files, PSE.OUT and PBLEND.OUT for the safe end and blend radius locations, respectively.

For the pressure loading specified (1,000 psig), the total stress intensity at Node 192 and Node 657 were determined to be 8,891 psi and 28,300 psi, respectively. The membrane plus bending stress intensity at Node 192 and Node 657 were determined to be 8,693 and 27,490 psi, respectively. Table 3 shows the pressure results.

Results were also extracted from the vessel portion of the model to verify the accuracy of the pressure results obtained from the ANSYS model, and to check the results due to the use of the 1.5 multiplier on the vessel radius. These results are contained in the file, PVESS.OUT. Based on earlier work [1], the radius of the finite element model (FEM) was multiplied by a factor of 1.5 to account for the fact that the vessel portion of the two-dimensional (2D) axisymmetric model is a sphere, but the true geometry is the intersection of two cylinders.

The equation for the membrane hoop stress for a sphere is:

$$\sigma = \frac{(pressure) \times (radius)}{2 \times thickness}$$

Considering a vessel base metal radius, R, of 105.90625 inches increased by a factor of 1.5, a vessel base metal thickness, t, of 5.4375 inches, and an applied pressure, P, of 1,000 psi, the calculated stress for a sphere is $PR/(2t) = 14,608$ psi. This compares very well with the remote vessel wall membrane hoop stress from the ANSYS result file, PVESS.OUT, of 13,410 psi. Thus, considering the peak total pressure stress of 28,300 psi reported above, the stress concentrating effect of the nozzle corner is $28,300/14,608 = 1.94$. In other words, the peak nozzle corner stress is 1.94 times higher than nominal vessel wall stress for the 2D axisymmetric model.

The equation for the membrane hoop stress in a cylinder is:

$$\sigma = \frac{(pressure) \times (radius)}{thickness}$$



Based on the previous dimensions, the calculated stress for a cylinder without the 1.5 factor is 19,477 psi. Increasing this by a factor of 1.94 yields an expected peak nozzle corner stress of 37,785 psi, which would be expected from a cylindrical geometry that is representative of the nozzle configuration. Therefore, the result from the ANSYS file for the peak nozzle corner stress (28,300 psi) is lower than the peak nozzle corner stress for a cylindrical geometry because of the use of the 1.5 multiplier. This is consistent with SI's experience where a factor of two increase in radius is typical for representing the three-dimensional (3D) effect in a 2D axisymmetric model.

Based on the foregoing, the ANSYS pressure stresses for the vessel blend radius are increased for use in the subsequent fatigue analysis by 1.33 (2.0/1.5). Thus, the blend radius results presented in Table 3 were obtained by multiplying the ANSYS stresses for the pressure loading by a 1.33X multiplication factor.

5.0 REFERENCES

1. SI Calculation No. VY-10Q-301, Revision 0, "Feedwater Nozzle Finite Element Model and Heat Transfer Coefficients."
2. ANSYS, Release 8.1 (w/Service Pack 1), ANSYS, Inc., June 2004.
3. American Society of Mechanical Engineers, Boiler and Pressure Vessel Code, Section II, Part D, 1998 Edition, 2000 Addenda.
4. VY Calculation Change Notice (CCN), CCN Number 1 for Calculation VYC1005 Revision 2, "This CCN Provides a Basis for the Power Uprate Safety Analysis Report being submitted as part of the power uprate project. The 50.59 assessment will be handled by the EPU design change and NRC SER for this submittal." SI File Number VY-05Q-208.
5. J. P. Holman, "Heat Transfer," 4th Edition, McGraw-Hill, 1976.
6. J. P. Holman, "Heat Transfer," 5th Edition, 1981.
7. GE Nuclear Energy Certified Design Specification, "Reactor Vessel – Extended Power Uprate," Revision 1, SI File No. VY-05Q-236.
8. N. P. Cheremisinoff, "Heat Transfer Pocket Handbook," Gulf Publishing Co, 1984.

Table 1: Material Properties @ 300°F ⁽¹⁾

Material Ident.	Young's Modulus, $E \times 10^6$ (psi)	Instantaneous Coefficient of Thermal Expansion, $\alpha \times 10^{-6}$ (in/in-°F)	Density, ρ (lb/in ³) (assumed)	Conductivity, k (BTU/hr-ft-°F)	Diffusivity, d (ft ² /hr)	Specific Heat, C_p (BTU/lbm-°F) (see Note 5)	Poisson's Ratio (assumed)
SA533 Grade B, A508 Class II (see Note 2)	26.7	7.3	0.283	23.4	0.401	0.119	0.3
SS Clad (see Note 3)	27.0	9.8	0.283	9.8	0.160	0.125	0.3
A508 Class I (see Note 4)	28.1	7.3	0.283	32.3	0.561	0.118	0.3
A106 Grade B (see Note 4)	28.3	7.3	0.283	32.3	0.561	0.118	0.3

Notes

1. The material properties applied in the analyses are taken from ASME Section II Part D 1998 Edition with 2000 Addenda. This is consistent with information provided in the Design Input Record (page 13 of VY EC No. 1773, SI File No. VY-16Q-209). The use of a later code edition than that used for the original design code is acceptable since later editions typically reflect more accurate material properties than was published in prior Code editions. Material Properties are evaluated at 300°F from the 1998 ASME Code, 2000 Addenda, Section II, Part D, except for density and Poisson's ratio, which are assumed typical values [3].
2. Properties of A508 Class II are used (3/4Ni-1/2Mo-1/3Cr-V).
3. Properties of 18Cr - 8Ni austenitic stainless steel are used.
4. Composition = C-Si.
5. Calculated as $[k/(\rho d)]/12^3$.

Table 2: Nodal Force Calculation for End Cap Load

Node Number	Element Number	Radius (in)	Δ Radius (in)	$R_o^2 - R_i^2$ (in ²)	$F_{element}$ (lb)	F_{node} (lb)
1		5.42				7678.0
	1022		0.1171	1.25565	15356.1	
2		5.3029				15188.4
	1021		0.1171	1.22823	15020.7	
3		5.1858				14853.0
	1020		0.1171	1.20080	14685.3	
4		5.0687				14517.6
	1019		0.1171	1.17338	14349.9	
5		4.9516				14182.2
	1018		0.1171	1.14595	14014.5	
6		4.8345				7007.3

Table 3: Pressure Results

Location	Membrane Plus Bending Stress Intensity (psi)	Total Stress Intensity (psi)
Safe End	8693	8891
Blend Radius	36653	37733

Note: The results for the Blend Radius have been increased by a factor of 1.33 (2.0/1.5) as discussed in Section 4.0.

Table 4: Heat Transfer Coefficients for Region 1 (40% Flow)
Calculation of Heat Transfer Coefficients for Feedwater Nozzle Flow Path

Pipe Inside Diameter, D =	9.669	inches =	0.806	ft	100% rated flow =	4,590	gpm		
		=	0.246	m	@ T =	391.9	*F		
Flow, % of rated =	40%				Density, ρ =	53.8997	lbm/ft ³		
Fluid Velocity, V =	8.022	ft/sec =	1,836.0	gpm =	0.793742524	Mlb/hr			
Characteristic Length, L = D =	0.806	ft =	0.246	m					
T _{fluid} - T _{surface} , ΔT = assumed to be 12% of fluid temperature =	8.40	12.00	24.00	36.00	48.00	60.00	72.00	*F	
Note: The above assumption is based on experience with past RPV heat transfer analyses.	4.67	6.67	13.33	20.00	26.67	33.33	40.00	*C	
		Value at Fluid Temperature, T [8]						Units	
Water Property	Conversion Factor [5]	70	100	200	300	400	500	600	*F
k (Thermal Conductivity)	1.7307	0.5997	0.6300	0.6784	0.6836	0.6611	0.6040	0.5071	W/m ² *C
C _p (Specific Heat)	4.1869	4.185	4.179	4.229	4.313	4.522	4.982	6.322	Btu/hr-ft ² *F
ρ (Density)	16.018	1.000	0.998	1.010	1.030	1.080	1.190	1.510	kJ/kg* ² C
β (Volumetric Rate of Expansion)	1.8	997.1	994.7	962.7	917.8	858.6	784.9	679.2	Btu/lbm* ² F
g (Gravitational Constant)	0.3048	62.3	62.1	60.1	57.3	53.6	49.0	42.4	kg/m ³
μ (Dynamic Viscosity)	1.4881	1.89E-04	3.24E-04	6.66E-04	1.01E-03	1.40E-03	1.98E-03	3.15E-03	lbm/ft ³
Pr (Prandtl Number)		1.05E-04	1.80E-04	3.70E-04	5.60E-04	7.80E-04	1.10E-03	1.75E-03	m ³ /m ³ *C
Calculated Parameter	Formula	70	100	200	300	400	500	600	ft ³ /m ³ *F
Reynold's Number, Re	ρVD/μ	6.0147E+05	8.7645E+05	1.8859E+06	2.8491E+06	3.7255E+06	4.5248E+06	4.7336E+06	m/s ²
Grashof Number, Gr	gβΔTL ³ /(μρ) ²	1.2852E+08	6.6834E+08	1.2721E+10	6.5918E+10	2.0931E+11	5.4429E+11	1.1372E+12	ft/s ²
Rayleigh Number, Ra	GrPr	8.9710E+08	3.0142E+09	2.4297E+10	8.0420E+10	1.9885E+11	4.6755E+11	1.2168E+12	kg/m-s
From [5]:		6.980	4.510	1.910	1.220	0.950	0.859	1.070	lbm/ft-s
Inside Surface Forced Convection Heat Transfer Coefficient:									---
H _{forced} = 0.023Re ^{0.8} Pr ^{0.4} /k/D		5,132.76	6,119.10	8,626.61	10,107.53	10,960.57	11,236.63	10,678.39	W/m ² *C
		903.95	1,077.66	1,519.26	1,780.07	1,930.31	1,978.92	1,880.61	Btu/hr-ft ² *F
		1.744E-03	2.079E-03	2.931E-03	3.434E-03	3.724E-03	3.817E-03	3.628E-03	Btu/sec-in ² *F
From [5]:									
Inside Surface Natural Convection Heat Transfer Coefficient:									
Case: Enclosed cylinder									
H _{free} = C(GrPr) ⁿ /k/L		232.43	330.57	599.85	815.28	988.69	1,118.54	1,192.73	W/m ² *C
		40.93	58.22	105.64	143.58	174.12	196.99	210.06	Btu/hr-ft ² *F
		7.896E-05	1.123E-04	2.038E-04	2.770E-04	3.359E-04	3.800E-04	4.052E-04	Btu/sec-in ² *F

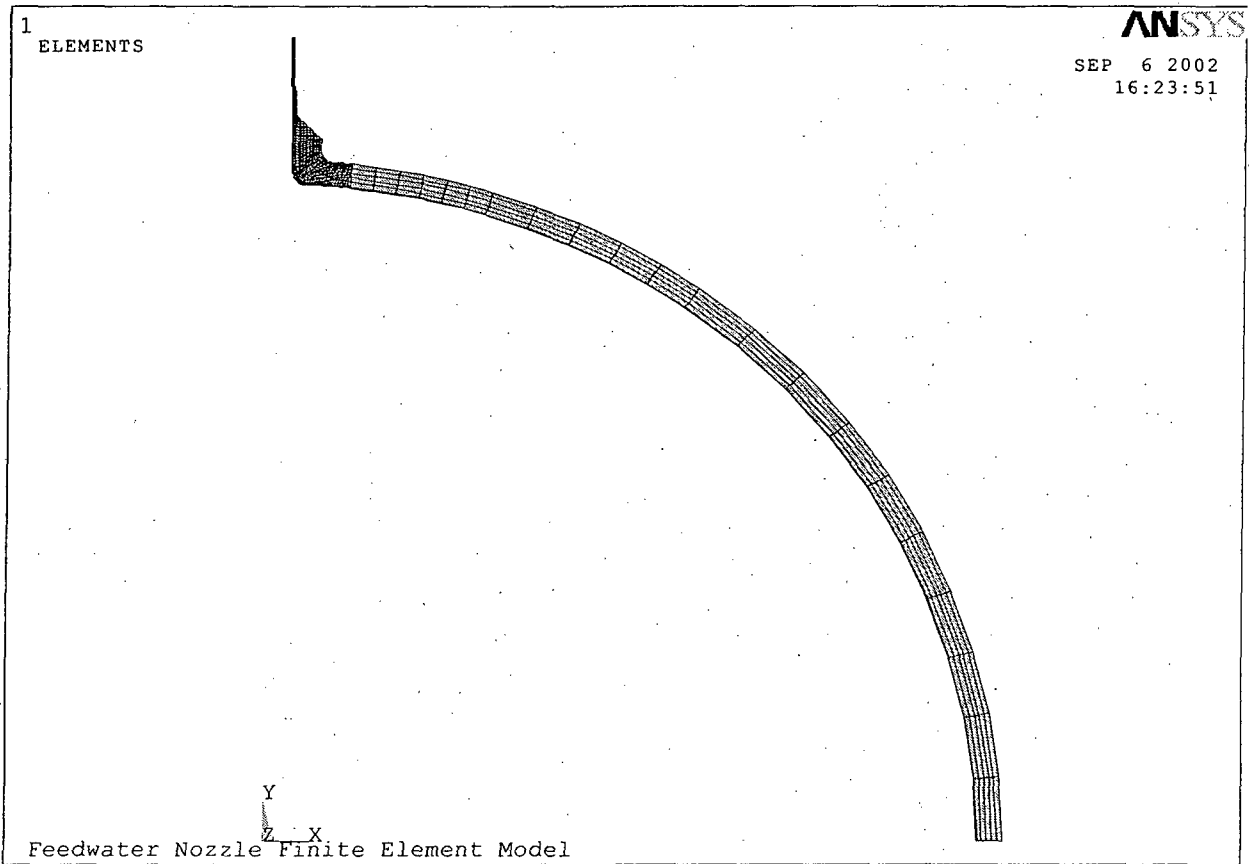


Figure 1: ANSYS Finite Element Model

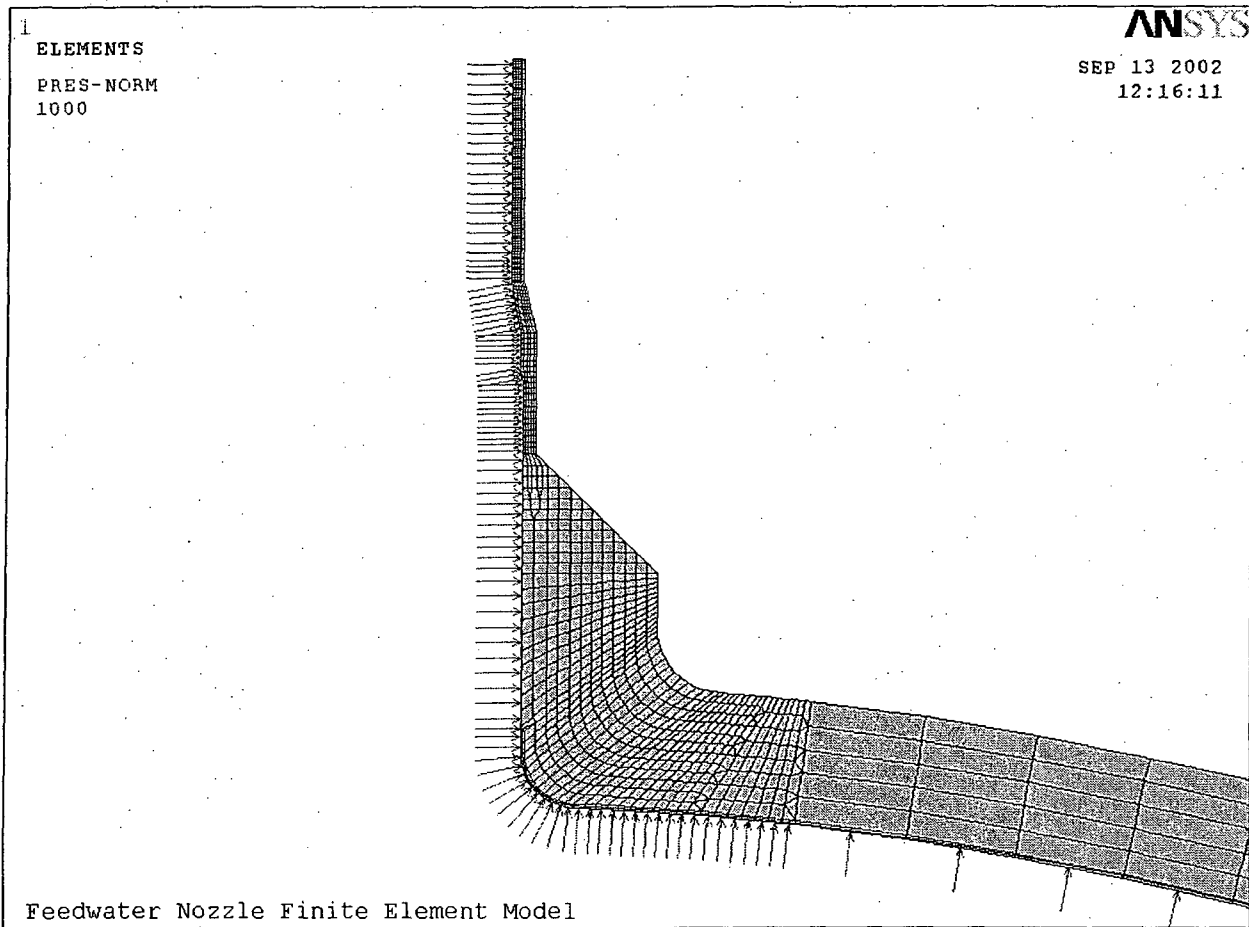


Figure 2: Feedwater Nozzle Internal Pressure Distribution

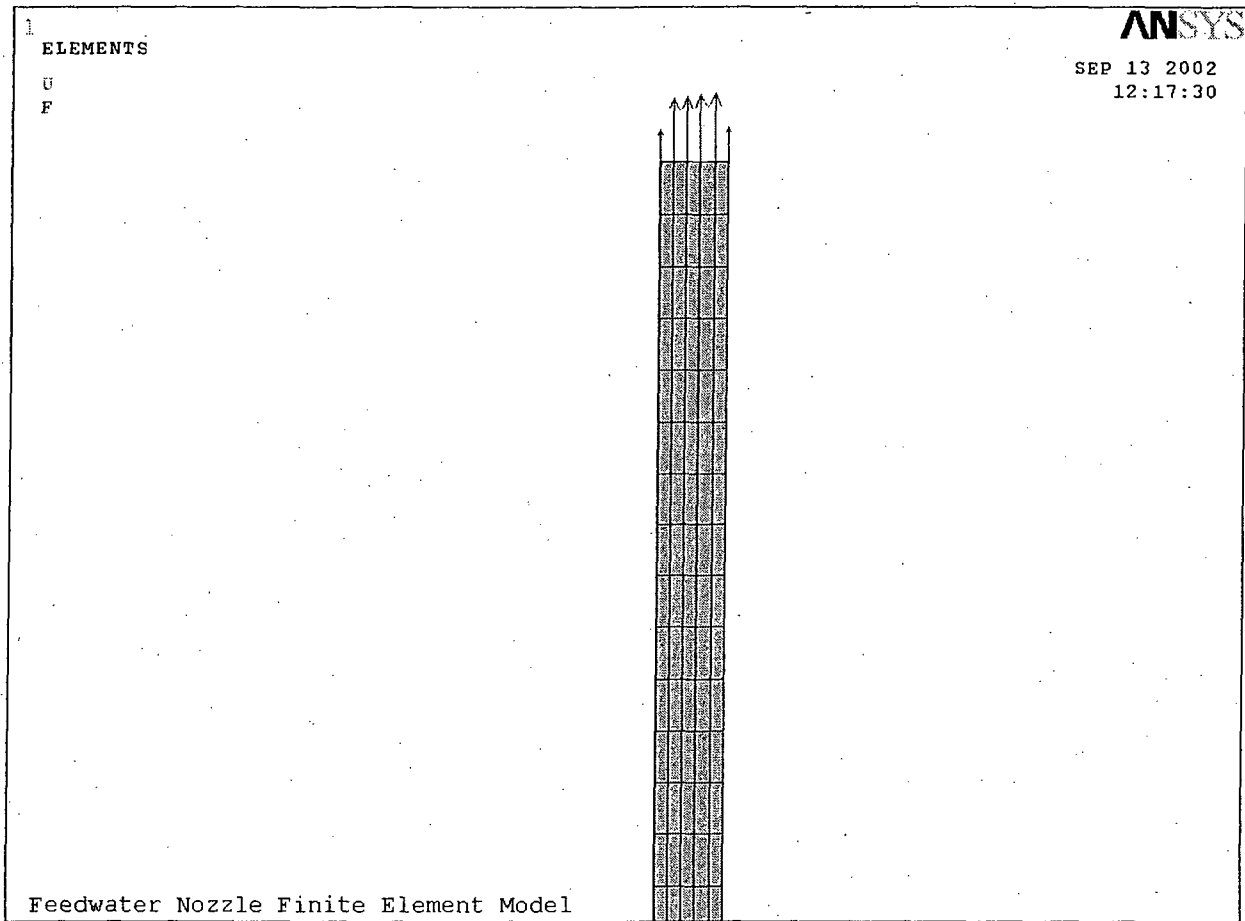


Figure 3: Feedwater Nozzle Pressure Cap Load

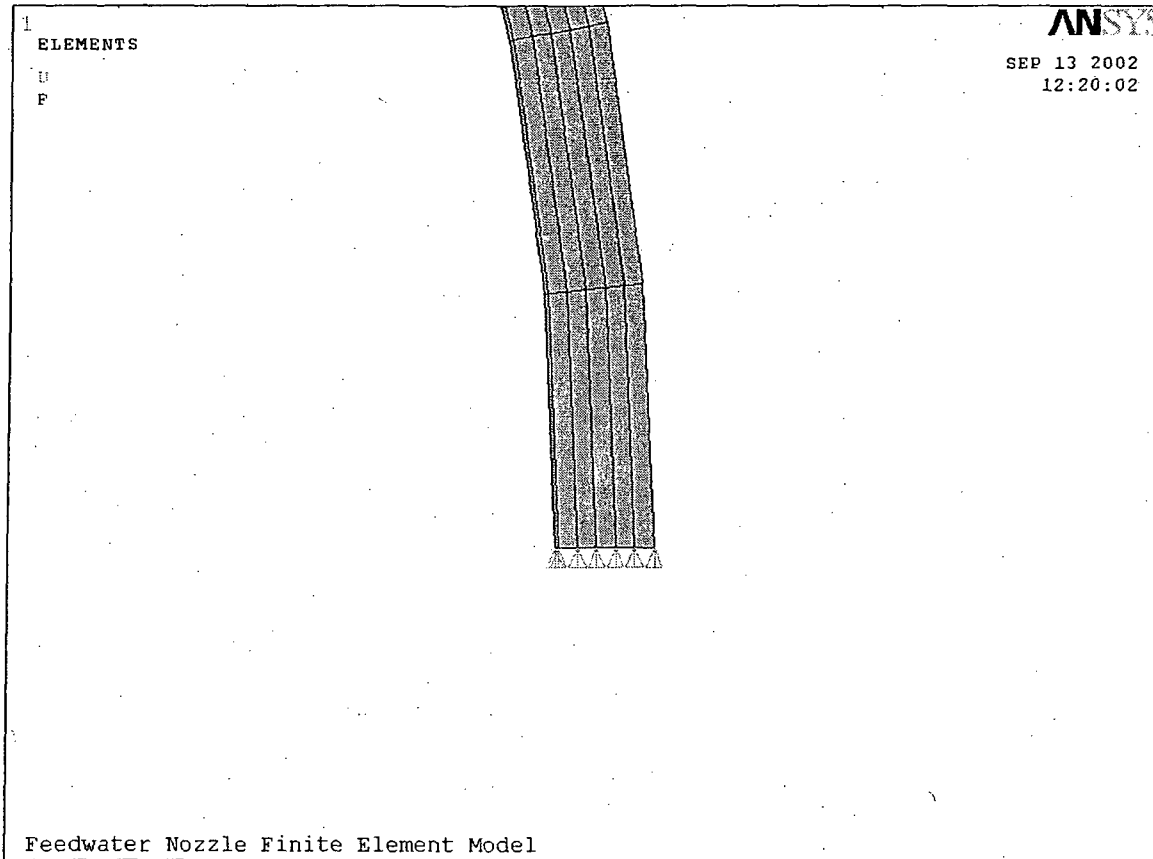
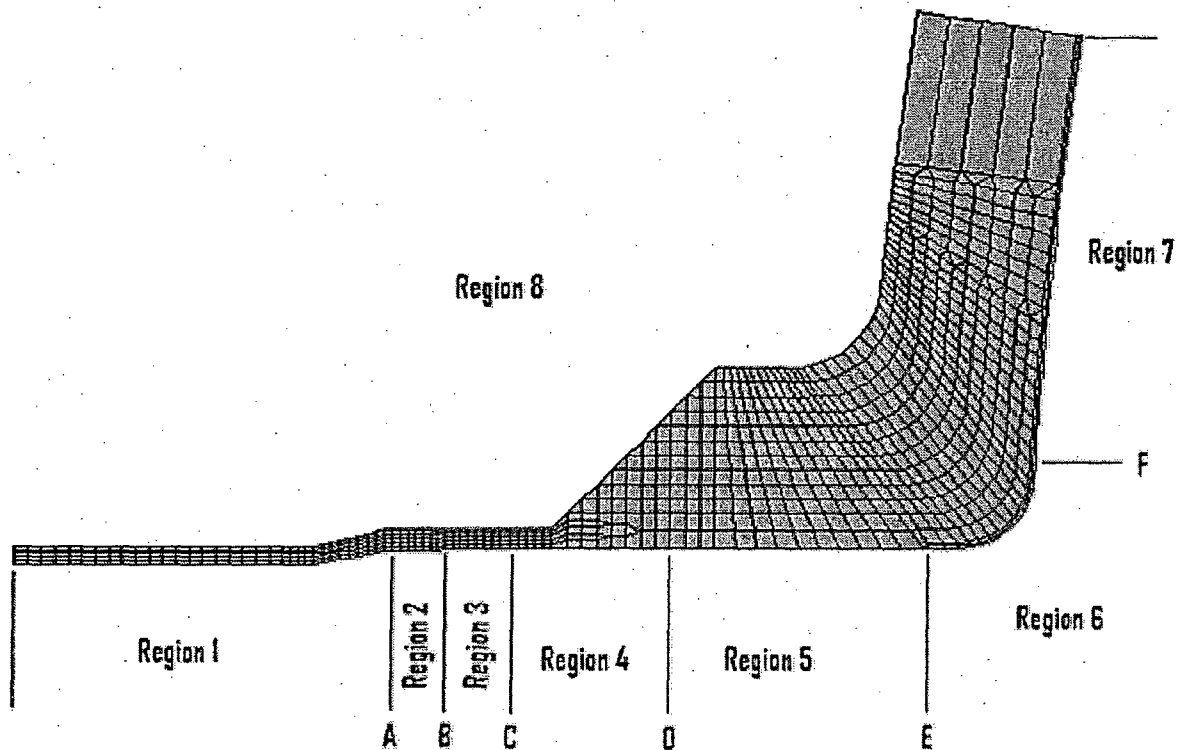


Figure 4: Feedwater Nozzle Vessel Boundary Conditions



- Notes: Point A: End of thermal sleeve = Node 204 = 0.25" from feedwater inlet side of thermal sleeve flat.
Point B: Beginning of annulus = Node 252.
Point C: Beginning of thermal sleeve transition = approximately 4.0" from Point A = Node 294.
Point D: End of thermal sleeve transition = approximately 9.5" from Point A = Node 387.
Point E: End of inner blend radius (nozzle side) = Node 553.
Point F: End of inner blend radius (vessel wall side) = Node 779.

Figure 5: Nozzle and Vessel Wall Thermal and Heat Transfer Boundaries [1]

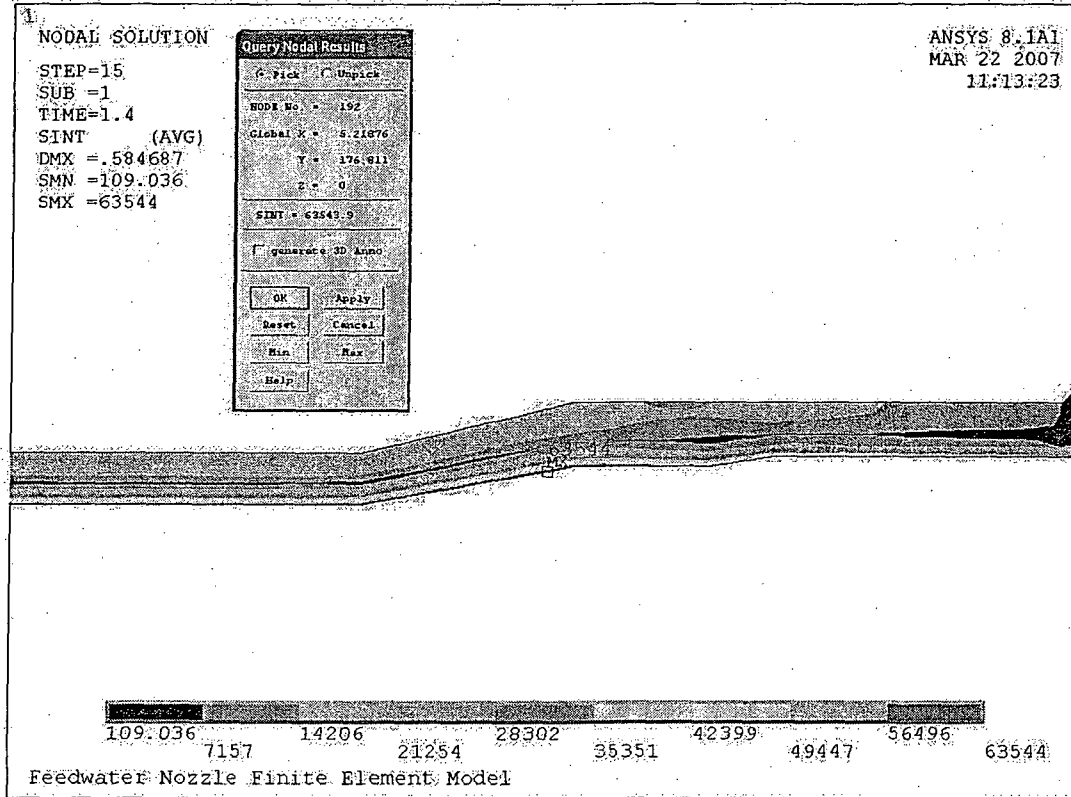


Figure 6: Safe End Critical Thermal Stress Location

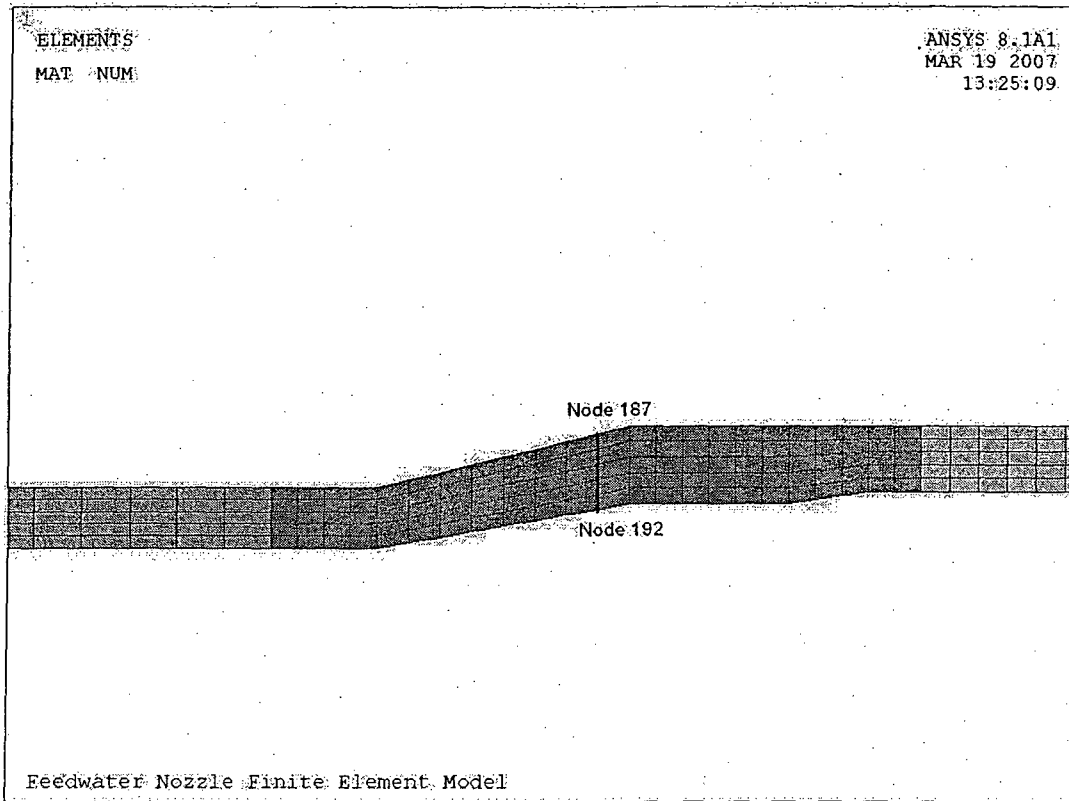


Figure 7: Safe End Limiting Linearized Stress Paths

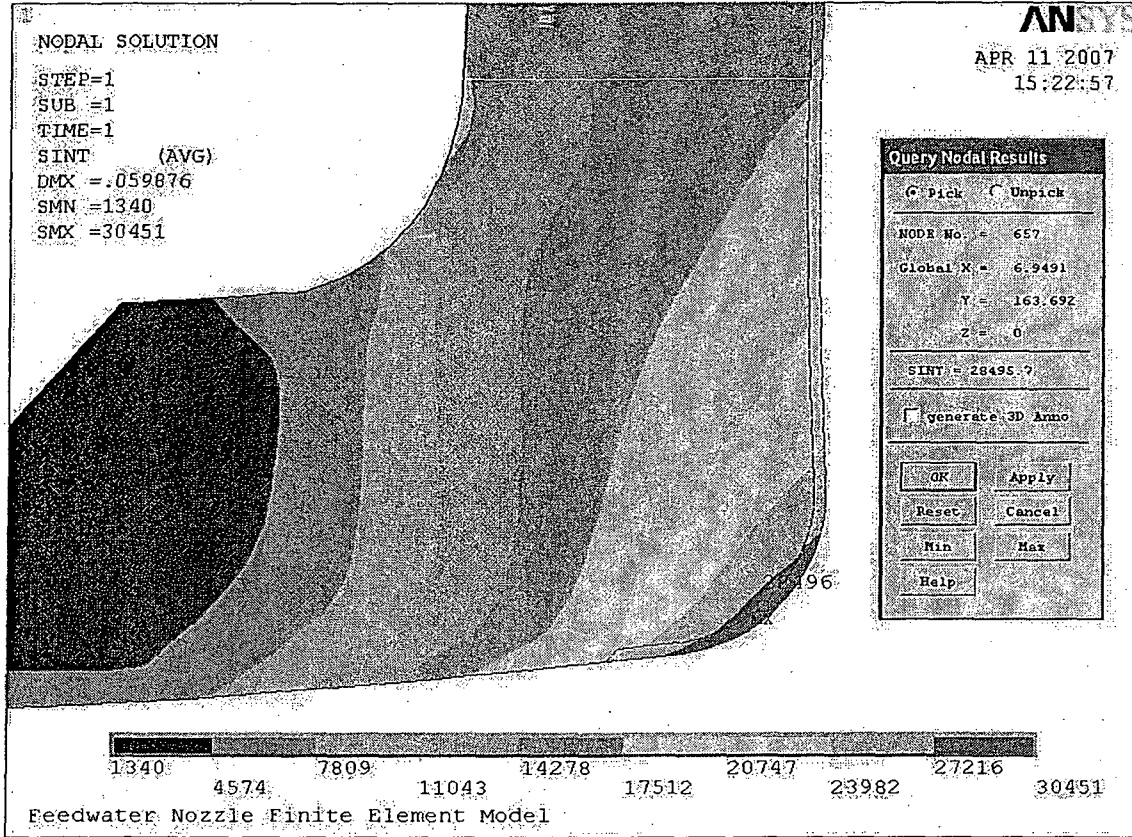


Figure 8: Blend Radius Limiting Pressure Stress Location

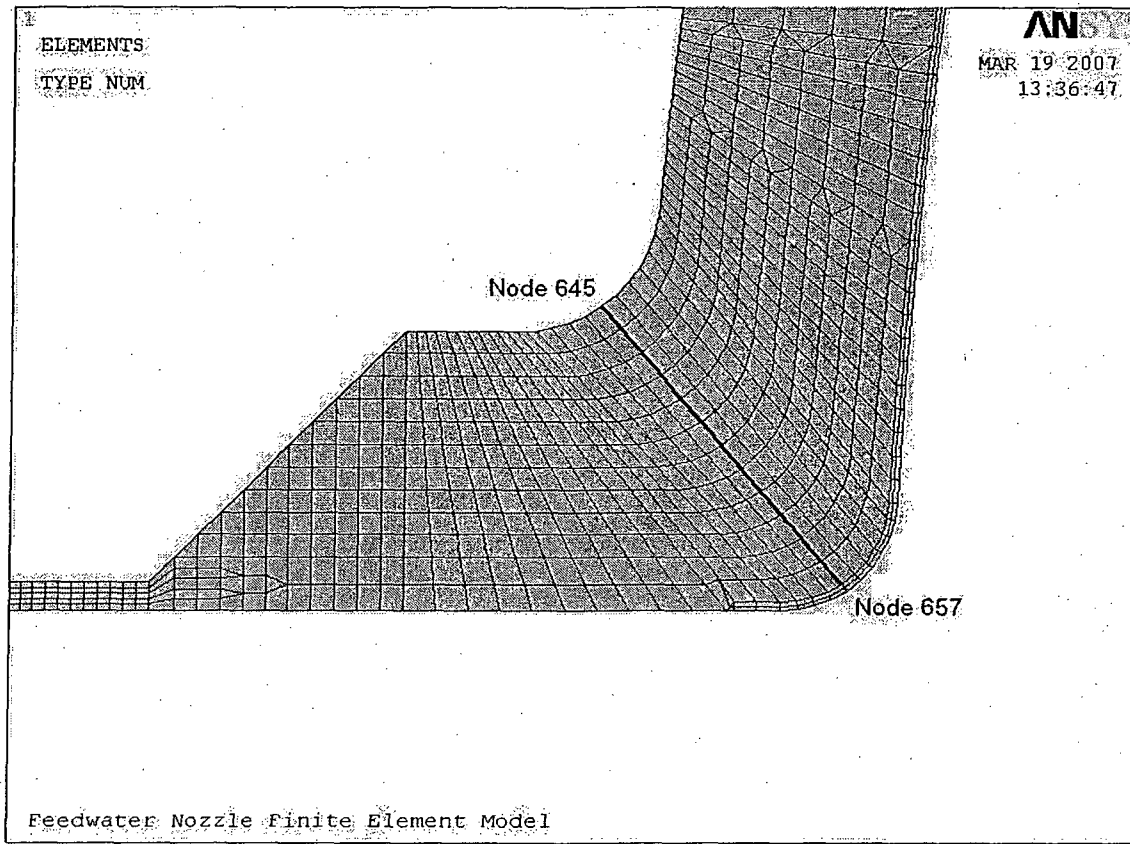


Figure 9: Blend Radius Linearized Stress Path

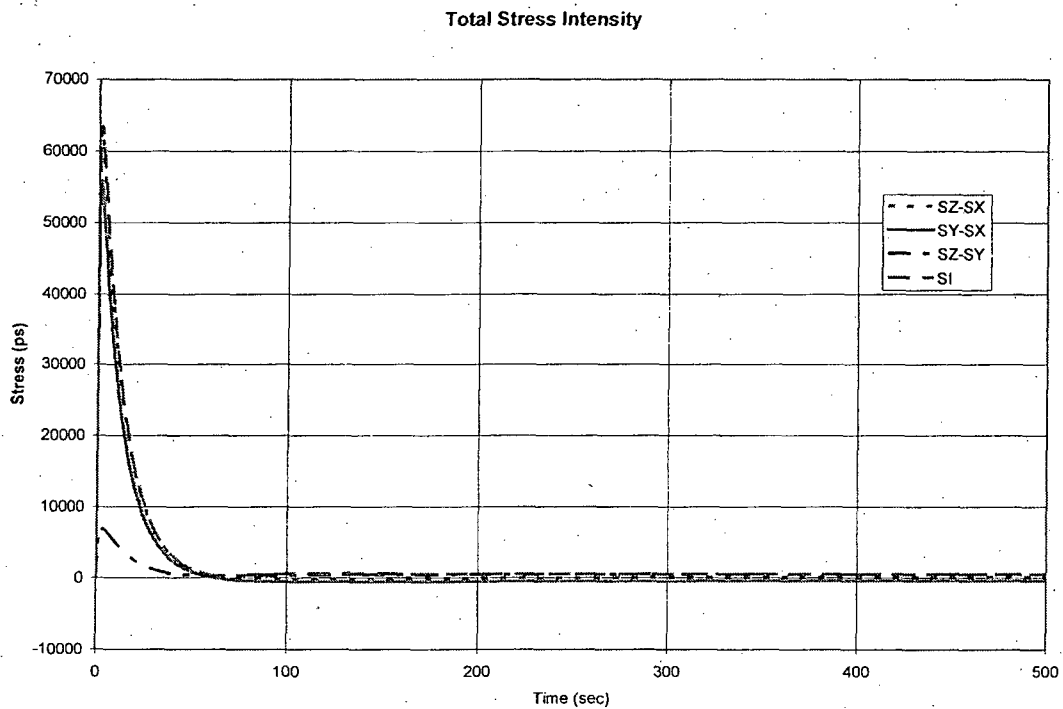


Figure 10: Safe End 100% Flow Total Stress Intensity

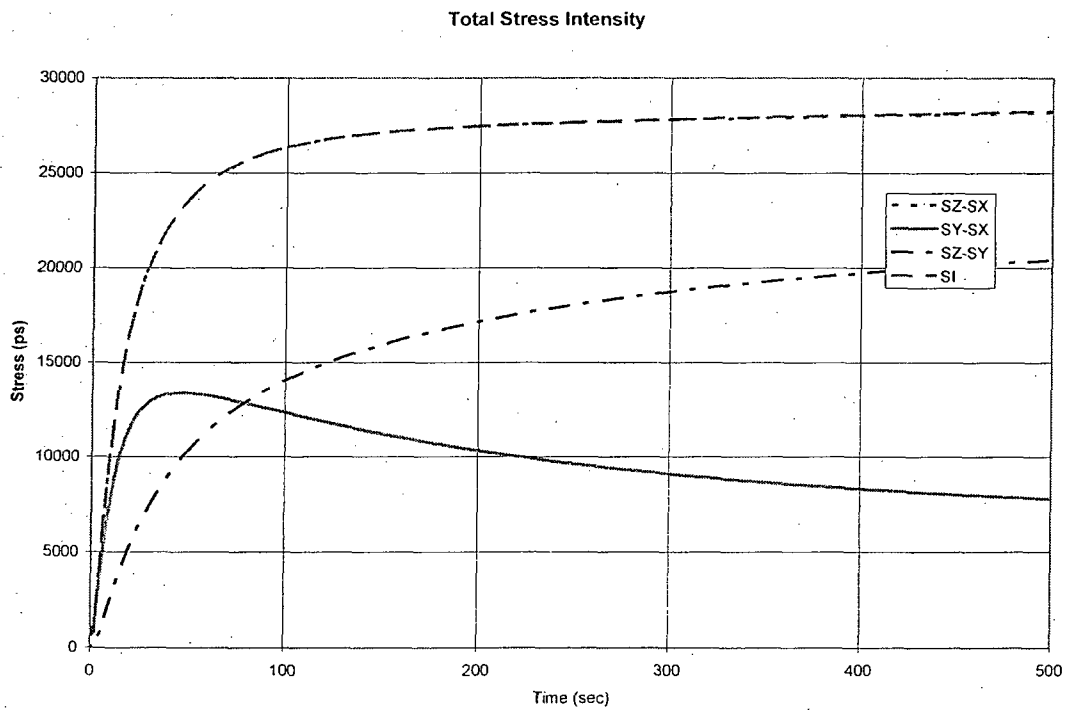


Figure 11: Blend Radius 100% Flow Total Stress Intensity

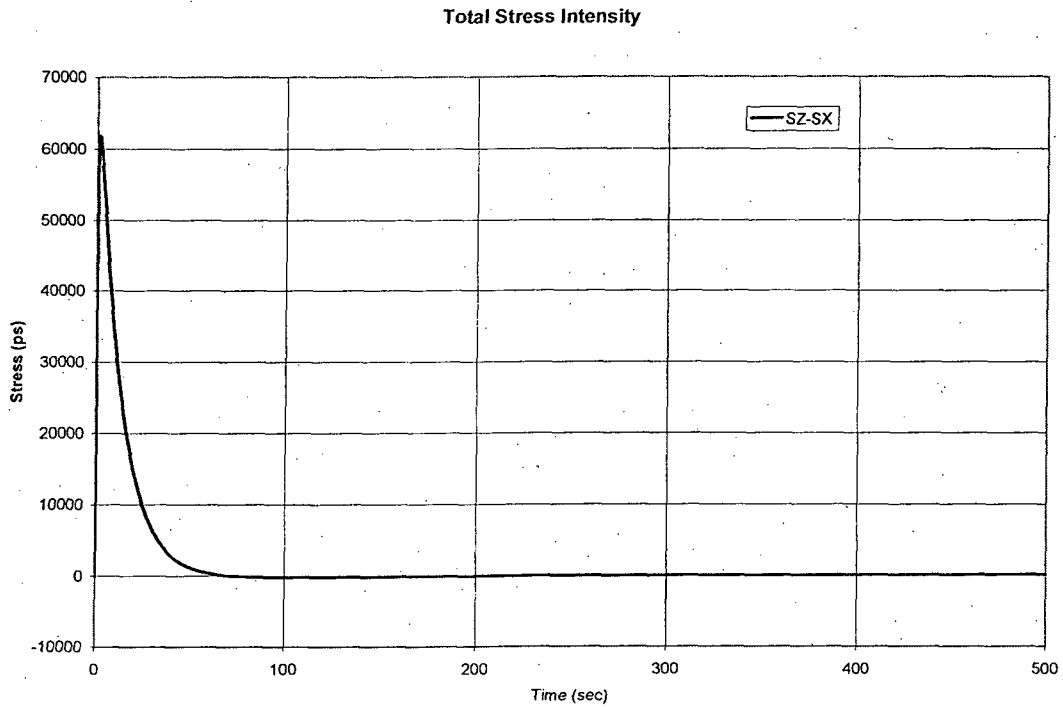


Figure 12: Safe End Total Stress History for 100% Flow

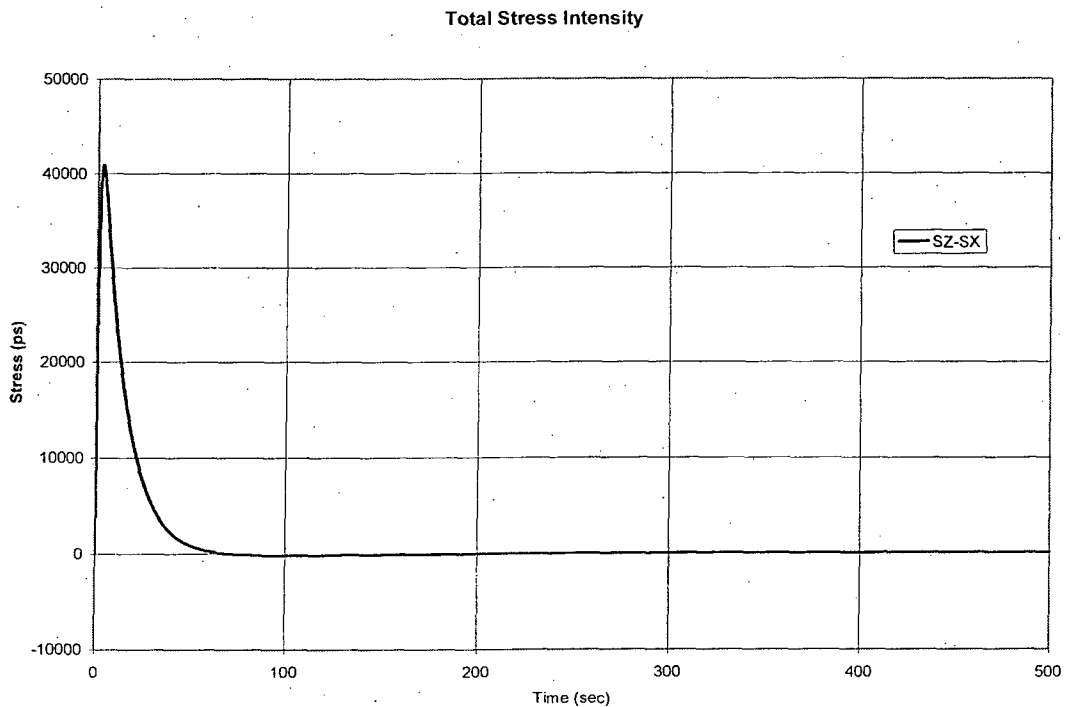


Figure 13: Safe End Membrane Plus Bending Stress History for 100% Flow

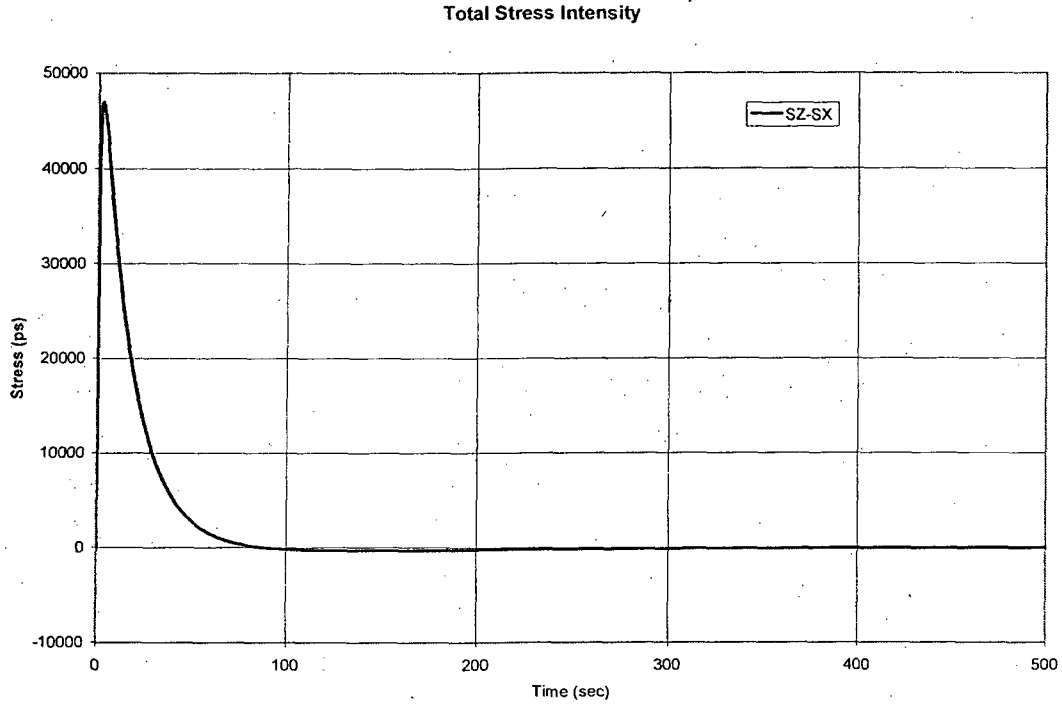


Figure 14: Safe End Total Stress History for 40% Flow

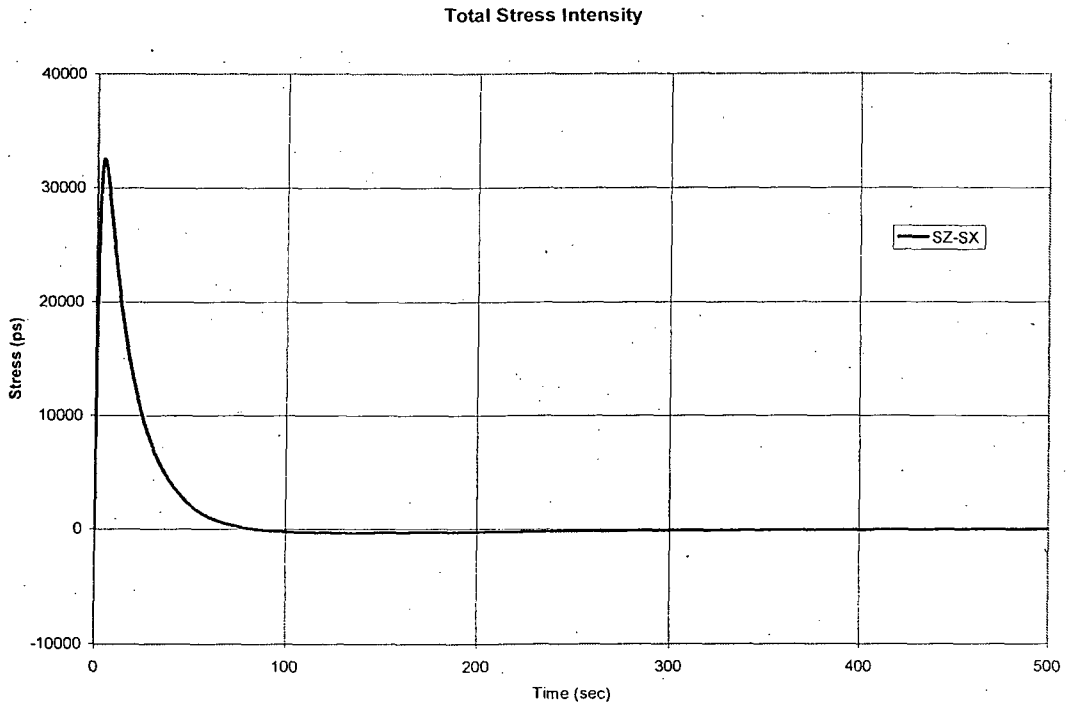


Figure 15: Safe End Membrane Plus Bending Stress History for 40% Flow

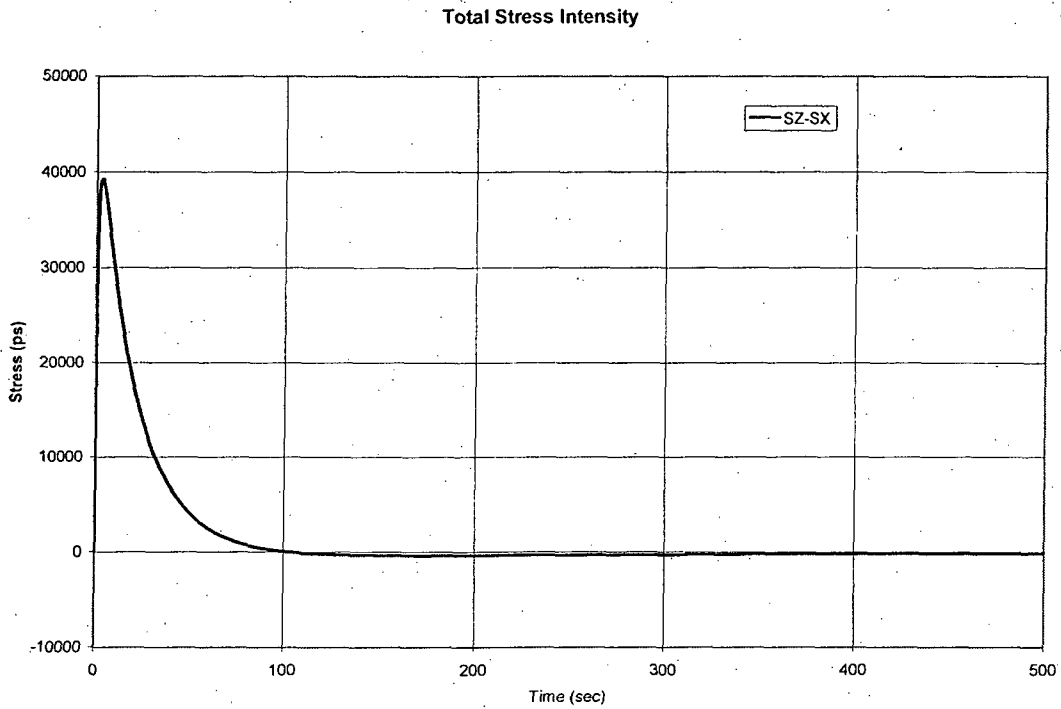


Figure 16: Safe End Total Stress History for 25% Flow

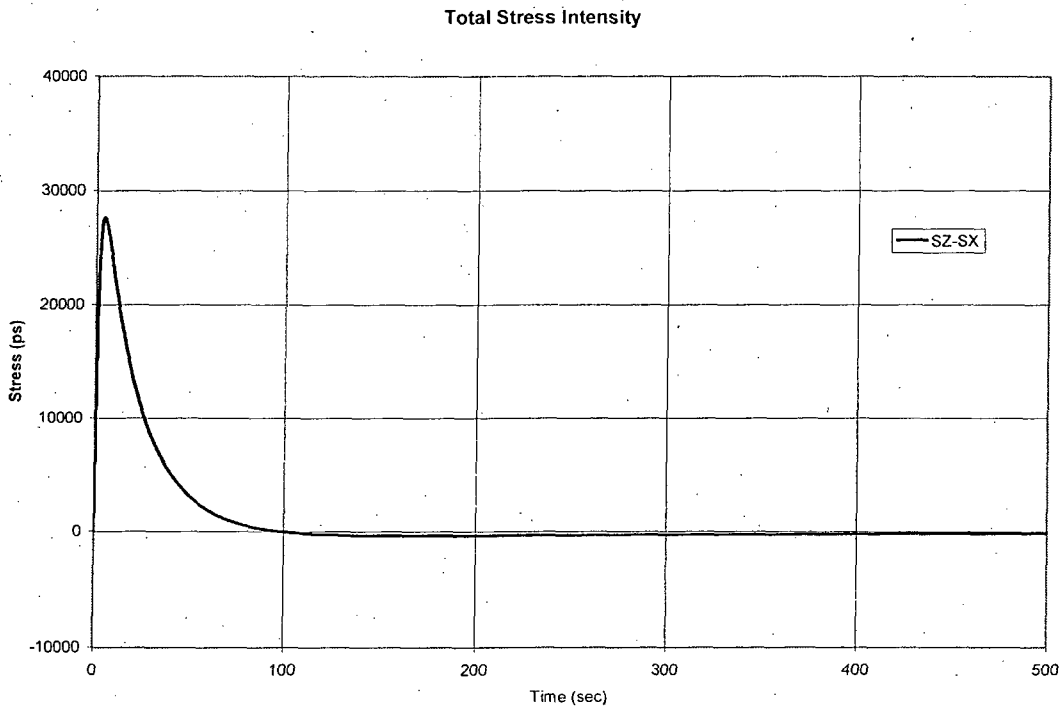


Figure 17: Safe End Membrane Plus Bending Stress History for 25% Flow

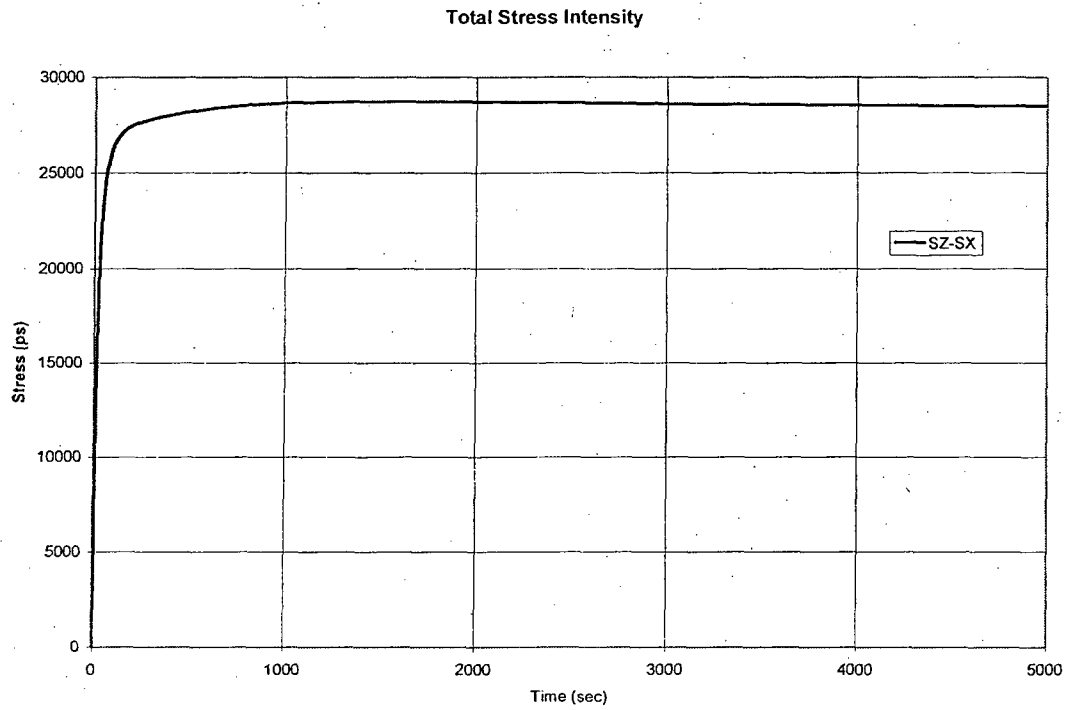


Figure 18: Blend Radius Total Stress History for 100% Flow

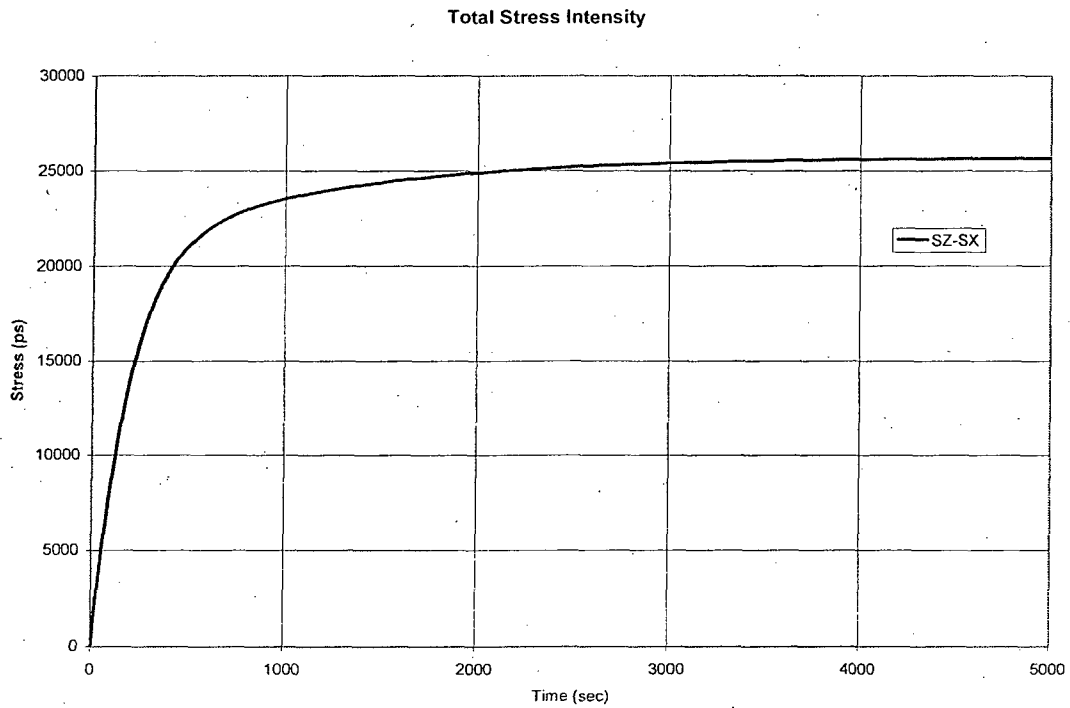


Figure 19: Blend Radius Membrane Plus Bending Stress History for 100% Flow

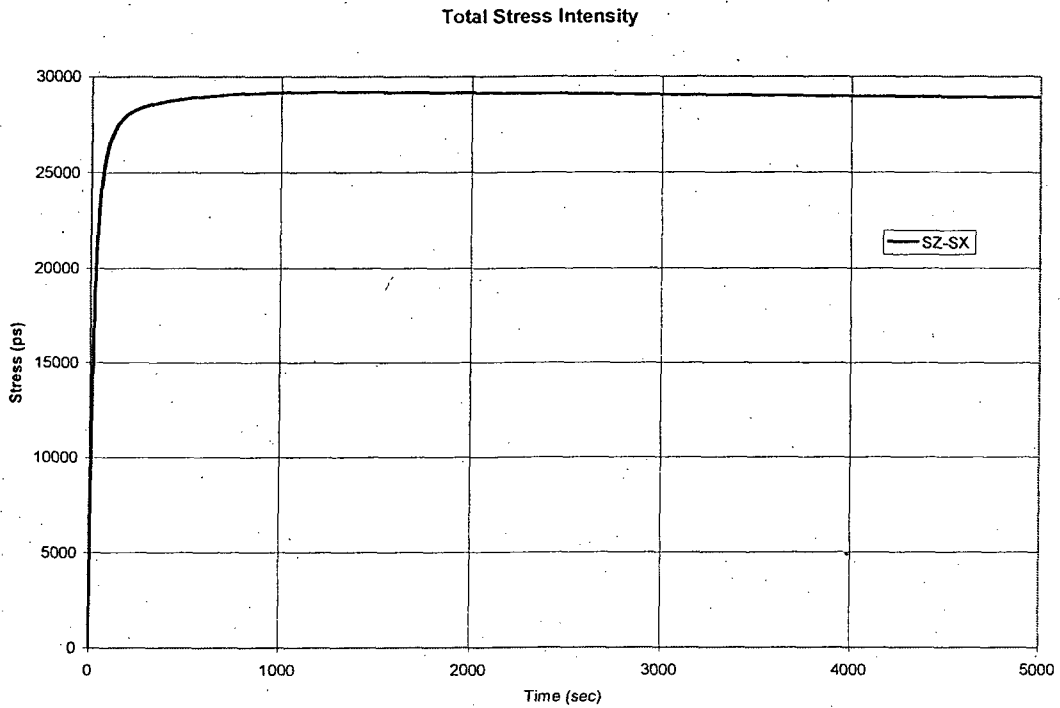


Figure 20: Blend Radius Total Stress History for 40% Flow

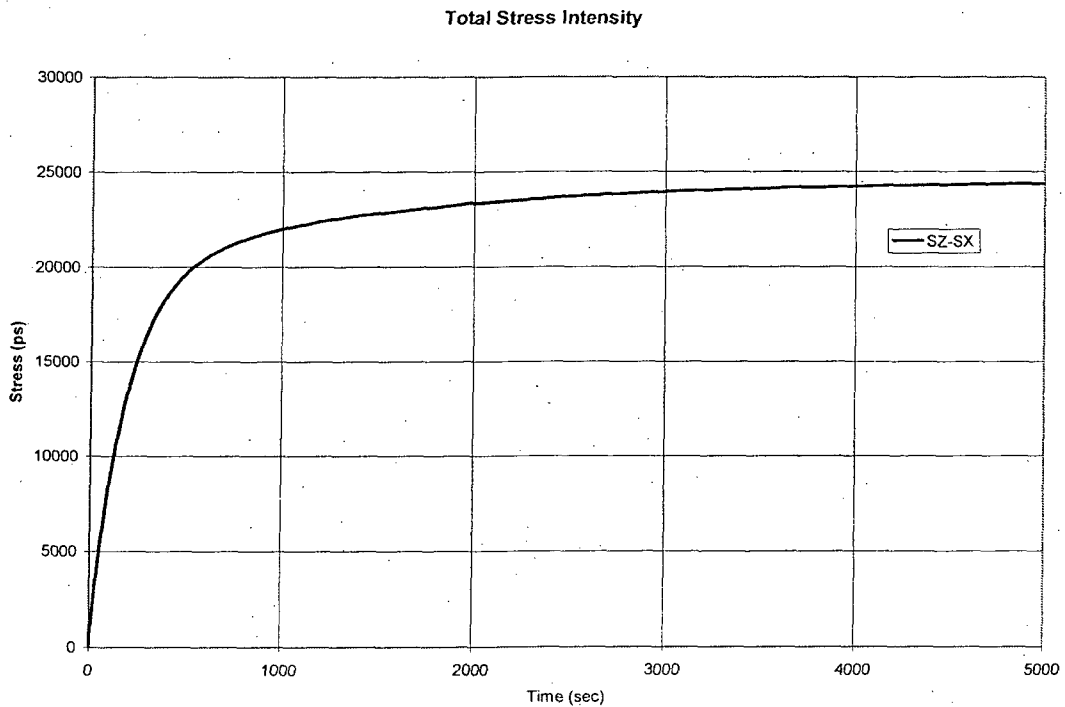


Figure 21: Blend Radius Membrane Plus Bending Stress History for 40% Flow

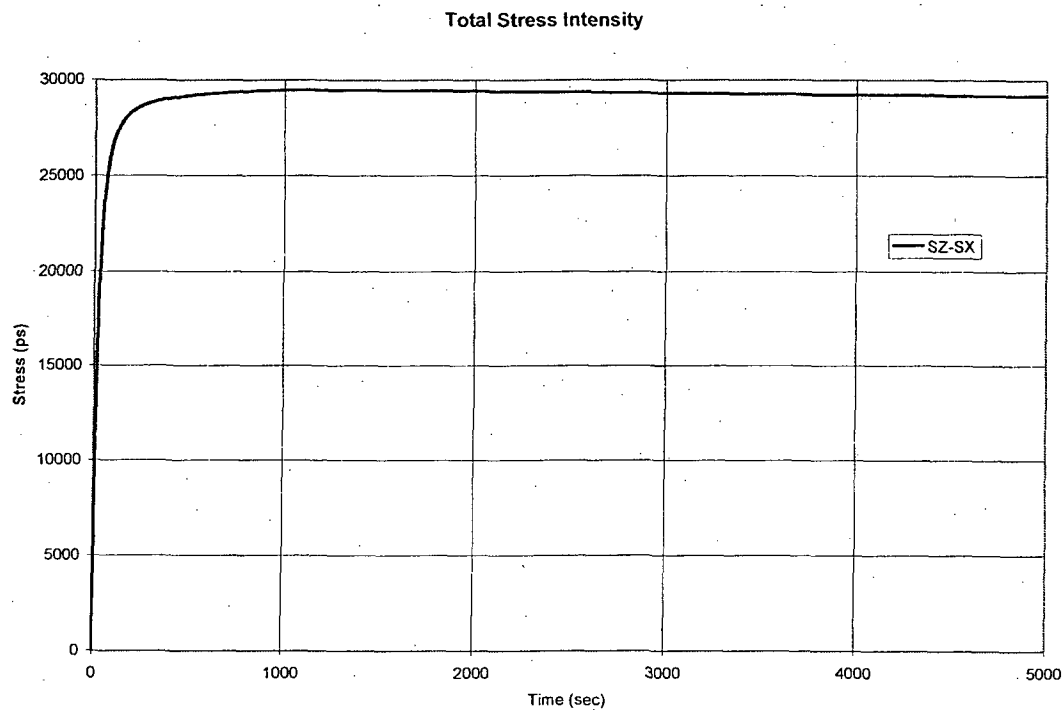


Figure 22: Blend Radius Total Stress History for 25% Flow

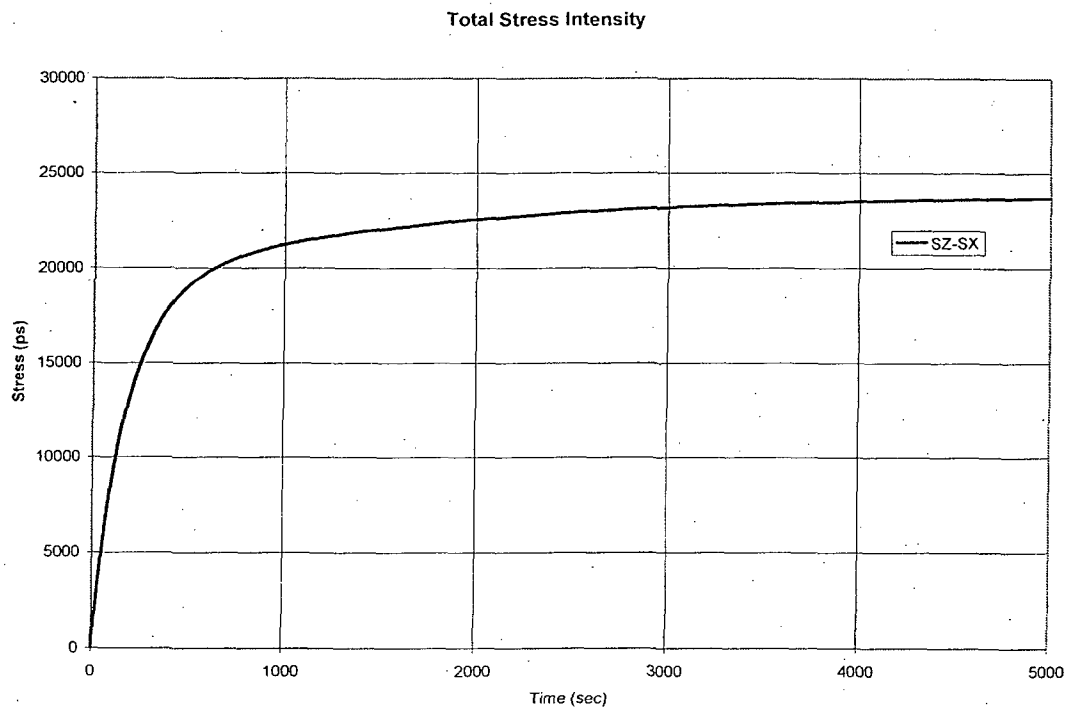


Figure 23: Blend Radius Membrane Plus Bending Stress History for 25% Flow

APPENDIX A
FINITE ELEMENT ANALYSIS FILES



FWP_VY.INP	Input File for Pressure Load	In Computer files
FWT_VY_100.INP	Input File for 100% Flow Thermal Analysis	In Computer files
FWS_VY_100.INP	Input File for 100% Flow Stress Analysis	In Computer files
FWT_VY_40.INP	Input File for 40% Flow Thermal Analysis	In Computer files
FWS_VY_40.INP	Input File for 40% Flow Stress Analysis	In Computer files
FWT_VY_25.INP	Input File for 25% Flow Thermal Analysis	In Computer files
FWS_VY_25.INP	Input File for 25% Flow Stress Analysis	In Computer files
PSE.OUT	Stress Output at Safe End with Pressure Load	In Computer files
PBLEND.OUT	Stress Output at Blend Radius with Pressure Load	In Computer files
PVESH.OUT	Stress Output at Vessel with Pressure Load	In Computer files
#FSE.OUT	Stress Output at Safe End	In Computer files
#FBLEND.OUT	Stress Output at Blend Radius	In Computer files
#FSE_INSIDE.RED	Stress Extracted at Safe End	In Computer files
#FBLEND_INSIDE.RED	Stress Extracted at Blend Radius	In Computer files
#FSE_T-Green.XLS	Green Function with Total Stress at Safe End	In Computer files
#FSE_M+B-Green.XLS	Green Function with Membrane plus Bending Stress at Safe End	In Computer files
#FBR_T-Green.XLS	Green Function with Total Stress at Blend Radius	In Computer files
#FBR_M+B-Green.XLS	Green Function with Membrane plus Bending Stress at Blend Radius	In Computer files

Where # is H, M, L meaning 100%, 40%, and 25% flow rate, respectively.



Structural Integrity Associates, Inc.

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CALCULATION PACKAGE

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PROJECT NAME:

Environmental Fatigue Analysis of VYNPS

CONTRACT NO.:

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CLIENT:

Entergy Nuclear Operations, Inc.

PLANT:

Vermont Yankee Nuclear Power Station

CALCULATION TITLE:

Fatigue Analysis of Feedwater Nozzle

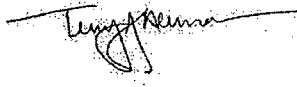

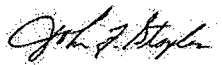
Document Revision	Affected Pages	Revision Description	Project Manager Approval Signature & Date	Preparer(s) & Checker(s) Signatures & Date
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1.0 OBJECTIVE

The purpose of this calculation is to perform a revised fatigue analysis for the feedwater nozzle. Two locations will be analyzed for fatigue acceptance: the safe end (SA508 Class 1) and the blend radius (SA508 Class 2). Both locations are chosen based on the highest overall stress of the analysis performed in Reference [1]. A revised cumulative fatigue factor (CUF) will be determined for both locations, the nozzle forging and safe end, respectively. In the end, the environmental fatigue usage factors will be determined for both locations.

2.0 METHODOLOGY

In order to provide an overall approach and strategy for evaluating the feedwater nozzle, the Green's Function methodology and associated ASME Code stress and fatigue analyses are described in this section.

Revised stress and fatigue analyses are being performed for the feedwater nozzle using ASME Code, Section III methodology. These analyses are being performed to address license renewal requirements to evaluate environmental fatigue for this component in response to Generic Aging Lessons Learned (GALL) Report [11] requirements. The revised analysis is being performed to refine the fatigue usage so that an environmental fatigue factor can be determined for subsequent license renewal efforts.

Two sets of rules are available under ASME Code, Section III, Class 1 [10]. Subparagraph NB-3600 of Section III provides simplified rules for analysis of piping components, and NB-3200 allows for more detailed analysis of vessel components. The NB-3600 piping equations combine by absolute sum the stresses due to pressure, moments and through wall thermal gradient effects, regardless of where within the pipe cross-section the maximum value of the components of stress are located. By considering stress signs, affected surface (inside or outside) and azimuthal position, the stress ranges may be significantly reduced. In addition, NB-3600 assigns stress indices by which the stresses are multiplied to conservatively incorporate the effects of geometric discontinuities. In NB-3200, stress indices are not required, as the stresses are calculated by finite element analysis and consider applicable stress concentration factors. In addition, NB-3200 methodology accounts for the different locations within a component where stresses due to thermal, pressure or other mechanical loading are a maximum. This generally results in a net reduction of the stress ranges and consequently, in the calculated fatigue usage. Article 4 [12] methodology was originally used to evaluate the feedwater nozzle. NB-3200 methodology, which is the modern day equivalent to Article 4, is used in this analysis to be consistent with the Section III design bases for this component, as well as to allow a more detailed analysis of this component. In addition, several of the conservatisms originally used in the original feedwater nozzle evaluation (such as grouping of transients) are removed in the current evaluation so as to achieve a more accurate CUF.

For the feedwater nozzle evaluated as a part of this work, stress histories will be computed by a time integration of the product of a pre-determined Green's Function and the transient data. This Green's



Function integration scheme is similar in concept to the well-known Duhamel theory used in structural dynamics. A detailed derivation of this approach and examples of its application to specific plant locations is contained in Reference [2]. A general outline is provided in this section.

The steps involved in the evaluation are as follows:

- Develop finite element model
- Develop heat transfer coefficients and boundary conditions for the finite element model
- Develop Green's Functions
- Develop thermal transient definitions
- Perform stress analysis to determine stresses for thermal transients
- Perform fatigue analysis

A Green's Function is derived by using finite-element methods to determine the transient stress response of the component to a step change in loading (usually a thermal shock). The critical location in the component is identified based on the maximum stress, and the thermal stress response over time is extracted for this location. This response to the input thermal step is the "Green's Function." Figure 1 shows a typical set of two Green's Functions, each for a different set of heat transfer coefficients (representing different flow rate conditions).

To compute the thermal stress response for an arbitrary transient, the loading parameter (usually local fluid temperature) is deconstructed into a series of step-loadings. By using the Green's Function, the response to each step can be quickly determined. By the principle of superposition, these can be added (algebraically) to determine the response to the original load history. The result is demonstrated in Figure 2. The input transient temperature history contains five step-changes of varying size, as shown in the upper plot in Figure 2. These five step changes produce the five successive stress responses in the second plot shown in Figure 2. By adding all five response curves, the real-time stress response for the input thermal transient is computed.

The Green's Function methodology produces identical results compared to running the input transient through the finite element model. The advantage of using Green's Functions is that many individual transients can be run with a significant reduction of effort compared to running all transients through the finite element model. The trade-off in this process is that the Green's Functions are based on constant material properties and heat transfer coefficients. Therefore, these parameters are chosen to bound all transients that constitute the majority of fatigue usage, i.e., the heat transfer coefficients at 300°F bound the cold water injection transient. In addition, the instantaneous value for the coefficient of thermal expansion is used instead of the mean value for the coefficient of thermal expansion. This conservatism is more than offset by the benefit of not having to analyze every transient, which was done in the VY reactor feedwater nozzle evaluation.

Once the stress history is obtained for all transients using the Green's Function approach, the remainder of the fatigue analysis is carried out using traditional methodologies in accordance with ASME Code, Section III requirements.

Fatigue calculations are performed in accordance with ASME Code, Section III, Subsection NB-3200 methodology. Fatigue analysis is performed for the two limiting locations (one in the safe end and one in the nozzle forging, representing the two materials of the nozzle assembly) using the Green's Functions developed for the three feedwater flow conditions and 60-year projected cycle counts.

Three Structural Integrity utility programs will be used to perform the fatigue analysis. The first two calculate stresses in response to transients. The transients analyzed are those described in the thermal cycle diagrams [3] for the feedwater nozzle. These transients are shown in Figures 4 – 20. The temperatures and pressures for these transients have been modified to account for power uprate [4]. The power uprate pressures and temperatures were used for this analysis. The last program calculates fatigue based on the stress output. The three programs are STRESS.EXE, P-V.EXE, and FATIGUE.EXE. The first program, STRESS.EXE, calculates a stress history in response to a thermal transient using a Green's Function. The second program, P-V.EXE, reduces the stress history to peaks and valleys, as required by ASME Code fatigue evaluation methods. The third program, FATIGUE.EXE, calculates fatigue from the reduced peak and valley history using ASME Code, Section III range-pair methodology. All three programs are explained in detail and have been independently verified for generic use in the Reference [5] calculation.

In order to perform the fatigue analysis, Green's Functions are developed using the finite element model. Then, input files with the necessary data are prepared and the three utility computer programs are run. The first program (STRESS.EXE) requires the following three input files:

- Input file "GREEN.DAT": This file contains the Green's Function for the location being evaluated. For each flow condition, two Green's Functions are determined: a membrane plus bending stress intensity Green's Function and a total stress intensity Green's Function. This allows computation of total stress, as well as membrane plus bending stress, which is necessary to compute K_e per ASME Code, Section III requirements.
- Input file "GREEN.CFG": This file is a configuration file containing parameters that define the Green's Function (i.e., number of points, temperature drop analyzed, etc.).
- Input file "TRANSNT.INP": This file contains the input transient history for all thermal transients to be analyzed for the location being evaluated.

Pressure and piping stress intensities are also included for each transient case, based on pressure stress results from finite element analysis and attached piping load calculations.

The second program (P-V.EXE) simply extracts only the maxima and minima stress (i.e., the peaks and valleys) from the stress histories generated by program STRESS.EXE.

The third program (FATIGUE.EXE) performs the ASME Code peak event-pairing required to calculate a fatigue usage value. The input data consists of the output peak and valley history from program P-V.EXE and a configuration input file that provides ASME Code configuration data relevant to the fatigue analysis (i.e., K_e parameters, S_m , Young's modulus, etc.). The output is the final fatigue calculation for the location being evaluated.



The Green's Function methodology described above uses standard industry stress and fatigue analysis practices, and is the same as the methodology used in typical stress reports. Special approval for the use of this methodology is therefore not required.

3.0 ANALYSIS

The fatigue analysis involves the preparing of input files for, and running of three programs verified and described in Reference [5]. The programs STRESS.EXE and P-V.EXE are run together through the use of a batch file. The program FATIGUE.EXE is run after processing the output from P_V.EXE. The steps associated with this process are described in the following sub-sections.

3.1 Transient Definitions (for program STRESS.EXE)

The program STRESS.EXE requires the following three input files for analyzing an individual transient:

- Green.dat. There are 12 stress history functions obtained from Reference [1]. They represent the membrane plus bending and total stress intensities at the blend radius and safe end locations. Both of the blend radius and the safe end have two stress history functions for each of the following flow conditions; 100%, 40%, and 25% flow.
- Green.cfg is configured as described in Reference [5].
- Transnt.inp. These files are created to represent the transients shown on the thermal cycle diagrams and redefined by power uprate. Note that transients 12, 13, and 15 are nearly identical on the thermal cycle diagram [3] and the results from running transient 12 will be used for all three transients. Transient 16, 17 and 18 will not be considered since there is no temperature change. Tables 1 and 2 show the thermal history used to represent each transient. Based upon the thermal cycle diagram for the feedwater nozzle [3], the transients are split into the following groups based upon flow rate:
 - Transients 3, 20, 20A, and 21-23 are run at 25% flow. Although Reference [3] shows 15% flow rate, it is conservative to use 25% flow rate for these transients. Transient 20, Hot Standby, is split up into two parts. The first portion is "Heatup portion" and the second portion is "Feedwater Injection portion" that are defined from Reference [3].
 - Transient 11 is run at 40% flow. Transient 11 starts off and ends at 100% flow.
 - Transients 5, 6, 9, 10, and 19 are run at 100% flow.
 - Transient 4 is run at 100% flow only to obtain the last stress point. The remainder of the stress points for transient 4 is obtained from the 25% flow stress results. The results are pulled from the two flow case results based upon the flow rates defined in the thermal cycle diagram [3].
 - Transients 12, 13, 14 and 15 were run at 100% flow. Heat transfer coefficients were not re-calculated for the 1 minute intervals each of these transients is at 110% flow. The effect of this small flow rate increase for such a relatively short duration should be minor.
 - Transients 1, 2, 24, and 25 are set as no thermal stress due to very small temperature changes (70°F to 100°F) at these transients.



3.2 Peak and Valley Points of the Stress History (for program P-V.EXE)

The program P-V.exe is then run to extract the peaks and valleys from the STRESS.OUT file produced by the STRESS.EXE program. The only input required for this program is STRESS.OUT and it outputs all the peaks and valleys to P-V.OUT. Columns 2 through 5 of Tables 4 (for the blend radius) and 5 (for the safe end) show the final peak and valley output. The pressure for column 6 is then filled in using the thermal cycle diagrams. Pressure and piping loads have to be added to the peak and valley points to calculate the final stress values used for fatigue analysis.

3.3 Pressure Load

The pressure stress associated with a 1000 psi internal pressure was determined in Reference [1]. These values are as follows:

Pressure stress for the safe end:

- 8693 psi membrane plus bending stress intensity.
- 8891 psi total linearized stress intensity.

Pressure stress for the blend radius:

- 36653 psi membrane plus bending stress intensity.
- 37733 psi total linearized stress intensity.

These pressure stress values for each location were linearly scaled with pressure. The actual pressure for column 6 of Tables 4 and 5 is obtained from Tables 1 and 2. The scaled pressure stress values are shown in columns 7 and 8 of Tables 4 and 5.

The pressure stress is combined with the thermal and piping loads to calculate the final stress values used for fatigue analysis.

3.4 Attached Piping Loads

Additionally, the piping stress intensity (stress caused by the attached piping) was determined. These piping forces and moments are determined as shown in Figure 3.

The following formulas are used to determine the maximum stress intensity in the nozzle at the two locations of interest. From engineering statics, the piping loads at the end of the model can be translated to the first and second cut locations using the following equations:

$$\begin{aligned} \text{For Cut I:} \quad (M_x)_1 &= M_x - F_y L_1 \\ (M_y)_1 &= M_y + F_x L_1 \end{aligned}$$

$$\begin{aligned} \text{For Cut II:} \quad (M_x)_2 &= M_x - F_y L_2 \\ (M_y)_2 &= M_y + F_x L_2 \end{aligned}$$

The total bending moment and shear loads are obtained using the equations below:

$$\text{For Cut I: } M_{xy} = \sqrt{(M_x)_1^2 + (M_y)_1^2}$$

$$F_{xy} = \sqrt{(F_x)_1^2 + (F_y)_1^2}$$

$$\text{For Cut II: } M_{xy} = \sqrt{(M_x)_2^2 + (M_y)_2^2}$$

$$F_{xy} = \sqrt{(F_x)_2^2 + (F_y)_2^2}$$

The distributed loads for a thin-walled cylinder are obtained using the equations below:

$$N_z = \frac{1}{\pi R_N} \left[\frac{1}{2} F_z + \frac{M_{xy}}{R_N} \right]$$

$$q_N = \frac{1}{\pi R_N} \left[F_{xy} - \frac{M_z}{2R_N} \right]$$

To determine the primary stresses, P_M , due to internal pressure and piping loads, the following equations are used.

For Cut I, using thin-walled equations:

$$(P_M)_z = \frac{Pa_N}{2t_N} + \frac{Nz}{t_N}$$

$$(P_M)_\theta = \frac{Pa_N}{t_N}$$

$$(P_M)_R = -P$$

$$\tau_M = \frac{q_N}{t_N}$$

$$SI_{MAX} = 2 \sqrt{\left(\frac{(P_M)_\theta - (P_M)_R}{2} \right)^2 + (\tau_M)_{z\theta}^2}$$

or

$$SI_{MAX} = 2 \sqrt{\left(\frac{(P_M)_z - (P_M)_R}{2} \right)^2 + (\tau_M)_{z\theta}^2}$$

Because pressure was considered separately in this analysis, the equations used for Cut I are valid for Cut II.



- where:
- L_1 = The length from the end of the nozzle where the piping loads are applied to the location of interest in the safe end.
 - L_2 = The length from the end of the nozzle where the piping loads are applied to the location of interest in the blend radius.
 - M_{xy} = The maximum bending moment in the xy plane.
 - F_{yx} = The maximum shear force in the xy plane.
 - N_z = The normal force per inch of circumference applied to the end of the nozzle in the z direction.
 - q_N = The shear force per inch of circumference applied to the nozzle.
 - R_N = The mid-wall nozzle radius.

Since the pressure was considered separately in this analysis, the equations can be simplified as follows:

$$(P_M)_z = \frac{N_z}{t_N}$$

$$(P_M)_\theta = 0$$

$$(P_M)_R = 0$$

$$\tau_M = \frac{q_N}{t_N}$$

$$SI_{MAX} = 2(\tau_M)_{z\theta}$$

or

$$SI_{MAX} = 2 \sqrt{\left(\frac{N_z}{2t_N}\right)^2 + (\tau_M)_{z\theta}^2}$$

Per Reference [6], the feedwater nozzle piping loads are as follows:

$$F_x = 3,000 \text{ lbs}$$

$$F_y = 15,000 \text{ lbs}$$

$$F_z = 3,200 \text{ lbs}$$

$$M_x = 28,000 \text{ ft-lb} = 336,000 \text{ in-lb}$$

$$M_y = 13,000 \text{ ft-lb} = 156,000 \text{ in-lb}$$

$$M_z = 40,000 \text{ ft-lb} = 480,000 \text{ in-lb}$$

The loads are applied at the connection of the piping and safe end. Therefore, the L_1 is equal to 12.0871 inches and the L_2 is equal to 27.572 inches. The calculations for the safe end and blend radius are shown in Table 3. The first cut location is the same as the Green's Function cross section per [1] at the safe end, and the second cut is from Node 645 (outside) to Node 501 (inside). The maximum stress intensities due to piping loads are 5707.97 psi at the safe end and 265.47 psi at the blend radius, respectively. The piping load sign is set as the same as the thermal stress sign.

These piping stress values are scaled assuming no stress occurs at an ambient temperature of 70°F and the full values are reached at reactor design temperature, 575°F. The scaled piping stress values

are shown in columns 9 and 10 of Tables 4 and 5. Columns 11 and 12 of Tables 4 and 5 show the summation of all stresses for each thermal peak and valley stress point.

3.5 Fatigue Analysis (for program FATIGUE.EXE)

The number of cycles projected for the 60-year operating life is used for each transient [3]:

Column 13 in Tables 4 and 5 shows the number of cycles associated with each transient. The number of cycles for 60 years was obtained from Reference [3].

The program FATIGUE.EXE performs the "ASME Code style" peak event pairing required to calculate a fatigue usage value. The input data for FATIGUE.CFG is as follows:

	Blend Radius	Safe End
Parameters m and n for Computing K_e	2.0 & 0.2 (low alloy steel) [10]	3.0 & 0.2 (carbon steel) [10]
Design Stress Intensity Values, S_m	26700 psi [8] @ 600°F	17800 psi [8] @ 600°F
Elastic Modulus from Applicable Fatigue Curve	30.0×10^6 psi [10]	30.0×10^6 psi [10]
Elastic Modulus Used in Finite Element Model	26.7×10^6 psi	28.1×10^6 psi
The Geometric Stress Concentration Factor K_t	1.0	1.34 [7, page 35 of S4]

The results of the fatigue analyses are presented in Tables 6 and 7 for the blend radius and safe end for 60 years, respectively.

The results described are contained in EXCEL files *BRresults.xls* and *SEresults.xls*, which are contained in the computer files.

4.0 FATIGUE USAGE RESULTS

The blend radius cumulative usage factor (CUF) from system cycling is 0.0636 for 60 years. The safe end CUF is 0.1471 for 60 years.

5.0 ENVIRONMENTAL FATIGUE ANALYSIS

In the response to NRC request for additional information (RAI) 4.3-H-02, VYNPS states that they have conservatively assumed that fatigue cracks may be present in the clad. VYNPS manages this cracking by performing periodic inspections that were implemented in response to Generic Letters 80-095 and



81-11, and NUREG-0619. The inspection frequency is based on the calculated fatigue crack growth of a postulated flaw in the nozzle inner blend radius. The VYNPS fatigue crack growth calculation uses methods in compliance with GE BWR Owners Group Topical Report "Alternate BWR Feedwater Nozzle Inspection Requirements", GE-NE-523-A71-0594, Revision 1, August 1999 and the associated NRC Final Safety Evaluation (TAC No. MA6787) dated March 10, 2000. The NRC has reviewed and approved this approach to handling FW nozzle inner blend radius cracking (Letter D.H. Dorman (USNRC) to D.A. Reid (VYNPC), Subject: Evaluation of Request for Relief from NUREG-0619 for VYNPS dated 2/6/95, (TAC No. M88803)).

The analysis performed for the feedwater nozzle calculated fatigue in the blend radius base metal, not the clad. This is consistent with the VYNPS position stated in the response to RAI 4.3-H-02, and is also consistent with ASME Code methodology since cladding is structurally neglected in fatigue analyses, per ASME Code, Section III, NB-3122.3.

Per Reference [9], the dissolved Oxygen (DO) calculation shows the overall HWC availability is 47%. This means the time ratio under NWC (pre-HWC) is 53%.

For the safe end location, the environmental fatigue factors for post-HWC and pre-HWC are all 1.74 from Table 3 of Reference [9]. It results in an EAF adjusted CUF of $1.74 \times 0.1471 = 0.2560$ for 60 years, which is acceptable (i.e., less than the allowable value of 1.0). The overall environmental multiplier is 1.74.

For the blend radius location, the environmental fatigue factors for post-HWC and pre-HWC are 11.14 and 8.82 from Table 4 of Reference [9]. These results in an EAF adjusted CUF of $(11.14 \times 53\% + 8.82 \times 47\%) \times 0.0636 = 0.6392$ for 60 years, which is acceptable (i.e., less than the allowable value of 1.0). The overall environmental multiplier is 10.0496.

6.0 REFERENCES

1. SI Calculation No. VY-16Q-301, Revision 0, "Feedwater Nozzle Stress History Development for Green Functions."
2. Kuo, A. Y., Tang, S. S., and Riccardella, P. C., "An On-Line Fatigue Monitoring System for Power Plants, Part I - Direct Calculation of Transient Peak Stress Through Transfer Matrices and Green's Functions," ASME PVP Conference, Chicago, 1986.
3. Entergy Design Input Record (DIR) EC No. 1773, Revision 0; "Environmental Fatigue Analysis for Vermont Yankee Nuclear Power Station," 7/3/07, SI File No. VY-16Q-209.
4. GE Certified Design Specification No. 26A6019, Revision 1, "Reactor Vessel - Extended Power Uprate," SI File No. VY-05Q-236.
5. Structural Integrity Associates Calculation (Generic) No. SW-SPVF-01Q-301, Revision 0, "STRESS.EXE, P-V.EXE, and FATIGUE.EXE Software Verification."
6. GE Drawing No. 919D294, Revision 11, Sht. No. 7, "Reactor Vessel," SI File No. VY-05Q-241.
7. Chicago Bridge & Iron Company Contractor 9-6201, Revision 2, "Section S4, Stress Analysis Feedwater Nozzle Vermont Yankee Reactor Vessel," SI File No. VY-05Q-238.



8. American Society of Mechanical Engineers, Boiler and Pressure Vessel Code, Section II, Part D, 1998 Edition, 2000 Addenda.
9. SI Calculation No. VY-16Q-303, Revision 0, "Environmental Fatigue Evaluation of Reactor Recirculation Inlet Nozzle and Vessel Shell Bottom Head."
10. American Society of Mechanical Engineers, Boiler and Pressure Vessel Code, Section III Subsection NB, 1998 Edition, 2000 Addenda.
11. NUREG-1801, Revision 1, "Generic Aging Lessons Learned (GALL) Report," U. S. Nuclear Regulatory Commission, September 2005.
12. American Society of Mechanical Engineers, Boiler and Pressure Vessel Code, Section III, Subsection A, Article 4, 1965 Edition with Winter 1966 Addenda.

Table 1: Blend Radius Transients

Transient Number	Time (s)	Temp (°F)	Time Step (s)	Pressure (psig)	Transient Number	Time (s)	Temp (°F)	Time Step (s)	Pressure (psig)	Transient Number	Time (s)	Temp (°F)	Time Step (s)	Pressure (psig)
1. Bolt-up	0	70		0	10. FW Heater	0	392		1010	14. SRV	0	392		1010
123 Cycles	10	70	10	0	Bypass	90	265	90	1010	Blowdown	60	275	60	885
2. Design	0	70		0	70 Cycles	1890	265	1600	1010	1 Cycles	960	100	900	50
HYD Test	1080	100	1080	0	HF_100	2070	392	180	1010	HF_100	5960	100	5000	50
120 Cycles	5280	100	3600	1100	11. Loss of FW Pumps	1	565	1	1010	18. Reduction to 0% Power 300 Cycles	1800	265	1800	1010
	5880	100	600	50	10 Cycles	3.5	565	2.5	1190	HF_100	6800	265	5000	1010
	10880	100	5000	50	MF_40, HF_100	4.5	50	1	1185	20. Hot Standby (Heatup Portion)	1	440	1	1010
3. Startup	0	100		50		13.5	50	9	1135	300 Cycles	3925	549	3924	1010
300 Cycles	16164	549	16164	1010		184.5	50	171	1135	LF_25	8925	549	5000	1010
LF_25	21164	549	5000	1010		1564.5	440	1380	1135	20A. Hot Standby (FW Injection Portion)	0	549	1	1010
4. Turbine Roll and increased to Rated	0	549		1010		1565.5	565	1	1135	300 Cycles	181	100	180	1010
Power	1801	100	1800	1010		2165.5	565	600	1135	LF_25	241	290	60	1010
300 cycles	1802	260	1	1010		2166.5	50	1	1135		451	549	210	1010
LF_25, HF_100	3602	392	1800	1010		2346.5	50	180	885		5451	549	5000	1010
5. Daily Reduction	0	392		1010		5406.5	440	3060	1055	21-23. Shutdown	0	549		1010
75% Power	800	310	900	1010		5407.5	565	1	1055	300 Cycles	6264	375	6264	50
10,000 Cycles	2700	310	1800	1010		6727.5	565	1320	1135	LF_25	6864	330	600	50
HF_100	3600	392	900	1010		6728.5	50	1	1135		15144	100	8280	50
	8600	392	5000	1010		7448.5	300	300	675		20144	100	5000	50
6. Weekly Reduct	0	392		1010		11048.5	400	3600	232	24. Hydrostatic Test	0	100		50
60% Power	1800	280	1800	1010		16411.5	549	5363	885		600	100	600	1563
2,000 Cycles	3600	280	1800	1010		16412.5	549	1	1010	1 Cycles	1200	100	600	1563
HF_100	5400	392	5000	1010		18212.5	549	1800	1010		1800	100	600	50
	10400	392	5000	1010		18213.5	100	1	1010		2400	100	600	50
8. Turbine Trip at 25% Power	0	392		1010		20013.5	100	1800	1010	25. Unbolt	0	100		0
Power	1800	265	1800	1010		20014.5	260	1	1010	123 Cycles	1080	70	1080	0
10 Cycles	1980	265	180	1010		21814.5	392	1800	1010		6980	70	5000	0
HF_100	2340	90	360	1010		26814.5	392	5000	1010					
	2520	90	180	1010	12. Turbine Generator Trip	0	392		1010					
	3420	265	900	1010		10	392	10	1135/1375 ⁽²⁾					
	3600	265	180	1010		15	392	5	1135/1375 ⁽²⁾					
	5400	392	1800	1010	13. Reactor Overpressure	30	392	15	940					
	10400	392	5000	1010		90	275	60	940					
						990	100	900	940					
						15. Other	2790	100	1800	940				
						SCRAMs	2791	260	1	940				
						228 cycles	3210	291	419	1010				
						HF_100	4591	392	1381	1010				
							8591	392	5000	1010				

Note: 1. The indicated time or pressure was assumed.
 2. 1375 psi is for Transient 13 only.

Table 2: Safe End Transient

Transient Number	Time (s)	Temp (°F)	Time Step (s)	Pressure (psig)	Transient Number	Time (s)	Temp (°F)	Time Step (s)	Pressure (psig)	Transient Number	Time (s)	Temp (°F)	Time Step (s)	Pressure (psig)
1. Bolt-up	0	70		0	10. FW Heater	0	392		1010	14. SRV	0	392		1010
123 Cycles	10	70	10	0	Bypass	90	265	90	1010	Blowdown	60	275	60	885
2. Design	0	70		0	70 Cycles	1890	265	1600	1010	1 Cycles	960	100	900	50
HYD Test	1080	100	1080	0	HF_100	2070	392	180	1010	HF_100	1460	100	500	50
120 Cycles	5280	100	3600	1100	11. Loss of FW Pumps	1	565	1	1010	18. Reduction to 0% Power 300 Cycles	1800	265	1800	1010
	5880	100	600	50	10 Cycles	3.5	565	2.5	1190	HF_100	2300	265	500	1010
	6380	100	500	50	MF_40, HF_100	4.5	50	1	1185	20. Hot Standby (Heatup Portion)	1	440	1	1010
3. Startup	0	100		50		13.5	50	9	1135	300 Cycles	3925	549	3924	1010
300 Cycles	16164	549	16164	1010		184.5	50	171	1135	LF_25	4425	549	500	1010
LF_25	16664	549	500	1010		1564.5	440	1380	1135	20A. Hot Standby (FW Injection Portion)	0	549	1	1010
4. Turbine Roll and increased to Rated	0	549		1010		1565.5	565	1	1135	300 Cycles	181	100	180	1010
Power	1801	100	1800	1010		2165.5	565	600	1135	LF_25	241	290	60	1010
300 cycles	1802	260	1	1010		2166.5	50	1	1135		451	549	210	1010
LF_25, HF_100	3602	392	1800	1010		2346.5	50	180	885		5451	549	500	1010
5. Daily Reduction	0	392		1010		5406.5	440	3060	1055	21-23. Shutdown	0	549		1010
75% Power	800	310	900	1010		5407.5	565	1	1055	300 Cycles	6264	375	6264	50
10,000 Cycles	2700	310	1800	1010		6727.5	565	1320	1135	LF_25	6864	330	600	50
HF_100	3600	392	900	1010		6728.5	50	1	1135		15144	100	8280	50
	8600	392	5000	1010		7448.5	300	300	675		15644	100	500	50
6. Weekly Reduct	0	392		1010		11048.5	400	3600	232	24. Hydrostatic Test	0	100		50
60% Power	1800	280	1800	1010		16411.5	549	5363	885		600	100	600	1563
2,000 Cycles	3600	280	1800	1010		16412.5	549	1	1010	1 Cycles	1200	100	600	1563
HF_100	5400	392	5000	1010		18212.5	549	1800	1010		1800	100	600	50
	10400	392	5000	1010		18213.5	100	1	1010		2400	100	600	50
8. Turbine Trip at 25% Power	0	392		1010		20013.5	100	1800	1010	25. Unbolt	0	100		0
Power	1800	265	1800	1010		20014.5	260	1	1010	123 Cycles	1080	70	1080	0
10 Cycles	1980	265	180	1010		21814.5	392	1800	1010		6980	70	5000	0
HF_100	2340	90	360	1010		22314.5	392	500	1010					
	2520	90	180	1010	12. Turbine Generator Trip	0	392		1010					
	3420	265	900	1010		10	392	10	1135/1375 ⁽²⁾					
	3600	265	180	1010		15	392	5	1135/1375 ⁽²⁾					
	5400	392	1800	1010	13. Reactor Overpressure	30	392	15	940					
	5900	392	500	1010		90	275	60	940					
						990	100	900	940					
						15. Other	2790	100	1800	940				
						SCRAMs	2791	260	1	940				
						228 cycles	3210	291	419	1010				
						HF_100	4591	392	1381	1010				
							5091	392	500	1010				

Note: 1. These transients are the same as in Table 1 with the exception of the 500 second steady state time increment that is used. The transients in Table 1 are plotted using a 5000 second steady state increment. The difference is due to the length of the Green's Function for the safe end which is shorter compared to the blend Radius.
 2. The indicated time or pressure was assumed.
 3. 1375 psi is for Transient 13 only.

Table 3: Maximum Piping Stress Intensity Calculations

Safe End External Piping Loads			Blend Radius External Piping Loads		
Parameters			Parameters		
$F_x =$	3.00	kips	$F_x =$	3.00	kips
$F_y =$	15.00	kips	$F_y =$	15.00	kips
$F_z =$	3.20	kips	$F_z =$	3.20	kips
$M_x =$	336.00	in-kips	$M_x =$	336.00	in-kips
$M_y =$	156.00	in-kips	$M_y =$	156.00	in-kips
$M_z =$	480.00	in-kips	$M_z =$	480.00	in-kips
OD=	11.86	in	OD=	22.67	in
ID=	10.409	in	ID=	10.750	in
$R_N =$	5.57	in	$R_N =$	8.35	in
L =	12.09	in	L =	27.57	in
$t_N =$	0.72	in	$t_N =$	5.96	in
$(M_x)_1 =$	154.69	in-kips	$(M_x)_2 =$	-77.58	in-kips
$(M_y)_1 =$	192.26	in-kips	$(M_y)_2 =$	238.72	in-kips
$M_{xy} =$	246.77	in-kips	$M_{xy} =$	251.01	in-kips
$F_{xy} =$	15.30	kips	$F_{xy} =$	15.30	kips
$N_z =$	2.63	kips/in	$N_z =$	1.21	kips/in
$q_N =$	-1.59	kips/in	$q_N =$	-0.51	kips/in
Primary Membrane Stress Intensity			Primary Membrane Stress Intensity		
$PM_z =$	3.63	ksi	$PM_z =$	0.20	ksi
$\tau =$	-2.20	ksi	$\tau =$	-0.09	ksi
$SI_{max} =$	5.71	ksi	$SI_{max} =$	0.27	ksi
$SI_{max} =$	5707.97	psi	$SI_{max} =$	265.47	psi

Note: The locations for Cut I and Cut II were defined in Reference [1] for safe end and blend radius paths, respectively.

Table 4: Blend Radius Stress Summary

1	2	3	4	5	6	7	8	9	10	11	12	13
Transient Number	Time (s)	Total Stress (psi)	M+B Stress (psi)	Temperature F	Pressure (psig)	Total Pressure Stress (psi)	M+B Pressure Stress (psi)	Total Piping Stress (psi)	M+B Piping Stress (psi)	Total Total Stress (psi)	Total M+B Stress (psi)	Number of Cycles (60 years)
1	0	0	0	70	0	0	0	0	0	0.00	0.00	123
	0	0	0	70	0	0	0	0	0	0.00	0.00	120
2	1680	0	0	100	1100	41506.3	40318.3	15.77042	15.77042	41522.07	40334.07	120
	10880	0	0	100	50	1886.65	1832.65	15.77042	15.77042	1902.42	1848.42	120
	0	29166	23676	100	50	1886.65	1832.65	15.77042	15.77042	31068.42	25524.42	300
3	16782.8	-3577	-3138	549	1010	38110.33	37019.53	-251.801	-251.801	34281.53	33629.73	300
	21164	-3532	-3138	549	1010	38110.33	37019.53	-251.801	-251.801	34326.53	33629.73	300
	0	-3530	-3158	549	1010	38110.33	37019.53	-251.801	-251.801	34328.53	33609.73	300
4	1801.9	29465	22266	244.004	1010	38110.33	37019.53	91.47053	91.47053	67666.80	59377.00	300
	8602	7720	6749	392	1010	38110.33	37019.53	169.2692	169.2692	45999.60	43937.80	300
	0	7720	6752	392	1010	38110.33	37019.53	169.2692	169.2692	45999.60	43940.80	10000
5	2229.8	13598	11941	311.002	1010	38110.33	37019.53	126.6901	126.6901	51835.02	49087.22	10000
	8600	7720	6752	392	1010	38110.33	37019.53	169.2692	169.2692	45999.60	43940.80	10000
	0	7720	6752	392	1010	38110.33	37019.53	169.2692	169.2692	45999.60	43940.80	2000
6	2820.3	15742	13892	280.691	1010	38110.33	37019.53	110.7562	110.7562	53963.09	51022.29	2000
	10400	7720	6752	392	1010	38110.33	37019.53	169.2692	169.2692	45999.60	43940.80	2000
	0	7720	6752	392	1010	38110.33	37019.53	169.2692	169.2692	45999.60	43940.80	10
9	2524	29006	23417	118.311	1010	38110.33	37019.53	25.39616	25.39616	67141.73	60461.93	10
	10400	7720	6752	392	1010	38110.33	37019.53	169.2692	169.2692	45999.60	43940.80	10
	0	7720	6752	392	1010	38110.33	37019.53	169.2692	169.2692	45999.60	43940.80	70
10	1632.4	16828	14701	267.399	1010	38110.33	37019.53	103.7688	103.7688	55042.10	51824.30	70
	7070	7720	6752	392	1010	38110.33	37019.53	169.2692	169.2692	45999.60	43940.80	70
	0	7720	6752	392	1010	38110.33	37019.53	169.2692	169.2692	45999.60	43940.80	10
	3.5	6620	6632	565	1190	44902.27	43617.07	260.2119	260.2119	51782.48	50509.28	10
	4.5	6190	6608	50	1185	44713.61	43433.81	10.51361	10.51361	50914.12	50052.32	10
	194.5	31720	21067	109.348	1135	42826.96	41601.16	20.68448	20.68448	74567.64	62688.84	10
	2166.3	-4761	-1859	513.483	972	36676.48	35626.72	-233.1304	-233.1304	31682.35	33534.59	10
	2362.5	31268	22070	102.255	1010	38110.33	37019.53	16.95583	16.95583	69395.29	59106.49	10
	6728.3	-4913	-3149	513.448	1010	38110.33	37019.53	-233.112	-233.112	32964.22	33637.42	10
	7149.9	32114	21472	83.333	1010	38110.33	37019.53	7.0089	7.0089	70231.34	58498.54	10
	18213.3	-3565	-3162	503.978	1010	38110.33	37019.53	-228.1338	-228.1338	34317.20	33629.40	10
	19122.6	29156	23083	100.048	1010	38110.33	37019.53	15.79565	15.79565	67282.13	60118.33	10
	26814.5	7720	6410	392	1010	38110.33	37019.53	169.2692	169.2692	45999.60	43598.80	10
	0	7720	6752	392	1010	38110.33	37019.53	169.2692	169.2692	45999.60	43940.80	60
	10	7720	6752	392	1135	42826.96	41601.16	169.2692	169.2692	50716.22	48522.42	60
12	30	7720	6752	392	940	35469.02	34453.82	169.2692	169.2692	43358.29	41375.09	60
	2033.7	28648	25301	132.007	940	35469.02	34453.82	32.59588	32.59588	64149.62	59787.42	60
	9591	7720	6752	392	1010	38110.33	37019.53	169.2692	169.2692	45999.60	43940.80	60
	0	7720	6752	392	1010	38110.33	37019.53	169.2692	169.2692	45999.60	43940.80	1
	10	7720	6752	392	1375	51882.88	50397.88	169.2692	169.2692	59772.14	57319.14	1
	30	7720	6752	392	940	35469.02	34453.82	169.2692	169.2692	43358.29	41375.09	1
13	2033.7	28648	25301	132.007	1010	38110.33	37019.53	32.59588	32.59588	66790.93	62353.13	1
	9591	7720	6752	392	1010	38110.33	37019.53	169.2692	169.2692	45999.60	43940.80	1
	0	7720	6752	392	1010	38110.33	37019.53	169.2692	169.2692	45999.60	43940.80	1
14	5960	28487	25650	100	50	1886.65	1832.65	15.77042	15.77042	30389.42	27498.42	1
	0	7720	6752	392	1010	38110.33	37019.53	169.2692	169.2692	45999.60	43940.80	228
	10	7720	6752	392	1135	42826.96	41601.16	169.2692	169.2692	50716.22	48522.42	228
	30	7720	6752	392	940	35469.02	34453.82	169.2692	169.2692	43358.29	41375.09	228
15	2033.7	28648	25301	132.007	1010	38110.33	37019.53	32.59588	32.59588	66790.93	62353.13	228
	9591	7720	6752	392	1010	38110.33	37019.53	169.2692	169.2692	45999.60	43940.80	228
	0	7720	6752	392	1010	38110.33	37019.53	169.2692	169.2692	45999.60	43940.80	300
19	6800	16752	14971	265	1010	38110.33	37019.53	102.5077	102.5077	54964.84	52093.04	300
	0	17151	13815	265	1010	38110.33	37019.53	102.5077	102.5077	55363.84	50937.04	300
20	8925	-3531	-3146	549	1010	38110.33	37019.53	-251.801	-251.801	34327.53	33621.73	300
	0	-3530	-3158	549	1010	38110.33	37019.53	-251.801	-251.801	34328.53	33609.73	300
20A	183	28102	12153	233	1010	38110.33	37019.53	85.68595	85.68595	66298.02	49258.22	300
	5451	-3530	-3158	549	1010	38110.33	37019.53	-251.801	-251.801	34328.53	33609.73	300
	0	-3530	-3158	549	1010	38110.33	37019.53	-251.801	-251.801	34328.53	33609.73	300
21-23	20144	29168	23656	100	50	1886.65	1832.65	15.77042	15.77042	31070.42	25504.42	300
	0	0	0	100	50	1886.65	1832.65	15.77042	15.77042	1902.42	1848.42	1
24	600	0	0	100	1563	58976.68	57288.64	15.77042	15.77042	58992.45	57304.41	1
	2400	0	0	100	50	1886.65	1832.65	15.77042	15.77042	1902.42	1848.42	1
	0	0	0	100	0	0	0	15.77042	15.77042	15.77	15.77	123
25	1580	0	0	70	0	0	0	0	0	0.00	0.00	123



Table 4: Blend Radius Stress Summary (Continue)

- NOTES: Column 1: Transient number identification.
Column 2: Time during transient where a maxima or minima stress intensity occurs from P-V.OUT output file.
Column 3: Maxima or minima total stress intensity from P-V.OUT output file.
Column 4: Maxima or minima membrane plus bending stress intensity from P-V.OUT output file.
Column 5: Temperature per total stress intensity.
Column 6: Pressure per Table 1.
Column 7: Total pressure stress intensity from the quantity (Column 6 x 37733)/1000 [Table 3, 1].
Column 8: Membrane plus bending pressure stress intensity from the quantity (Column 6 x 36653)/1000 [Table 3, 1].
Column 9: Total external stress from calculation in Table 3, $265.47 \text{ psi} * (\text{Column 5} - 70^\circ\text{F}) / (575^\circ\text{F} - 70^\circ\text{F})$.
Column 10: Same as Column 9, but for M+B stress.
Column 11: Sum of total stresses (Columns 3, 7, and 9).
Column 12: Sum of membrane plus bending stresses (Columns 4, 8, and 10).
Column 13: Number of cycles for the transient (60 years).

Table 5: Safe End Stress Summary

1	2	3	4	5	6	7	8	9	10	11	12	13
Transient Number	Time (s)	Total Stress (psi)	M+B Stress (psi)	Temperature F	Pressure (psig)	Total Pressure Stress (psi)	M+B Pressure Stress (psi)	Total Piping Stress (psi)	M+B Piping Stress (psi)	Total Total Stress (psi)	Total M+B Stress (psi)	Number of Cycles (60 years)
1	0	0	0	70	0	0	0	0	0	0.00	0.00	123
2	0	0	0	70	0	0	0	0	0	0.00	0.00	120
	1680	0	0	100	1100	9780.1	9562.3	339.0875	339.0875	10119.19	9901.39	120
	6960	0	0	100	50	444.55	434.65	339.0875	339.0875	783.64	773.74	120
3	0	-170	-165	100	50	444.55	434.65	-339.0875	-339.0875	-64.54	-69.44	300
	153.2	-235	-212	104.256	50	444.55	434.65	-387.1927	-387.1927	-177.64	-164.54	300
	16328.2	2	3	549	1010	8979.91	8779.93	5414.097	5414.097	14396.01	14197.03	300
	16664	-1	0	549	1010	8979.91	8779.93	-5414.097	-5414.097	3564.81	14194.03	300
4	0	-3	-2	549	1010	8979.91	8779.93	-5414.097	-5414.097	3562.81	3363.83	300
	3.6	44060	30988	100	1010	8979.91	8779.93	339.0875	339.0875	53379.00	40107.02	300
	1804.6	-15889	-11224	260.286	1010	8979.91	8779.93	-2150.787	-2150.787	-9059.88	-4594.86	300
	4102	21	23	392	1010	8979.91	8779.93	3639.539	3639.539	12640.45	12442.47	300
5	0	22	23	392	1010	8979.91	8779.93	3639.539	3639.539	12641.45	12442.47	10000
	900.1	244	189	310	1010	8979.91	8779.93	2712.7	2712.7	11936.61	11681.63	10000
	3600	-169	-110	392	1010	8979.91	8779.93	-3639.539	-3639.539	5171.37	5030.39	10000
	3684.4	33	35	392	1010	8979.91	8779.93	3639.539	3639.539	12652.45	12454.47	10000
6	4100	22	23	392	1010	8979.91	8779.93	3639.539	3639.539	12641.45	12442.47	10000
	0	22	23	392	1010	8979.91	8779.93	3639.539	3639.539	12641.45	12442.47	2000
	1800.1	196	159	280	1010	8979.91	8779.93	2373.612	2373.612	11549.52	11312.54	2000
	5400.2	-108	-68	392	1010	8979.91	8779.93	-3639.539	-3639.539	5232.37	5072.39	2000
	5496.6	29	31	392	1010	8979.91	8779.93	3639.539	3639.539	12648.45	12450.47	2000
	5900	22	23	392	1010	8979.91	8779.93	3639.539	3639.539	12641.45	12442.47	2000
9	0	22	23	392	1010	8979.91	8779.93	3639.539	3639.539	12641.45	12442.47	10
	97.3	180	137	385.135	1010	8979.91	8779.93	3561.945	3561.945	12721.85	12478.87	10
	1884.1	63	65	265	1010	8979.91	8779.93	2204.069	2204.069	11246.98	11049.00	10
	2059.2	1161	859	226.597	1010	8979.91	8779.93	1770.003	1770.003	11910.91	11408.93	10
	3420.1	-334	-211	265	1010	8979.91	8779.93	-2204.069	-2204.069	6441.84	6364.86	10
	3490.2	97	98	265	1010	8979.91	8779.93	2204.069	2204.069	11280.98	11082.00	10
	5400.1	-126	-80	392	1010	8979.91	8779.93	-3639.539	-3639.539	5214.37	5060.39	10
	5470.6	31	32	392	1010	8979.91	8779.93	3639.539	3639.539	12650.45	12451.47	10
	5900	22	23	392	1010	8979.91	8779.93	3639.539	3639.539	12641.45	12442.47	10
	10	0	23	22	392	1010	8979.91	8779.93	3639.539	3639.539	12642.45	12441.47
77.1		2308	3188	285.461	1010	8979.91	8779.93	2435.338	2435.338	13723.25	14403.27	70
169.4		-12	-13	265	1010	8979.91	8779.93	-2204.069	-2204.069	6763.84	6562.86	70
1890		74	72	265	1010	8979.91	8779.93	2204.069	2204.069	11257.98	11056.00	70
1968.2		-1069	-1511	322.362	1010	8979.91	8779.93	-2852.427	-2852.427	5058.48	4416.50	70
2147.2		91	90	392	1010	8979.91	8779.93	3639.539	3639.539	12710.45	12509.47	70
2570		23	22	392	1010	8979.91	8779.93	3639.539	3639.539	12642.45	12441.47	70
0		-29	-27	392	1010	8979.91	8779.93	-3639.539	-3639.539	5311.37	5113.39	10
11	2.9	-20317	-13859	565	1147	10197.98	9970.871	-5594.944	-5594.944	-15713.97	-9483.07	10
	6.8	42852	29563	565	1172	10420.25	10188.2	5594.944	5594.944	58867.20	45346.14	10
	1567.4	-15216	-10526	565	1135	10091.29	9866.555	-5594.944	-5594.944	-10719.66	-6254.39	10
	2168.4	60377	41773	50	1134	10082.39	9857.862	-226.0583	-226.0583	70233.34	51404.80	10
	5409.4	-14924	-10329	565	1054	9371.114	9162.422	-5594.944	-5594.944	-11147.83	-6761.52	10
	6730.4	60377	41773	50	1133	10073.5	9849.169	-226.0583	-226.0583	70224.44	51396.11	10
	7243.2	-1965	-1434	128.917	675	6001.425	5867.775	-665.9339	-665.9339	3370.49	3767.84	10
	18215.4	52636	36417	100	1010	8979.91	8779.93	339.0875	339.0875	61955.00	45536.02	10
	20015.5	-24511	-16189	260.183	1010	8979.91	8779.93	-2149.623	-2149.623	-17680.71	-9558.69	10
	22314.5	22	23	392	937	8330.867	8145.341	3639.539	3639.539	11992.41	11807.88	10
12	0	23	22	392	1010	8979.91	8779.93	3639.539	3639.539	12642.45	12441.47	60
	10	23	22	392	1135	10091.29	9866.555	3639.539	3639.539	13753.82	13528.09	60
	30	23	22	392	940	8357.54	8171.42	3639.539	3639.539	12020.08	11832.96	60
	90	3174	4383	275	940	8357.54	8171.42	2317.098	2317.098	13848.64	14871.52	60
	2793.5	-16189	-24511	260.183	941	8366.431	8180.113	-2149.623	-2149.623	-9972.19	-18480.51	60
	5091	23	22	392	1010	8979.91	8779.93	3639.539	3639.539	12642.45	12441.47	60
13	0	23	22	392	1010	8979.91	8779.93	3639.539	3639.539	12642.45	12441.47	1
	10	23	22	392	1375	12225.13	11952.88	3639.539	3639.539	15887.66	15614.41	1
	30	23	22	392	940	8357.54	8171.42	3639.539	3639.539	12020.08	11832.96	1
	90	3174	4383	275	940	8357.54	8171.42	2317.098	2317.098	13848.64	14871.52	1
	2793.5	-16189	-24511	260.183	941	8366.431	8180.113	-2149.623	-2149.623	-9972.19	-18480.51	1
5091	23	22	392	1010	8979.91	8779.93	3639.539	3639.539	12642.45	12441.47	1	

Table 5: Safe End Stress Summary (continue)

1	2	3	4	5	6	7	8	9	10	11	12	13
Transient Number	Time (s)	Total Stress (psi)	M+B Stress (psi)	Temperature F	Pressure (psig)	Total Pressure Stress (psi)	M+B Pressure Stress (psi)	Total Piping Stress (psi)	M+B Piping Stress (psi)	Total Total Stress (psi)	Total M+B Stress (psi)	Number of Cycles (60 years)
14	0	22	23	392	1010	8979.91	8779.93	3639.539	3639.539	12641.45	12442.47	1
	60	4383	3174	275	885	7868.535	7693.305	2317.098	2317.098	14568.63	13184.40	1
	148	420	300	258.492	803	7139.473	6980.479	2130.509	2130.509	9689.98	9410.99	1
	960	544	424	100	50	444.55	434.65	339.0875	339.0875	1327.64	1197.74	1
	1460	137	139	100	50	444.55	434.65	339.0875	339.0875	920.64	912.74	1
15	0	23	22	392	1010	8979.91	8779.93	3639.539	3639.539	12642.45	12441.47	228
	10	23	22	392	1135	10091.29	9866.555	3639.539	3639.539	13753.82	13528.09	228
	30	23	22	392	940	8357.54	8171.42	3639.539	3639.539	12020.08	11832.96	228
	90	3174	4383	275	940	8357.54	8171.42	2317.098	2317.098	13848.64	14871.52	228
	2793.5	-16189	-24511	260.183	941	8366.431	8180.113	-2149.623	-2149.623	-9972.19	-18480.51	228
19	0	22	23	392	1010	8979.91	8779.93	3639.539	3639.539	12641.45	12442.47	300
	1800	219	177	265	1010	8979.91	8779.93	2204.069	2204.069	11402.98	11161.00	300
	2300	72	74	265	1010	8979.91	8779.93	2204.069	2204.069	11255.98	11058.00	300
	0	-109	-105	265	1010	8979.91	8779.93	-2204.069	-2204.069	6666.84	6470.86	300
20	4	-17288	-12189	440.106	1010	8979.91	8779.93	-4183.277	-4183.277	-12491.37	-7592.35	300
	4425	-2	-1	549	1010	8979.91	8779.93	-5414.097	-5414.097	3563.81	3363.83	300
	0	-3	-2	549	1010	8979.91	8779.93	-5414.097	-5414.097	3562.81	3363.83	300
	4	44060	30988	100	1010	8979.91	8779.93	339.0875	339.0875	53379.00	40107.02	300
20A	241	-7461	-5525	290.247	1010	8979.91	8779.93	-2489.433	-2489.433	-970.52	765.50	300
	572	128	132	549	1010	8979.91	8779.93	5414.097	5414.097	14522.01	14326.03	300
	951	-3	-2	549	1010	8979.91	8779.93	-5414.097	-5414.097	3562.81	3363.83	300
	0	-3	-2	549	1010	8979.91	8779.93	-5414.097	-5414.097	3562.81	3363.83	300
21-23	138	62	45	545.167	989	8793.199	8597.377	5370.773	5370.773	14225.97	14013.15	300
	6264	-5	-20	374.97	50	444.55	434.65	-3447.05	-3447.05	-3007.50	-3032.40	300
	6390	104	59	366.172	50	444.55	434.65	3347.607	3347.607	3896.16	3841.26	300
	15644	-173	-167	100	50	444.55	434.65	-339.0875	-339.0875	-67.54	-71.44	300
24	0	0	0	100	50	444.55	434.65	339.0875	339.0875	783.64	773.74	1
	600	0	0	100	1563	13896.63	13587.16	339.0875	339.0875	14235.72	13926.25	1
	2400	0	0	100	50	444.55	434.65	339.0875	339.0875	783.64	773.74	1
25	0	0	0	100	0	0	0	339.0875	339.0875	339.09	339.09	123
	1580	0	0	70	0	0	0	0	0	0.00	0.00	123

- NOTES: Column 1: Transient number identification.
 Column 2: Time during transient where a maxima or minima stress intensity occurs from P-V.OUT output file.
 Column 3: Maxima or minima total stress intensity from P-V.OUT output file.
 Column 4: Maxima or minima membrane plus bending stress intensity from P-V.OUT output file.
 Column 5: Temperature per total stress intensity.
 Column 6: Pressure per Table 2.
 Column 7: Total pressure stress intensity from the quantity (Column 6 x 8891)/1000 [Table 3, 1].
 Column 8: Membrane plus bending pressure stress intensity from the quantity (Column 6 x 8693)/1000 [Table3, 1].
 Column 9: Total external stress from calculation in Table 3, 5707.97 psi*(Column 5-70°F)/(575°F-70°F).
 Column 10: Same as Column 9, but for M+B stress.
 Column 11: Sum of total stresses (Columns 3, 7, and 9).
 Column 12: Sum of membrane plus bending stresses (Columns 4, 8, and 10).
 Column 13: Number of cycles for the transient (60 years).

Table 6: Fatigue Results for Blend Radius (60 Years)

LOCATION = LOCATION NO. 2 -- BLEND RADIUS
 FATIGUE CURVE = 1 (1 = CARBON/LOW ALLOY, 2 = STAINLESS STEEL)
 m = 2.0
 n = .2
 Sm = 26700. psi
 Ecurve = 3.000E+07 psi
 Eanalysis = 2.670E+07 psi
 Kt = 1.00

MAX	MIN	RANGE	MEM+BEND	Ke	Salt	Napplied	Nallowed	U
74568.	0.	74568.	62689.	1.000	41892.	1.000E+01	7.488E+03	.0013
70231.	0.	70231.	58499.	1.000	39456.	1.000E+01	8.944E+03	.0011
69395.	0.	69395.	59106.	1.000	38986.	1.000E+01	9.268E+03	.0011
67667.	0.	67667.	59377.	1.000	38015.	9.300E+01	9.988E+03	.0093
67667.	0.	67667.	59377.	1.000	38015.	1.200E+02	9.988E+03	.0120
67667.	0.	67667.	59377.	1.000	38015.	8.700E+01	9.988E+03	.0087
67282.	0.	67282.	60118.	1.000	37799.	1.000E+01	1.018E+04	.0010
67142.	0.	67142.	60462.	1.000	37720.	1.000E+01	1.025E+04	.0010
66791.	0.	66791.	62353.	1.000	37523.	1.000E+00	1.044E+04	.0001
66791.	0.	66791.	62353.	1.000	37523.	1.500E+01	1.044E+04	.0014
66791.	16.	66775.	62337.	1.000	37514.	1.230E+02	1.045E+04	.0118
66791.	1902.	64889.	60505.	1.000	36454.	9.000E+01	1.152E+04	.0078
66298.	1902.	64396.	47410.	1.000	36177.	3.000E+01	1.182E+04	.0025
66298.	1902.	64396.	47410.	1.000	36177.	1.000E+00	1.182E+04	.0001
66298.	1902.	64396.	47410.	1.000	36177.	1.000E+00	1.182E+04	.0001
66298.	30389.	35909.	21760.	1.000	20173.	1.000E+00	9.581E+04	.0000
66298.	31068.	35230.	23734.	1.000	19792.	2.670E+02	1.038E+05	.0026
64150.	31068.	33081.	34263.	1.000	18585.	3.300E+01	1.303E+05	.0003
64150.	31070.	33079.	34283.	1.000	18584.	2.700E+01	1.303E+05	.0002
59772.	31070.	28702.	31815.	1.000	16125.	1.000E+00	2.222E+05	.0000
58992.	31070.	27922.	31800.	1.000	15687.	1.000E+00	2.519E+05	.0000
55364.	31070.	24293.	25433.	1.000	13648.	2.710E+02	4.757E+05	.0006
55364.	31682.	23681.	17402.	1.000	13304.	1.000E+01	5.703E+05	.0000
55364.	32964.	22400.	17300.	1.000	12584.	1.000E+01	9.414E+05	.0000
55364.	34282.	21082.	17307.	1.000	11844.	9.000E+00	1.912E+06	.0000
55042.	34282.	20761.	18195.	1.000	11663.	7.000E+01	2.231E+06	.0000
54965.	34282.	20683.	18463.	1.000	11620.	2.210E+02	2.310E+06	.0001
54965.	34317.	20648.	18464.	1.000	11600.	1.000E+01	2.348E+06	.0000
54965.	34327.	20638.	18463.	1.000	11595.	6.900E+01	2.358E+06	.0000
53963.	34327.	19637.	17393.	1.000	11032.	2.310E+02	3.757E+06	.0001
53963.	34328.	19636.	17401.	1.000	11031.	3.000E+02	3.758E+06	.0001
53963.	34329.	19635.	17413.	1.000	11031.	3.000E+02	3.760E+06	.0001
53963.	34329.	19635.	17413.	1.000	11031.	3.000E+02	3.760E+06	.0001
53963.	34329.	19635.	17413.	1.000	11031.	3.000E+02	3.760E+06	.0001
53963.	34329.	19635.	17413.	1.000	11031.	3.000E+02	3.760E+06	.0001
53963.	41522.	12441.	10688.	1.000	6989.	1.200E+02	1.000E+20	.0000
53963.	43358.	10605.	9647.	1.000	5958.	6.000E+01	1.000E+20	.0000
53963.	43358.	10605.	9647.	1.000	5958.	1.000E+00	1.000E+20	.0000
53963.	43358.	10605.	9647.	1.000	5958.	8.800E+01	1.000E+20	.0000
51835.	43358.	8477.	7712.	1.000	4762.	1.400E+02	1.000E+20	.0000
51835.	46000.	5835.	5149.	1.000	3278.	3.000E+02	1.000E+20	.0000
51835.	46000.	5835.	5146.	1.000	3278.	9.560E+03	1.000E+20	.0000
51782.	46000.	5783.	6568.	1.000	3249.	1.000E+01	1.000E+20	.0000



50914.	46000.	4915.	6112.	1.000	2761.	1.000E+01	1.000E+20	.0000
50716.	46000.	4717.	4582.	1.000	2650.	6.000E+01	1.000E+20	.0000
50716.	46000.	4717.	4582.	1.000	2650.	2.280E+02	1.000E+20	.0000
46000.	46000.	0.	0.	1.000	0.	1.320E+02	1.000E+20	.0000
46000.	46000.	0.	0.	1.000	0.	1.000E+04	1.000E+20	.0000
46000.	46000.	0.	0.	1.000	0.	2.000E+03	1.000E+20	.0000
46000.	46000.	0.	0.	1.000	0.	2.000E+03	1.000E+20	.0000
46000.	46000.	0.	0.	1.000	0.	1.000E+01	1.000E+20	.0000
46000.	46000.	0.	0.	1.000	0.	1.000E+01	1.000E+20	.0000
46000.	46000.	0.	0.	1.000	0.	7.000E+01	1.000E+20	.0000
46000.	46000.	0.	0.	1.000	0.	7.000E+01	1.000E+20	.0000
46000.	46000.	0.	0.	1.000	0.	1.000E+01	1.000E+20	.0000
46000.	46000.	0.	0.	1.000	0.	1.000E+01	1.000E+20	.0000
46000.	46000.	0.	0.	1.000	0.	6.000E+01	1.000E+20	.0000
46000.	46000.	0.	0.	1.000	0.	6.000E+01	1.000E+20	.0000
46000.	46000.	0.	0.	1.000	0.	1.000E+00	1.000E+20	.0000
46000.	46000.	0.	0.	1.000	0.	1.000E+00	1.000E+20	.0000
46000.	46000.	0.	0.	1.000	0.	1.000E+00	1.000E+20	.0000
46000.	46000.	0.	0.	1.000	0.	2.280E+02	1.000E+20	.0000
46000.	46000.	0.	0.	1.000	0.	2.280E+02	1.000E+20	.0000

=====
TOTAL USAGE FACTOR = .0636

Table 7: Fatigue Results for Safe End (60 Years)

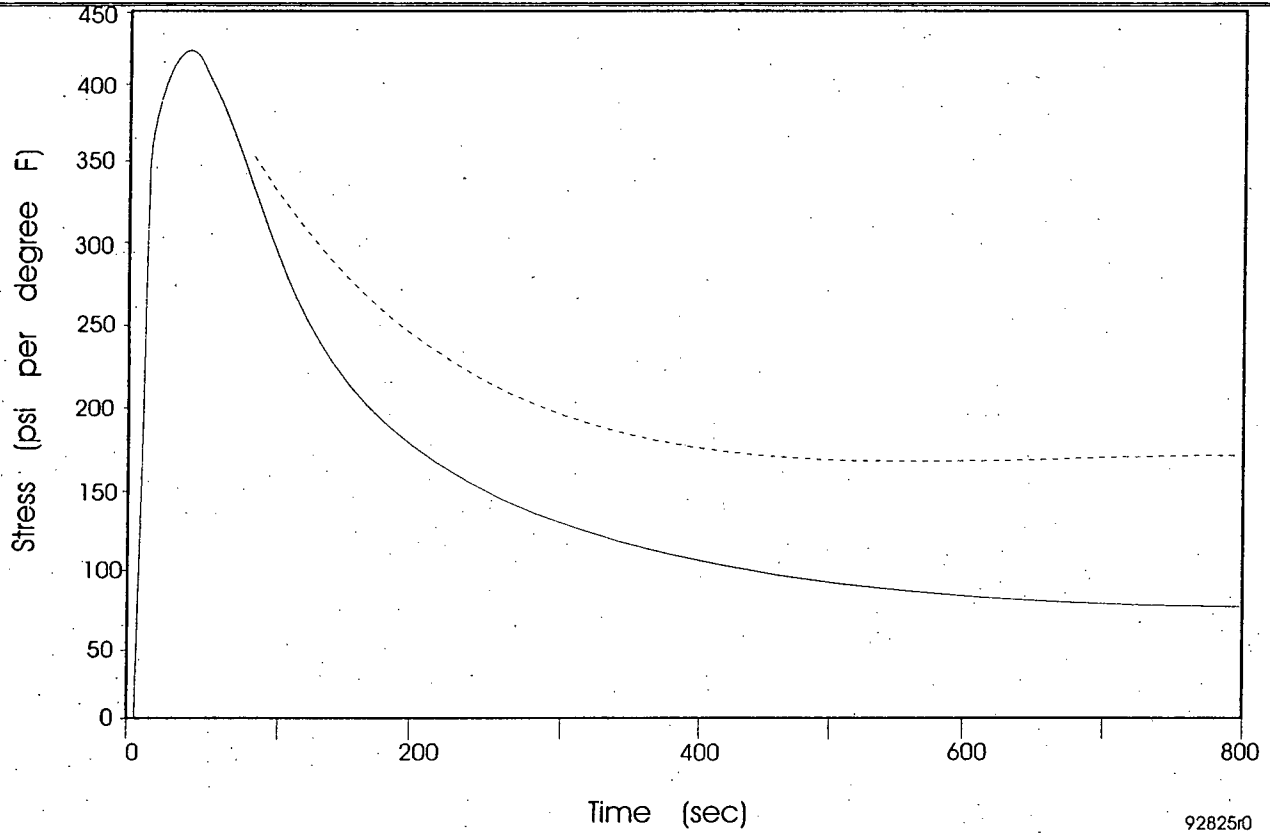
LOCATION = LOCATION NO. 1 -- SAFE END
 FATIGUE CURVE = 1 (1 = CARBON/LOW ALLOY, 2 = STAINLESS STEEL)
 m = 3.0
 n = .2
 Sm = 17800. psi
 Ecurve = 3.000E+07 psi
 Eanalysis = 2.810E+07 psi
 Kt = 1.34

MAX	MIN	RANGE	MEM+BEND	Ke	Salt	Napplied	Nallowed	U
70233.	-17681.	87914.	60963.	1.283	74422.	1.000E+01	1.338E+03	.0075
70224.	-15714.	85938.	60879.	1.280	72869.	1.000E+01	1.415E+03	.0071
61955.	-12491.	74446.	53128.	1.000	49383.	1.000E+01	4.568E+03	.0022
58867.	-12491.	71359.	52938.	1.000	47700.	1.000E+01	5.094E+03	.0020
53379.	-12491.	65870.	47699.	1.000	43819.	2.800E+02	6.552E+03	.0427
53379.	-11148.	64527.	46869.	1.000	42951.	1.000E+01	6.953E+03	.0014
53379.	-10720.	64099.	46361.	1.000	42631.	1.000E+01	7.109E+03	.0014
53379.	-9972.	63351.	58588.	1.194	53087.	6.000E+01	3.628E+03	.0165
53379.	-9972.	63351.	58588.	1.194	53087.	1.000E+00	3.628E+03	.0003
53379.	-9972.	63351.	58588.	1.194	53087.	2.280E+02	3.628E+03	.0628
53379.	-9060.	62439.	44702.	1.000	41444.	1.100E+01	7.731E+03	.0014
15888.	-9060.	24948.	20209.	1.000	16985.	1.000E+00	1.802E+05	.0000
14569.	-9060.	23629.	17779.	1.000	15840.	1.000E+00	2.410E+05	.0000
14522.	-9060.	23582.	18921.	1.000	16022.	2.870E+02	2.287E+05	.0013
14522.	-3008.	17530.	17358.	1.000	12508.	1.300E+01	9.944E+05	.0000
14396.	-3008.	17404.	17229.	1.000	12417.	2.870E+02	1.083E+06	.0003
14396.	-971.	15367.	13432.	1.000	10641.	1.300E+01	5.165E+06	.0000
14236.	-971.	15206.	13161.	1.000	10506.	1.000E+00	5.563E+06	.0000
14226.	-971.	15196.	13248.	1.000	10516.	2.860E+02	5.531E+06	.0001
14226.	-178.	14404.	14178.	1.000	10262.	1.400E+01	6.379E+06	.0000
13849.	-178.	14026.	15036.	1.000	10216.	6.000E+01	6.547E+06	.0000
13849.	-178.	14026.	15036.	1.000	10216.	1.000E+00	6.547E+06	.0000
13849.	-178.	14026.	15036.	1.000	10216.	2.250E+02	6.547E+06	.0000
13849.	-68.	13916.	14943.	1.000	10141.	3.000E+00	6.837E+06	.0000
13754.	-68.	13821.	13600.	1.000	9846.	6.000E+01	8.117E+06	.0000
13754.	-68.	13821.	13600.	1.000	9846.	2.280E+02	8.117E+06	.0000
13723.	-68.	13791.	14475.	1.000	9989.	9.000E+00	7.465E+06	.0000
13723.	-65.	13788.	14473.	1.000	9987.	6.100E+01	7.474E+06	.0000
12722.	-65.	12786.	12548.	1.000	9103.	1.000E+01	1.729E+07	.0000
12710.	-65.	12775.	12579.	1.000	9102.	7.000E+01	1.730E+07	.0000
12652.	-65.	12717.	12524.	1.000	9061.	1.590E+02	1.833E+07	.0000
12652.	0.	12652.	12454.	1.000	9014.	1.230E+02	1.959E+07	.0000
12652.	0.	12652.	12454.	1.000	9014.	1.200E+02	1.959E+07	.0000
12652.	0.	12652.	12454.	1.000	9014.	1.230E+02	1.959E+07	.0000
12652.	339.	12313.	12115.	1.000	8772.	1.230E+02	2.905E+07	.0000
12652.	784.	11869.	11681.	1.000	8456.	1.200E+02	4.952E+07	.0000
12652.	784.	11869.	11681.	1.000	8456.	1.000E+00	4.952E+07	.0000
12652.	784.	11869.	11681.	1.000	8456.	1.000E+00	4.952E+07	.0000
12652.	784.	11869.	11681.	1.000	8456.	1.000E+00	4.952E+07	.0000
12652.	921.	11732.	11542.	1.000	8357.	1.000E+00	5.462E+07	.0000
12652.	1328.	11325.	11257.	1.000	8088.	1.000E+00	7.100E+07	.0000
12652.	3370.	9282.	8687.	1.000	6531.	1.000E+01	1.000E+20	.0000
12652.	3563.	9090.	9091.	1.000	6502.	3.000E+02	1.000E+20	.0000
12652.	3563.	9090.	9091.	1.000	6502.	3.000E+02	1.000E+20	.0000

12652.	3563.	9090.	9091.	1.000	6502.	3.000E+02	1.000E+20	.0000
12652.	3563.	9090.	9091.	1.000	6502.	3.000E+02	1.000E+20	.0000
12652.	3564.	9089.	9090.	1.000	6501.	3.000E+02	1.000E+20	.0000
12652.	3565.	9088.	-1740.	1.000	4535.	3.000E+02	1.000E+20	.0000
12652.	3896.	8756.	8613.	1.000	6237.	3.000E+02	1.000E+20	.0000
12652.	5058.	7594.	8038.	1.000	5513.	7.000E+01	1.000E+20	.0000
12652.	5171.	7481.	7424.	1.000	5341.	7.048E+03	1.000E+20	.0000
12650.	5171.	7479.	7421.	1.000	5339.	1.000E+01	1.000E+20	.0000
12648.	5171.	7477.	7420.	1.000	5338.	2.000E+03	1.000E+20	.0000
12642.	5171.	7471.	7411.	1.000	5333.	7.000E+01	1.000E+20	.0000
12642.	5171.	7471.	7411.	1.000	5333.	7.000E+01	1.000E+20	.0000
12642.	5171.	7471.	7411.	1.000	5333.	6.000E+01	1.000E+20	.0000
12642.	5171.	7471.	7411.	1.000	5333.	6.000E+01	1.000E+20	.0000
12642.	5171.	7471.	7411.	1.000	5333.	1.000E+00	1.000E+20	.0000
12642.	5171.	7471.	7411.	1.000	5333.	1.000E+00	1.000E+20	.0000
12642.	5171.	7471.	7411.	1.000	5333.	2.280E+02	1.000E+20	.0000
12642.	5171.	7471.	7411.	1.000	5333.	2.280E+02	1.000E+20	.0000
12641.	5171.	7470.	7412.	1.000	5333.	2.240E+02	1.000E+20	.0000
12641.	5214.	7427.	7382.	1.000	5304.	1.000E+01	1.000E+20	.0000
12641.	5232.	7409.	7370.	1.000	5293.	2.000E+03	1.000E+20	.0000
12641.	5311.	7330.	7329.	1.000	5243.	1.000E+01	1.000E+20	.0000
12641.	6442.	6200.	6078.	1.000	4412.	1.000E+01	1.000E+20	.0000
12641.	6667.	5975.	5972.	1.000	4273.	3.000E+02	1.000E+20	.0000
12641.	6764.	5878.	5880.	1.000	4205.	7.000E+01	1.000E+20	.0000
12641.	9690.	2951.	3031.	1.000	2126.	1.000E+00	1.000E+20	.0000
12641.	10119.	2522.	2541.	1.000	1808.	1.200E+02	1.000E+20	.0000
12641.	11247.	1394.	1393.	1.000	997.	1.000E+01	1.000E+20	.0000
12641.	11256.	1385.	1384.	1.000	991.	3.000E+02	1.000E+20	.0000
12641.	11258.	1383.	1386.	1.000	990.	7.000E+01	1.000E+20	.0000
12641.	11281.	1360.	1360.	1.000	973.	1.000E+01	1.000E+20	.0000
12641.	11403.	1238.	1281.	1.000	894.	3.000E+02	1.000E+20	.0000
12641.	11550.	1092.	1130.	1.000	788.	2.000E+03	1.000E+20	.0000
12641.	11911.	731.	1034.	1.000	578.	1.000E+01	1.000E+20	.0000
12641.	11937.	705.	761.	1.000	514.	4.555E+03	1.000E+20	.0000
12641.	11937.	705.	761.	1.000	514.	5.445E+03	1.000E+20	.0000
12641.	11992.	649.	635.	1.000	462.	1.000E+01	1.000E+20	.0000
12641.	12020.	621.	610.	1.000	442.	6.000E+01	1.000E+20	.0000
12641.	12020.	621.	610.	1.000	442.	1.000E+00	1.000E+20	.0000
12641.	12020.	621.	610.	1.000	442.	2.280E+02	1.000E+20	.0000
12641.	12640.	1.	0.	1.000	1.	3.000E+02	1.000E+20	.0000
12641.	12641.	0.	0.	1.000	0.	3.956E+03	1.000E+20	.0000
12641.	12641.	0.	0.	1.000	0.	2.000E+03	1.000E+20	.0000
12641.	12641.	0.	0.	1.000	0.	2.000E+03	1.000E+20	.0000
12641.	12641.	0.	0.	1.000	0.	1.000E+01	1.000E+20	.0000
12641.	12641.	0.	0.	1.000	0.	1.000E+01	1.000E+20	.0000
12641.	12641.	0.	0.	1.000	0.	1.000E+00	1.000E+20	.0000

=====

TOTAL USAGE FACTOR = .1471



Note: A typical set of two Green's Functions is shown, each for a different set of heat transfer coefficients (representing different flow rate conditions).

Figure 1: Typical Green's Functions for Thermal Transient Stress

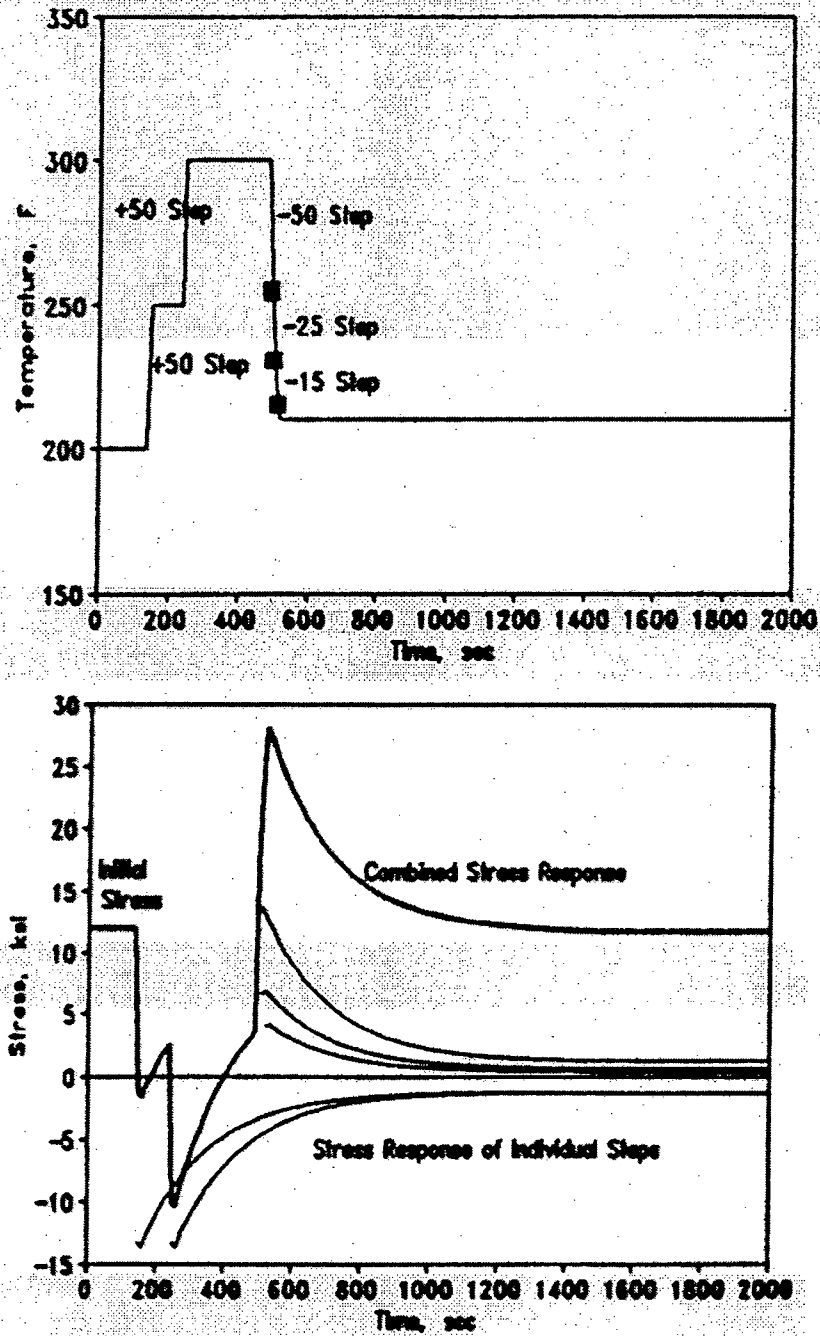


Figure 2: Typical Stress Response Using Green's Functions

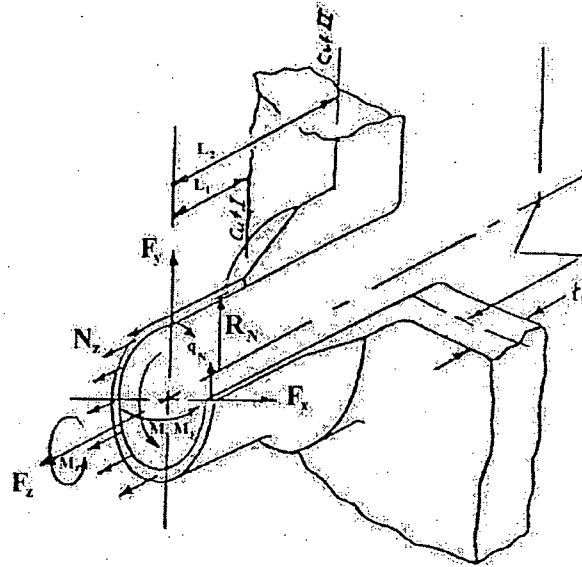


Figure 3: External Forces and Moments on the Feedwater Nozzle

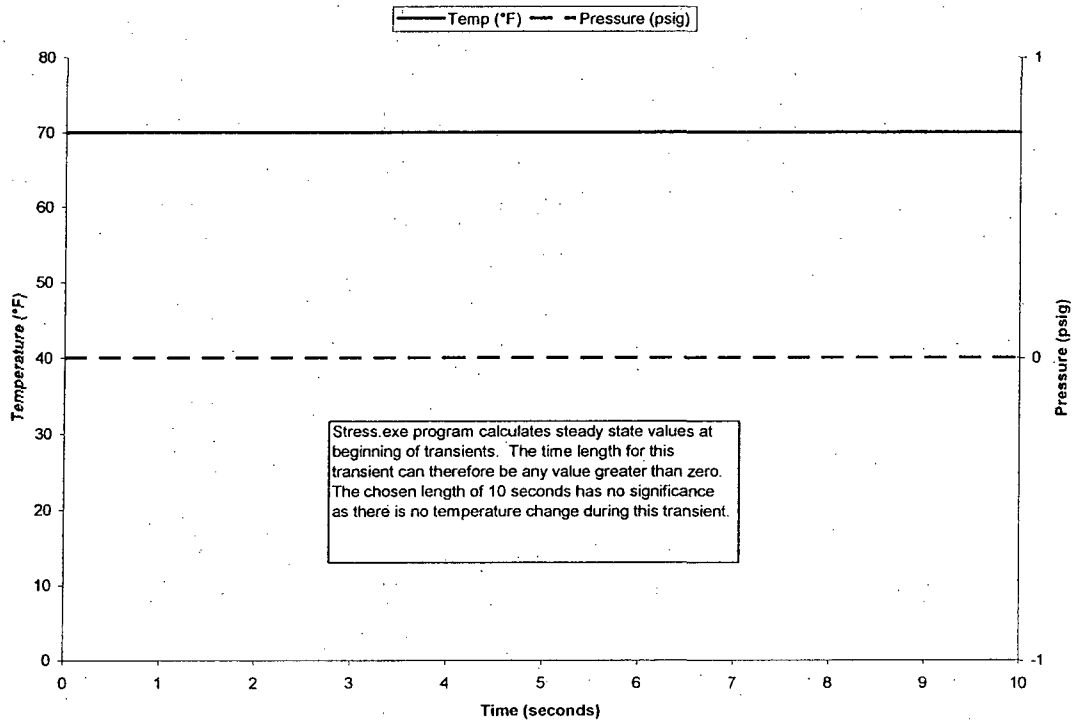


Figure 4: Transient 1, Bolt-up

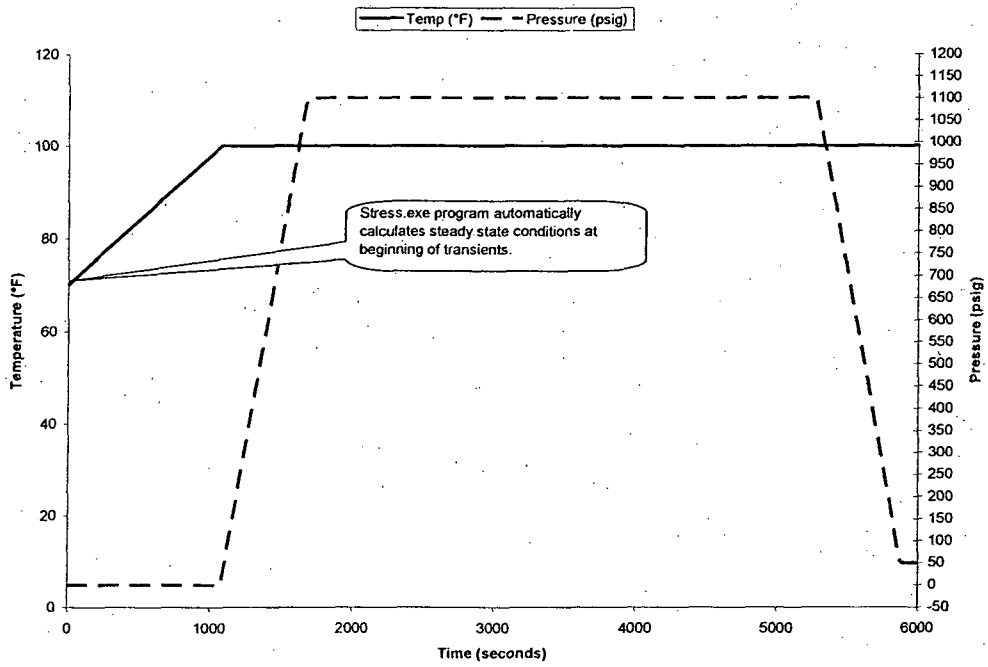


Figure 5: Transient 2, Design HYD Test

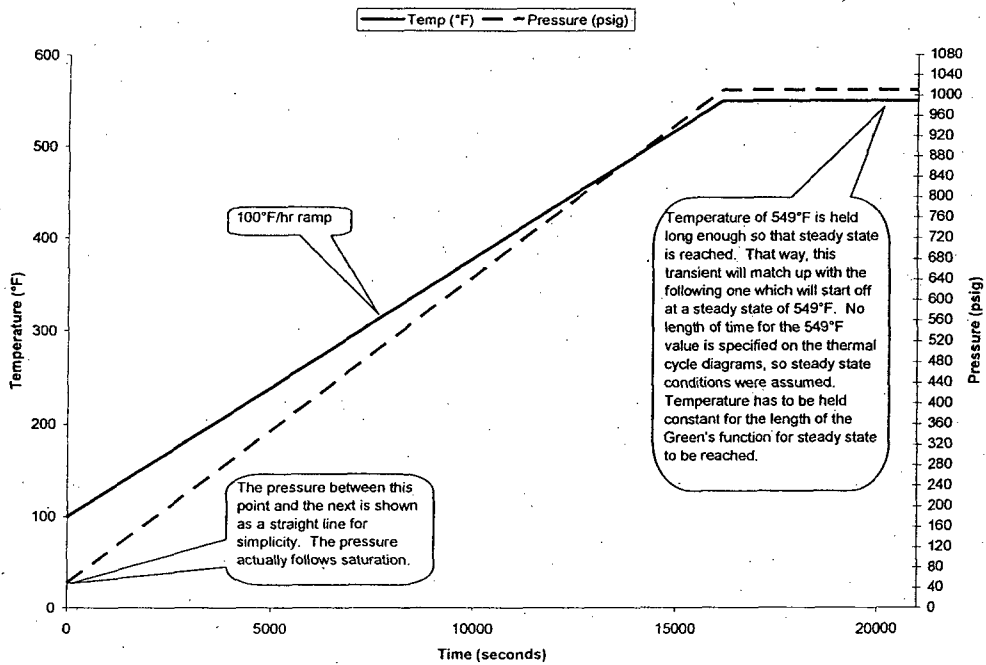


Figure 6: Transient 3, Startup

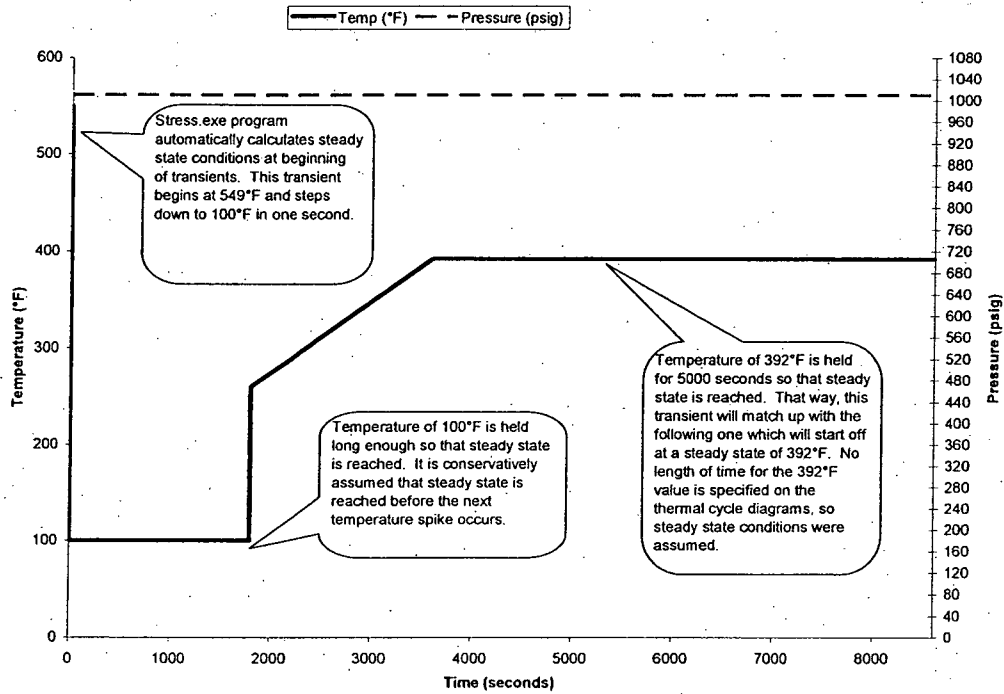


Figure 7: Transient 4, Turbine Roll and Increased to Rated Power

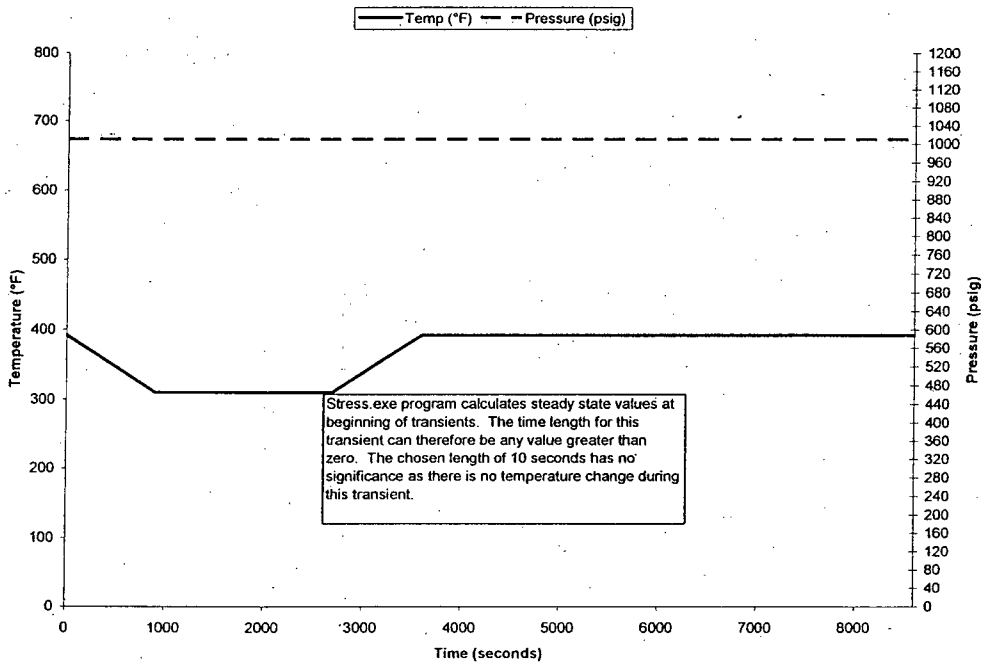


Figure 8: Transient 5, Daily Reduction 75% Power

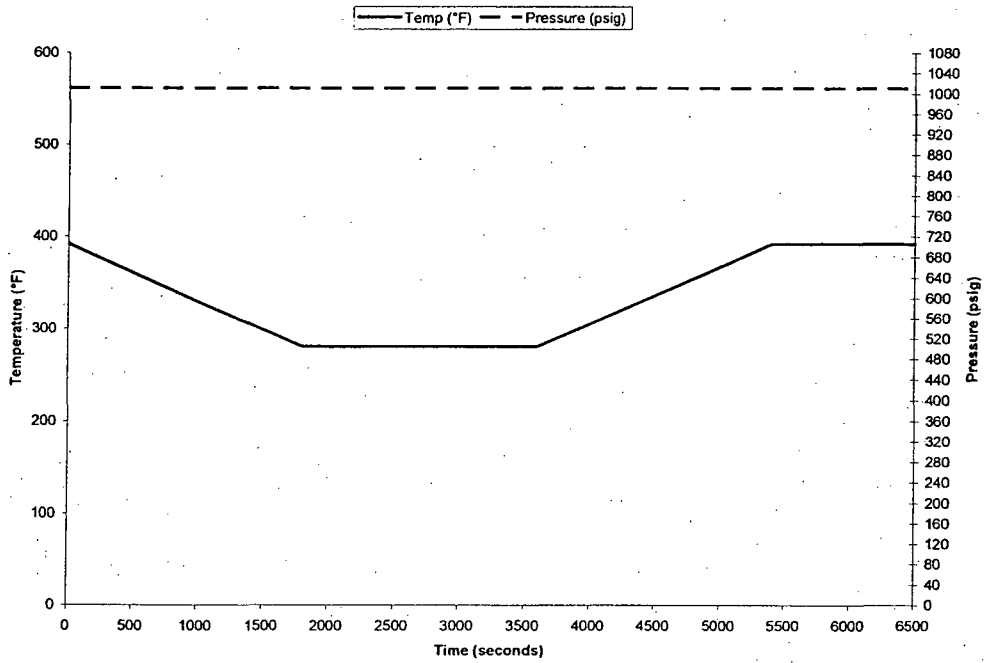


Figure 9: Transient 6, Weekly Reduction 50% Power

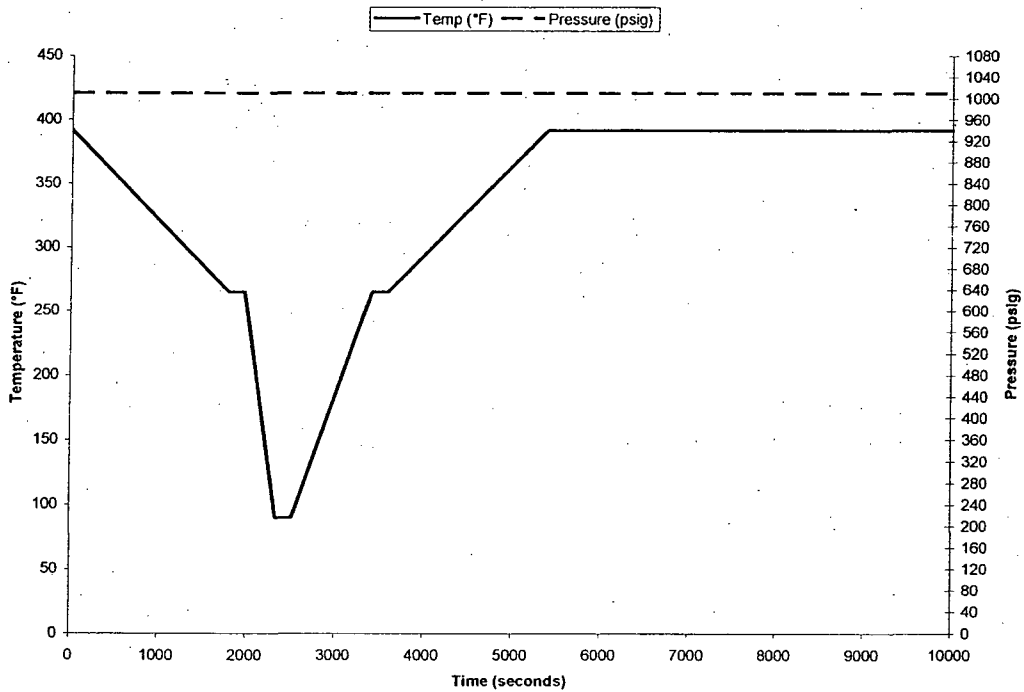


Figure 10: Transient 9, Turbine Trip at 25% Power

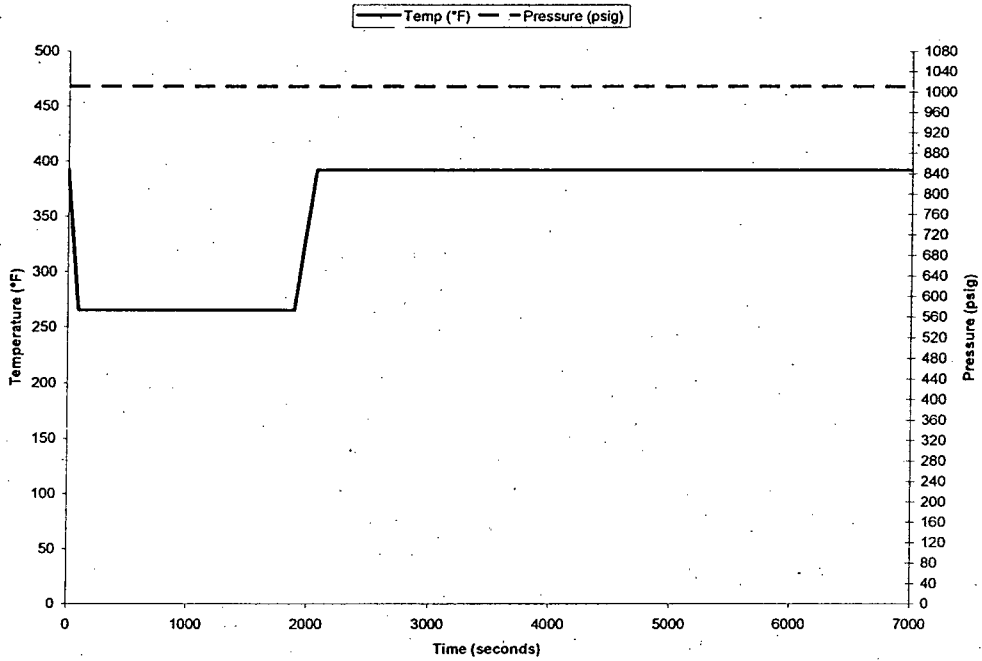


Figure 11: Transient 10, Feedwater Bypass

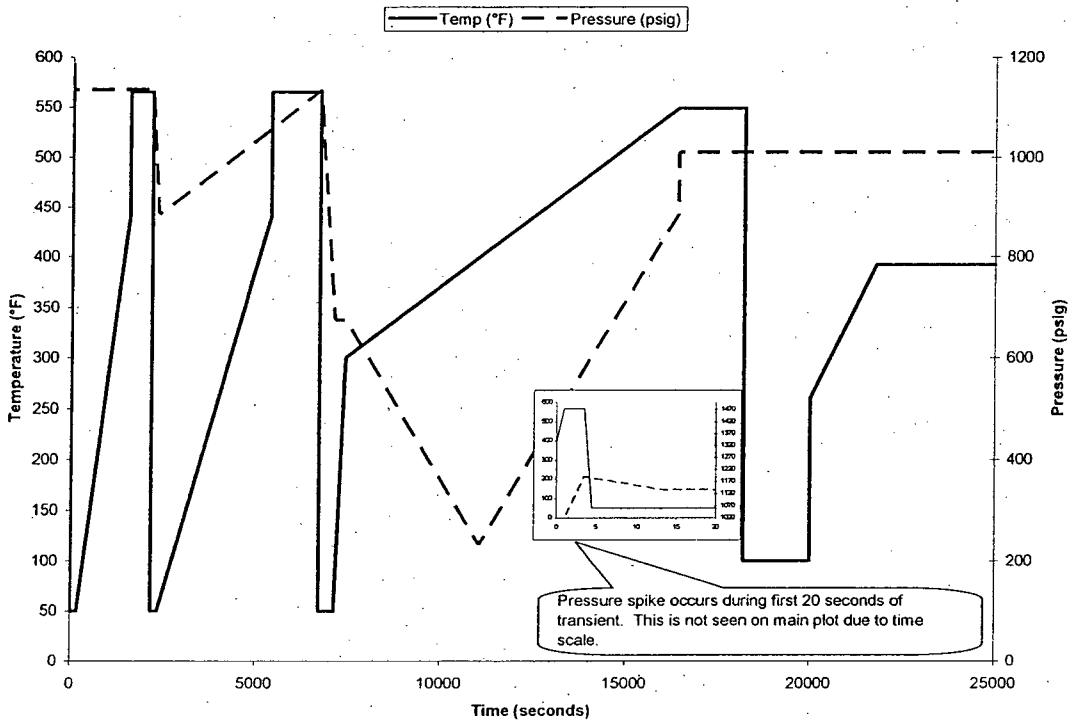


Figure 12: Transient 11, Loss of Feedwater Pumps

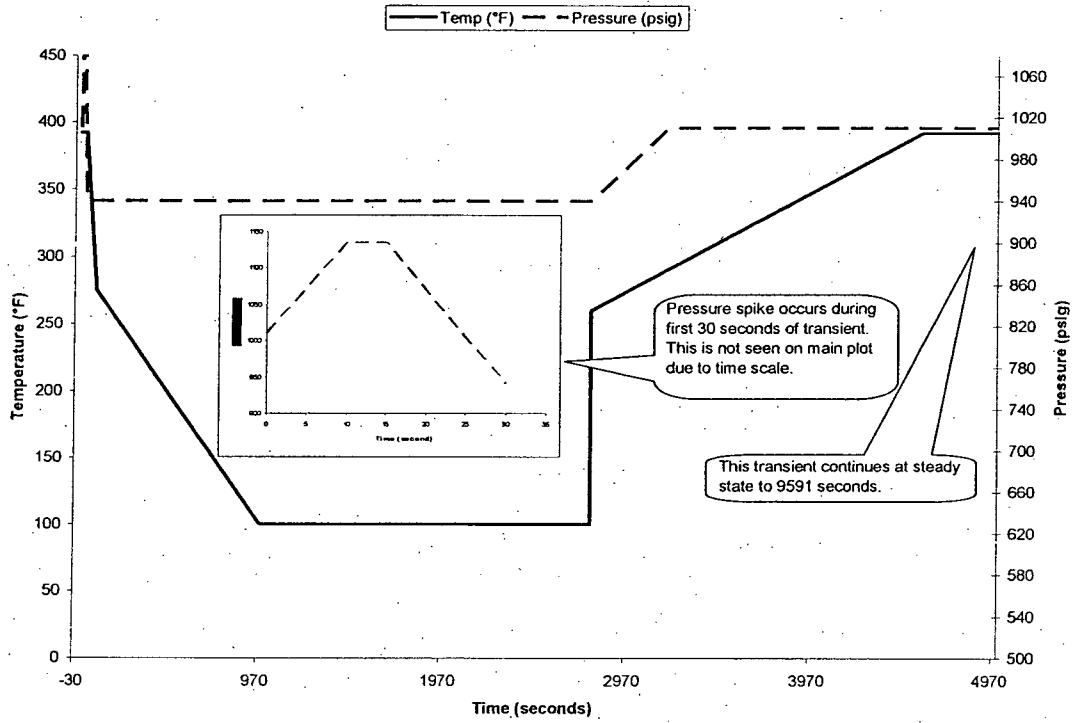


Figure 13: Transient 12, Turbine Generator Trip

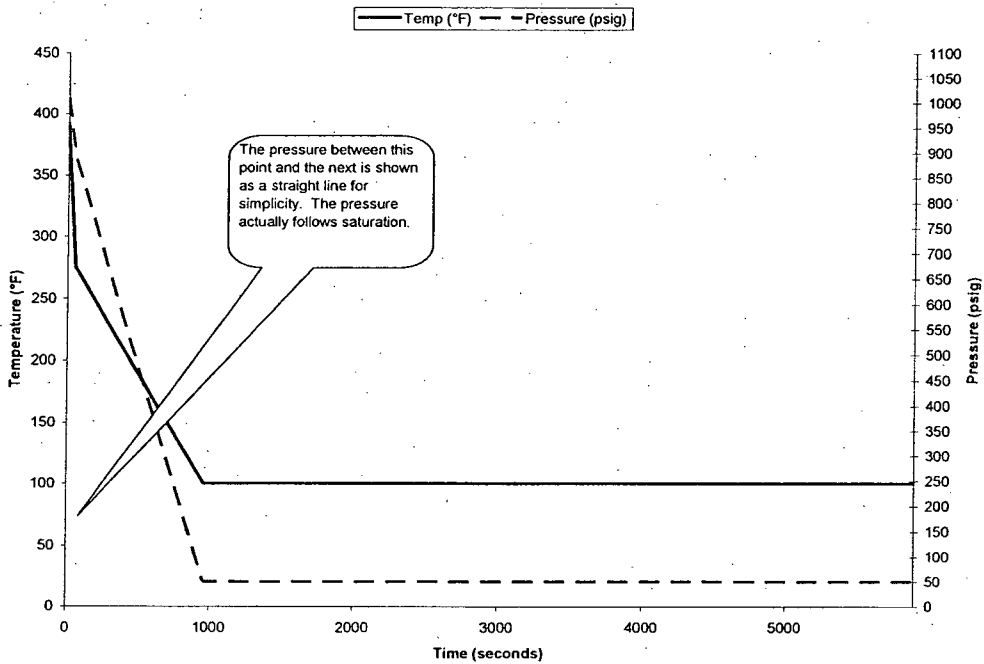


Figure 14: Transient 14, SRV Blowdown

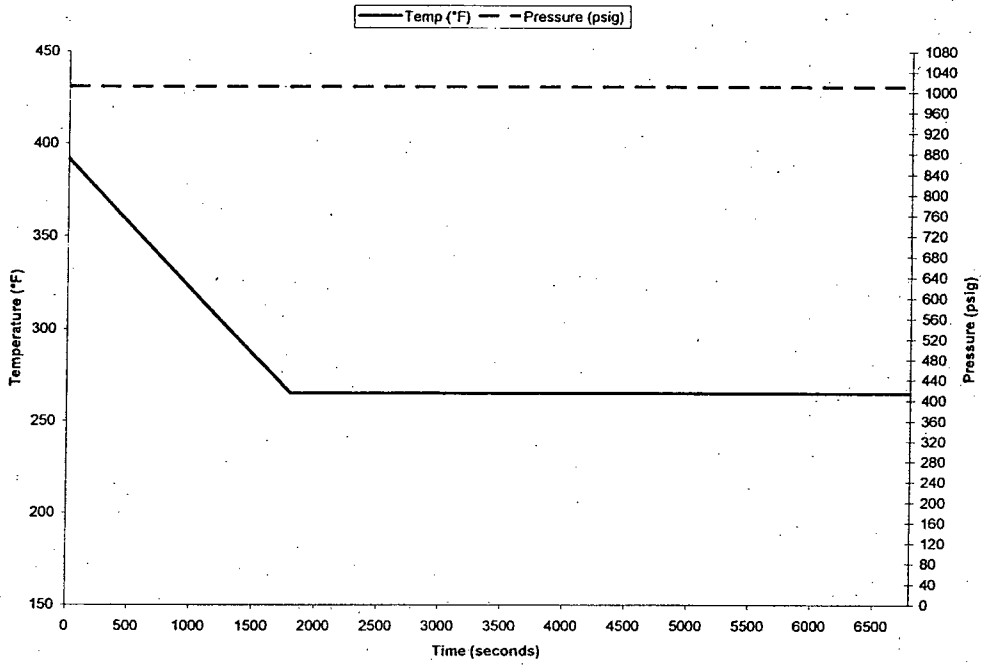


Figure 15: Transient 19, Reduction to 0% Power

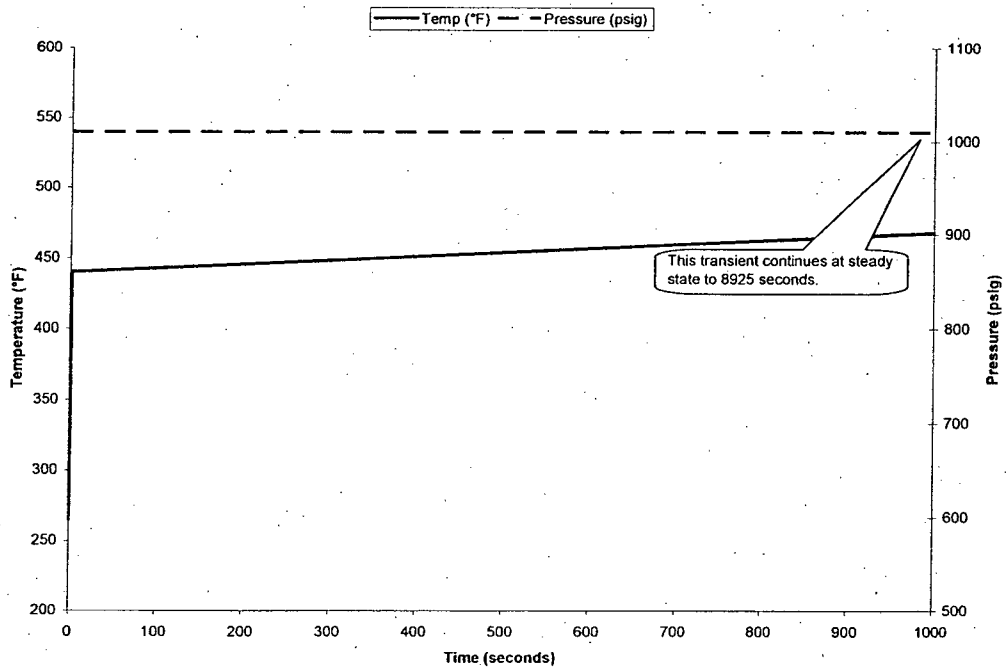


Figure 16: Transient 20, Hot Standby (Heatup Portion)

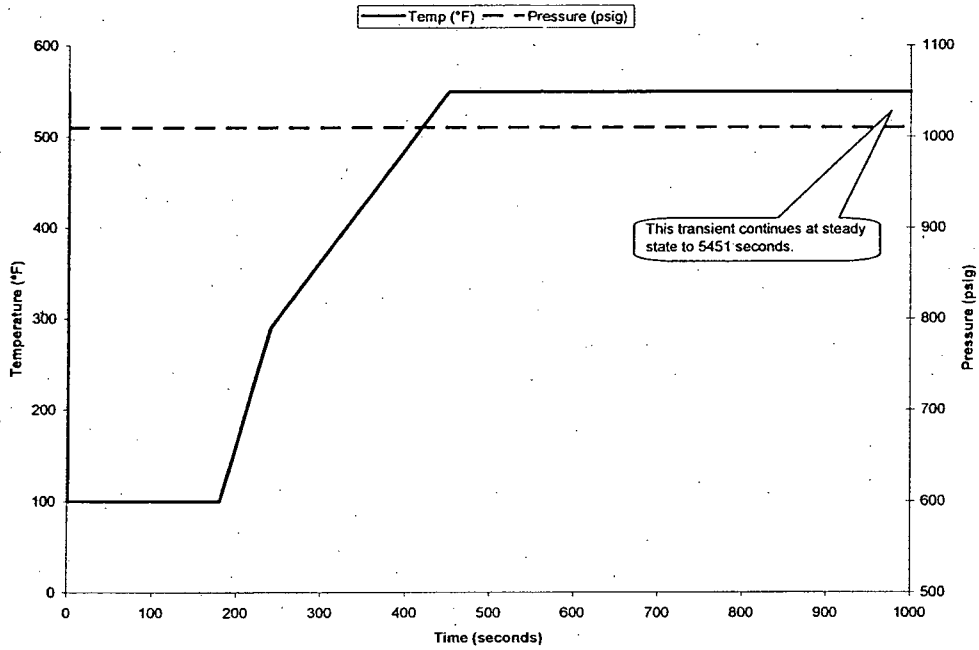


Figure 17: Transient 20A, Hot Standby (Feedwater Injection Portion)

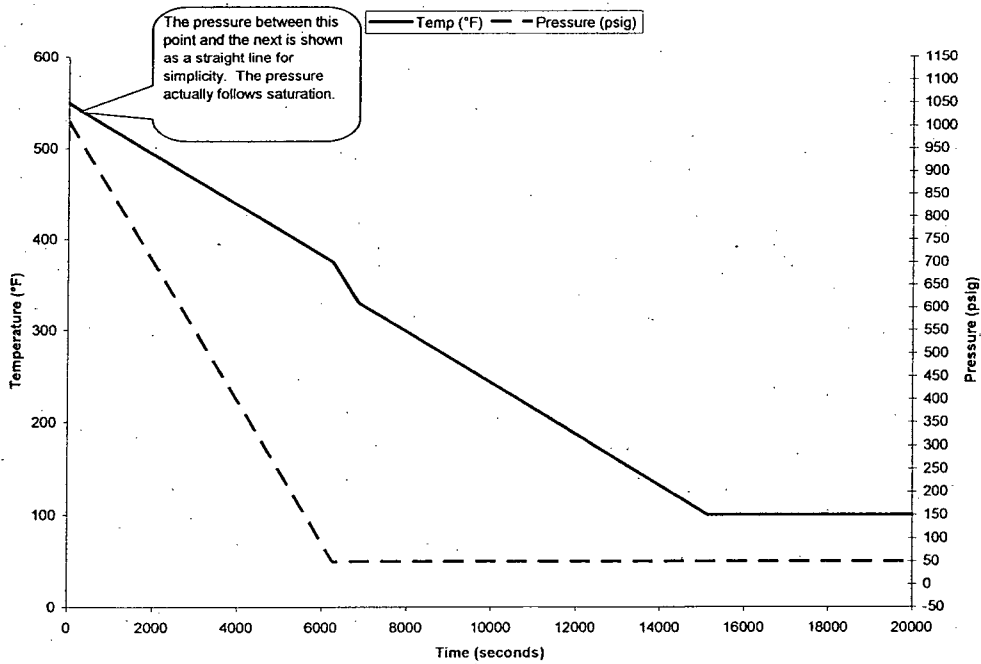


Figure 18: Transient 21-23, Shutdown

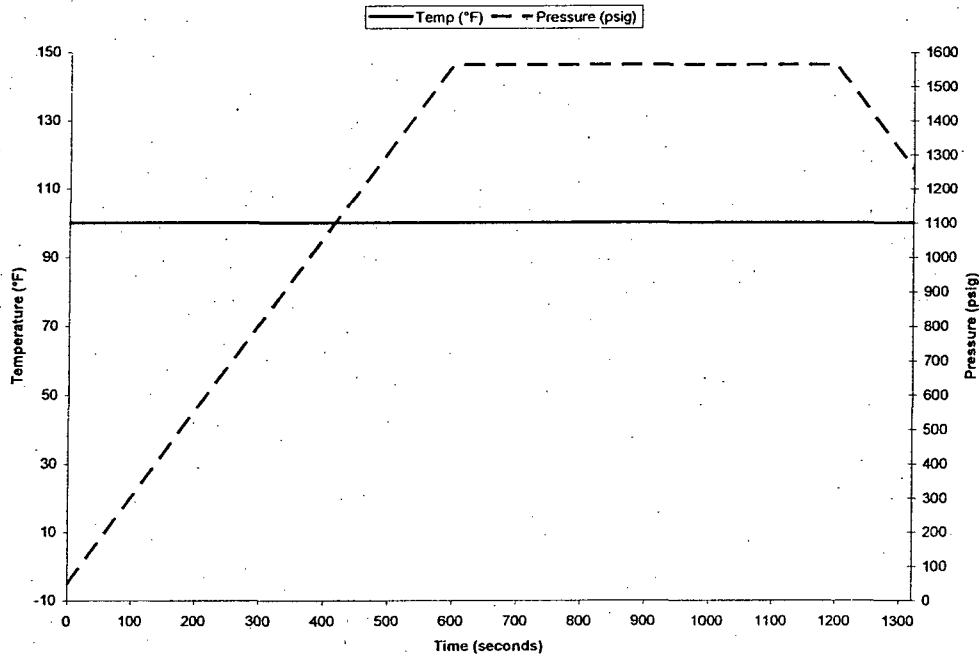


Figure 19: Transient 24, Hydrostatic Test

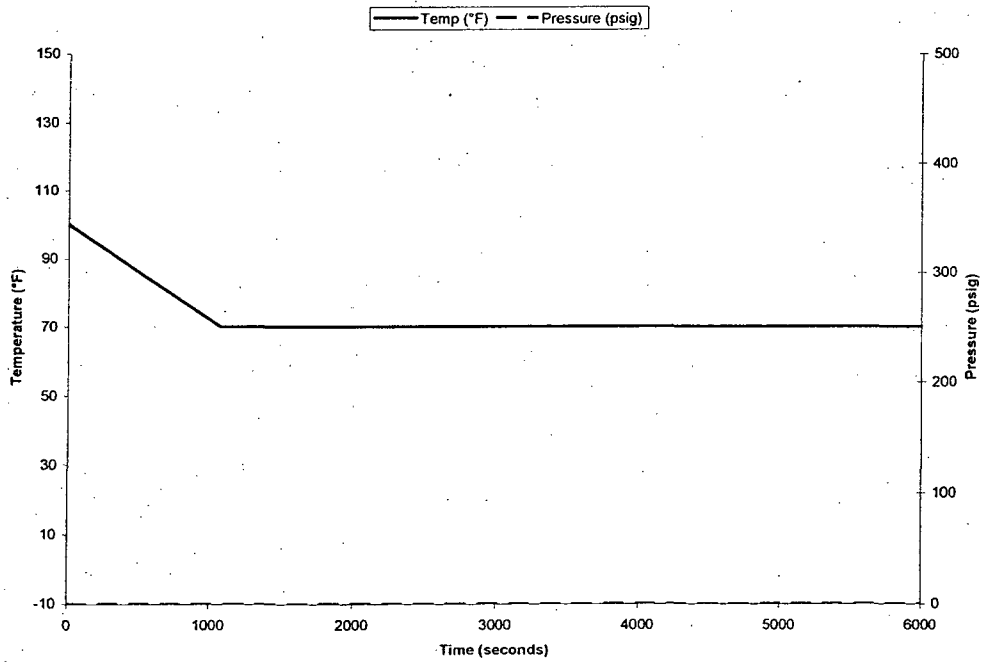


Figure 20: Transient 25, Unbolt



APPENDIX A
SUMMARY OF OUTPUT FILES



Transient Table.xls	Definition of Transients	In Computer files
BRresults.xls	Blend Radius Stress Summary	In Computer files
SEresults.xls	Safe End Stress Summary	In Computer files
TRANSNT XX.INP	Input File for Each Transient	In Computer files
Green.dat	Input File for Green Functions	In Computer files
P-V XX.OUT	Output File for Stress Analysis	In Computer files
GREEN.CFG	Input File for Defining Green Function	In Computer files
FATIGUE.CFG	Input File for Defining Fatigue Analysis	In Computer files
FATIGUE.DAT	Input File for Fatigue Curves	In Computer files
FATIGUE.inp	Input file for Fatigue Analysis from BRresults.xls or SEresults.xls	In Computer files
FATIGUE.OUT	Fatigue Output File	In Computer files

Where XX is defined for each transient.



Structural Integrity Associates, Inc.

CALCULATION PACKAGE

File No.: VY-16Q-303

Project No.: VY-16Q **NEC-JH_06**

PROJECT NAME:

Environmental Fatigue Analysis of VYNPS

CONTRACT NO.:

10150394

CLIENT:

Entergy Nuclear Operations, Inc.

PLANT:

Vermont Yankee

CALCULATION TITLE:

Environmental Fatigue Evaluation of Reactor Recirculation Inlet Nozzle and Vessel Shell/Bottom Head

Document Revision	Affected Pages	Revision Description	Project Manager Approval Signature & Date	Preparer(s) & Checker(s) Signatures & Date
0	1 - 24, Appendices: A1 - A2, B1 - B2 In computer files	Initial issue.	Terry J. Herrmann 07/05/07 	Gary L. Stevens 07/05/07 Terry J. Herrmann 07/05/07

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1.0 INTRODUCTION/STATEMENT OF PROBLEM/ OBJECTIVE

The purpose of this calculation is to perform a plant-specific evaluation of reactor water environmental effects for the reactor recirculation (RR) inlet nozzle and the reactor pressure vessel (RPV) shell/bottom head locations identified within NUREG/CR-6260 [1] for the older vintage General Electric (GE) plant for the Vermont Yankee Nuclear Power Plant (VY).

The water chemistry input used in this calculation covers several portions of the RPV, as well as the feedwater and recirculation lines. Although these regions encompass more areas than needed to address the two components of interest in this calculation, environmental fatigue multipliers are developed for all of these regions in this calculation for potential use in other evaluations associated with this project.

2.0 TECHNICAL APPROACH OR METHODOLOGY

Per Chapter X, "Time-Limited Aging Analyses Evaluation of Aging Management Programs Under 10 CFR 54.21(c)(1)(iii)," Section X.M1, "Metal Fatigue of Reactor Coolant Pressure Boundary," of the Generic Aging Lessons Learned (GALL) Report [2], detailed, vintage-specific, fatigue calculations are required for plants applying for license renewal for the locations identified for the appropriate vintage plant in NUREG/CR-6260.

In this calculation, detailed environmentally assisted fatigue (EAF) calculations are performed for VY for two of the locations associated with the older vintage GE plant in NUREG/CR-6260. The older-vintage GE plant is the appropriate comparison to VY since the original piping design at VY was in accordance with USAS B31.1 [3], as well as the fact that the older-vintage boiling water reactor (BWR) in NUREG/CR-6260 was a BWR-4 plant, which is the same as VY.

Entergy performed an initial assessment of EAF effects for VY in their License Renewal Application (LRA) that was submitted to the NRC in January 2006. Table 4.3-3 of the VY LRA provides the results of those evaluations. All but two of the VY locations evaluated for EAF in the LRA did not yield acceptable results for 60 years of operation. Further refined analyses are currently underway in other calculations associated with this project to address those components. This calculation documents the EAF evaluation for the RR inlet nozzle and RPV shell/bottom head locations, where it is expected that acceptable EAF results can be achieved based on the existing analyses without the need for additional refined evaluations.

3.0 ASSUMPTIONS / DESIGN INPUTS

Per Section X.M1 of the GALL Report [2], the EAF evaluation must use the appropriate F_{en} relationships from NUREG/CR-6583 [4] (for carbon/low alloy steels) and NUREG/CR-5704 [5] (for stainless steels), as appropriate for the material for each location. These expressions are:

For Carbon Steel [4, p. 69]:
$$F_{en} = \exp (0.585 - 0.00124T^* - 0.101S^*T^*O^* \dot{\epsilon}^*)$$

Substituting $T^* = 25^\circ\text{C}$ in the above expression, as required by NUREG/CR-6583 to relate room temperature air data to service temperature data in water [6], the following is obtained:

$$\begin{aligned} F_{en} &= \exp (0.585 - 0.00124(25^\circ\text{C}) - 0.101 S^* T^* O^* \dot{\epsilon}^*) \\ &= \exp (0.554 - 0.101 S^* T^* O^* \dot{\epsilon}^*) \end{aligned}$$

For Low Alloy Steel [4, p. 69]:
$$F_{en} = \exp (0.929 - 0.00124T^* - 0.101S^*T^*O^* \dot{\epsilon}^*)$$

Substituting $T^* = 25^\circ\text{C}$ in the above expression, as required by NUREG/CR-6583 to relate room temperature air data to service temperature data in water [6], the following is obtained:

$$\begin{aligned} F_{en} &= \exp (0.929 - 0.00124(25^\circ\text{C}) - 0.101 S^* T^* O^* \dot{\epsilon}^*) \\ &= \exp (0.898 - 0.101 S^* T^* O^* \dot{\epsilon}^*) \end{aligned}$$

where [4, pp. 60 and 65]:

- F_{en} = fatigue life correction factor
- S^* = S for $0 < \text{sulfur content}, S \leq 0.015 \text{ wt. } \%$
= 0.015 for $S > 0.015 \text{ wt. } \%$
- T^* = 0 for $T < 150^\circ\text{C}$
= $(T - 150)$ for $150 \leq T \leq 350^\circ\text{C}$
- T = fluid service temperature ($^\circ\text{C}$)
- O^* = 0 for dissolved oxygen, $\text{DO} < 0.05 \text{ parts per million (ppm)}$
= $\ln(\text{DO}/0.04)$ for $0.05 \text{ ppm} \leq \text{DO} \leq 0.5 \text{ ppm}$
= $\ln(12.5)$ for $\text{DO} > 0.5 \text{ ppm}$
- $\dot{\epsilon}^*$ = 0 for strain rate, $\dot{\epsilon} > 1\%/ \text{sec}$
= $\ln(\dot{\epsilon}^*)$ for $0.001 \leq \dot{\epsilon} \leq 1\%/ \text{sec}$
= $\ln(0.001)$ for $\dot{\epsilon} < 0.001\%/ \text{sec}$

For Types 304 and 316 Stainless Steel [5, p. 31]: $F_{en} = \exp(0.935 - T^* \dot{\epsilon}^* O^*)$

where [5, pp. 25 and 31]:

- F_{en} = fatigue life correction factor
- T^* = 0 for $T < 200^\circ\text{C}$
= 1 for $T \geq 200^\circ\text{C}$
- T = fluid service temperature ($^\circ\text{C}$)
- $\dot{\epsilon}^*$ = 0 for strain rate, $\dot{\epsilon} > 0.4\%/sec$
= $\ln(\dot{\epsilon}/0.4)$ for $0.0004 \leq \dot{\epsilon} \leq 0.4\%/sec$
= $\ln(0.0004/0.4)$ for $\dot{\epsilon} < 0.0004\%/sec$
- O^* = 0.260 for dissolved oxygen, DO < 0.05 parts per million (ppm)
= 0.172 for DO ≥ 0.05 ppm

Bounding F_{en} values are determined or, where necessary, computed for each load pair in the detailed fatigue calculation for each component. The environmental fatigue is then determined as $U_{env} = (U) (F_{en})$, where U is the original fatigue usage and U_{env} is the environmentally assisted fatigue (EAF) usage factor. All calculations can be found in Excel spreadsheet "VY-16Q-303 (Env. Fat. Calcs).xls" associated with this calculation.

From Reference [7], for the BWR, typical DO levels range from just over 200 ppb for normal water chemistry (NWC) conditions to less than 10 ppb for hydrogen water chemistry (HWC) conditions. Typical HWC system availabilities are greater than 90%. Based on VY-specific water chemistry input for Entergy [8], which is also contained in Appendix A of this calculation, the input shown in Table 1 is defined for use in this calculation.

The water chemistry input covers several portions of the RPV, as well as the feedwater and recirculation lines. Although these regions encompass more areas than needed to address the two components of interest in this calculation, environmental fatigue multipliers are developed for all of these regions in this calculation for potential use in other evaluations associated with this project.

Therefore, based on Table 1 and for the purposes of this calculation, the following is assumed:

- Over the 60-year operating life of the plant, HWC conditions exist for 47% of the time, and NWC conditions exist for 53% of the time.
- All operation through 11/1/2003 was assumed as NWC using the dissolved oxygen values from the "Pre-NMCA" column in Appendix A, and all operation after 11/1/2003 was assumed as HWC using the maximum oxygen values from the "Post-NMCA + HWC (OLP)", "Post-NMCA + HWC (EPU)", and "Future Operation" columns in Appendix A.
- Recirculation line DO is 122 ppb pre-HWC and 48 ppb post-HWC.
- Feedwater line DO is 40 ppb for pre-HWC and 40 ppb for post-HWC conditions.
- RPV Upper Region DO is 114 ppb pre-HWC and 97 ppb post-HWC.
- RPV Beltline DO is 123 ppb pre-HWC and 46 ppb post-HWC.
- RPV Bottom Head Region DO is 128 ppb pre-HWC and 69 ppb post-HWC.



Based on the above typical DO levels, bounding F_{en} multipliers for each of the three applicable materials (carbon, low alloy, and stainless steels) are shown in Tables 2 through 6 for the various RPV and piping regions.

The projected number of cycles used in this calculation is based on the number of cycles actually experienced by the plant in the past and forward-projected with some additional margin for 60 years of operation, as documented in Reference [9]. In addition, the latest governing stress analysis for each location was utilized, and any relevant effects of Extended Power Uprate (EPU) operation were incorporated as necessary. With these assumptions, the cumulative usage factor (CUF) values documented in this calculation are considered applicable for sixty years of operation including all relevant EAF and EPU effects.

4.0 CALCULATIONS

The analyses for the NUREG/CR-6260 locations identified in Section 2.0 are provided in this section. As previously noted, the fatigue calculations for 60 years for all locations make use of the 60-year projected cycles for VY from Reference [9], and incorporate EPU effects.

Since the F_{en} methodology documented in References [4] and [5] is relatively "new" technology, it is intended to apply to "modern-day" fatigue analyses, i.e., applied to fatigue analyses that use current ASME Code fatigue curves, etc. Therefore, to be consistent with this approach, the evaluation for the all locations will also utilize modern-day fatigue calculation methodology using the 1998 Edition, 2000 Addenda of the ASME Code [11]. This involves applying a Young's Modulus correction factor (i.e., $E_{\text{fatigue curve}}/E_{\text{analysis}}$) to the calculated stresses, applying K_c where appropriate, and utilizing the 2000 Addenda fatigue curve.

NOTE: It is recognized that some of the references used in this calculation are not the latest revision; for example, Reference [12] (VYC-378, Revision 0) has been revised. However, the details necessary to perform the evaluations in this calculation are not necessarily contained in the latest revision of all documents. Therefore, wherever necessary, the appropriate revision of the governing document is referenced in order to obtain all appropriate inputs necessary to perform the EAF calculations. So, it should be recognized that, despite using what appear to be outdated revisions of some references, use of these references is for input data use only. All calculations represent the latest available analyses for all locations.

NOTE: Hand calculations may yield results slightly different than the values shown in the tables of this calculation due to round-off based on the significant figures utilized by the spreadsheet used for these calculations.

4.1 RPV Lower Head

The 60-year CUF value (without EAF effects) for the RPV shell/bottom head location was reported in Table 4.3-3 of the VY LRA submittal to be 0.400. The EAF CUF estimated by Entergy for this location was 0.98, based on an overall F_{en} of 2.45. Based on this result, further refined analysis would not normally be necessary to show acceptable EAF CUF results for this component. However, the calculation for this location is updated in this section to reflect the updated water chemistry information supplied for this project.

The CUF value reported in the VY LRA for the RPV shell/bottom head location is 0.400. This value is the original design basis CUF from the RPV Stress Report, as noted on page B8 of Reference [12]. However, as noted on page A61 of Reference [12], this CUF corresponds to Point 8, which is located on the outside surface of the RPV bottom head at the junction with the support skirt. Therefore, this location is not exposed to the reactor coolant, and EAF effects do not apply. Based on this, evaluation of the limiting location along the inside surface of the RPV bottom head was performed.

Based on a review of the primary plus secondary stresses tabulated for all locations along the bottom head on page A52 of Reference [12], Point 14 was selected for EAF evaluation. Per Section 3.2.1.2 of Reference [13], none of the CUF values for the RPV bottom head region were evaluated for the effects of EPU, as the CUF values are below the EPU screening criteria value of 0.5. Therefore, as a part of the evaluation for this location, EPU effects were included. Per References [14] and [19], the RPV shell material is low alloy steel (A-533, Grade B).

The new CUF calculation for Point 14 for 40 years, which includes the use of updated methodology and incorporates EPU effects [14], is shown at the top portion of Table 7. The CUF for 40 years (without EAF effects) is 0.0057.

The fatigue calculation for 60 years for the RPV shell/bottom head location is also shown in Table 7. The results show a CUF (without EAF effects) of 0.0085 for 60 years. The fatigue calculation for 60 years makes use of the 60-year projected cycles for VY from Reference [9].

The resulting environmental fatigue calculation for the RPV shell/bottom head location is shown in Table 7. Bounding F_{en} multipliers were applied in the calculations. RPV bottom head water chemistry conditions from Tables 1 and 6 are used for this location. The results show an EAF adjusted CUF of 0.0809 for 60 years, which is acceptable (i.e., less than the allowable value of 1.0).

The CUF determined for Point 14 is very low. Comparison to other locations of the RPV shell/bottom head region indicates it is not the limiting location from a fatigue perspective. Review of the CUF values in Table 3-1 of Reference [15] reveals that the shroud support (at vessel wall junction) location is potentially more limiting, so EAF evaluation of that location is also performed.

Per page S3-99f of Reference [16], the design basis CUF of 0.06 is for Point 9. Page S3-85 of Reference [16] reveals that this point is on the RPV shell at the junction of the shroud support plate. Per References [14] and [19], the RPV shell material is low alloy steel (A-533, Grade B).



The revised and updated CUF calculation for Point 9 for 40 years, which includes the use of updated methodology and incorporates EPU effects, is shown at the top portion of Table 8. The CUF for 40 years (without EAF effects) is 0.0549. This CUF value is more limiting than the RPV shell/bottom head location evaluated in Table 7, so it is considered to be the governing location for VY with respect to the equivalent NUREG/CR-6260 RPV shell/bottom head location.

The fatigue calculation for 60 years for the RPV shell/shroud support location is also shown in Table 8. The results show a CUF (without EAF effects) of 0.0774 for 60 years. The fatigue calculation for 60 years makes use of the 60-year projected cycles for VY from Reference [9].

The resulting environmental fatigue calculation for the RPV shell/shroud support location is shown in Table 8. Bounding F_{en} multipliers were applied in the calculations. RPV bottom head water chemistry conditions from Table 6 are used for this location. The results show an EAF adjusted CUF of 0.7364 for 60 years, which is acceptable (i.e., less than the allowable value of 1.0).



4.2 RR Inlet Nozzle

For conservatism due to the different materials involved, two locations are evaluated for the RR inlet nozzle: (1) the limiting location in the nozzle forging, and (2) the limiting location in the safe end.

The 60-year CUF value (without EAF effects) for the RR inlet nozzle in the VY LRA submittal is 0.610. However, that analysis used conservative transient definitions and cyclic projections for 60 years of operation that have since been updated. The applicable CUF values are those shown in Table 3-1 of Reference [15] (0.1058 for the safe end, and 0.03 for the nozzle for 40-years), except that these values are pre-EPU.

For the RR inlet nozzle forging, the governing CUF calculation is shown on page B28 of Reference [12], where a value of 0.03 was obtained. From pages A269 and A270 of Reference [12], the CUF calculation corresponds to Point 12 in the nozzle forging, which is on the outside surface of the nozzle on the outboard end of the nozzle transition. Although this location is not exposed to the reactor coolant, it will be conservatively evaluated for EAF effects as it is the bounding fatigue location in the nozzle forging. As a part of the evaluation for this location, EPU effects were included. Per page I-S8-4 of Reference [17], the RR inlet nozzle material is low alloy steel (A-508 Class II).

The new CUF calculation for Point 12 for 40 years, which includes the use of updated methodology and incorporates EPU effects [14], is shown at the top portion of Table 9. The CUF for 40 years (without EAF effects) is 0.0433.

The fatigue calculation for 60 years for the RR inlet nozzle forging location is also shown in Table 9. The results show a CUF (without EAF effects) of 0.0650 for 60 years. The fatigue calculation for 60 years makes use of the 60-year projected cycles for VY from Reference [9].

The resulting environmental fatigue calculation for the RR inlet nozzle forging location is shown in Table 9. Bounding F_{en} multipliers were applied in the calculations. RPV beltline water chemistry conditions from Table 5 are used for this location. The results show an EAF adjusted CUF of 0.5034 for 60 years, which is acceptable (i.e., less than the allowable value of 1.0)

For the RR inlet nozzle safe end, the governing CUF calculation is shown on page B27 of Reference [12], where a value of 0.1058 was obtained. From pages A257 and A259 of Reference [12], the CUF calculation corresponds to Line 6 at the inside surface of the safe end. Page A238 of Reference [12] reveals that this location is location at the nozzle-to-safe end weld. Per Section 3.2.1.2 of Reference [13], the CUF value for the RR inlet nozzle safe end was evaluated for the effects of EPU, since the original CUF calculated in Reference [18] was 0.551 (which was adjusted downward to 0.1058 by Entergy in Reference [12] based on further refined evaluation). Therefore, as a part of the evaluation for this location, EPU effects were included. Per page 8 of Reference [18], the RR inlet nozzle safe end material is 316L stainless steel.



The new CUF calculation for the RR inlet nozzle safe end for 40 years, which includes the use of updated methodology and incorporates EPU effects [14], is shown at the top portion of Table 10. The CUF for 40 years (without EAF effects) is 0.0017.

The fatigue calculation for 60 years for the RR inlet nozzle safe end location is also shown in Table 10. The results show a CUF (without EAF effects) of 0.0017 for 60 years. The fatigue calculation for 60 years makes use of the 60-year projected cycles for VY from Reference [9].

The resulting environmental fatigue calculation for the RR inlet nozzle safe end location is shown in Table 10. Bounding F_{en} multipliers were applied in the calculations. Recirculation line water chemistry conditions from Table 2 are used for this location. The results show an EAF adjusted CUF of 0.0199 for 60 years, which is acceptable (i.e., less than the allowable value of 1.0)

5.0 RESULTS OF ANALYSIS

The final environmental fatigue results contained in Sections 4.1 and 4.2 (and associated Tables 7 through 10) for the RPV shell/bottom head and RR inlet nozzle locations are summarized in Table 11.

6.0 CONCLUSIONS AND DISCUSSION

In this calculation, EAF calculations were performed in accordance with the GALL Report [2] for the following VY locations:

- RR inlet nozzle, consisting of the following bounding locations:
 - Nozzle forging (low alloy steel)
 - Safe end (stainless steel)
- RPV shell/bottom head, consisting of the following bounding locations:
 - Limiting bottom head shell inside surface location (low alloy steel)
 - Limiting RPV shell/shroud support location (low alloy steel)

The above locations were selected based on the locations identified in NUREG/CR-6260 for the older vintage GE plant and plant-specific fatigue calculations that determined the limiting locations for VY. Calculations for the remaining NUREG/CR-6260 locations will be documented in other analyses performed under this project.

The EAF results for the locations identified above are shown in Table 11. These results indicate that the fatigue usage factors, including environmental effects, are within the allowable value for 60 years of operation for all locations evaluated. The calculations for all locations make use of the 60-year projected cycles for VY and incorporate EPU effects. Therefore, no additional evaluation is required for these components, and the GALL requirements are satisfied.

7.0 REFERENCES

1. NUREG/CR-6260 (INEL-95/0045), "Application of NUREG/CR-5999 Interim Fatigue Curves to Selected Nuclear Power Plant Components," March 1995.
2. NUREG-1801, Revision 1, "Generic Aging Lessons Learned (GALL) Report," U. S. Nuclear Regulatory Commission, September 2005.
3. USAS B31.1.0 – 1967, USA Standard Code for Pressure Piping, "Power Piping," American Society of Mechanical Engineers, New York.
4. NUREG/CR-6583 (ANL-97/18), "Effects of LWR Coolant Environments on Fatigue Design Curves of Carbon and Low-Alloy Steels," March 1998.
5. NUREG/CR-5704 (ANL-98/31), "Effects of LWR Coolant Environments on Fatigue Design Curves of Austenitic Stainless Steels," April 1999.
6. EPRI/BWRVIP Memo No. 2005-271, "Potential Error in Existing Fatigue Reactor Water Environmental Effects Analyses," July 1, 2005.

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8. "Vermont Yankee Dissolved Oxygen (DO) Levels for Use in EAF Evaluations," page 11 of Entergy Design Input Record (DIR) EC No. 1773, Revision 0, "Environmental Fatigue Analysis for Vermont Yankee Nuclear Power Station," 7/3/07, SI File No. VY-16Q-209.
9. "Reactor Thermal Cycles for 60 Years of Operation," Attachment 1 of Entergy Design Input Record (DIR) EC No. 1773, Revision 0, "Environmental Fatigue Analysis for Vermont Yankee Nuclear Power Station," 7/3/07, SI File No. VY-16Q-209.
10. VY LRA, page 1-4 (*included as Appendix B to this calculation*).
11. American Society of Mechanical Engineers Boiler & Pressure Vessel Code, Section III, Rules for Construction of Nuclear Facility Components, and Section II, Materials, Part D, "Properties (Customary)," 1998 Edition including the 2000 Addenda.
12. Yankee Atomic Electric Company Calculation No. VYC-378, Revision 0, "Vermont Yankee Reactor Cyclic Limits for Transient Events," 10/16/85, SI File No. VY-05Q-211.

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14. GE Nuclear Energy Certified Design Specification No. 26A6019, Revision 1, "Reactor Vessel – Extended Power Uprate," June 2, 2003, SI File No. VY-05Q-236.
15. Structural Integrity Associates Report No. SIR-01-130, Rev. 0, "System Review and Recommendations for a Transient and Fatigue Monitoring System at the Vermont Yankee Nuclear Power Station," February 2002, SI File No. W-VY-05Q-401.
16. CB&I RPV Stress Report, Section S3, Revision 4, "Stress Analysis, Shroud Support, Vermont Yankee Reactor Vessel, CB&I Contract 9-6201," 2-3-70, SI File No. VY-16Q-203.
17. CB&I RPV Stress Report, Section S8, Revision 4, "Stress Analysis, Recirculation Inlet Nozzle, Vermont Yankee Reactor Vessel, CB&I Contract 9-6201," 2-3-70, SI File No. VY-16Q-203.
18. GE Nuclear Energy Certified Stress Report No. 23A4292, Revision 4, "Reactor Vessel – Recirculation Inlet Safe End Nozzle," March 12, 1986, SI File No. VY-16Q-203.
19. Entergy Drawing No. 5920-5752, Revision 3 (CB&I Drawing No. R15, Revision 1), "Vessel & Attachments Mat'l. Identifications," 1/20/88, SI File No. VY-16Q-209.

Table 1: Water Chemistry Calculations

Date of HWC Implementation:	11/01/2003	<i>(see Appendix A)</i>
Availability of HWC System Since HWC Implementation:	98.54%	<i>(see Appendix A)</i>
Projected Future HWC System Availability:	98.5%	<i>(see Appendix A, assume same as recent experience)</i>
<u>Recirculation Line DO</u>		
pre-HWC:	122	ppb <i>(see Appendix A)</i>
post-HWC:	48	ppb <i>(see Appendix A)</i>
<u>Feedwater Line DO</u>		
pre-HWC:	40	ppb <i>(see Appendix A)</i>
post-HWC:	40	ppb <i>(see Appendix A)</i>
<u>RPV Upper Region DO</u>		
pre-HWC:	114	ppb <i>(see Appendix A)</i>
post-HWC:	97	ppb <i>(see Appendix A)</i>
<u>RPV Beltline Region DO</u>		
pre-HWC:	123	ppb <i>(see Appendix A)</i>
post-HWC:	46	ppb <i>(see Appendix A)</i>
<u>RPV Bottom Head Region DO</u>		
pre-HWC:	128	ppb <i>(see Appendix A)</i>
post-HWC:	69	ppb <i>(see Appendix A)</i>
Plant Startup Date:	03/22/1972	<i>(see Appendix B)</i>
Time at pre-HWC Conditions:	31.61	years <i>(calculated, includes leap years.)</i>
Date of Calculations:	04/30/2007	
Time Since HWC Implementation:	3.49	years <i>(calculated, includes leap years.)</i>
Projected Future Time for HWC Operation:	24.90	years <i>(calculated, includes leap years.)</i>
Overall HWC Availability:	47%	

Note: All operation through 11/1/2003 was assumed as NWC using the dissolved oxygen values from the "Pre-NMCA" column in Appendix A, and all operation after 11/1/2003 was assumed as HWC using the maximum oxygen values from the "Post-NMCA + HWC (OLP)", "Post-NMCA + HWC (EPU)", and "Future Operation" columns in Appendix A.

Table 2: Bounding F_{en} Multipliers for Recirculation Line

Low Alloy Steel: $F_{en} = \exp(0.898 - 0.101S^*T^*O^*e^*)$

Assume $S^* = 0.015$ (maximum)
Assume $e^* = \ln(0.001) = -6.908$ (minimum)

For a BWR with HWC environment (post-HWC implementation):
DO = 48 ppb = 0.048 ppm
DO < 0.050 ppm, so $O^* = 0$
Thus:

T (°C)	T (°F)	F_{en}
0	32	2.45
50	122	2.45
100	212	2.45
150	302	2.45
200	392	2.45
250	482	2.45
288	550	2.45

Thus, maximum $F_{en} = 2.45$ [$T^* = (T-150)$ for $T > 150^\circ\text{C}$]

For a BWR with NWC environment (pre-HWC implementation):
DO = 122 ppb = 0.122 ppm, so $O^* = \ln(0.122/0.04) = 1.115$
Thus:

T (°C)	T (°F)	F_{en}
0	32	2.45
50	122	2.45
100	212	2.45
150	302	2.45
200	392	4.40
250	482	7.89
288	550	12.29

Thus, maximum $F_{en} = 12.29$

Carbon Steel: $F_{en} = \exp(0.554 - 0.101S^*T^*O^*e^*)$

Assume $S^* = 0.015$ (maximum)
Assume $e^* = \ln(0.001) = -6.908$ (minimum)

For a BWR with HWC environment (post-HWC implementation):
DO = 48 ppb = 0.048 ppm
DO < 0.050 ppm, so $O^* = 0$
Thus:

T (°C)	T (°F)	F_{en}
0	32	1.74
50	122	1.74
100	212	1.74
150	302	1.74
200	392	1.74
250	482	1.74
288	550	1.74

Thus, maximum $F_{en} = 1.74$ [$T^* = (T-150)$ for $T > 150^\circ\text{C}$]

For a BWR with NWC environment (pre-HWC implementation):
DO = 122 ppb = 0.122 ppm, so $O^* = \ln(0.122/0.04) = 1.115$
Thus:

T (°C)	T (°F)	F_{en}
0	32	1.74
50	122	1.74
100	212	1.74
150	302	1.74
200	392	3.12
250	482	5.59
288	550	8.71

Thus, maximum $F_{en} = 8.71$

Stainless Steel: $F_{en} = \exp(0.935 - T^*e^*O^*)$

For a BWR with HWC environment (post-HWC implementation):
DO = 48 ppb = 0.048 ppm < 0.050 ppm, so $O^* = 0.260$
Conservatively use $T^* = 1$ for $T > 200^\circ\text{C}$
Thus:

$e^* = 0$ for $e > 0.4\%/sec$ so $F_{en} = 2.55$
 $e^* = \ln(e/0.4)$ for $0.0004 \leq e \leq 0.4\%/sec$ so F_{en} ranges from 2.55 to 15.35
 $e^* = \ln(0.0004/0.4)$ for $e < 0.0004\%/sec$ so $F_{en} = 15.35$

Thus, maximum $F_{en} = 15.35$

For a BWR with NWC environment (pre-HWC implementation):
DO = 122 ppb = 0.122 ppm > 0.05 ppm, so $O^* = 0.172$
Conservatively use $T^* = 1$ for $T > 200^\circ\text{C}$
Thus:

so $F_{en} = 2.55$
so F_{en} ranges from 2.55 to 8.36
so $F_{en} = 8.36$

Thus, maximum $F_{en} = 8.36$

Table 3: Bounding F_{en} Multipliers for Feedwater Line

<u>Low Alloy Steel:</u>			$F_{en} = \exp(0.898 - 0.101S^*T^*O^*_{i'})$		
Assume $S^* = 0.015$ (maximum)			Assume $i'_{i'} = \ln(0.001) = -6.908$ (minimum)		
For a BWR with HWC environment (post-HWC implementation): DO = 40 ppb = 0.040 ppm < 0.050 ppm so $O^* = 0$ Thus:			For a BWR with NWC environment (pre-HWC implementation): DO = 40 ppb = 0.040 ppm < 0.050 ppm so $O^* = 0$ Thus:		
T (°C)	T (°F)	F_{en}	T (°C)	T (°F)	F_{en}
0	32	2.45	0	32	2.45
50	122	2.45	50	122	2.45
100	212	2.45	100	212	2.45
150	302	2.45	150	302	2.45
200	392	2.45	200	392	2.45
250	482	2.45	250	482	2.45
288	550	2.45	288	550	2.45
Thus, maximum $F_{en} =$		2.45	[T = (T-150) for T > 150°C]		Thus, maximum $F_{en} =$
					2.45

<u>Carbon Steel:</u>			$F_{en} = \exp(0.554 - 0.101S^*T^*O^*_{i'})$		
Assume $S^* = 0.015$ (maximum)			Assume $i'_{i'} = \ln(0.001) = -6.908$ (minimum)		
For a BWR with HWC environment (post-HWC implementation): DO = 40 ppb = 0.040 ppm < 0.050 ppm so $O^* = 0$ Thus:			For a BWR with NWC environment (pre-HWC implementation): DO = 40 ppb = 0.040 ppm < 0.050 ppm so $O^* = 0$ Thus:		
T (°C)	T (°F)	F_{en}	T (°C)	T (°F)	F_{en}
0	32	1.74	0	32	1.74
50	122	1.74	50	122	1.74
100	212	1.74	100	212	1.74
150	302	1.74	150	302	1.74
200	392	1.74	200	392	1.74
250	482	1.74	250	482	1.74
288	550	1.74	288	550	1.74
Thus, maximum $F_{en} =$		1.74	[T = (T-150) for T > 150°C]		Thus, maximum $F_{en} =$
					1.74

There is no stainless steel in the Class 1 feedwater line.

Table 4: Bounding F_{en} Multipliers for RPV Upper Region

<u>Low Alloy Steel:</u>			$F_{en} = \exp(0.898 - 0.101S^*T^*O^*\epsilon^*)$																																																																																																																																																		
Assume $S^* = 0.015$ (maximum)			Assume $\epsilon^* = \ln(0.001) = -6.908$ (minimum)																																																																																																																																																		
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$\epsilon^* = 0$ for $\epsilon_r > 0.4\%/sec$	so $F_{en} =$	2.55	$\epsilon^* = 0$ for $\epsilon_r > 0.4\%/sec$	so $F_{en} =$	2.55																																																																																																																																																
$\epsilon^* = \ln(\epsilon_r/0.4)$ for $0.0004 \leq \epsilon_r \leq 0.4\%/sec$	so F_{en} ranges from	2.55	$\epsilon^* = \ln(\epsilon_r/0.4)$ for $0.0004 \leq \epsilon_r \leq 0.4\%/sec$	so F_{en} ranges from	2.55																																																																																																																																																
	to	8.36		to	8.36																																																																																																																																																
$\epsilon^* = \ln(0.0004/0.4)$ for $\epsilon_r < 0.0004\%/sec$	so $F_{en} =$	8.36	$\epsilon^* = \ln(0.0004/0.4)$ for $\epsilon_r < 0.0004\%/sec$	so $F_{en} =$	8.36																																																																																																																																																
Thus, maximum $F_{en} = 8.36$			Thus, maximum $F_{en} = 8.36$																																																																																																																																																		

Table 5: Bounding F_{en} Multipliers for RPV Beltline Region

<u>Low Alloy Steel:</u>			$F_{en} = \exp(0.898 - 0.101S^*T^*O^*\epsilon^*)$					
Assume $S^* = 0.015$ (maximum) Assume $\epsilon^* = \ln(0.001) = -6.908$ (minimum)			Assume $S^* = 0.015$ (maximum) Assume $\epsilon^* = \ln(0.001) = -6.908$ (minimum)					
For a BWR with HWC environment (post-HWC implementation): DO = 46 ppb = 0.046 ppm DO < 0.050 ppm, so $O^* = 0$ Thus:			For a BWR with NWC environment (pre-HWC implementation): DO = 123 ppb = 0.123 ppm, so $O^* = \ln(0.123/0.04) = 1.123$ Thus:					
T (°C)	T (°F)	F_{en}	T (°C)	T (°F)	F_{en}			
0	32	2.45	0	32	2.45			
50	122	2.45	50	122	2.45			
100	212	2.45	100	212	2.45			
150	302	2.45	150	302	2.45			
200	392	2.45	200	392	4.42			
269.45	517.01	2.45	269.45	517.01	10.00			
288	550	2.45	288	550	12.43			
Thus, maximum $F_{en} =$			Thus, maximum $F_{en} =$					
2.45			[T* = (T-150) for T > 150°C]			12.43		
<u>Carbon Steel:</u>			$F_{en} = \exp(0.554 - 0.101S^*T^*O^*\epsilon^*)$					
Assume $S^* = 0.015$ (maximum) Assume $\epsilon^* = \ln(0.001) = -6.908$ (minimum)			Assume $S^* = 0.015$ (maximum) Assume $\epsilon^* = \ln(0.001) = -6.908$ (minimum)					
For a BWR with HWC environment (post-HWC implementation): DO = 46 ppb = 0.046 ppm DO < 0.050 ppm, so $O^* = 0$ Thus:			For a BWR with NWC environment (pre-HWC implementation): DO = 123 ppb = 0.123 ppm, so $O^* = \ln(0.123/0.04) = 1.123$ Thus:					
T (°C)	T (°F)	F_{en}	T (°C)	T (°F)	F_{en}			
0	32	1.74	0	32	1.74			
50	122	1.74	50	122	1.74			
100	212	1.74	100	212	1.74			
150	302	1.74	150	302	1.74			
200	392	1.74	200	392	3.13			
250	482	1.74	250	482	5.64			
288	550	1.74	288	550	8.81			
Thus, maximum $F_{en} =$			Thus, maximum $F_{en} =$					
1.74			[T* = (T-150) for T > 150°C]			8.81		
<u>Stainless Steel:</u>			$F_{en} = \exp(0.935 - T^*\epsilon^*O^*)$					
For a BWR with HWC environment (post-HWC implementation): DO = 46 ppb = 0.046 ppm < 0.050 ppm, so $O^* = 0.260$ Conservatively use $T^* = 1$ for T > 200°C Thus:			For a BWR with NWC environment (pre-HWC implementation): DO = 123 ppb = 0.123 ppm > 0.05 ppm, so $O^* = 0.172$ Conservatively use $T^* = 1$ for T > 200°C Thus:					
$\epsilon^* = 0$ for $\epsilon > 0.4\%/sec$	so $F_{en} =$	2.55	$\epsilon^* = 0$ for $\epsilon > 0.4\%/sec$	so $F_{en} =$	2.55			
$\epsilon^* = \ln(\epsilon/0.4)$ for $0.0004 \leq \epsilon \leq 0.4\%/sec$	so F_{en} ranges from	2.55	$\epsilon^* = \ln(\epsilon/0.4)$ for $0.0004 \leq \epsilon \leq 0.4\%/sec$	so F_{en} ranges from	2.55			
	to	15.35		to	8.36			
$\epsilon^* = \ln(0.0004/0.4)$ for $\epsilon < 0.0004\%/sec$	so $F_{en} =$	15.35	$\epsilon^* = \ln(0.0004/0.4)$ for $\epsilon < 0.0004\%/sec$	so $F_{en} =$	8.36			
Thus, maximum $F_{en} =$			Thus, maximum $F_{en} =$					
15.35			8.36					

Table 6: Bounding F_{en} Multipliers for RPV Bottom Head Region

<u>Low Alloy Steel:</u>			$F_{en} = \exp(0.898 - 0.101S^*T^*O^*c^*)$		
			Assume $S^* = 0.015$ (maximum) Assume $c^* = \ln(0.001) = -6.908$ (minimum)		
For a BWR with HWC environment (post-HWC implementation): DO = 69 ppb = 0.069 ppm, so $O^* = \ln(0.069/0.04) = 0.545$ Thus:			For a BWR with NWC environment (pre-HWC implementation): DO = 128 ppb = 0.128 ppm, so $O^* = \ln(0.128/0.04) = 1.163$ Thus:		
	T (°C)	T (°F)	F_{en}		F_{en}
	0	32	2.45		2.45
	50	122	2.45		2.45
	100	212	2.45		2.45
	150	302	2.45		2.45
	200	392	3.27		4.51
	250	482	4.34		8.29
	288	550	5.39		13.17
Thus, maximum $F_{en} =$			5.39	[$T^* = (T-150)$ for $T > 150^\circ\text{C}$]	Thus, maximum $F_{en} =$
			13.17		
<u>Carbon Steel:</u>			$F_{en} = \exp(0.554 - 0.101S^*T^*O^*c^*)$		
			Assume $S^* = 0.015$ (maximum) Assume $c^* = \ln(0.001) = -6.908$ (minimum)		
For a BWR with HWC environment (post-HWC implementation): DO = 69 ppb = 0.069 ppm, so $O^* = \ln(0.069/0.04) = 0.545$ Thus:			For a BWR with NWC environment (pre-HWC implementation): DO = 128 ppb = 0.128 ppm, so $O^* = \ln(0.128/0.04) = 1.163$ Thus:		
	T (°C)	T (°F)	F_{en}		F_{en}
	0	32	1.74		1.74
	50	122	1.74		1.74
	100	212	1.74		1.74
	150	302	1.74		1.74
	200	392	2.31		3.20
	250	482	3.08		5.88
	288	550	3.82		9.34
Thus, maximum $F_{en} =$			3.82	[$T^* = (T-150)$ for $T > 150^\circ\text{C}$]	Thus, maximum $F_{en} =$
			9.34		
<u>Stainless Steel:</u>			$F_{en} = \exp(0.935 - T^*c^*O^*)$		
For a BWR with HWC environment (post-HWC implementation): DO = 69 ppb = 0.069 ppm > 0.050 ppm, so $O^* = 0.172$ Conservatively use $T^* = 1$ for $T > 200^\circ\text{C}$ Thus:			For a BWR with NWC environment (pre-HWC implementation): DO = 128 ppb = 0.128 ppm > 0.05 ppm, so $O^* = 0.172$ Conservatively use $T^* = 1$ for $T > 200^\circ\text{C}$ Thus:		
			so $F_{en} =$		so $F_{en} =$
			2.55		2.55
			so F_{en} ranges from		so F_{en} ranges from
			2.55		2.55
			to		to
			8.36		8.36
			so $F_{en} =$		so $F_{en} =$
			8.36		8.36
Thus, maximum $F_{en} =$			8.36		Thus, maximum $F_{en} =$
			8.36		

Table 7: EAF Evaluation for RPV Shell/Bottom Head Location

Component: RPV Shell/Bottom Head
 NUREG/CR-6260 CUF: 0.032 (for reference only)
 Reference: NUREG/CR-6260, p. 5-102
 Stress Report CUF: 0.0057 (for Point 14, see below)
 Material: Low Alloy Steel (Material = A-533 Gr. B per References [14] and [19])

Design Basis CUF Calculation for 40 years:

$E_{\text{fatigue curve}}/E_{\text{analysis}} =$	1.149	Conservatively used minimum E of 26.1 from Section S2 Appendix of RPV Stress Report.
Power Uprate =	1.0067	$=(549 - 100) / (546 - 100)$ per 4.4.1.b of 26A6019, Rev. 1 [14]
$K_t =$	1.000	stress concentration factor
$m =$	2.0	NB-3228.5 of ASME Code, Section III [11]
$n =$	0.2	NB-3228.5 of ASME Code, Section III [11]
$S_m =$	26,700	psi (ASME Code, Section II, Part D [11])

P_L+P_B+Q (see Note 1)	K_e (see Note 2)	S_{alt} (see Note 3)	n (see Note 4)	N (see Note 5)	U
44,526	1.00	25,762	200	35,300	0.0057
Total, $U_{40} =$					0.0057

- Notes:
- P_L+P_B+Q is obtained for Point 14 from p. A52 of VYC-378, Rev. 0.
 - K_e computed in accordance with NB-3228.5 of ASME Code, Section III.
 - $S_{\text{alt}} = 0.5 * K_e * K_t * E_{\text{fatigue curve}}/E_{\text{analysis}} * \text{Power Uprate} * (P_L+P_B+Q)$.
 - n for 40 years is the number of Heatup-Cooldown cycles, per p. B8 of VYC-378, Rev. 0.
 - N obtained from Figure I-9.1 of Appendix I of ASME Code, Section III.
 - n for 60 years is the projected number of Heatup-Cooldown cycles.

Revised CUF Calculation for 60 Years:

P_L+P_B+Q (see Note 1)	K_e (see Note 2)	S_{alt} (see Note 3)	n (see Note 6)	N (see Note 4)	U
44,526	1.00	25,762	300	35,300	0.0085
Total, $U_{60} =$					0.0085

Environmental CUF Calculation for 60 Years:

Maximum $F_{\text{en-HWC}}$ Multiplier for HWC Conditions = 5.39 (from Table 6)
 Maximum $F_{\text{en-NWC}}$ Multiplier for NWC Conditions = 13.17 (from Table 6)

$$U_{\text{env-60}} = U_{60} * F_{\text{en-NWC}} * 0.53 + U_{60} * F_{\text{en-HWC}} * 0.47 = 0.0809$$

Overall Multiplier = $U_{\text{env-60}}/U_{60} = 9.51$

Table 8: EAF Evaluation for Limiting RPV Shell/Shroud Support Location

Component: RPV Shell at Shroud Support
 NUREG/CR-6260 CUF: 0.032 (for reference only)
 Reference: NUREG/CR-6260, p. 5-102
 Stress Report CUF: 0.0549 (for Point 9, see below)
 Material: Low Alloy Steel (Material = A-533 Gr. B per References [14] and [19])

Design Basis CUF Calculation for 40 years:

Hydrotest σ_p =	26,240	psi (p. S3-97 of RPV Stress Report)
Hydrotest σ_r =	-1,250	psi (p. S3-97 of RPV Stress Report)
Stress Concentration Factor, K_1 =	2.40	(p. S3-99d of RPV Stress Report)
Hydrotest $K_1\sigma_p$ =	62,976	psi (p. S3-97 of RPV Stress Report)
Improper Startup σ_p =	28,060	psi (p. S3-98 of RPV Stress Report)
Improper Startup σ_r =	-1,025	psi (p. S3-98 of RPV Stress Report)
Improper Startup Skin Stress =	156,099	psi (p. S3-98 of RPV Stress Report)
Improper Startup $K_1\sigma_p$ + Skin Stress =	223,443	psi (p. S3-98 of RPV Stress Report)
Warmup σ_p =	-5,707	psi (p. S3-99a of RPV Stress Report)
Warmup σ_r =	-102	psi (p. S3-99a of RPV Stress Report)
Warmup $K_1\sigma_p$ =	-13,696	psi (p. S3-99a of RPV Stress Report)
$E_{fatigue\ curve}/E_{analysis}$ =	1.0417	30.0 / 28.8 per S3-99f of RPV Stress Report and ASME Code fatigue curve
Power Uprate =	1.0067	=(549 - 100) / (546 - 100) per 4.4.1.b of 26A6019, Rev. 1 [14]
m =	2.0	NB-3228.5 of ASME Code, Section III [11]
n =	0.2	NB-3228.5 of ASME Code, Section III [11]
S_m =	26,700	psi (ASME Code, Section II, Part D [11])

$P_L + P_B + Q$ (see Note 1)	Events	K_p (see Note 2)	S_{alt} (see Note 3)	n (see Note 4)	N (see Note 5)	U
34,690	Improper Startup - Warmup	1.00	124,825	5	332	0.0151
33,095	Hydrotest - Warmup	1.00	40,804	322	8,095	0.0398
Total, U_{40} =						0.0549

- Notes: 1. $P_L + P_B + Q$ is computed for Point 9 based on the $[(\sigma_p \cdot \sigma_r)_{Event 1} + (\sigma_p \cdot \sigma_r)_{Event 2}]$ stress intensity.
 2. K_p computed in accordance with NB-3228.5 of ASME Code, Section III.
 3. $S_{alt} = 0.5 \cdot K_p \cdot E_{fatigue\ curve} / E_{analysis} \cdot Power\ Uprate \cdot [(K_1\sigma_p \cdot \sigma_r)_{Event 1} + (K_1\sigma_p \cdot \sigma_r)_{Event 2}]$
 4. n for 40 years is the number of cycles as follows per p. S3-99e and S3-99f of the RPV Stress Report:

Improper Startup =	5	cycles
Hydrotest =	2	cycles
Isothermal at 70°F and 1,000 psi =	120	cycles (same as number of Startup events)
Warmup-Cooldown =	199	cycles
Warmup-Blowdown =	1	cycle
TOTAL =	327	cycles

5. N obtained from Figure I-9.1 of Appendix I of ASME Code, Section III.

6. n for 60 years is the projected number of cycles as follows:

Improper Startup =	1	cycles
Hydrotest =	1	cycles
Isothermal at 70°F and 1,000 psi =	300	cycles (same as number of Startup events)
Warmup-Cooldown =	300	cycles
Warmup-Blowdown =	1	cycle
TOTAL =	603	cycles

Revised CUF Calculation for 60 Years:

$P_L + P_B + Q$ (see Note 1)	Events	K_p (see Note 2)	S_{alt} (see Note 3)	n (see Note 6)	N (see Note 4)	U
34,690	Improper Startup - Warmup	1.00	124,825	1	332	0.0030
33,095	Hydrotest - Warmup	1.00	40,804	602	8,095	0.0744
Total, U_{60} =						0.0774

Environmental CUF Calculation for 60 Years:

Maximum F_{en-HWC} Multiplier for HWC Conditions =	5.39	(from Table 6)
Maximum F_{en-NWC} Multiplier for NWC Conditions =	13.17	(from Table 6)
$U_{env-60} = U_{60} \times F_{en-NWC} \times 0.53 + U_{60} \times F_{en-HWC} \times 0.47 =$	0.7364	
Overall Multiplier = $U_{env-60} / U_{60} =$	9.51	

Table 9: EAF Evaluation for RR Inlet Nozzle Forging Location

Component: Recirculation Inlet Nozzle Forging
 NUREG/CR-6260 CUF: 0.310 (for reference only)
 Reference: NUREG/CR-6260, p. 5-105
 Stress Report CUF: 0.0433 (updated for Point 12, see below)
 Material: Low Alloy Steel (Material = A-508 Cl. II per p. I-S8-4 of CBIN Stress Report Section S8)

Design Basis CUF Calculation for 40 years:

$E_{fatigue\ curve} / E_{analysis} = 1.1278$ = 30.0 / 26.6 (per p. I-S8-24 of CBIN Stress Report Section S8 and ASME Code fatigue curve)
 Power Uprate = 1.0067 = (549 - 100) / (546 - 100) per 4.4.1.b of 26A6019, Rev. 1 [14]
 $K_1 = 1.660$ stress concentration factor (p. A270 of VYC-378, Rev. 0 [12])
 $m = 2.0$ NB-3228.5 of ASME Code, Section III [11]
 $n = 0.2$ NB-3228.5 of ASME Code, Section III [11]
 $S_m = 26,700$ psi (ASME Code, Section II, Part D [11])

$P_L + P_B + Q$ (see Note 1)	Skin Stress (see Note 2)	K_e (see Note 3)	S_{alt} (see Note 4)	n (see Note 5)	N (see Note 6)	U
43,110	15,145	1.00	49,224	200	4,614	0.0433
Total, $U_{40} =$						0.0433

- Notes:
- $P_L + P_B + Q$ is obtained for Point 12 from p. A270 of VYC-378, Rev. 0.
 - Skin Stress is obtained for Point 12 from p. A270 of VYC-378, Rev. 0.
 - K_e computed in accordance with NB-3228.5 of ASME Code, Section III.
 - $S_{alt} = 0.5 * K_e * E_{fatigue\ curve} / E_{analysis} * Power\ Uprate * [(P_L + P_B + Q) K_1 + Skin\ Stress]$.
 - n for 40 years is the number of Heatup-Cooldown cycles, per p. B28 of VYC-378, Rev. 0.
 - N obtained from Figure I-9.1 of Appendix I of ASME Code, Section III.
 - n for 60 years is the projected number of Heatup-Cooldown cycles.

Revised CUF Calculation for 60 Years:

$P_L + P_B + Q$ (see Note 1)	Skin Stress (see Note 2)	K_e (see Note 3)	S_{alt} (see Note 4)	n (see Note 7)	N (see Note 6)	U
43,110	15,145	1.00	49,224	300	4,614	0.0650
Total, $U_{60} =$						0.0650

Environmental CUF Calculation for 60 Years:

Maximum F_{en-HWC} Multiplier for HWC Conditions = 2.45 (from Table 5)
 Maximum F_{en-NWC} Multiplier for NWC Conditions = 12.43 (from Table 5)
 $U_{env-60} = U_{60} * F_{en-NWC} * 0.53 + U_{60} * F_{en-HWC} * 0.47 = 0.5034$
 Overall Multiplier = $U_{env-60} / U_{60} = 7.74$

Table 10: EAF Evaluation for RR Inlet Nozzle Safe End Location

Component: Recirculation Inlet Nozzle Safe End
 NUREG/CR-6260 CUF: 0.310 (for reference only)
 Reference: NUREG/CR-6260, p. 5-105
 Stress Report CUF: 0.0017 (updated for Location 6-I, see below)
 Material: Stainless Steel (316L per p. 8 of 23A4292, Rev. 4)

Design Basis CUF Calculation for 40 years:

$E_{fatigue\ curve} / E_{analysis} = 1.1076$ = 28.3 / 25.55 (per p. 62 of Reference [18] and ASME Code fatigue curve)
 Power Uprate = 1.0067 = (549 - 100) / (546 - 100) per 4.4.1.b of 26A6019, Rev. 1 [14]
 $K_t = 1.280$ stress concentration factor (p. B27 of VYC-378, Rev. 0 [12])
 $m = 1.7$ NB-3228.5 of ASME Code, Section III [11]
 $n = 0.3$ NB-3228.5 of ASME Code, Section III [11]
 $S_m = 16,600$ psi (ASME Code, Section II, Part D [11])

$P_L + P_B + Q$ (see Note 1)	$P + Q + F$ (see Note 2)	K_e (see Note 3)	S_{alt} (see Note 4)	n (see Note 5)	N (see Note 6)	U
47,183	36,972	1.00	26,385	2,076	1,242,266	0.0017
Total, $U_{40} =$						0.0017

- Notes: 1. $P_L + P_B + Q$ is obtained for Surface I (after weld overlay) from p. 117 of Reference [18].
 2. $P + Q + F$ is obtained for Point 6-I from p. 118 of Reference [18] (BEFORE weld overlay).
 3. K_e computed in accordance with NB-3228.5 of ASME Code, Section III.
 4. $S_{alt} = 0.5 * K_e * E_{fatigue\ curve} / E_{analysis} * Power\ Uprate * [(P+Q+F) K_t]$.
 5. n for 40 years is the number of cycles as follows per p. B26 of VYC-378, Rev. 0:

Design Hydrotest = 130	
<u>Loss of Feedpumps Composite:</u>	
Startup/Shutdown =	290
SRV Blowdown =	8
Loss of Feedwater Pumps =	30
SCRAM =	270
Normal +/- Seismic =	11
Normal =	739
Zero-load =	598
Total number of cycles = 2,076	

10 events x 3 up/down cycles per event
 10 cycles of upset seismic, plus 1 Level C seismic event
 = Sum of all of above events
 = Startup/Shutdown + SRV Blowdown + Scram + LOFP

6. N obtained from Figure I-9.2 of Appendix I of ASME Code, Section III.
 7. n for 60 years is the projected number of cycles as follows:

Design Hydrotest = 120	
<u>Loss of Feedpumps Composite:</u>	
Startup/Shutdown =	300
SRV Blowdown =	1
Loss of Feedwater Pumps =	30
SCRAM =	289
Normal +/- Seismic =	11
Normal =	751
Zero-load =	620
Total number of cycles = 2,122	

10 events x 3 up/down cycles per event
 All remaining scrams
 Assume the same
 = Sum of all of above events
 = Startup/Shutdown + SRV Blowdown + Scram + LOFP

Revised CUF Calculation for 60 Years:

$P_L + P_B + Q$ (see Note 1)	$P + Q + F$ (see Note 2)	K_e (see Note 3)	S_{alt} (see Note 4)	n (see Note 5)	N (see Note 7)	U
47,183	36,972	1.00	26,385	2,122	1,242,266	0.0017
Total, $U_{60} =$						0.0017

Environmental CUF Calculation for 60 Years:

Maximum F_{en-HWC} Multiplier for HWC Conditions = 15.35 (from Table 2)
 Maximum F_{en-NWC} Multiplier for NWC Conditions = 8.36 (from Table 2)
 $U_{env-60} = U_{60} * F_{en-NWC} * 0.53 + U_{60} * F_{en-HWC} * 0.47 = 0.0199$
 Overall Multiplier = $U_{env-60} / U_{60} = 11.64$

Table 11: Summary of EAF Evaluation Results for VY

No.	Component	Material	40-Year Design CUF ⁽¹⁾	60-Year CUF ⁽²⁾	Overall Environmental Multiplier	60-Year Environmental CUF ^(2,3)
1	RPV Shell/Bottom Head	Low Alloy Steel	0.0057	0.0085	9.51	0.0809
2	RPV Shell at Shroud Support	Low Alloy Steel	0.0549	0.0774	9.51	0.7364
3	Recirculation Inlet Nozzle Safe End	Stainless Steel	0.0017	0.0017	11.64	0.0199
4	Recirculation Inlet Nozzle Forging	Low Alloy Steel	0.0433	0.0650	7.74	0.5034

- Notes:
1. Updated 40-year CUF calculation based on recent ASME Code methodology and design basis cycles.
 2. CUF results using updated ASME Code methodology and actual cycles accumulated to-date and projected to 60 years.
 3. An F_{en} multiplier was used for each respective component with the following conditions:
 - + 47% HWC conditions and 53% NWC conditions



APPENDIX A

VY WATER CHEMISTRY INFORMATION [8]

Location	Pre-NMCA 1593 MWth (OLP)	Post-NMCA + HWC 1593 MWth (OLP)	Post-NMCA + HWC 1912 MWth (EPU)	Future Operation Post-NMCA + HWC 1912 MWth (EPU)
	---	Average Availability 98.5%	Average Availability 98.5%	Average Availability 99%
	Implementation Date = 11/1972	NMCA Application Date = 04/27/2001 HWC Implementation Date = 11/01/2003	EPU Implementation Date = 5/2006	---
FW Line	40 ppb	40 ppb	40 ppb	40 ppb
Recirc. Line	122 ppb	48 ppb	34 ppb	34 ppb
RPV Bottom Head **	128 ppb	69 ppb	55 ppb	55 ppb
RPV Upper Region	114 ppb	97 ppb	90 ppb	90 ppb
RPV Beltline Region	123 ppb	46 ppb	31 ppb	31 ppb

** RPV Bottom head at "Lower Plenum, Downflow" (i.e. outside core support columns)



APPENDIX B
VY LICENSE DATE [10]

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Theodore A. Sullivan Vice President - Fitzpatrick Nuclear Power Station	Fitzpatrick Nuclear Power Station 268 Lake Road East Lycoming, New York 13093
Jay K. Thayer Vice President - Vermont Yankee Nuclear Power Station	Entergy Nuclear Vermont Yankee Corporate Office P.O. Box 0500 185 Old Ferry Road Brattleboro, VT 05302-0500

1.1.5 Class and Period of License Sought

ENO requests renewal of the facility operating license for VYNPS (facility operating license DPR-28) for a period of 20 years. The license was issued under Section 104b of the Atomic Energy Act of 1954 as amended. License renewal would extend the facility operating license from midnight March 21, 2012, to midnight March 21, 2032.

This application also applies to renewal of those NRC source materials, special nuclear material, and by-product material licenses that are subsumed or combined with the facility operating license.

1.1.6 Alteration Schedule

ENO does not propose to construct or alter any production or utilization facility in connection with this renewal application.



Structural Integrity Associates, Inc.

File No.: VY-16Q-304

CALCULATION PACKAGE

Project No.: VY-16Q **NEC-JH_07**

PROJECT NAME:

Environmental Fatigue Analysis of VYNPS

CONTRACT NO.:

10150394

CLIENT:

Entergy Nuclear Operations, Inc

PLANT:

Vermont Yankee Nuclear Power Station

CALCULATION TITLE:

Recirculation Outlet Nozzle Finite Element Model

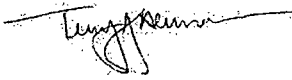

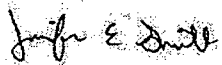
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0	1-6, Appendix: A1-A20	Initial Issue	Terry J. Herrmann 7/12/2007 	Minghao Qin 7/12/2007  Jennifer E. Smith 7/12/2007 



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1.0 OBJECTIVE

The objective of this calculation is to create a finite element model of the Vermont Yankee Nuclear Power Station recirculation outlet nozzle. This model will be used to develop a Green's Function to be used in a subsequent fatigue analysis.

2.0 GEOMETRY / MATERIAL PROPERTIES

A 2-D axisymmetric finite element model (FEM) of the nozzle was developed with element type PLANE182. The developed model includes the safe end, the nozzle forging, a portion of the vessel shell, and cladding. The model used the vessel radius multiplied by a factor 2.0 due to the model being axisymmetric.

The 2-D axisymmetric FEM was constructed using the dimensions and information from References [4 and 5] based on ANSYS [2] finite element software. Figure 1 shows the resulting finite element model.

The materials of the various components of the model are listed below:

- Safe End – SA182 F316 [4] (16Cr-12Ni-2Mo)
- Piping – SA376 TP316 [7] (16Cr-12Ni-2Mo)
- Nozzle Forging – SA508 Class 2 [5] (3/4Ni-1/2Mo-1/3Cr-V)
- Vessel – SA533 Grade B [6] (Mn-1/2Mo-1.2Ni)
- Cladding – SA240 Type 304 [1, Sheet 7] (18Cr-8Ni)

Material properties for these materials are based upon the 1998 ASME Code, Section II, Part D, with 2000 Addenda [3] and are shown in Table 1. The properties are taken at an average temperature of 300°F. This average temperature is based on a thermal shock of 500°F to 100°F which will be applied to the FEM model for Green's Function development.

3.0 PROGRAM INPUT

The input file, RON_VY.INP (included in Appendix A), creates the finite element model for the recirculation outlet nozzle.



4.0 REFERENCES

1. GE. Stress Report No. 23A4316, Revision 0, "Reactor Vessel Recirculation Outlet Safe End," SI File No. VY-16Q-204.
2. ANSYS, Release 8.1 (w/Service Pack 1), ANSYS, Inc., June 2004.
3. American Society of Mechanical Engineers, Boiler and Pressure Vessel Code, Section II, Part D, 1998 Edition, 2000 Addenda.
4. Vermont Yankee Drawing 5920-06623, Rev. 0, (Hitachi, Ltd. Drawing No IOR290-127), "Recirc. Outlet Safe End," SI File No. VY-16Q-204.
5. Vermont Yankee Drawing 5920-00238, Rev. 4, (Chicago Bridge & Iron Company, Contract No. 9-6201, Drawing No. 21), "36"x28" Nozzles Mk N1A/B," SI File No. VY-16Q-204.
6. Vermont Yankee Drawing 5920-05752, Rev. 3, "Vessel & Attachments Material Identifications," SI File No. VY-16Q-209.
7. SI File No. VY-16Q-103, "Vermont Yankee Comments on VY-16Q-304."

Table 1: Material Properties @ 300°F ⁽¹⁾

Material	SA533 Grade B (Mn-1/2Mo- 1/2Ni)	SA508 Class 2 (3/4Ni-1/2Mo- 1/3Cr-V)	SA240 Type 304 (18Cr-8Ni)	SA182 F316/ SA376 TP316 (16Cr-12Ni-2Mo)
Modulus of Elasticity, e-6 psi	28.0	26.7	27.0	27.0
Coefficient of Thermal Expansion, e-6, in/in/°F	7.7	7.3	9.8	9.8
Thermal Conductivity, Btu/hr-ft-°F	23.4	23.4	9.8	9.3
Thermal Diffusivity, ft ² /hr	0.401	0.401	0.160	0.150
Specific Heat, Btu/lb-°F ⁽²⁾	0.119	0.119	0.125	0.127
Density, lb/in ³	0.283	0.283	0.283	0.283
Poisson's Ratio	0.3	0.3	0.3	0.3

Notes:

1. The material properties applied in the analyses are taken from ASME Section II Part D 1998 Edition with 2000 Addenda. This is consistent with information provided in the Design Input Record (page 13 of VY EC No. 1773, SI File No. VY-16Q-209). The use of a later code edition than that used for the original design code is acceptable since later editions typically reflect more accurate material properties than was published in prior Code editions. Material Properties are evaluated at 300°F from the 1998 ASME Code, Section II, Part D, with 2000 Addenda, except for density and Poisson's ratio, which are assumed typical values.
2. Calculated as $[k/(pd)]/12^3$.

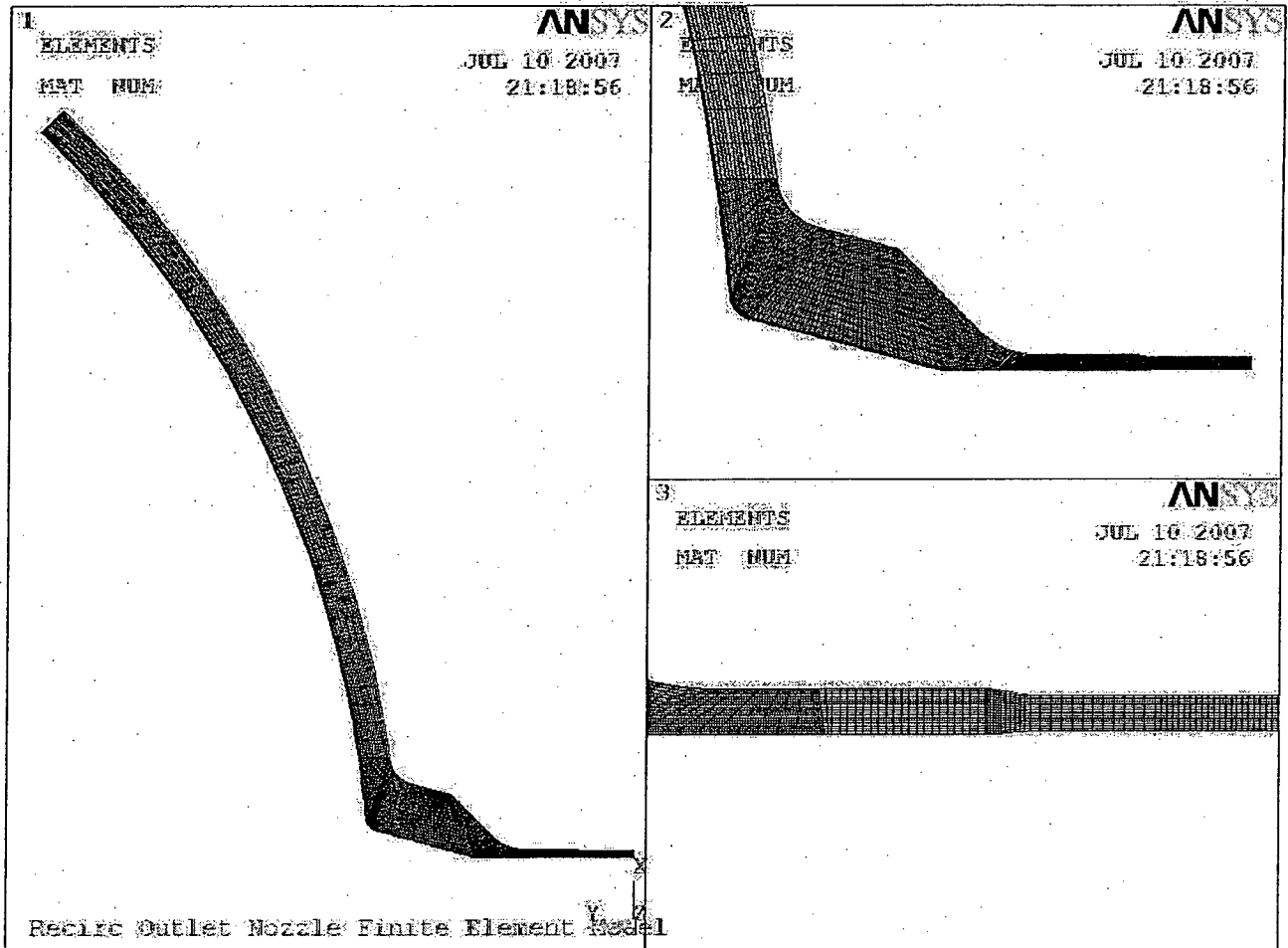


Figure 1: ANSYS Finite Element Model

APPENDIX A

RON_VY.inp



```
finish
/clear,start
/prep7
/title, Recirc Outlet Nozzle Finite Element Model

/com, PLANE182, 2-D Solid
et,1,PLANE182,,1 !Axisymmetric

/com, *****
/com, Material Properties @T=300F
/com, *****

/COM, Material #1 (Safe-End and Piping) SA-182 F316 (16Cr-12Ni-2Mo)
mp,ex,1,27E+06
mp,alpx,1,9.8E-06
mp,kxx,1,9.3/3600/12
mp,c,1,0.127
mp,nuxy,1,0.3
mp,dens,1,0.283

/COM, Material #2 (Nozzle Forging) SA-508 Class 2 (3/4Ni-1/2Mo-1/3Cr-V)
mp,ex,2,26.7E+06
mp,alpx,2,7.3E-06
mp,kxx,2,23.4/3600/12
mp,c,2,0.119
mp,nuxy,2,0.3
mp,dens,2,0.283

/COM, Material #3 (Cladding) SA-240 Type 304 (18Cr-8Ni)
mp,ex,3,27E+06
mp,alpx,3,9.8E-06
mp,kxx,3,9.8/3600/12
mp,c,3,0.125
mp,nuxy,3,0.3
mp,dens,3,0.283

/COM, Material #4 (Vessel) SA-533, GR. B (Mn-1/2Mo-1/2Ni)
mp,ex,4,28.0E+06
mp,alpx,4,7.7E-06
mp,kxx,4,23.4/3600/12
mp,c,4,0.119
mp,nuxy,4,0.3
mp,dens,4,0.283

*AFUN,DEG
/com, *** Geometric Parameters ***
*set,vira,(103+3/16) !Actual Vessel Inner Radius to base metal used for model
*set,vir,2.0*vira !2.0 time of Vessel Inner Radius to base metal used for model
*set,tvw,5+5/8-3/16 !Vessel Wall Thickness
*set,ri1,25.75/2
*set,ro1,28.375/2
*set,L1,5
*set,ro2,28.375/2
```

```

*set,L2,4.25
*set,ro3,28.875/2
*set,ro4,48.75/2
*set,L3,1.5
*set,L4,5.25
*set,L5,7+1/16
*set,L6,12+13/16
*set,L7,9+7/8
*set,L8,9+3/8
*set,L9,31+15/16
*set,L10,L9-12-13/16-tvw
*set,ra,7
*set,rb,1
*set,rc,5.25
*set,rd,2.5
*set,tv,3/16
*set,dimA,vir-(tv*2.0)+L9+11+L1 !Vessel Centerline to End of Safe End used for model
*set,L21,1
*set,L22,4.25
*set,ri21,(25+15/16)/2

```

```

/com,*****
/com, Geometry
/com,*****
local,13,0,,dimA,,

```

```

csys,13

```

```

/com, Begin at end of Safe-End - Carbon Section

```

```

k, 1, ri1, -1*(dimA)
k, 2, ri1+tv, -1*(dimA)
k, 3, ro1, -1*(dimA)
k, 4, ri1, -1*(dimA-L1)
k, 5, ri1+tv, -1*(dimA-L1)
k, 6, ro1, -1*(dimA-L1)
k, 7, ri1, -1*(dimA-L1-L2)
k, 8, ri1+tv, -1*(dimA-L1-L2)
k, 9, ro2, -1*(dimA-L1-L2)
k, 10, ri1, -1*(dimA-L1-L2-L3)
k, 11, ri1+tv, -1*(dimA-L1-L2-L3)
k, 12, ro3, -1*(dimA-L1-L2-L3)
k, 13, ri1, -1*(dimA-L1-L2-L3-L4)
k, 14, ri1+tv, -1*(dimA-L1-L2-L3-L4)
k, 15, ro3, -1*(dimA-L1-L2-L3-L4)
k, 16, ri1, -1*(dimA-L1-L2-L3-L4-L5)
k, 17, ri1+tv, -1*(dimA-L1-L2-L3-L4-L5)
k, 18, ro3, -1*(dimA-L1-L2-L3-L4-L5)

k,19, ro4, -1*(dimA-L1-L2-L3-L4-L5-L7)! Temporary Point
l,19,18
l,18,15
!fillt,1,2,ra
k,22, ro4+(L8+6)*tan(15), -1*(dimA-L1-L2-L3-L4-L5-L7-(L8+6))
l,19,22

```

LFILLT,1,4,rb

k, 25, ri1, -1*(dimA-L1-L2-L3-L4-L6)
 k, 26, ri1+tv, -1*(dimA-L1-L2-L3-L4-L6)

k, 27, ri1+(L10+tvw+tv+4)*tan(15), -1*(vir-tv-4)
 k, 28, ri1+tv+(L10+tvw+tv+4)*tan(15), -1*(vir-tv-4)

k,29, (vir+tvw+tv)*sin(45), -1*(vir+tvw+tv)*cos(45)
 k,30, 0, -1*(vir+tvw+tv) ! Temporary Point
 k,31, 0, 0 ! Temporary Point

larc,29,30,31,vir+tvw+tv

k,32, (vir+tv)*sin(45), -1*(vir+tv)*cos(45)
 k,33, 0, -1*(vir+tv) ! Temporary Point
 larc,32,33,31,vir+tv

k,34, vir*sin(45), -1*vir*cos(45)
 k,35, 0, -1*vir ! Temporary Point
 larc,34,35,31,vir

- LSTR, 4, 5
- LSTR, 5, 6
- LSTR, 6, 9
- LSTR, 9, 12
- LSTR, 12, 15
- LSTR, 5, 8
- LSTR, 4, 7
- LSTR, 7, 10
- LSTR, 8, 11
- LSTR, 11, 14
- LSTR, 10, 13
- LSTR, 13, 16
- LSTR, 14, 17
- LSTR, 16, 25
- LSTR, 17, 26
- LSTR, 26, 28
- LSTR, 25, 27
- LSTR, 4, 1
- LSTR, 1, 2
- LSTR, 2, 3
- LSTR, 3, 6
- LSTR, 5, 2
- LSTR, 7, 8
- LSTR, 8, 9
- LSTR, 12, 11
- LSTR, 11, 10
- LSTR, 13, 14
- LSTR, 14, 15

FLST,2,2,4,ORDE,2
 FITEM,2,4
 FITEM,2,6
 LPTN,P51X

FLST,2,2,4,ORDE,2
FITEM,2,8
FITEM,2,25
LPTN,P51X

FLST,2,2,4,ORDE,2
FITEM,2,7
FITEM,2,24
LPTN,P51X

FLST,2,6,4,ORDE,6
FITEM,2,6
FITEM,2,25
FITEM,2,37
FITEM,2,40
FITEM,2,42
FITEM,2,44
LDELE,P51X,, ,1

!*
LFILLT,4,41,rd, ,
!*
LFILLT,43,8,rd, ,
!*
LFILLT,39,38,rc, ,

FLST,2,3,4,ORDE,3
FITEM,2,1
FITEM,2,3
FITEM,2,5
LCOMB,P51X, ,0
LSTR, 16, 17
LSTR, 17, 21
LSTR, 25, 26
LSTR, 26, 24
LSTR, 22, 30
LSTR, 30, 35
LSTR, 27, 28
LSTR, 28, 33
LSTR, 29, 32
LSTR, 32, 34

k,39, 0, -1*(vir+tvw+tv)

!Create Areas
FLST,2,4,4
FITEM,2,27
FITEM,2,30
FITEM,2,26
FITEM,2,9
AL,P51X
FLST,2,4,4
FITEM,2,28
FITEM,2,29



FITEM,2,10
FITEM,2,30
AL,P51X
FLST,2,4,4
FITEM,2,11
FITEM,2,32
FITEM,2,10
FITEM,2,14
AL,P51X
FLST,2,4,4
FITEM,2,15
FITEM,2,14
FITEM,2,9
FITEM,2,31
AL,P51X
FLST,2,4,4
FITEM,2,32
FITEM,2,33
FITEM,2,12
FITEM,2,17
AL,P51X
FLST,2,4,4
FITEM,2,16
FITEM,2,17
FITEM,2,31
FITEM,2,34
AL,P51X
FLST,2,4,4
FITEM,2,36
FITEM,2,13
FITEM,2,33
FITEM,2,18
AL,P51X
FLST,2,4,4
FITEM,2,19
FITEM,2,18
FITEM,2,35
FITEM,2,34
AL,P51X
FLST,2,4,4
FITEM,2,2
FITEM,2,5
FITEM,2,36
FITEM,2,21
AL,P51X
FLST,2,4,4
FITEM,2,20
FITEM,2,21
FITEM,2,3
FITEM,2,35
AL,P51X
FLST,2,4,4
FITEM,2,1
FITEM,2,37
FITEM,2,23



FITEM,2,5
AL,P51X
FLST,2,4,4
FITEM,2,22
FITEM,2,23
FITEM,2,25
FITEM,2,3
AL,P51X
FLST,2,4,4
FITEM,2,38
FITEM,2,42
FITEM,2,37
FITEM,2,8
AL,P51X
FLST,2,4,4
FITEM,2,4
FITEM,2,8
FITEM,2,25
FITEM,2,40
AL,P51X
FLST,2,4,4
FITEM,2,24
FITEM,2,45
FITEM,2,7
FITEM,2,42
AL,P51X
FLST,2,4,4
FITEM,2,6
FITEM,2,7
FITEM,2,44
FITEM,2,40
AL,P51X
FLST,2,4,4
FITEM,2,41
FITEM,2,43
FITEM,2,47
FITEM,2,44
AL,P51X
FLST,2,4,4
FITEM,2,39
FITEM,2,46
FITEM,2,45
FITEM,2,43
AL,P51X

! define materials
FLST,5,8,5,ORDE,2
FITEM,5,1
FITEM,5,-8
CM,_Y,AREA
ASEL, , , ,P51X
CM,_Y1,AREA
CMSEL,S,_Y
!*
CMSEL,S,_Y1

```
AATT, 1, 1, 0,
CMSEL,S,_Y
CMDELE,_Y
CMDELE,_Y1
!*
FLST,5,5,5,ORDE,5
FITEM,5,9
FITEM,5,11
FITEM,5,13
FITEM,5,15
FITEM,5,18
CM,_Y,AREA
ASEL,,,P51X
CM,_Y1,AREA
CMSEL,S,_Y
!*
CMSEL,S,_Y1
AATT, 2, 1, 0,
CMSEL,S,_Y
CMDELE,_Y
CMDELE,_Y1
!*
FLST,5,5,5,ORDE,5
FITEM,5,10
FITEM,5,12
FITEM,5,14
FITEM,5,16
FITEM,5,-17
CM,_Y,AREA
ASEL,,,P51X
CM,_Y1,AREA
CMSEL,S,_Y
!*
CMSEL,S,_Y1
AATT, 3, 1, 0,
CMSEL,S,_Y
CMDELE,_Y
CMDELE,_Y1
!*
```

```
!/com, Map mesh areas
FLST,5,10,4,ORDE,10
FITEM,5,5
FITEM,5,10
FITEM,5,28
FITEM,5,32
FITEM,5,-33
FITEM,5,36
FITEM,5,-37
FITEM,5,42
FITEM,5,45
FITEM,5,-46
CM,_Y,LINE
LSEL,,,P51X
CM,_Y1,LINE
```



```
CMSEL,,_Y
!*
LESIZE,_Y1,,15,,,,,1
!*
FLST,5,10,4,ORDE,10
FITEM,5,3
FITEM,5,9
FITEM,5,25
FITEM,5,27
FITEM,5,31
FITEM,5,34
FITEM,5,-35
FITEM,5,40
FITEM,5,44
FITEM,5,47
CM,_Y,LINE
LSEL,,,P51X
CM,_Y1,LINE
CMSEL,,_Y
!*
LESIZE,_Y1,,,2,,,,,1
!*
FLST,5,3,4,ORDE,3
FITEM,5,39
FITEM,5,41
FITEM,5,43
CM,_Y,LINE
LSEL,,,P51X
CM,_Y1,LINE
CMSEL,,_Y
!*
LESIZE,_Y1,,,80,,,,,1
!*
FLST,5,3,4,ORDE,3
FITEM,5,6
FITEM,5,-7
FITEM,5,24
CM,_Y,LINE
LSEL,,,P51X
CM,_Y1,LINE
CMSEL,,_Y
!*
LESIZE,_Y1,,,20,,,,,1
!*
FLST,5,3,4,ORDE,3
FITEM,5,4
FITEM,5,8
FITEM,5,38
CM,_Y,LINE
LSEL,,,P51X
CM,_Y1,LINE
CMSEL,,_Y
!*
LESIZE,_Y1,,,40,,,,,1
!*
```



```
FLST,5,3,4,ORDE,3
FITEM,5,1
FITEM,5,22
FITEM,5,-23
CM,_Y,LINE
LSEL,, , ,P51X
CM,_Y1,LINE
CMSEL,,_Y
!*
LESIZE,_Y1, , ,30, , , ,1
!*
FLST,5,6,4,ORDE,6
FITEM,5,2
FITEM,5,20
FITEM,5,-21
FITEM,5,26
FITEM,5,29
FITEM,5,-30
CM,_Y,LINE
LSEL,, , ,P51X
CM,_Y1,LINE
CMSEL,,_Y
!*
LESIZE,_Y1, , ,40, , , ,1
!*
FLST,5,9,4,ORDE,2
FITEM,5,11
FITEM,5,-19
CM,_Y,LINE
LSEL,, , ,P51X
CM,_Y1,LINE
CMSEL,,_Y
!*
LESIZE,_Y1, , ,20, , , ,1
!*

! Meshing
FLST,5,18,5,ORDE,2
FITEM,5,1
FITEM,5,-18
CM,_Y,AREA
ASEL,, , ,P51X
CM,_Y1,AREA
CHKMSH,'AREA'
CMSEL,S,_Y
!*
MSHKEY,1
AMESH,_Y1
MSHKEY,0
!*
CMDELE,_Y
CMDELE,_Y1
CMDELE,_Y2
!*
```



!Modify the safe end ID

FLST,2,6,5,ORDE,2

FITEM,2,1

FITEM,2,-6

ACLEAR,P51X

FLST,2,6,5,ORDE,2

FITEM,2,1

FITEM,2,-6

ADELE,P51X

FLST,2,9,4,ORDE,7

FITEM,2,9

FITEM,2,14

FITEM,2,-17

FITEM,2,26

FITEM,2,-27

FITEM,2,30

FITEM,2,-31

LDELE,P51X,,1

FLST,2,3,4,ORDE,3

FITEM,2,10

FITEM,2,28

FITEM,2,32

LDELE,P51X,,1

FLST,3,2,3,ORDE,2

FITEM,3,3

FITEM,3,6

KGEN,2,P51X,,,-ro2+ri21,, ,0

FLST,3,1,3,ORDE,1

FITEM,3,2

KGEN,2,P51X,, ,L22,, ,0

FLST,3,3,3,ORDE,3

FITEM,3,1

FITEM,3,-2

FITEM,3,4

KGEN,2,P51X,, ,tv,, ,0

FLST,3,2,3,ORDE,2

FITEM,3,10

FITEM,3,-11

KGEN,2,P51X,, ,-(L3-L21), , ,0

FLST,3,1,3,ORDE,1

FITEM,3,23

KGEN,2,P51X,, ,5,, ,0

LSTR, 23, 40

FLST,2,2,4,ORDE,2

FITEM,2,9

FITEM,2,12

LPTN,P51X

LDELE, 16,, ,1

FLST,2,4,3

FITEM,2,11

FITEM,2,23

FITEM,2,41

FITEM,2,12

A,P51X



FLST,2,4,3
FITEM,2,23
FITEM,2,8
FITEM,2,9
FITEM,2,41
A,P51X
FLST,2,4,3
FITEM,2,8
FITEM,2,7
FITEM,2,6
FITEM,2,9
A,P51X
FLST,2,4,3
FITEM,2,7
FITEM,2,5
FITEM,2,3
FITEM,2,6
A,P51X
FLST,2,4,3
FITEM,2,10
FITEM,2,20
FITEM,2,23
FITEM,2,11
A,P51X
FLST,2,4,3
FITEM,2,20
FITEM,2,4
FITEM,2,8
FITEM,2,23
A,P51X
FLST,2,4,3
FITEM,2,4
FITEM,2,2
FITEM,2,7
FITEM,2,8
A,P51X
FLST,2,4,3
FITEM,2,2
FITEM,2,1
FITEM,2,5
FITEM,2,7
A,P51X
FLST,5,8,5,ORDE,4
FITEM,5,1
FITEM,5,-6
FITEM,5,19
FITEM,5,-20
CM,_Y,AREA
ASEL, , , P51X
CM,_Y1,AREA
CMSEL,S,_Y
!*
CMSEL,S,_Y1
AATT, 1, , 1, 0,
CMSEL,S,_Y



CMDELE,_Y
CMDELE,_Y1
!*
FLST,5,4,4,ORDE,4
FITEM,5,15
FITEM,5,-16
FITEM,5,26
FITEM,5,28
CM,_Y,LINE
LSEL,, ,P51X
CM,_Y1,LINE
CMSEL,,_Y
!*
LESIZE,_Y1,, ,15,, , ,1
!*
FLST,5,4,4,ORDE,4
FITEM,5,31
FITEM,5,48
FITEM,5,50
FITEM,5,52
CM,_Y,LINE
LSEL,, , ,P51X
CM,_Y1,LINE
CMSEL,,_Y
!*
LESIZE,_Y1,, ,2,, , , ,1
!*
FLST,5,6,4,ORDE,6
FITEM,5,9
FITEM,5,-10
FITEM,5,12
FITEM,5,14
FITEM,5,30
FITEM,5,32
CM,_Y,LINE
LSEL,, , ,P51X
CM,_Y1,LINE
CMSEL,,_Y
!*
LESIZE,_Y1,, ,6,, , , ,1
!*
FLST,5,3,4,ORDE,3
FITEM,5,11
FITEM,5,17
FITEM,5,49
CM,_Y,LINE
LSEL,, , ,P51X
CM,_Y1,LINE
CMSEL,,_Y
!*
LESIZE,_Y1,, ,12,, , , ,1
!*
FLST,5,3,4,ORDE,3
FITEM,5,27
FITEM,5,29

```
FITEM,5,51
CM,_Y,LINE
LSEL,,,P51X
CM,_Y1,LINE
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LESIZE,_Y1,,,25,,,,,1
!*
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FITEM,5,-6
FITEM,5,19
FITEM,5,-20
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ASEL,,,P51X
CM,_Y1,AREA
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CMSEL,S,_Y
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MSHKEY,1
AMESH,_Y1
MSHKEY,0
!*
CMDELE,_Y
CMDELE,_Y1
CMDELE,_Y2
!*

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ACLEAR,P51X

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lplo
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FITEM,2,41
FITEM,2,43
FITEM,2,53
LPTN,P51X
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FITEM,2,-61
LDELE,P51X,,,1
FLST,2,4,4
FITEM,2,54
FITEM,2,62
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FITEM,2,44
AL,P51X
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FITEM,2,63
FITEM,2,58
FITEM,2,45
AL,P51X
FLST,2,4,4
FITEM,2,63
FITEM,2,56
FITEM,2,57
FITEM,2,46
AL,P51X
FLST,2,4,4
FITEM,2,47
FITEM,2,59
FITEM,2,57
FITEM,2,62
AL,P51X

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CMDELE,_Y1
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FITEM,5,58
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LESIZE,_Y1,, ,8, , , , ,1
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AMESH,_Y1
MSHKEY,0
!*
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CMDELE,_Y1
CMDELE,_Y2
!*
FLST,5,2,5,ORDE,2
FITEM,5,21
FITEM,5,-22
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ASEL,, , ,P51X
CM,_Y1,AREA
CHKMSH,'AREA'
CMSEL,S,_Y
!*
MSHKEY,1
AMESH,_Y1
MSHKEY,0
!*
```



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CMDELE,_Y1
CMDELE,_Y2
!*
!

!Simulating Butter
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FLST,2,2,5,ORDE,2
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ADELE,P51X

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KGEN,2,44,,, -0.25,,0
KGEN,2,14,,,11/16-1.375*tan(7.5),,0
KGEN,2,46,,, -0.25,,0

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LSTR, 17, 46
LSTR, 46, 47
LSTR, 47, 14
LSTR, 46, 44
LSTR, 45, 47
LSTR, 13, 16

FLST,3,2,3,ORDE,2
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FITEM,3,-47
KGEN,2,P51X,,, -0.25,,,0

LSTR, 48, 46
LSTR, 49, 47

FLST,2,3,4,ORDE,3
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FITEM,2,64
FITEM,2,-65

LPTN,P51X
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FITEM,2,-71
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FITEM,2,68
FITEM,2,3
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FLST,2,4,4
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FITEM,2,69
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FITEM,2,43
AL,P51X

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!*
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CMSEL,S,_Y
CMDELE,_Y
CMDELE,_Y1
!*
FLST,5,3,5,ORDE,3
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FITEM,5,23
FITEM,5,-24
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ASEL,,,P51X
CM,_Y1,AREA
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!*
CMSEL,S,_Y1
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CMDELE,_Y

```
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!*
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ASEL,, ,P51X
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CMSEL,S,_Y
!*
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!*
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FITEM,5,72
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CMSEL,,_Y
!*
LESIZE,_Y1, , ,2, , , ,1
!*
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FITEM,5,-10
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CM,_Y1,AREA
CHKMSH,'AREA'
CMSEL,S,_Y
!*
MSHKEY,1
AMESH,_Y1
MSHKEY,0
!*
CMDELE,_Y
```



```
CMDELE,_Y1
CMDELE,_Y2
!*
FLST,5,4,5,ORDE,2
FITEM,5,23
FITEM,5,-26
CM,_Y,AREA
ASEL,, ,P51X
CM,_Y1,AREA
CHKMSH,'AREA'
CMSEL,S,_Y
!*
MSHKEY,1
AMESH,_Y1
MSHKEY,0
!*
CMDELE,_Y
CMDELE,_Y1
CMDELE,_Y2
!*
```

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save
finish
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Structural Integrity Associates, Inc.

File No.: VY-16Q-305

CALCULATION PACKAGE

Project No.: VY-16Q **NEC-JH_08**

PROJECT NAME:

Environmental Fatigue Analysis of VYNPS

CONTRACT NO.:

10150394

CLIENT:

Entergy Nuclear Operations, Inc.

PLANT:

Vermont Yankee Nuclear Power Station

CALCULATION TITLE:

Recirculation Outlet Stress History Development for Nozzle Green Function

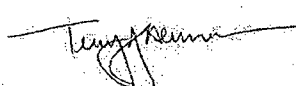
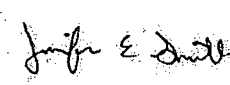

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1.0 OBJECTIVE

The objective of this calculation is to compute the pressure stresses, thermal stresses, and the Green's Functions for high (100%), mid (50%), and no (0%) flow thermal loading of the Vermont Yankee Nuclear Power Station recirculation outlet nozzle.

2.0 RECIRCULATION OUTLET NOZZLE MODEL

An axisymmetric finite element model of the recirculation outlet nozzle was developed in Reference [1] using ANSYS [2]. The geometry and model in Reference [1] is used in this calculation. The material properties are taken at an average temperature of 300°F. This average temperature is based on a thermal shock of 500°F to 100°F which will be applied to the FE model for Green's Function development. Table 1 listed the material properties at 300°F. The meshed model is shown in Figure 1.

3.0 APPLIED LOADS

Both pressure and thermal loads will be applied to the finite element model.

3.1 Pressure Load

A uniform pressure of 1000 psi was applied along the inside surface of the recirculation outlet nozzle and the vessel wall. A pressure load of 1000 psi was used because it is easily scaled up or down to account for different pressures that occur during transients. In addition, a cap load was applied to the piping at the end of the nozzle. This cap load was calculated as follows:

$$P_{cap} = P \cdot \frac{(D_i^2)}{(D_o^2 - D_i^2)}$$

where:

- P = Pressure = 1,000 psi
- D_i = Inner Radius = 12.96875 in
- D_o = Outer Radius = 14.18750 in
- P_{cap} = Tension stress on the end of the nozzle. (psi)

Therefore, the cap load is 5081.7 psi. The calculated value was given a negative sign in order for it to exert tension on the end of the model. The ANSYS input file VY_RON_P.INP, in the computer files, applies the pressure loading to the geometry in file RON_VY.INP. Figures 2, 3, and 4 show the internal pressure distribution, cap load, and symmetry condition applied to the vessel end of the model, respectively.

3.2 Thermal Load

Thermal loads are applied to the recirculation outlet nozzle model. The heat transfer coefficients after power uprate were determined by scaling the values from Reference [4]. These values were determined for various regions of the finite element model and for 100% (28,294 GPM, converted from 12.3 Mlbm/hr [7]), 50% (14,147 GPM), and 0% (0 GPM) flow rates. The temperatures used are based upon a thermal shock from 500°F to 100°F. The calculated heat transfer coefficients for each region are shown below. The GPM values are calculated from the Mlbm/hr values at an average temperature of 300°F.

3.2.1 Heat Transfer Coefficients

The heat transfer coefficients for the 100% flow and 50% flow cases were calculated from Reference [4] as follows:

$$h_{Df} = h_{300} \left(\frac{f_{Df}}{25} \right)^{0.8} \left(\frac{26}{D_{Df}} \right)^{0.2}$$

Where:

- h_{Df} = the heat transfer coefficient at a Diameter and flow rate
- h_{300} = the heat transfer coefficient from Reference [4] at 300°F
- f_{Df} = the flow rate corresponding to h_{Df} (ft/sec)
- D_{Df} = the diameter corresponding to h_{Df} (in)

The heat transfer coefficients for 0% flow were calculated in spreadsheet Ht_coefs.xls for natural convection and are shown in Tables 3 and 4.

As shown in Figure 5, the following heat transfer coefficients were applied:

Region 1

The heat transfer coefficient, h , for 100% flow is $4789 \left(\frac{17.364}{25} \right)^{0.8} = 3577.8$ BTU/hr-ft²-°F at 300°F. [4]

where 17.364 ft/sec is converted from 28,294 GPM and 25.8 in ID.

The heat transfer coefficient, h , for 50% flow is $4789 \left(\frac{8.682}{25} \right)^{0.8} = 2054.9$ BTU/hr-ft²-°F at 300°F. [4]

where 8.682 ft/sec is converted from 14,147 GPM and 25.8 in ID.

The heat transfer coefficient, h , for 0% flow is 112.34 BTU/hr-ft²-°F at 300°F. [Table 3, for natural convection]



Region 2

The heat transfer coefficient for Region 2 is linearly transitioned from the value of the heat transfer coefficient used in Region 1 to the value used for Region 3.

Region 3 (the point between Region 2 and Region 4)

The heat transfer coefficient, h, for 100% flow is $4789 \left(\frac{17.364}{25} \right)^{0.8} \cdot \left(\frac{26}{35.49} \right)^{0.2} = 3361$
BTU/hr-ft²-°F at 300°F. [4]

where the flow rate is the same as that for Region 1, and the ID is 35.49 in.

The heat transfer coefficient, h, for 50% flow is $4789 \left(\frac{8.682}{25} \right)^{0.8} \cdot \left(\frac{26}{35.49} \right)^{0.2} = 1930.9$
BTU/hr-ft²-°F at 300°F. [4]

where the flow rate is the same as that for Region 1, and the ID is 35.49 in.

The heat transfer coefficient, h, for 0% flow is 112.34 BTU/hr-ft²-°F at 300°F. using the same HTC as Region 1 [Table 3, for natural convection]

Region 4

Per Reference [1], the heat transfer coefficient for Region 4 (Nozzle Blend Radius) is linearly transitioned from the value of the heat transfer coefficient used in Region 3 to the value used in Region 5.

Region 5

The heat transfer coefficient, h, for 100% flow is $0.5 \times 3577.8 = 1788.9$ BTU/hr-ft²-°F at 300°F. [4]

The heat transfer coefficient, h, for 50% flow is $0.5 \times 2054.9 = 1027.4$ BTU/hr-ft²-°F at 300°F. [4]

The heat transfer coefficient, h, for 0% flow is 101 BTU/hr-ft²-°F at 300°F. [Table 4, for natural convection] by using 40 in. hydraulic diameter [4].



Region 6

The heat transfer coefficient, h , is $0.4 \text{ BTU/hr-ft}^2\text{-}^\circ\text{F}$ [4].

3.2.2 Boundary Fluid Temperatures

For the Green's Functions, a 500°F to 100°F thermal shock is run to determine the stress response to a one-degree change in temperature. The following temperatures are valid when there is water flow. Values between defined points are linearly interpolated. For the 100%, 50%, and 0% flow cases, the thermal shock is run as follows:

Regions 1 to 5

$$T = 500^\circ\text{F} - 100^\circ\text{F}$$

Region 6

$$T = 120^\circ\text{F}$$

4.0 THERMAL AND PRESSURE LOAD RESULTS

The three flow dependent thermal load cases outlined in Section 3.0 were run on the finite element model. Appendix A contains the thermal transient input files VY_RON_T_100.INP, VY_RON_T_50.INP, and VY_RON_T_0.INP for 100%, 50%, and 0% flow rates, respectively. The three flow dependent input files for the stress runs are also included in Appendix A. The stress filenames are VY_RON_S_100.INP, VY_RON_S_50.INP, and VY_RON_S_0.INP for 100%, 50%, and 0% flow rates, respectively.

The critical safe end location was chosen as node 6395, which has the highest stress intensity due to thermal loading under high flow conditions. As shown in Figures 6 and 7, Node 6395 is located on the inside diameter of the nozzle safe end of the model and the maximum stress occurs at 5.1 seconds.

The critical blend radius location was chosen, based upon the highest pressure stress. Assumed the cladding has cracked, therefore, as shown in Figures 8 and 9, the critical location is selected as node 3829 at base metal of the nozzle.

The stress intensity for use in the Green's functions are calculated from the component stresses (X, Y, and Z) and compared to the stress intensity reported by ANSYS. As seen in Figure 10, the Y-X calculated total stress intensity best matches the ANSYS reported stress intensity for 100% flow at the safe end. Therefore, the Y-X stress will be used for the total and membrane plus bending Green's functions for all flow rates for the safe end. As seen in Figure 11, the Z-X calculated total stress intensity best matches the ANSYS reported stress intensity for 100% flow at the blend radius in very beginning. Therefore, the Z-X stress will be used for the total and membrane plus bending Green's functions for all flow rates for the blend radius.



The stress time history for the critical paths was extracted during the stress run for 100% flow rate. This produced two files, HFSE.OUT and HFBR.OUT, which contain the thermal stress history. The membrane plus bending stresses and total stresses for the Green's Functions were extracted from these files to produce the files HFSE_Inside.RED and HFBR_Inside.RED, where SE and BR corresponded to the safe end and blend radius locations, respectively. The total stress intensity (SI) was extracted from these files to produce the files HFSE.CLD and HFBR.CLD, where SE and BR corresponded to the safe end and the blend radius, respectively.

The stress time history for the critical paths was extracted during the stress run for 50% flow rate. This produced two files, MFSE.OUT and MFBR.OUT which contains the thermal stress history. The membrane plus bending stresses and total stresses for the Green's Functions were extracted from the file to produce the file MFSE_Inside.RED, where SE corresponds to the safe end location.

The stress time history for the critical paths was extracted during the stress run for 0% flow rate. This produced two files, LFSE.OUT and LFBR.OUT which contain the thermal stress history. The membrane plus bending stresses and total stresses for the Green's Functions were extracted from the file to produce the file LFSE_Inside.RED, where SE corresponds to the safe end location.

The stress time history for the recirculation outlet nozzle during 100% flow, 50% flow, and 0% flow are shown in Figures 12 to 23. The data for the Green's Functions is included in the files HFBR_M+B-Green.xls, HFBR_T-Green.xls, HFSE_M+B-Green.xls, HFSE_T-Green.xls, MFBR_M+B-Green.xls, MFBR_T_Green.xls, MFSE_M+B-Green.xls, MFSE_T-Green.xls, LFBR_M+B-Green.xls, LFBR_T-Green.xls, LFSE_M+B-Green.xls, and LFSE_T-Green.xls in the project Files. Where HF, MF, and LF corresponded to 100% flow, 50% flow, and 0% flow rate, respectively. M+B and T corresponded to membrane plus bending stress and total stress, respectively.

The pressure stress intensities for the path were extracted during the pressure run. The pressure stresses were extracted along the nodal path as shown in Figures 7 and 9. This produced two files, PSE.OUT and PBR.OUT for the safe end and blend radius locations, respectively.

For the pressure loading specified (1000 psig), the total stress intensities at Node 6395 and Node 3829 were determined to be 11490 psi and 31300 psi, respectively. The membrane plus bending stress intensities at Node 6395 and Node 3829 were determined to be 11350 psi and 33640 psi, respectively. Table 2 shows the final pressure results.

Results were also extracted from the vessel portion of the model to verify the accuracy of the results obtained from the ANSYS model, and to check the results due to the use of the 2.0 multiplier on the vessel radius. These results are contained in the file PVESS.OUT. The radius of the finite element model (FEM) was multiplied by a factor of 2.0 [1] to account for the fact that the vessel portion of the 2D axisymmetric model is a sphere but the true geometry is the intersection of two cylinders.



The equation for the membrane hoop stress for a sphere is:

$$\sigma = \left(\frac{(\text{pressure}) \times (\text{radius})}{2 \times \text{thickness}} \right)$$

Considering a vessel base metal radius, R, of 105.906 inches increased by a factor of 2.0, a vessel base metal thickness, t, of 5.4375 inches, and an applied pressure, P, of 1,000 psi, the calculated stress for a sphere is $PR/(2t) = 19,477$ psi. This compares very well with the remote vessel wall membrane hoop stress from the ANSYS result file, PVESS.OUT, of 19,540 psi. Thus, considering the peak total pressure stress of 31,300 psi reported above, the stress concentrating effect of the nozzle corner is $31,300/19,477 = 1.61$. In other words, the peak nozzle corner stress is 1.61 times higher than nominal vessel wall stress for the 2D axisymmetric model.

The equation for the membrane hoop stress in a cylinder is:

$$\sigma = \left(\frac{(\text{pressure}) \times (\text{radius})}{\text{thickness}} \right)$$

Based on the previous dimensions, the calculated stress for a cylinder without the 2.0 factor is 19,477 psi. Increasing this by a factor of 1.61 yields an expected peak nozzle corner stress of 31,358 psi, which would be expected from a cylindrical geometry that is representative of the nozzle configuration. Therefore, the result from the ANSYS file for the peak nozzle corner stress (31,300 psi) is close to the peak nozzle corner stress for a cylindrical geometry because of the use of the 2.0 multiplier. This is consistent with SI's experience where a factor of two increase in radius is typical for representing the three-dimensional (3D) effect in a 2D axisymmetric model.

5.0 REFERENCES

1. SI Calculation No. VY-16Q-304, Revision 0, "Recirculation Outlet Nozzle Finite Element Model"
2. ANSYS, Release 8.1 (w/Service Pack 1), ANSYS, Inc., June 2004.
3. American Society of Mechanical Engineers, Boiler and Pressure Vessel Code, Section II, Part D, 1998 Edition, 2000 Addenda.
4. CB&I, RPV Stress Report Section: T9 "Thermal Analysis Recirculation Outlet Nozzle Vermont Yankee Reactor Vessel." 9-6201, SI document, VY-16Q-204.
5. J. P. Holman, "Heat Transfer," 4th Edition, McGraw-Hill, 1976.
6. J. P. Holman, "Heat Transfer," 5th Edition, 1981.
7. Entergy Nuclear Northeast Engineering Report, Report No. VY-RPT-05-00022, "Task T0100 Reactor Heat Balance EPU Task Report for ER-04-1409," SI File No. VY-16Q-205.



Table 1: Material Properties @ 300°F⁽¹⁾

Material	SA-533 Gr B (Mn-1/2Mo- 1/2Ni)	SA-508 Cl 2 (3/4Ni-1/2Mo- 1/3Cr-V)	SA-240 Type 304 (18Cr-8Ni)	SA-182 F316/ SA 376 TP316 (16Cr-12Ni- 2Mo)
Modulus of Elasticity, e ⁻⁶ psi	28.0	26.7	27.0	27.0
Coefficient of Thermal Expansion, e ⁻⁶ , in/in/°F	7.7	7.3	9.8	9.8
Thermal Conductivity, Btu/hr-ft-°F	23.4	23.4	9.8	9.3
Thermal Diffusivity, ft ² /hr	0.401	0.401	0.160	0.150
Calculated Specific Heat, Btu/lb-°F ⁽²⁾	0.119	0.119	0.125	0.127
Density, lb/in ³	0.283	0.283	0.283	0.283
Poisson's Ratio	0.3	0.3	0.3	0.3

Notes: ⁽¹⁾The material properties applied in the analyses are taken from ASME Section II Part D 1998 Edition with 2000 Addenda. This is consistent with information provided in the Design Input Record (page 13 of VY EC No. 1773, SI File No. VY-16Q-209). The use of a later code edition than that used for the original design code is acceptable since later editions typically reflect more accurate material properties than was published in prior Code editions. Material Properties are evaluated at 300°F from the 1998 ASME Code, Section II, Part D, with 2000 Addenda, except for density and Poisson's ratio, which are assumed typical values.

⁽²⁾ Calculated as $[k/(pd)]/12^3$.

Table 2: Pressure Results

Location	Membrane Plus Bending Stress Intensity (psi)	Total Stress Intensity (psi)
Safe End	11350	11490
Blend Radius	33640	31300

Table 3: 0% Flow Regions 1 and 3 Heat Transfer Coefficients

Pipe Inside Diameter, D =	25.800	inches =	2.150	ft					
			=	0.655	m				
Outer Pipe, Inside radius, r _o =	12.9	inches =	1.075	ft					
			=	0.328	m				
Inner Pipe Outside Diameter, D =	n/a	inches =	0.000	ft					
			=	0.000	m				0.000
Inner Pipe, Outside radius, r _i =	0	inches =	0.000	ft					
			=	0.000	m				
Fluid Velocity, V =	17.364	ft/sec =	28293.595	gpm =	12.3	Mlb/hr			
Characteristic Length, L = D =	2.150	ft =	0.655	m					
(Outside) T _{fluid} - T _{surface} , ΔT =	8.40	12.00	24.00	36.00	48.00	60.00	72.00	°F	
=	4.67	6.67	13.33	20.00	26.67	33.33	40.00	°C	

Water Property	Conversion Factor [1]	Value at Fluid Temperature, T [3]							Units
		70	100	200	300	400	500	600	
k	1.7307	0.5997	0.6300	0.6784	0.6836	0.6611	0.6040	0.5071	W/m ² ·°C
(Thermal Conductivity)		0.3465	0.3640	0.3920	0.3950	0.3820	0.3490	0.2930	Btu/hr-ft ² ·°F
c _p	4.1869	4.185	4.179	4.229	4.313	4.522	4.982	6.322	kJ/kg·°C
(Specific Heat)		1.000	0.998	1.010	1.030	1.080	1.190	1.510	Btu/lbm·°F
ρ	16.018	997.1	994.7	962.7	917.8	858.6	784.9	679.2	kg/m ³
(Density)		62.3	62.1	60.1	57.3	53.6	49.0	42.4	lbm/ft ³
β	1.8	1.89E-04	3.24E-04	6.66E-04	1.01E-03	1.40E-03	1.98E-03	3.15E-03	m ³ /m ³ ·°C
(Volumetric Rate of Expansion)		1.05E-04	1.80E-04	3.70E-04	5.60E-04	7.80E-04	1.10E-03	1.75E-03	ft ³ /ft ³ ·°F
g	0.3048	9.806	9.806	9.806	9.806	9.806	9.806	9.806	m/s ²
(Gravitational Constant)		32.17	32.17	32.17	32.17	32.17	32.17	32.17	ft/s ²
μ	1.4881	9.96E-04	6.82E-04	3.07E-04	1.93E-04	1.38E-04	1.04E-04	8.62E-05	kg/m·s
(Dynamic Viscosity)		6.69E-04	4.58E-04	2.06E-04	1.30E-04	9.30E-05	7.00E-05	5.79E-05	lbm/ft·s
Pr		6.980	4.510	1.910	1.220	0.950	0.859	1.070	---
(Prandtl Number)									
Calculated Parameter	Formula	70	100	200	300	400	500	600	°F
Reynold's Number, Re	ρVD/μ	3473691	5061789	10891437	16454670	21515912	26132199	27337904	---
Grashof Number, Gr	gβΔTL ³ /(μρ) ²	2441754517	1.2697E+10	2.417E+11	1.252E+12	3.977E+12	1.034E+13	2.16049E+13	---
Grashof Number, Gr _s	gβΔT(r _o -r _i) ³ /(μρ) ³	3.05E+08	1.59E+09	3.02E+10	1.57E+11	4.97E+11	1.29E+12	2.70E+12	---
Rayleigh Number, Ra	GrPr	17043446531	5.7265E+10	4.616E+11	1.528E+12	3.778E+12	8.883E+12	2.31172E+13	---
Rayleigh Number, Ra	Gr _s Pr	2.13E+09	7.16E+09	5.77E+10	1.91E+11	4.72E+11	1.11E+12	2.89E+12	---
From [1]:									
Inside Surface Forced Convection Heat Transfer Coefficient:									
H _{forced}	0.023Re ^{0.8} Pr ^{0.4} /kD	7,823.02	9,326.34	13,148.12	15,405.24	16,705.40	17,126.15	16,275.32	W/m ² ·°C
		1,377.74	1,642.50	2,315.56	2,713.07	2,942.05	3,016.15	2,866.31	Btu/hr-ft ² ·°F
From [1]:									
Inside Surface Natural Convection Heat Transfer Coefficient:									
Case:	Enclosed cylinder			C =	0.55	n =	0.25		
H _{free}	C(GrPr) ⁿ /kL	181.85	258.65	469.34	637.89	773.57	875.17	933.22	W/m ² ·°C
		32.03	45.55	82.66	112.34	136.24	154.13	164.35	Btu/hr-ft ² ·°F

Table 4: 0% Flow Region 5 Heat Transfer Coefficient
Heat Transfer Coefficients

References: 1. J. P. Holman, "Heat Transfer," 4th Edition, McGraw-Hill, 1976.

2. J. P. Holman, "Heat Transfer," 5th Edition, 1981.

3. N. P. Chermisinoff, "Heat Transfer Pocket Handbook," Gulf Publishing Co., 1984.

(Required Inputs are Shaded!)

Title =	Piping								
Pipe Inside Diameter, D =	40.000	inches =	3.333	ft					
		=	1.016	m					
Outer Pipe, Inside radius, r _o =	20	inches =	1.667	ft					
		=	0.508	m					
Inner Pipe Outside Diameter, D =	n/a	inches =	0.000	ft					
		=	0.000	m				0.000	
Inner Pipe, Outside radius, r _i =	0	inches =	0.000	ft					
		=	0.000	m					
Fluid Velocity, V =	7.224	ft/sec =	28293.595	gpm =	12.3	Mlb/hr			
Characteristic Length, L = D =	3.333	ft =	1.016	m					
(Outside) T _{fluid} - T _{surface} , ΔT =	8.40	12.00	24.00	36.00	48.00	60.00	72.00	°F	
	=	4.67	6.67	13.33	20.00	26.67	33.33	40.00	°C

Water Property	Conversion Factor [1]	Value at Fluid Temperature, T [3]							Units
		70	100	200	300	400	500	600	°F
k	1.7307	21.11	37.78	93.33	148.89	204.44	260.00	315.56	W/m·°C
(Thermal Conductivity)		0.5997	0.6300	0.6784	0.6836	0.6611	0.6040	0.5071	Btu/hr-ft·°F
c _p	4.1869	4.185	4.179	4.229	4.313	4.522	4.982	6.322	kJ/kg·°C
(Specific Heat)		1.000	0.998	1.010	1.030	1.080	1.190	1.510	Btu/lbm·°F
ρ	16.018	997.1	994.7	962.7	917.8	858.6	784.9	679.2	kg/m ³
(Density)		62.3	62.1	60.1	57.3	53.6	49.0	42.4	lbm/ft ³
β	1.8	1.89E-04	3.24E-04	6.66E-04	1.01E-03	1.40E-03	1.98E-03	3.15E-03	m ³ /m ³ ·°C
(Volumetric Rate of Expansion)		1.05E-04	1.80E-04	3.70E-04	5.60E-04	7.80E-04	1.10E-03	1.75E-03	ft ³ /ft ³ ·°F
g	0.3048	9.806	9.806	9.806	9.806	9.806	9.806	9.806	m/s ²
(Gravitational Constant)		32.17	32.17	32.17	32.17	32.17	32.17	32.17	ft/s ²
μ	1.4881	9.96E-04	6.82E-04	3.07E-04	1.93E-04	1.38E-04	1.04E-04	8.62E-05	kg/m-s
(Dynamic Viscosity)		6.69E-04	4.58E-04	2.06E-04	1.30E-04	9.30E-05	7.00E-05	5.79E-05	lbm/ft-s
Pr		6.980	4.510	1.910	1.220	0.950	0.859	1.070	---
(Prandtl Number)									
Calculated Parameter	Formula	70	100	200	300	400	500	600	°F
Reynold's Number, Re	ρVD/μ	2240531	3264854	7024977	10613262	13877763	16855268	17632948	---
Grashof Number, Gr	gβΔT ³ /(μρ) ²	9099611606	4.732E+10	9.01E+11	4.667E+12	1.48E+13	3.85E+13	8.05143E+13	---
Grashof Number, Gr _o	gβΔT(r _o -r _i) ³ /(μρ) ³	1.14E+09	5.91E+09	1.13E+11	5.83E+11	1.85E+12	4.82E+12	1.01E+13	---
Rayleigh Number, Ra	GrPr	6.3515E+10	2.134E+11	1.72E+12	5.694E+12	1.41E+13	3.31E+13	8.61503E+13	---
Rayleigh Number, Ra	Gr _o Pr	7.94E+09	2.67E+10	2.15E+11	7.12E+11	1.76E+12	4.14E+12	1.08E+13	---
From [1]:									
Inside Surface Forced Convection Heat Transfer Coefficient:									
H _{forced}	0.023Re ^{0.8} Pr ^{0.4} /k/D	3,552.89	4,235.64	5,971.33	6,996.42	7,586.90	7,777.99	7,391.58	W/m ² ·°C
		625.71	745.95	1,051.63	1,232.17	1,336.16	1,369.81	1,301.76	Btu/hr-ft ² ·°F
From [1]:									
Inside Surface Natural Convection Heat Transfer Coefficient:									
Case:	Enclosed cylinder			C = 0.55		n = 0.25			
H _{free}	C(GrPr) ⁿ /k/L	162.97	231.79	420.60	571.66	693.25	784.30	836.32	W/m ² ·°C
		28.70	40.82	74.07	100.68	122.09	138.13	147.29	Btu/hr-ft ² ·°F

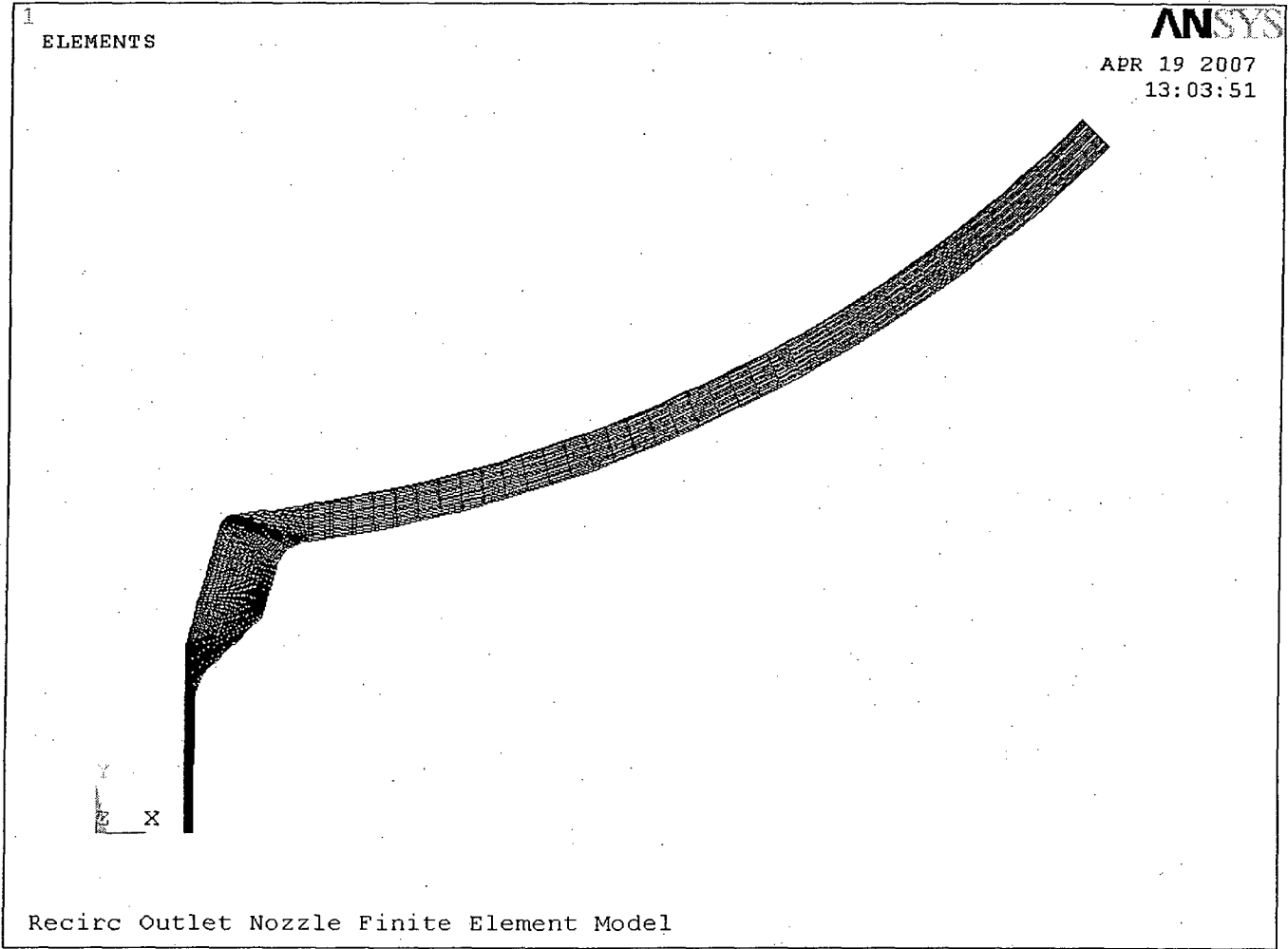


Figure 1: ANSYS Finite Element Model

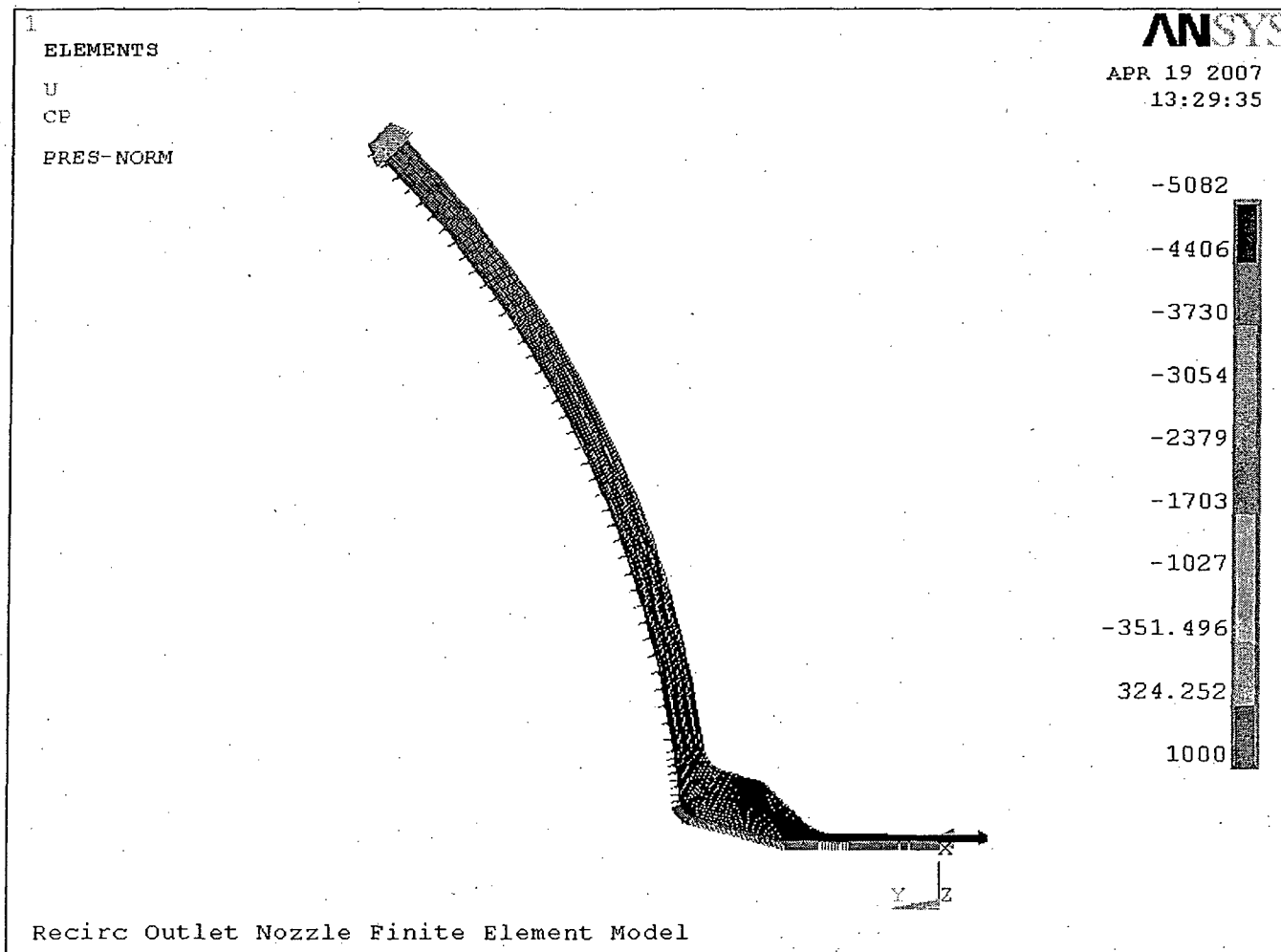


Figure 2: Recirculation Outlet Nozzle Internal Pressure Distribution

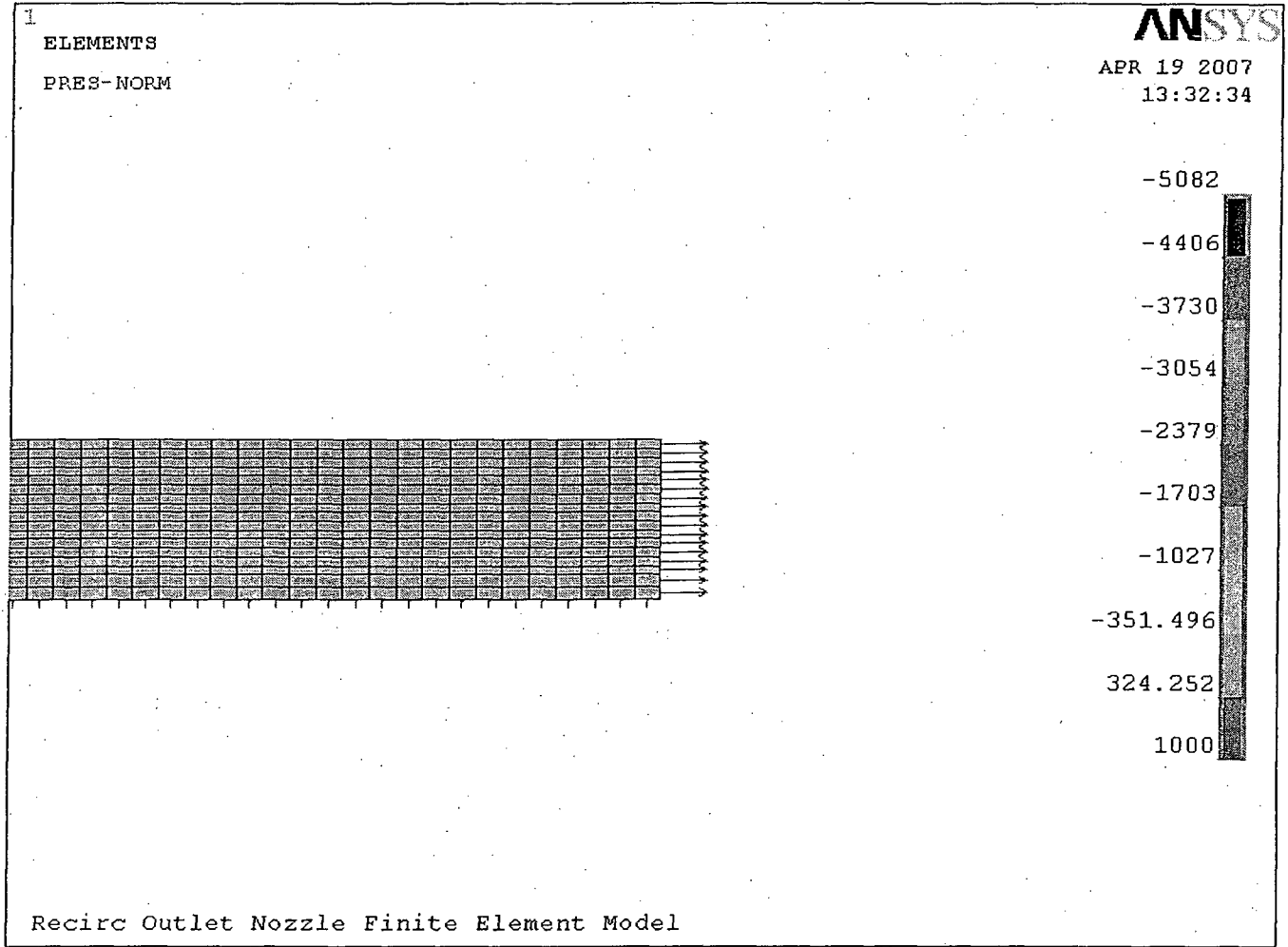


Figure 3: Recirculation Outlet Nozzle Pressure Cap Load

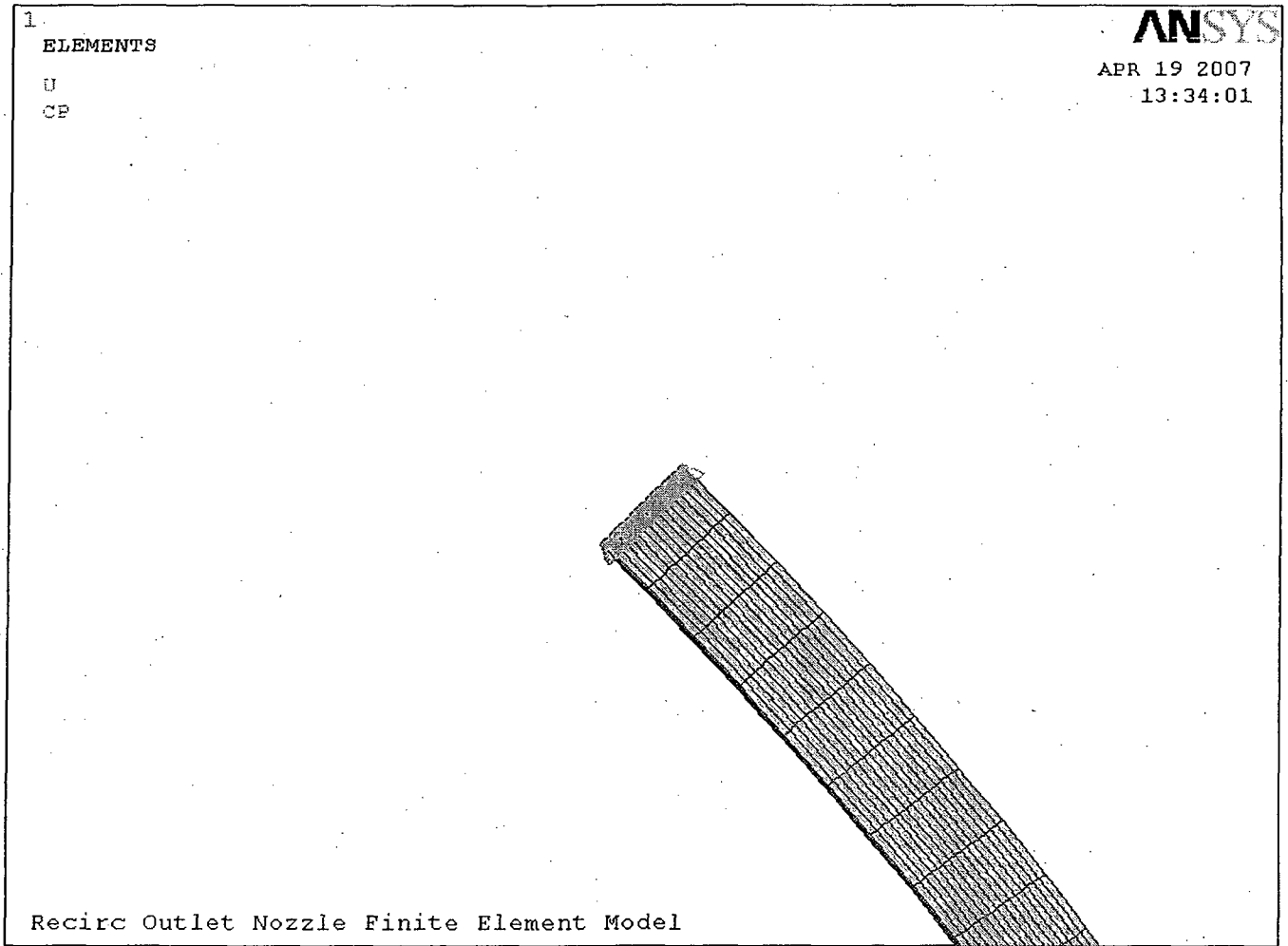


Figure 4: Recirculation Outlet Nozzle Vessel Boundary Conditions

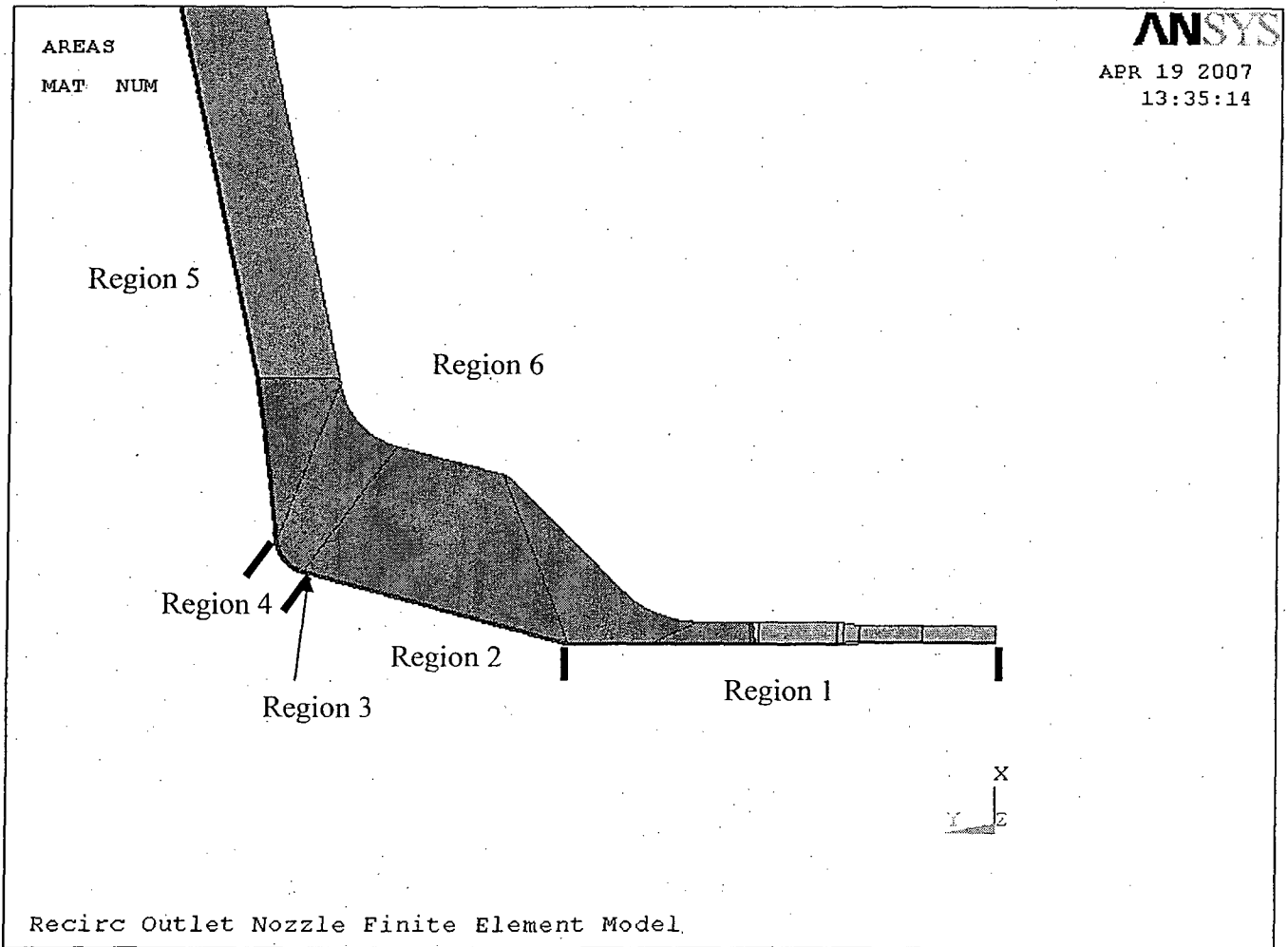


Figure 5: Nozzle and Vessel Wall Thermal and Heat Transfer Boundaries

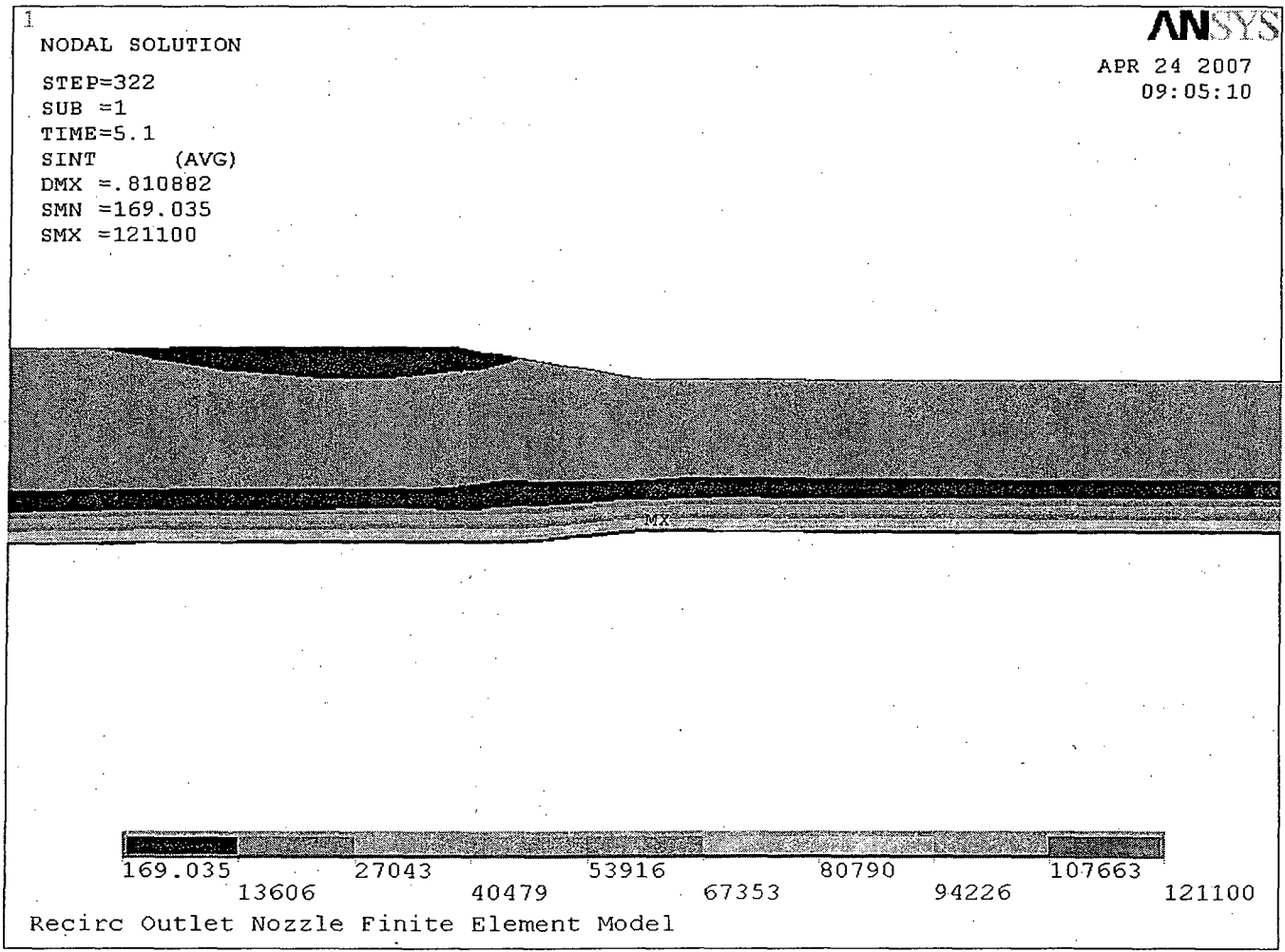


Figure 6: Safe End Critical Thermal Stress Location

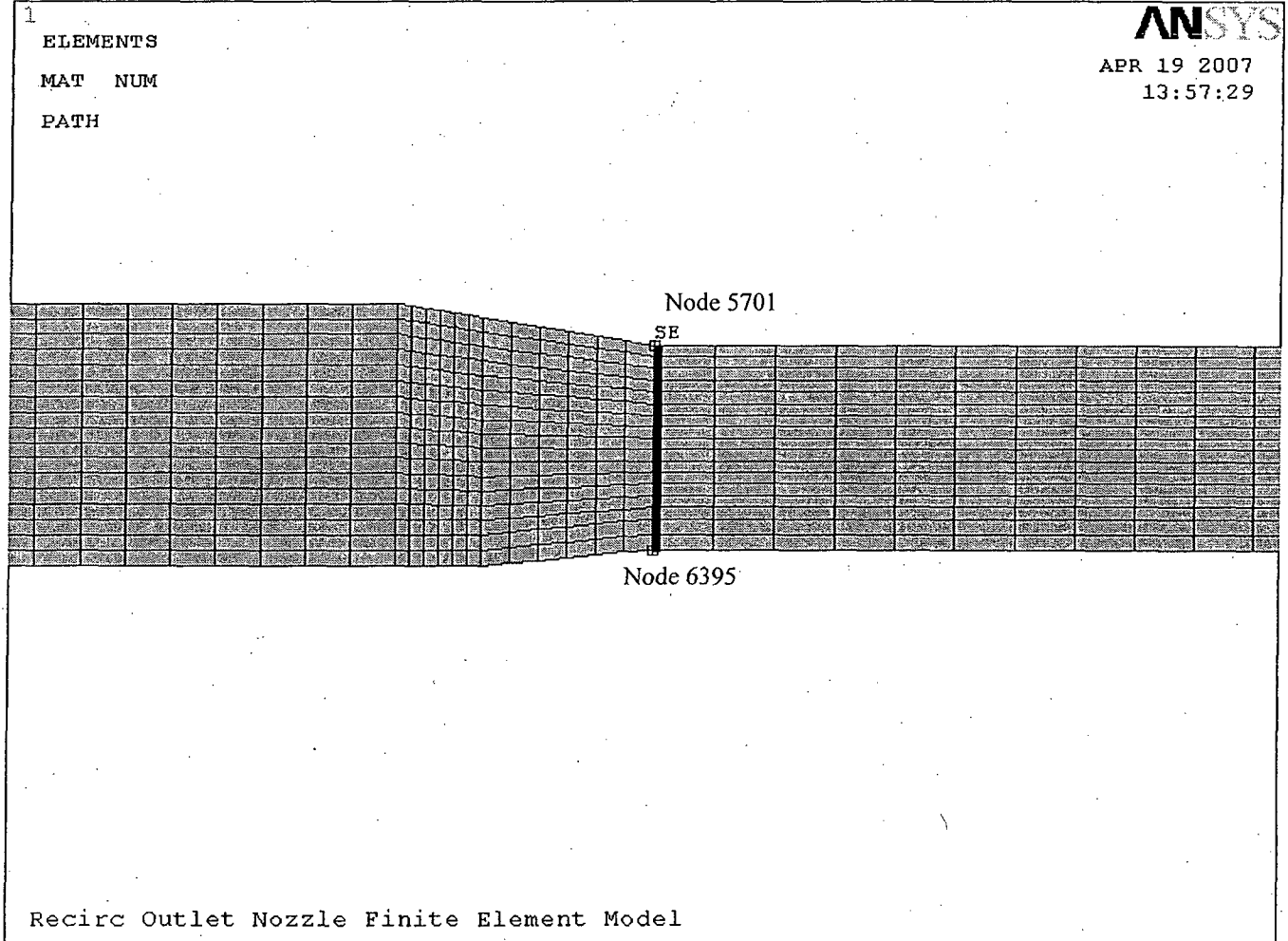


Figure 7: Safe End Limiting Linearized Stress Paths

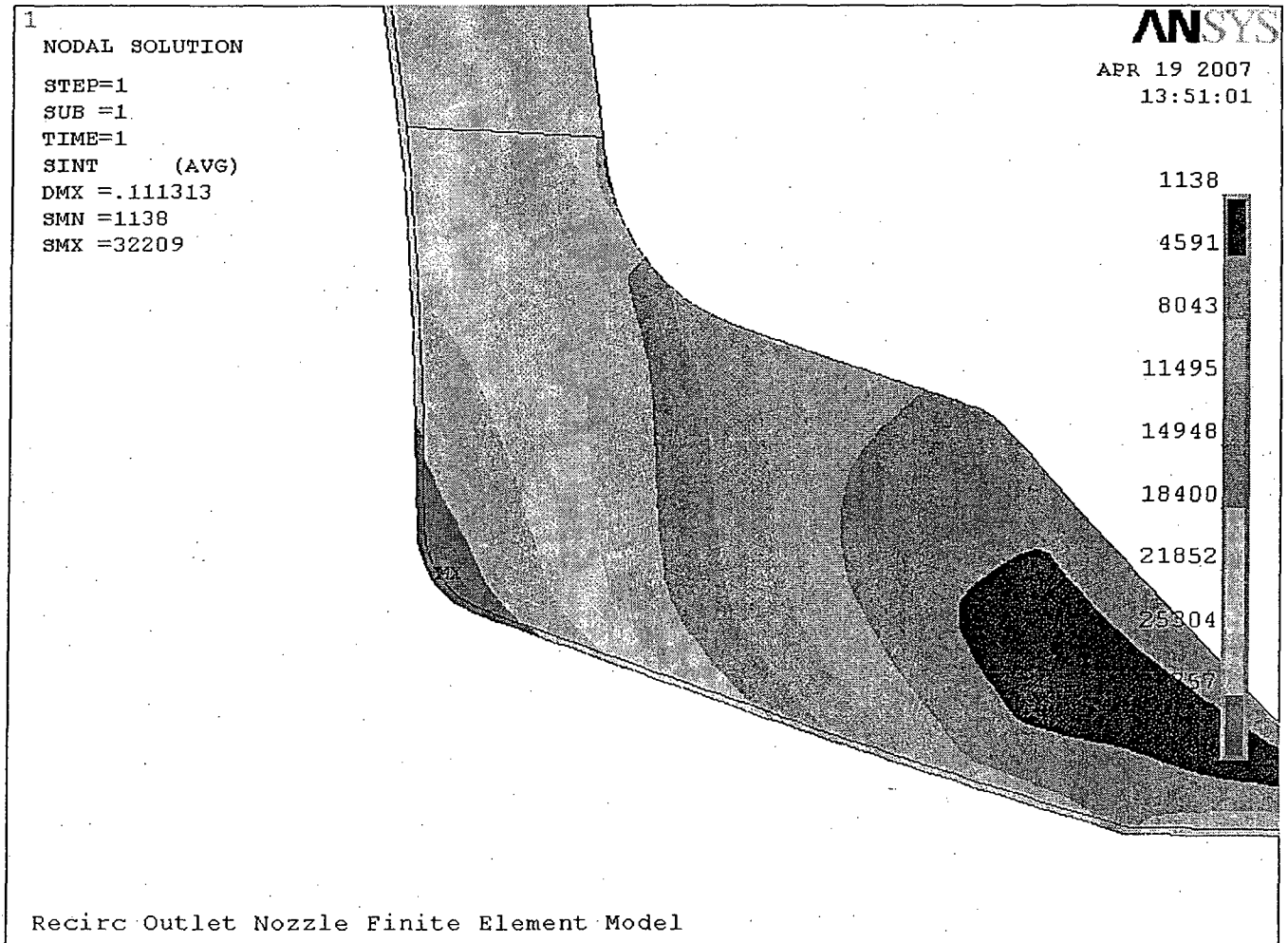


Figure 8: Blend Radius Limiting Pressure Stress Location

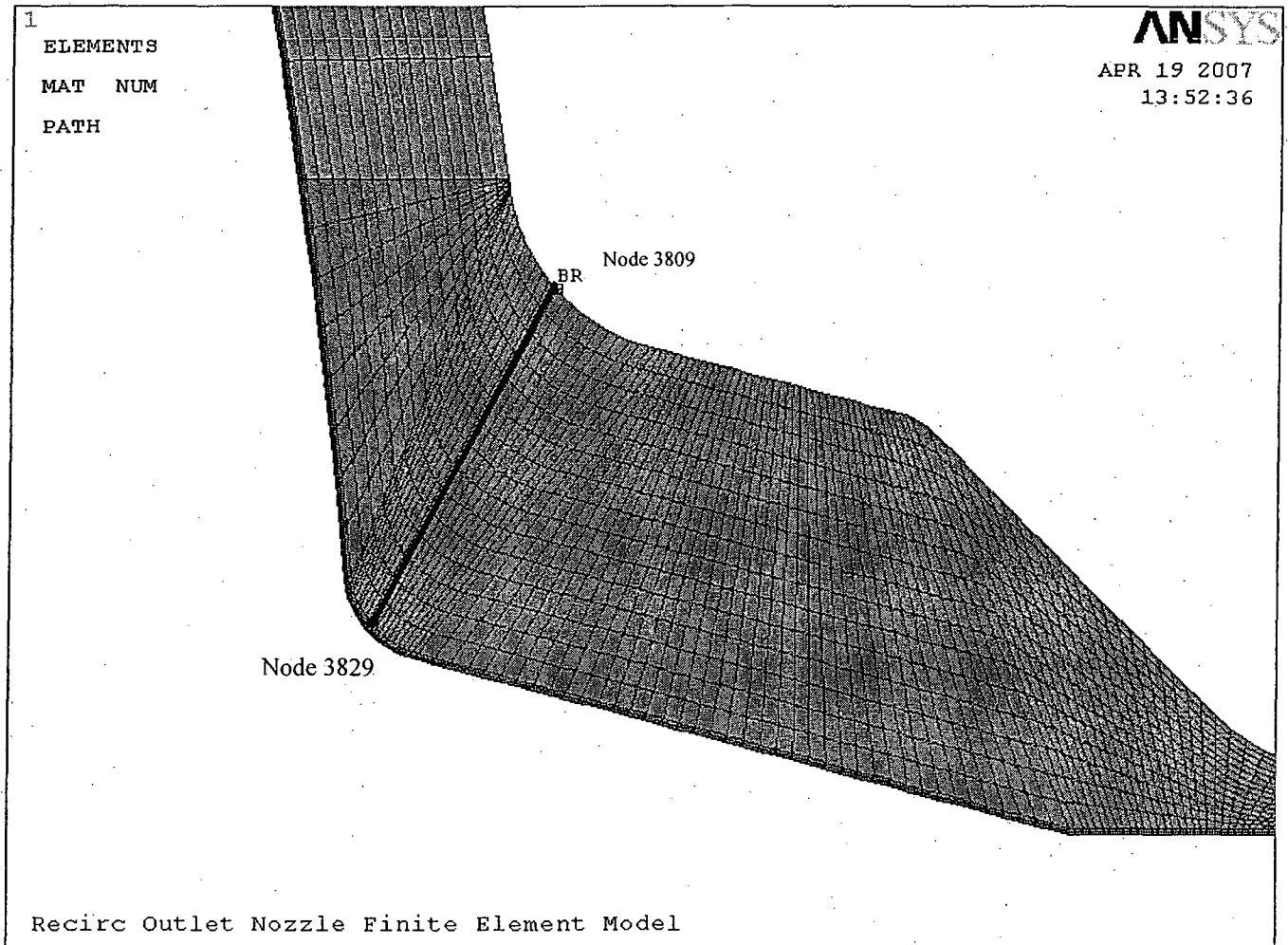


Figure 9: Blend Radius Linearized Stress Path

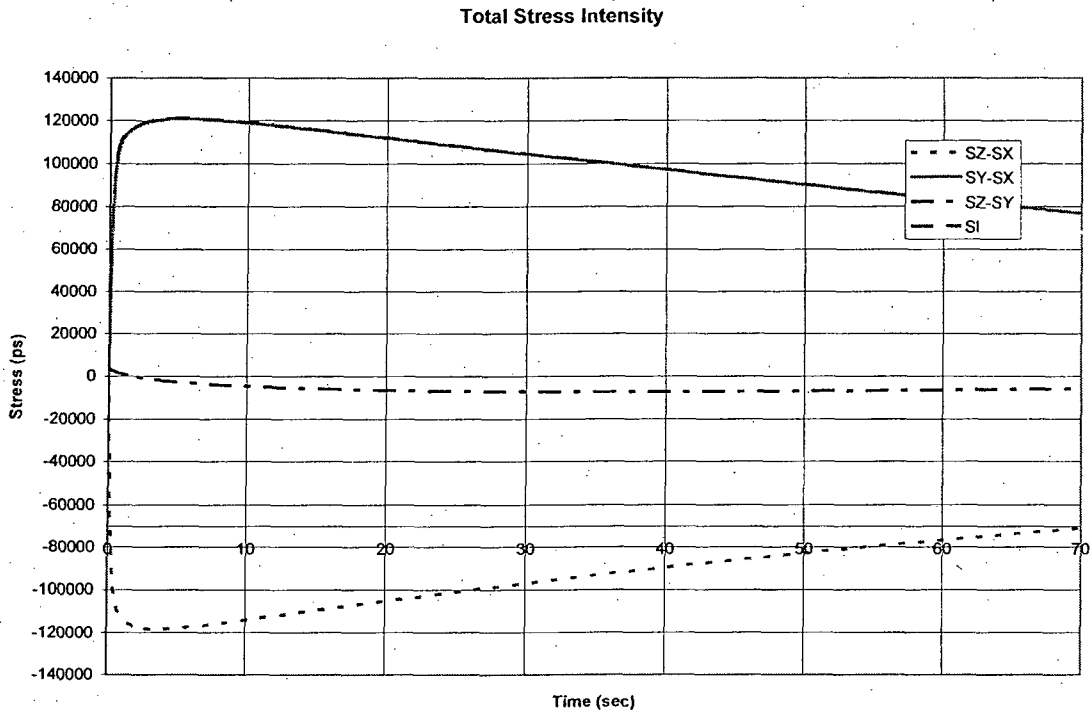


Figure 10: Safe End 100% Flow Total Stress Intensity

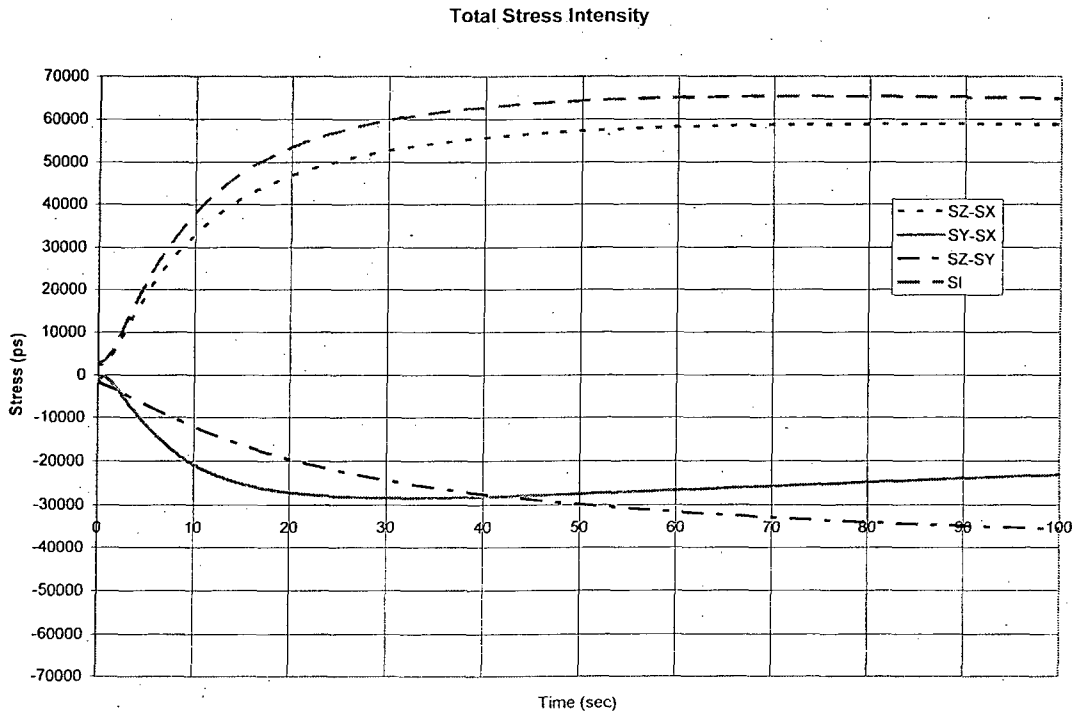


Figure 11: Blend Radius 100% Flow Total Stress Intensity

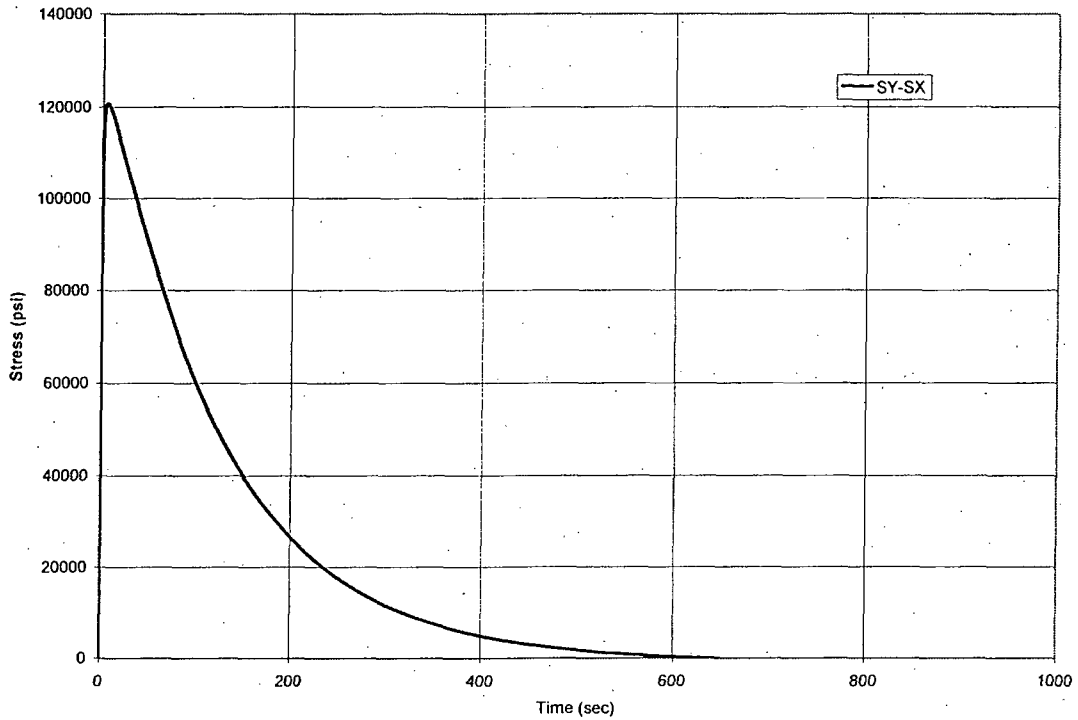


Figure 12: Safe End Total Stress History for 100% Flow

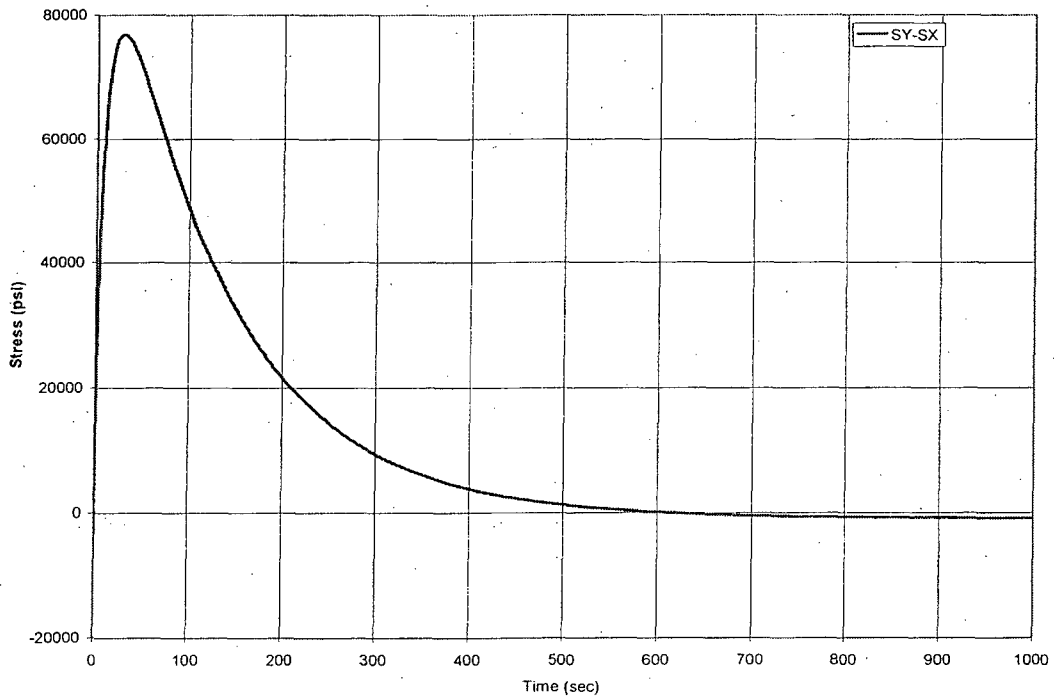


Figure 13: Safe End Membrane Plus Bending Stress History for 100% Flow

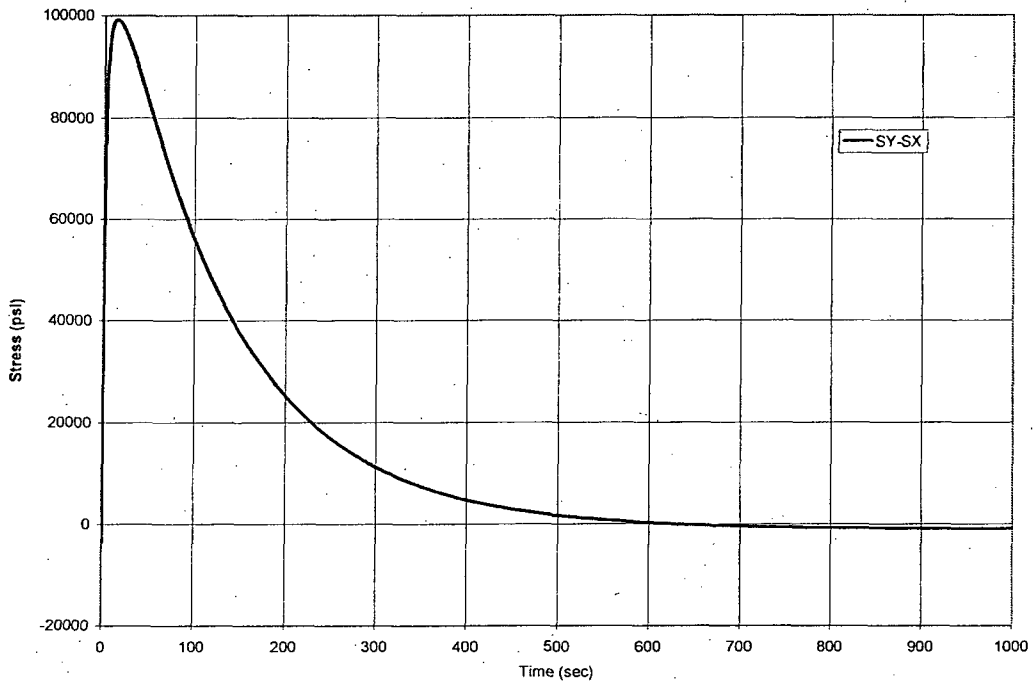


Figure 14: Safe End Total Stress History for 50% Flow

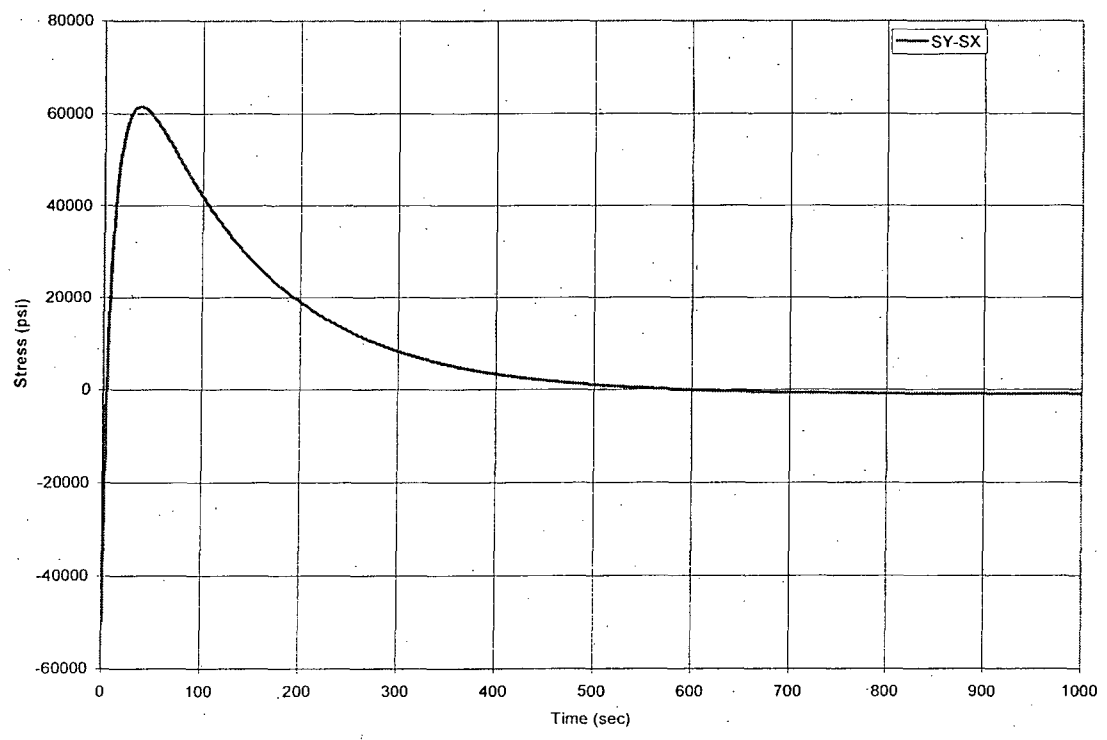


Figure 15: Safe End Membrane Plus Bending Stress History for 50% Flow

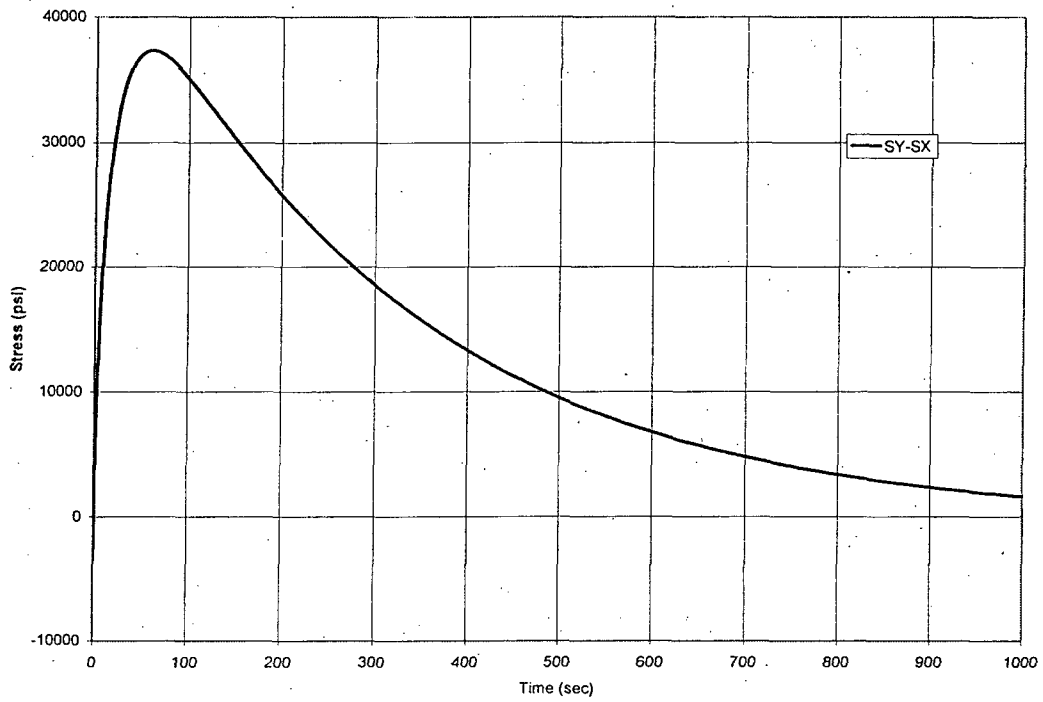


Figure 16: Safe End Total Stress History for 0% Flow

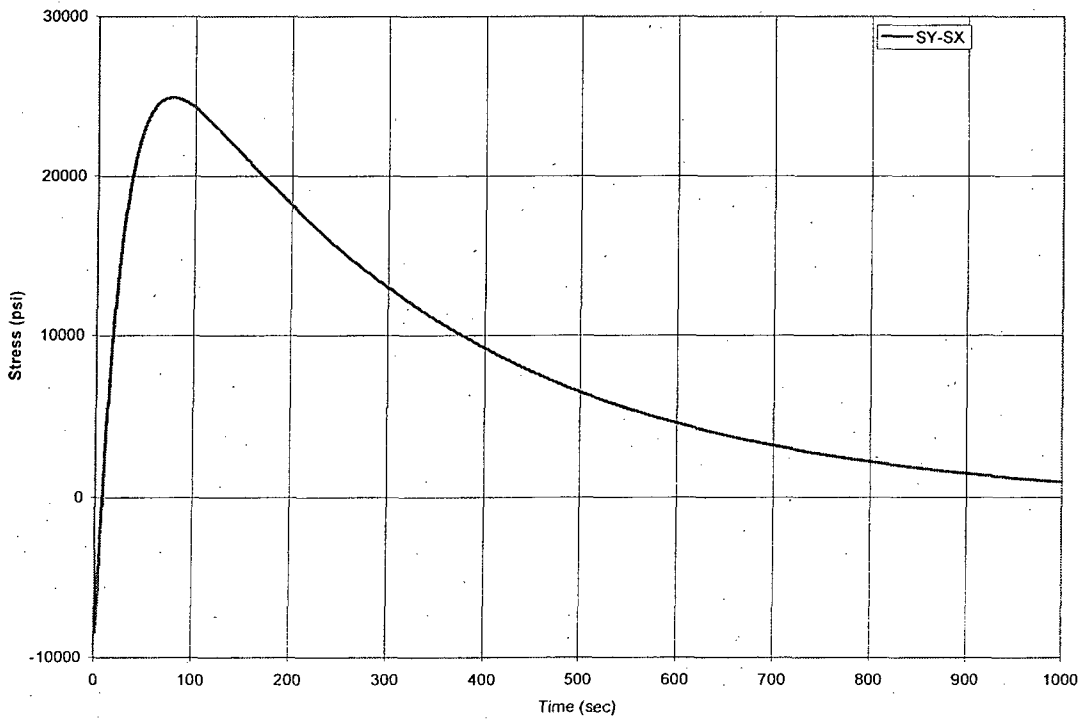


Figure 17: Safe End Membrane Plus Bending Stress History for 0% Flow

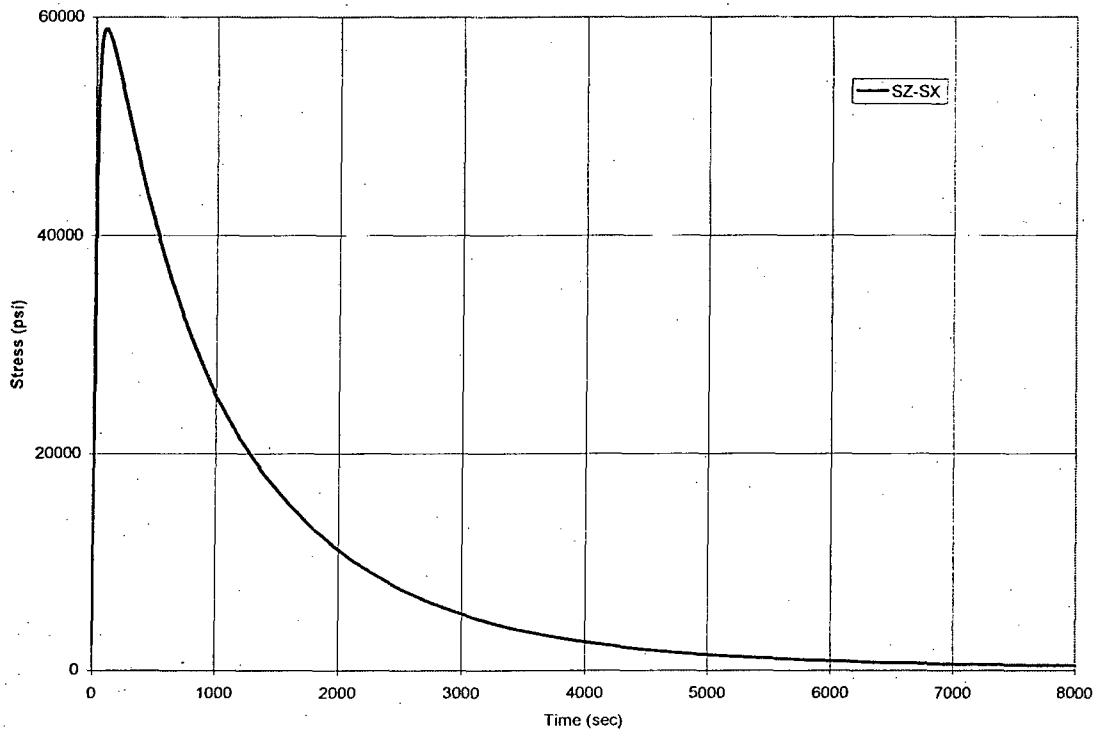


Figure 18: Blend Radius Total Stress History for 100% Flow

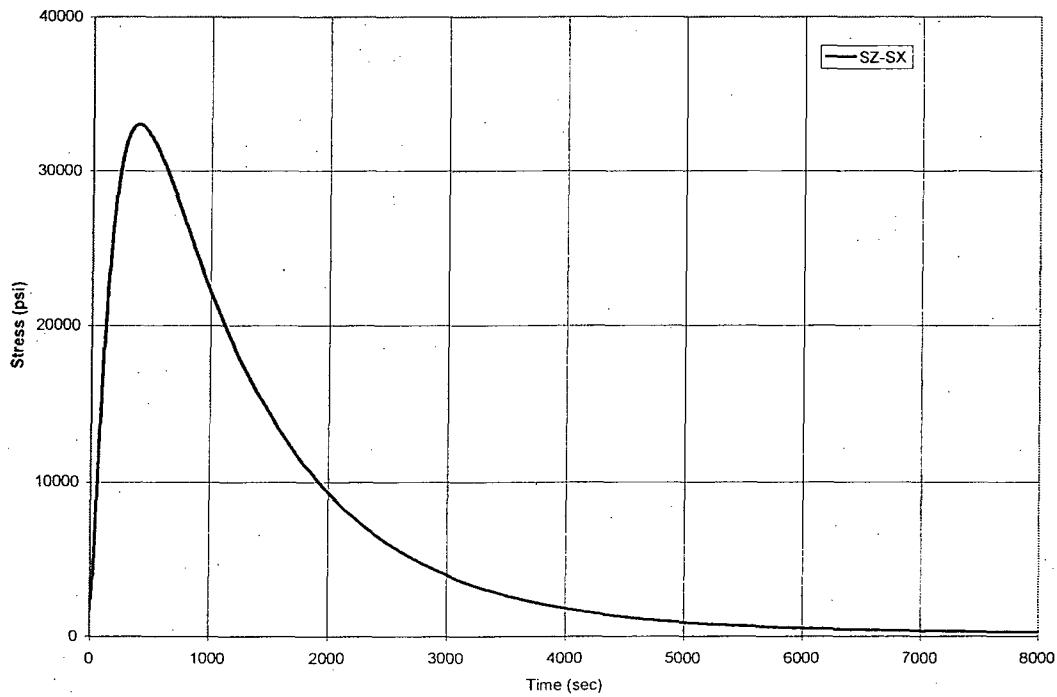


Figure 19: Blend Radius Membrane Plus Bending Stress History for 100% Flow

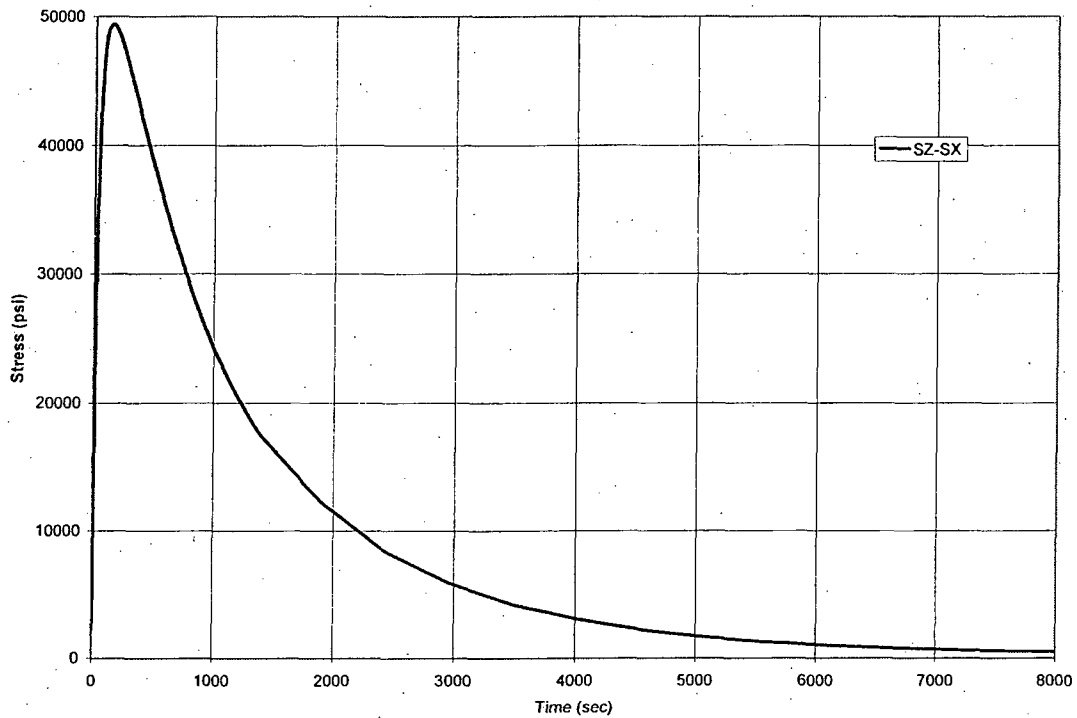


Figure 20: Blend Radius Total Stress History for 50% Flow

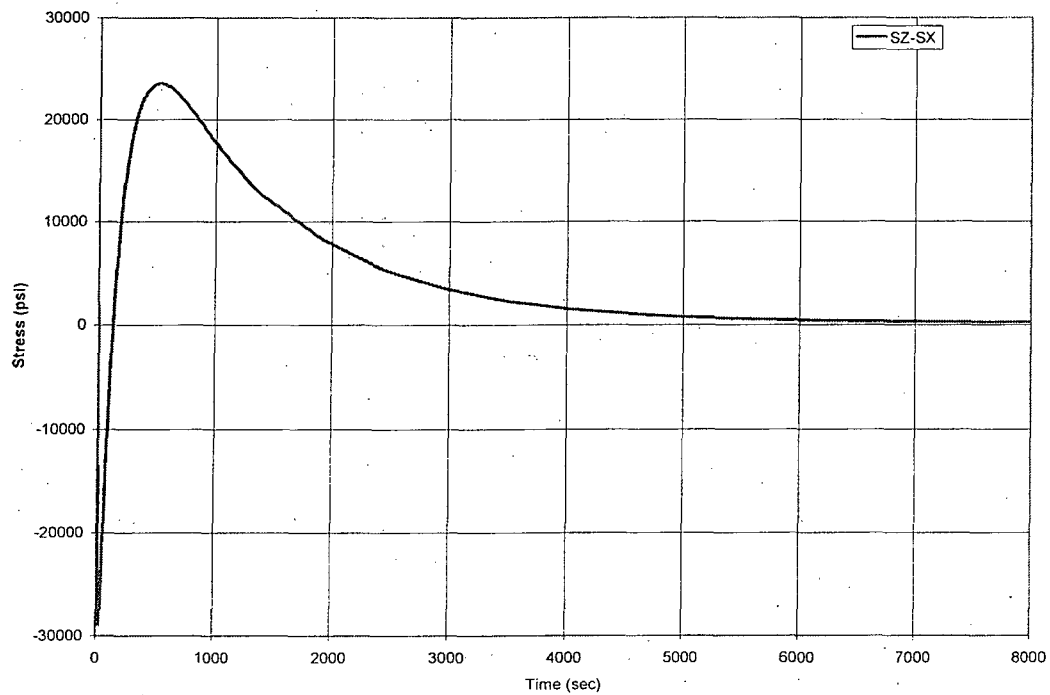


Figure 21: Blend Radius Membrane Plus Bending Stress History for 50% Flow

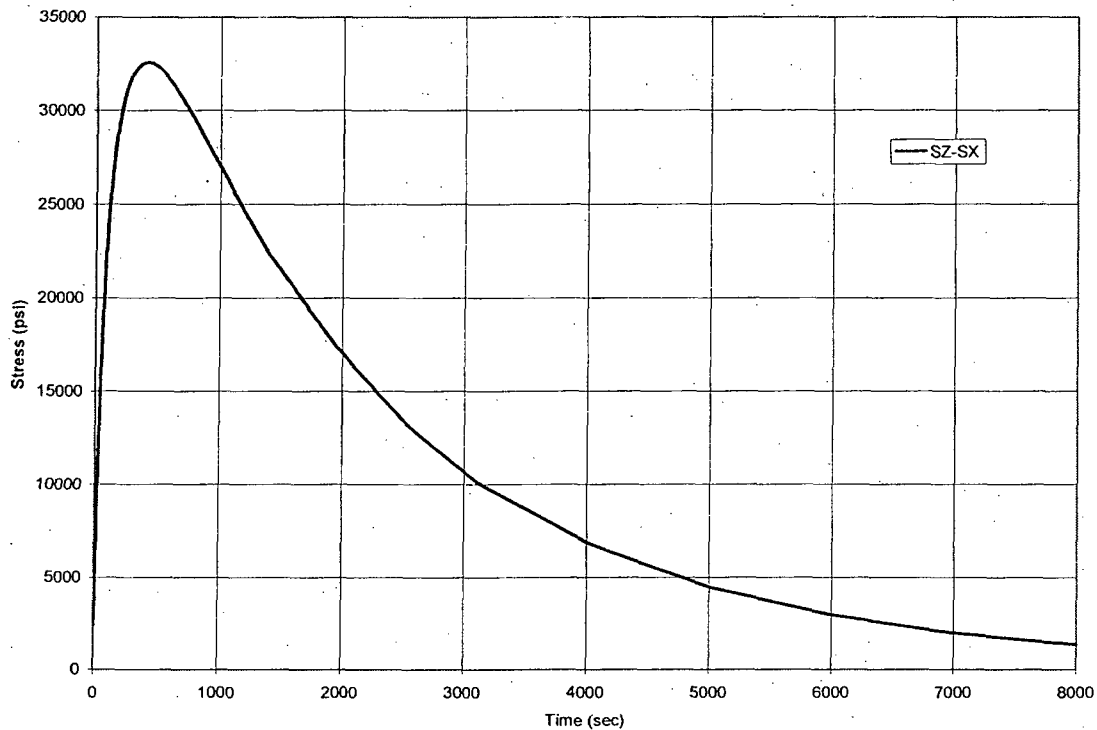


Figure 22: Blend Radius Total Stress History for 0% Flow

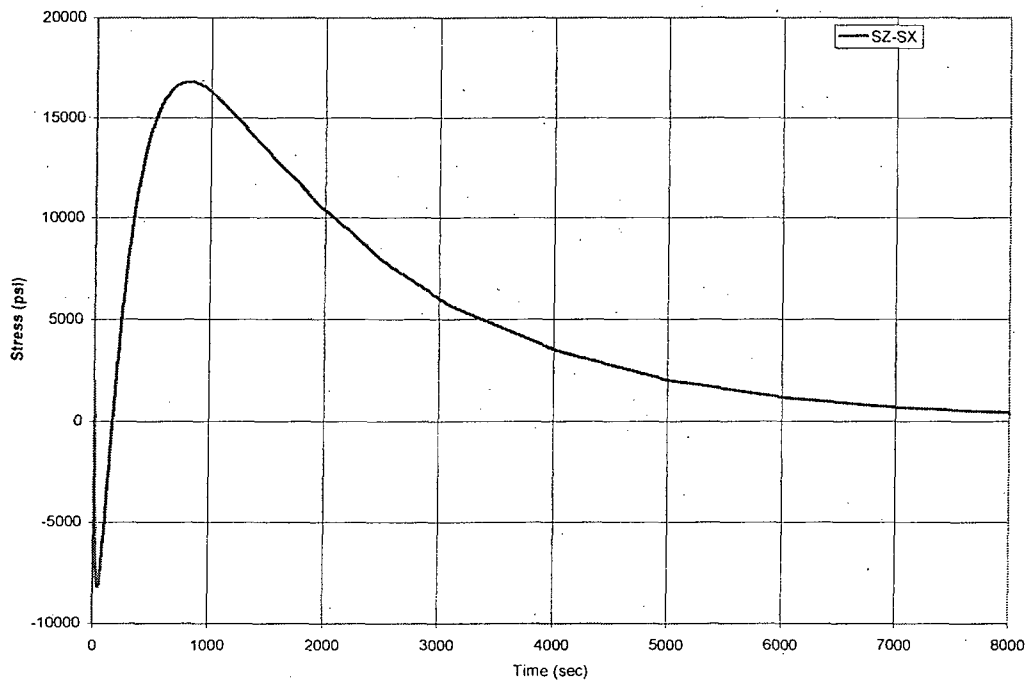


Figure 23: Blend Radius Membrane Plus Bending Stress History for 0% Flow

APPENDIX A
FINITE ELEMENT ANALYSIS FILES

RON_VY.INP	Input File for Pressure Load	In Computer files
VY RON T 100.INP	Input File for 100% Flow Thermal Analysis	In Computer files
VY RON S 100.INP	Input File for 100% Flow Stress Analysis	In Computer files
VY RON T 50.INP	Input File for 50% Flow Thermal Analysis	In Computer files
VY RON T 50.INP	Input File for 50% Flow Stress Analysis	In Computer files
VY RON 0.INP	Input File for 0% Flow Thermal Analysis	In Computer files
VY RON 0.INP	Input File for 0% Flow Stress Analysis	In Computer files
PVESH.OUT	Stress Output across the shell with Pressure Load	In Computer files
PSE.OUT	Stress Output at Safe End with Pressure Load	In Computer files
PBLEND.OUT	Stress Output at Blend Radius with Pressure Load	In Computer files
#FSE.OUT	Stress Output at Safe End	In Computer files
#FBR.OUT	Stress Output at Blend Radius	In Computer files
#FSE INSIDE.RED	Stress Extracted at Safe End	In Computer files
#FBR INSIDE.RED	Stress Extracted at Blend Radius	In Computer files
#FSE T-Green.XLS	Green Function with Total Stress at Safe End	In Computer files
#FSE_M+B-Green.XLS	Green Function with Membrane plus Bending Stress at Safe End	In Computer files
HFBR_T-Green.XLS	Green Function with Total Stress at Blend Radius at 100% flow	In Computer files
HFBR_M+B-Green.XLS	Green Function with Membrane plus Bending Stress at Blend Radius at 100% flow	In Computer files

Where # is H, M, L meaning 100%, 50%, and 0% flow rate, respectively.



Structural Integrity Associates, Inc.

File No.: VY-16Q-306

NEC-JH_09

CALCULATION PACKAGE

Project No.: VY-16Q

PROJECT NAME:

Environmental Fatigue Analysis of VYNPS

CONTRACT NO.:

10150394

CLIENT:

Entergy Nuclear Operations, Inc.

PLANT:

Vermont Yankee Nuclear Power Station

CALCULATION TITLE:

Fatigue Analysis of Recirculation Outlet Nozzle

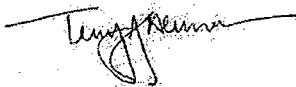
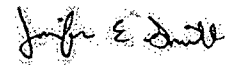

Document Revision	Affected Pages	Revision Description	Project Manager Approval Signature & Date	Preparer(s) & Checker(s) Signatures & Date
0	1-34, Appendix: A1-A1	Initial Issue	Terry J. Herrmann 7/27/2007 	J. E. Smith 7/27/2007  Minghao Qin 7/27/2007 



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1.0 OBJECTIVE

The purpose of this calculation is to perform a revised fatigue analysis for the Entergy Vermont Yankee (VY) reactor pressure vessel (RPV) recirculation outlet nozzle. Two locations will be analyzed for fatigue acceptance: the safe end (SA182 F316) and the nozzle inner corner blend radius (SA508 Class 2). Both locations are chosen based on the highest overall stress of the analysis performed in Reference [1]. Fatigue usage will be determined for each location, the nozzle forging and safe end, respectively. An environmental fatigue usage factor will also be determined for each of these locations.

2.0 METHODOLOGY

In order to provide an overall approach and strategy for evaluating the recirculation outlet nozzle, the Green's Function methodology and associated ASME Code stress and fatigue analyses are described in this section.

Revised stress and fatigue analyses are being performed for the recirculation outlet nozzle using ASME Code, Section III methodology. These analyses are being performed to address license renewal requirements to evaluate environmental fatigue for this component in response to Generic Aging Lessons Learned (GALL) Report [14] requirements. The revised analysis is being performed to refine the fatigue usage so that an environmental fatigue factor can be determined for subsequent license renewal efforts.

Two sets of rules are available under ASME Code, Section III, Class I [13]. Subparagraph NB-3600 of Section III provides simplified rules for analysis of piping components, and NB-3200 allows for more detailed analysis of vessel components. The NB-3600 piping equations combine by absolute sum the stresses due to pressure, moments and through wall thermal gradient effects, regardless of where within the pipe cross-section the maximum value of the components of stress are located. By considering stress signs, affected surface (inside or outside) and azimuthal position, the stress ranges may be significantly reduced. In addition, NB-3600 assigns stress indices by which the stresses are multiplied to conservatively incorporate the effects of geometric discontinuities. In NB-3200, stress indices are not required, as the stresses are calculated by finite element analysis and consider applicable stress concentration factors. In addition, NB-3200 methodology accounts for the different locations within a component where stresses due to thermal, pressure or other mechanical loading are a maximum. This generally results in a net reduction of the stress ranges and consequently, in the calculated fatigue usage. Article 4 [17] methodology was originally used to evaluate the recirculation outlet nozzle. NB-3200 methodology, which is the modern day equivalent to Article 4, is used in this analysis to be consistent with the Section III design bases for this component, as well as to allow a more detailed analysis of this component. In addition, several of the conservatisms originally used in the original recirculation outlet nozzle evaluation (such as grouping of transients) are removed in the current evaluation so as to achieve a more accurate CUF.

For the recirculation outlet nozzle evaluated as a part of this work, stress histories will be computed by a time integration of the product of a pre-determined Green's Function and the transient data.



This Green's Function integration scheme is similar in concept to the Duhamel theory used in structural dynamics. A detailed derivation of this approach and examples of its application to specific plant locations is contained in Reference [15]. A general outline is provided in this section.

The steps involved in the evaluation are as follows:

- Develop finite element model
- Develop heat transfer coefficients and boundary conditions for the finite element model
- Develop Green's Functions
- Develop thermal transient definitions
- Perform stress analysis to determine stresses for thermal transients
- Perform fatigue analysis

A Green's Function is derived by using finite-element methods to determine the transient stress response of the component to a step change in loading (usually a thermal shock). The critical location in the component is identified based on the maximum stress, and the thermal stress response over time is extracted for this location. This response to the input thermal step is the "Green's Function." Figure 13 shows a typical set of two Green's Functions, each for a different set of heat transfer coefficients (representing different flow rate conditions).

To compute the thermal stress response for an arbitrary transient, the loading parameter (usually local fluid temperature) is deconstructed into a series of step-loadings. By using the Green's Function, the response to each step can be quickly determined. By the principle of superposition, these can be added (algebraically) to determine the response to the original load history. The result is demonstrated in Figure 14. The input transient temperature history contains five step-changes of varying size, as shown in Figure 14. These five step changes produce the five successive stress responses in the second plot shown in Figure 14. By adding all five response curves, the real-time stress response for the input thermal transient is computed.

The Green's Function methodology produces identical results compared to running the input transient through the finite element model. The advantage of using Green's Functions is that many individual transients can be run with a significant reduction of effort compared to running all transients through the finite element model. The trade-off in this process is that the Green's Functions are based on constant material properties and heat transfer coefficients. Therefore, these parameters are chosen to bound all transients that constitute the majority of fatigue usage, i.e., the heat transfer coefficients at 300°F bound the cold water injection transient. In addition, the instantaneous value for the coefficient of thermal expansion is used instead of the mean value for the coefficient of thermal expansion. This conservatism is more than offset by the benefit of not having to analyze every transient, which was done in the VY reactor recirculation outlet nozzle evaluation.

Once the stress history is obtained for all transients using the Green's Function approach, the remainder of the fatigue analysis is carried out using traditional methodologies in accordance with ASME Code, Section III requirements.

Fatigue calculations are performed in accordance with ASME Code, Section III, Subsection NB-3200 methodology. Fatigue analysis is performed for the two limiting locations (one in the safe end and one in the nozzle forging, representing the two materials of the nozzle assembly) using the Green's Functions developed for these three Recirculation flow conditions and 60-year projected cycle counts.

Three Structural Integrity utility computer programs are used to facilitate the fatigue analysis process: STRESS.EXE, P-V.EXE, and FATIGUE.EXE. The first program, STRESS.EXE, calculates a stress history in response to a thermal transient using a Green's Function. The second program, P-V.EXE, reduces the stress history to peaks and valleys, as required by ASME Code fatigue evaluation methods. The third program, FATIGUE.EXE, calculates fatigue from the reduced peak and valley history using ASME Code, Section III range-pair methodology. All three programs are explained in detail and have been independently verified for generic use in the Reference [5] calculation.

In order to perform the fatigue analysis, Green's Functions are developed using the finite element model. Then, input files with the necessary data are prepared and the three utility computer programs are run. The first program (STRESS.EXE) requires the following three input files:

- Input file "GREEN.DAT": This file contains the Green's Function for the location being evaluated. For each flow condition, two Green's Functions are determined: a membrane plus bending stress intensity Green's Function and a total stress intensity Green's Function. This allows computation of total stress, as well as membrane plus bending stress, which is necessary to compute K_e per ASME Code, Section III requirements.
- Input file "GREEN.CFG": This file is a configuration file containing parameters that define the Green's Function (i.e., number of points, temperature drop analyzed, etc.).
- Input file "TRANSNT.INP": This file contains the input transient history for all thermal transients to be analyzed for the location being evaluated.

Pressure and piping stress intensities are also included for each transient case, based on pressure stress results from finite element analysis and attached piping load calculations.

The second program (P-V.EXE) simply extracts only the maxima and minima stress (i.e., the peaks and valleys) from the stress histories generated by program STRESS.EXE.

The third program (FATIGUE.EXE) performs the ASME Code peak event-pairing required to calculate a fatigue usage value. The input data consists of the output peak and valley history from program P-V.EXE and a configuration input file that provides ASME Code configuration data relevant to the fatigue analysis (i.e., K_e parameters, S_m , Young's modulus, etc.). The output is the final fatigue calculation for the location being evaluated.

The Green's Function methodology described above uses standard industry stress and fatigue analysis practices, and is the same as the methodology used in typical stress reports. Special approval for the use of this methodology is therefore not required.

The 10 transients to be analyzed are described in Reference [2], for the recirculation outlet nozzle. Transients 11 and 12 are hydrostatic tests that have only a small temperature change and are not



modeled. Transients 1 to 10 are shown in Figures 3 – 12. The analysis of transient 9 is an exception to this process because there are two different thermal shocks at the nozzle and vessel regions. Transient 9 is analyzed separately using ANSYS instead of STRESS.EXE and P-V.EXE. The results from ANSYS are input directly into FATIGUE.EXE with the other transient stress results.

3.0 ANALYSIS

The fatigue analysis involves preparing the input files and running the three programs. The programs STRESS.EXE and P-V.EXE are run together through the use of a batch file. The program FATIGUE.EXE is run after processing the output from P-V.EXE. The ANSYS results from transient 9 are added to the P-V.EXE results for the other transients and input into FATIGUE.EXE.

The steps associated with this process are described in the following sub-sections.

3.1 Transient Definitions (for program STRESS.EXE)

The program STRESS.EXE requires the following three input files for analyzing an individual transient:

- GREEN.DAT. There are 12 stress history functions (Green's Functions) obtained from Reference [1]. They represent the membrane plus bending and total stress intensities at the blend radius and safe end locations. The blend radius and the safe end have three stress history functions for the 100% flow, 50%, and no-flow conditions.
- GREEN.CFG is configured as described in Reference [5].
- Several TRANSNT.INP files are created to simulate the transients shown on Reference [2]. Tables 2 and 3 show the thermal history used to simulate each transient for the blend radius and safe end locations, respectively. The aforementioned transient information for each location is contained in EXCEL files *Blend_Radius_Transients.xls* and *Safe_End_Transients.xls*, which are contained in the computer files. Transients are split into the following groups based upon flow rate:
 - Transients 2, 3, 5, 6, 7, and 8 are run at 100% flow Green's Function
 - Transients 1 and 10 are run at 50% flow Green's Function
 - Transient 4 is run at no flow, 50% flow, and 100% flow Green's Functions, as shown in Tables 2 and 3.
 - Transient 9 is simulated by ANSYS [11] model and the thermal results are taken from ANSYS directly. See Section 4 for details.
 - Transients 11 and 12 have only small temperature change (70°F to 100°F). Therefore, the thermal stresses for these two transient are ignored. Only the piping load and the pressure load are considered in these two transients.
 - The loss of feedwater heaters (Feedwater Heater Bypass) event has a negligible temperature change (526 °F to 516 °F) associated with it. Therefore this transient is ignored.



3.2 Peak and Valley Points of the Stress History (for program P-V.EXE)

After STRESS.EXE runs are completed, the program P-V.EXE is run to extract only the peaks and valleys from the STRESS.OUT stress history file produced by the STRESS.EXE program. The only input required for this program is the stress history file (STRESS.OUT), and the program outputs all of the resulting peaks and valleys to output file P-V.OUT. The resulting peak and valley stress summaries for all transients are summarized in Tables 4 and 5 for both locations. Columns 2 through 5 of Tables 4 (for the blend radius) and 5 (for the safe end) show the final peak and valley output. These final peaks and valleys were selected from the total stress and membrane plus bending stress intensities that were calculated by STRESS.EXE and screened with P-V.EXE.

3.3 Pressure Load

The pressure stress associated with a 1,000 psi internal pressure was determined in Reference [1]. These values are as follows:

Pressure stress for the safe end:

- 11,350 psi membrane plus bending linearized stress intensity.
- 11,490 psi total stress intensity.

Pressure stress for the blend radius:

- 33,640 psi membrane plus bending linearized stress intensity.
- 31,300 psi total stress intensity.

The pressure stress intensity values for each transient were linearly scaled based on the pressure. The actual pressure for column 6 of Tables 4 and 5 is obtained from Tables 2 and 3, respectively. The scaled pressure stress values are shown in columns 7 and 8 of Tables 4 and 5.

The pressure stress is combined with the peak and valley points to calculate the final stress values used for fatigue analysis.

3.4 Attached Piping Loads

Additionally, the piping stress intensity (stress caused by the attached piping) was determined. These piping forces and moments are determined as shown in Figure 1.

The following formulas are used to determine the maximum stress intensity in the nozzle at the two locations of interest. From engineering statics, the piping loads at the end of the model can be translated to the first and second cut locations using the following equations:

$$\begin{aligned} \text{For Cut I: } & (M_x)_1 = M_x - F_y L_1 \\ & (M_y)_1 = M_y + F_x L_1 \end{aligned}$$

$$\begin{aligned} \text{For Cut II: } (M_x)_2 &= M_x - F_y L_2 \\ (M_y)_2 &= M_y + F_x L_2 \end{aligned}$$

The total bending moment and shear loads are obtained using the equations below:

$$\begin{aligned} \text{For Cut I: } M_{xy} &= \sqrt{(M_x)_1^2 + (M_y)_1^2} \\ F_{xy} &= \sqrt{(F_x)_1^2 + (F_y)_1^2} \end{aligned}$$

$$\begin{aligned} \text{For Cut II: } M_{xy} &= \sqrt{(M_x)_2^2 + (M_y)_2^2} \\ F_{xy} &= \sqrt{(F_x)_2^2 + (F_y)_2^2} \end{aligned}$$

The distributed loads for a thin-walled cylinder are obtained using the equations below:

$$\begin{aligned} N_z &= \frac{1}{\pi R_N} \left[\frac{1}{2} F_{zy} + \frac{M_{xy}}{R_N} \right] \\ q_N &= \frac{1}{\pi R_N} \left[F_{xy} - \frac{M_z}{2R_N} \right] \end{aligned}$$

To determine the primary stresses, P_M , due to internal pressure and piping loads, the following equations are used.

For Cut I, using thin-walled equations:

$$(P_M)_z = \frac{Pa_N}{2t_N} + \frac{Nz}{t_N}$$

$$(P_M)_\theta = \frac{Pa_N}{t_N}$$

$$(P_M)_R = -P$$

$$\tau_M = \frac{q_N}{t_N}$$

$$SI_{MAX} = 2 \sqrt{\left(\frac{(P_M)_\theta - (P_M)_R}{2} \right)^2 + (\tau_M)_{z\theta}^2}$$

or

$$SI_{MAX} = 2 \sqrt{\left(\frac{(P_M)_z - (P_M)_R}{2} \right)^2 + (\tau_M)_{z\theta}^2}$$

Because pressure was considered separately in this analysis, the equations used for Cut I are valid for Cut II.

- where:
- L_1 = The length from the end of the nozzle where the piping loads are applied to the location of interest in the safe end.
 - L_2 = The length from the end of the nozzle where the piping loads are applied to the location of interest in the blend radius.
 - M_{xy} = The maximum bending moment in the xy plane.
 - F_{yx} = The maximum shear force in the xy plane.
 - N_z = The normal force per inch of circumference applied to the end of the nozzle in the z direction.
 - q_N = The shear force per inch of circumference applied to the nozzle.
 - R_N = The mid-wall nozzle radius.

Since the pressure was considered separately in this analysis, the equations can be simplified as follows:

$$(P_M)_z = \frac{N_z}{t_N}$$

$$(P_M)_\theta = 0$$

$$(P_M)_R = 0$$

$$\tau_M = \frac{q_N}{t_N}$$

$$SI_{MAX} = 2(\tau_M)_{z\theta}$$

or

$$SI_{MAX} = 2 \sqrt{\left(\frac{N_z}{2t_N}\right)^2 + (\tau_M)_{z\theta}^2}$$

Per Reference [7], the recirculation outlet nozzle piping loads (Total thermal, weight and seismic loads) are as follows:

$$F_x = 20,000 \text{ lbs}$$

$$M_x = 2,004,000 \text{ in-lb}$$

$$F_y = 20,000 \text{ lbs}$$

$$M_y = 3,000,000 \text{ in-lb}$$

$$F_z = 30,000 \text{ lbs}$$

$$M_z = 2,004,000 \text{ in-lb}$$

L_1 is equal to 4.25 inches and the L_2 is equal to 42.77 inches. The calculations for the safe end and blend radius are shown in Table 1. The first cut location is the same as the Green's Function cross section per [1] at the safe end, and the second cut is from Node 3829 (inside) to Node 3809 (outside). This gives the maximum ID and minimum OD for the cross section calculation. The maximum stress intensities due to the piping loads are 5708.89 psi at the safe end and 280.16 psi at the blend radius. The piping load sign is set as the same as the thermal stress sign.



These piping stress values are scaled assuming no stress occurs at an ambient temperature of 70°F, and the full values are reached at reactor design temperature, 575°F [6]. The scaled piping stress values are shown in columns 9 and 10 of Tables 4 and 5. Columns 11 and 12 of Tables 4 and 5 show the summation of all stresses for each thermal peak and valley stress point.

3.5 Fatigue Analysis (for program FATIGUE.EXE)

The number of cycles projected for the 60-year operating life is used for each transient [2]:

Column 13 in Tables 4 and 5 shows the number of cycles associated with each transient. The number of cycles for 60 years was obtained from Reference [2] unless otherwise noted.

The program FATIGUE.EXE performs the “ASME Code style” peak event pairing required to calculate a fatigue usage value. The input data for FATIGUE.CFG is as follows:

	Blend Radius	Safe End
Parameters m and n for Computing K_e	2.0 & 0.2 (low alloy steel) [13]	1.7 & 0.3 (stainless steel) [13]
Design Stress Intensity Values, S_m	26700 psi [9] @ 600°F	17000 psi [9] @ 600°F
Elastic Modulus from Applicable Fatigue Curve	30.0×10^6 psi [13]	28.3×10^6 psi [13]
Elastic Modulus Used in Finite Element Model	26.7×10^6 psi [1]	27.0×10^6 psi [1]
The Geometric Stress Concentration Factor K_t	1.0	1.53 [3]

The results of the fatigue analyses are presented in Tables 6 and 7 for the blend radius and safe end for 60 years, respectively.

The fatigue run inputs described are contained in EXCEL files *BRresults.xls* and *SEresults.xls*, which are contained in the computer files.

4.0 CALCULATION OF THERMAL STRESSES FOR TRANSIENT 9

Per Tables 2 and 3, the thermal shocks are from 526°F to 268°F and from 526°F to 130°F at the blend radius and the safe end, respectively. Therefore, the average temperatures for these two locations are about 400°F and 330°F. Since there are two different temperature shocks in the same model, ANSYS [10] will be used to calculate stresses directly. In this section, ANSYS [10] is used to simulate this transient and the results will then be used as input to FATIGUE.EXE, as shown in Tables 4 and 5. This case corresponds to the downhill (RPV) side of the blend radius.

An additional case was also run to simulate the uphill (RPV) side of the blend radius, where the thermal shocks are from 526°F to 130°F at the safe end, and no temperature change at the blend

radius. This case at the uphill side of the blend radius was found to produce lower stresses than the previously mentioned downhill case. Due to this, the downhill case was used for the rest of the analysis in this calculation.

4.1 Thermal Load

Since the average temperatures in the blend radius and safe end respectively are 400°F and 330°F, the material properties for 400°F are used for the blend radius, cladding and vessel. Table 8 shows the material properties at 400°F. The flow rate at this transient is 3395.2 GPM (calculated from 12% of max flow rate [2]) and is shown in Tables 2 and 3.

Heat transfer coefficients listed on Reference [4] are for pre power uprate. The heat transfer coefficients can be scaled by power uprate flow rate and diameter to values corresponding to the flow and location conditions. Referring to Figure 2, heat transfer coefficients were applied as follows:

Region 1

Per [4], the heat transfer coefficient at 500°F, h, for 3395.2 GPM (2.084 ft/s) flow is

$$4911 \cdot \left(\frac{2.084}{25} \right)^{0.8} = 672.8 \text{ BTU/hr-ft}^2\text{-}^\circ\text{F.}$$

Per [4], the heat transfer coefficient at 100°F, h, for 3395.2 GPM (2.084 ft/s) flow is

$$2250 \cdot \left(\frac{2.084}{25} \right)^{0.8} = 308.24 \text{ BTU/hr-ft}^2\text{-}^\circ\text{F.}$$

The fluid temperature shock is:

$$T = 526^\circ\text{F} - 130^\circ\text{F} - 526^\circ\text{F}$$

Region 2

Per [4], the heat transfer coefficient at 500°F, h, for 3395.2 GPM (2.084 ft/s) flow is

$$4911 \cdot \left(\frac{2.084}{25} \right)^{0.8} \left(\frac{26}{35.49} \right)^{0.2} = 632.21 \text{ BTU/hr-ft}^2\text{-}^\circ\text{F.}$$

Per [4], the heat transfer coefficient at 300°F, h, for 3395.2 GPM (2.084 ft/s) flow is

$$4789 \cdot \left(\frac{2.084}{25} \right)^{0.8} \left(\frac{26}{35.49} \right)^{0.2} = 616.57 \text{ BTU/hr-ft}^2\text{-}^\circ\text{F.}$$



The fluid temperature shock is:

$$T = 526^{\circ}\text{F} - 268^{\circ}\text{F} - 526^{\circ}\text{F}$$

Region 3

Per [4], the heat transfer coefficient at 500°F, h, for 3395.2 GPM flow is

$$672.8(0.5) = 336.4 \text{ BTU/hr-ft}^2\text{-}^{\circ}\text{F}.$$

Per [4], the heat transfer coefficient at 300°F, h, for 3395.2 GPM flow is

$$336.4 \left(\frac{4789}{4911} \right) = 328.04 \text{ BTU/hr-ft}^2\text{-}^{\circ}\text{F}.$$

The fluid temperature shock is:

$$\text{Case 1: } T = 526^{\circ}\text{F} - 268^{\circ}\text{F} - 526^{\circ}\text{F}$$

$$\text{Case 2: } T = 526^{\circ}\text{F}$$

Region 4

The heat transfer coefficient, h, is 0.4 BTU/hr-ft²-°F [4].

The temperature is:

$$T = 120^{\circ}\text{F}$$

4.2 Thermal Results

The flow dependent thermal load case outlined in Section 4.1 was run on the finite element model. Appendix A contains the thermal transient input file VY_RON_T_T9.INP for 3395.2 GPM flow rate. The flow dependent input files for the stress run is also included in Appendix A. The stress filename is VY_RON_S_T9.INP for 3395.2 GPM flow rate.

The critical safe end and blend radius locations are defined in Reference [1] at nodes 6395 and 3829, respectively.

The stress time history for the critical paths was extracted during the stress run. This produced two files, T9SE.OUT and T9BR.OUT, which contain the thermal stress history. The membrane plus bending stresses and total stresses were extracted from these files to produce the files T9SE_Inside.RED and T9BR_Inside.RED, where SE and BR corresponded to the safe end and blend radius locations, respectively.



The data for the stress results is included in the files T9BR_M+B.xls, T9BR_T.xls, T9SE_M+B.xls, and T9SE_T.xls in the project Files. Where SE and BR corresponded to the safe end and blend radius locations, respectively. M+B and T corresponded to membrane plus bending stress and total stress, respectively.

5.0 FATIGUE USAGE RESULTS

The blend radius cumulative usage factor (CUF) from system cycling is 0.0108 for 60 years (Table 6). The safe end CUF is 0.0015 for 60 years (Table 7).

6.0 ENVIRONMENTAL FATIGUE ANALYSIS

The Recirculation Outlet nozzle has three materials: a Ni-Cr-Fe dissimilar metal weld (DMW), a low alloy steel forging, and a stainless steel safe end. To ensure the maximum CUF considering environmental effects was identified, locations in the safe end and nozzle forging were selected. This selection produces bounding environmental fatigue results for the entire nozzle assembly for the following reasons:

- The highest thermal stresses from the FEM analysis occur in the stainless steel safe end. Stainless steel F_{en} multipliers are significantly higher than Ni-Cr-Fe multipliers (F_{en} values are 2.55 or higher for stainless steel [12] vs. a constant value of 1.49 for Ni-Cr-Fe [16]). Therefore, evaluation of the safe end bounds the Ni-Cr-Fe weld material.
- The highest pressure stresses from the FEM analysis occur in the low alloy steel nozzle forging. Low alloy steel F_{en} multipliers are higher than Ni-Cr-Fe multipliers (F_{en} values are 2.45 or higher for low alloy steel [12] vs. a constant value of 1.49 for Ni-Cr-Fe [16]). Therefore, evaluation of the nozzle forging bounds the Ni-Cr-Fe weld material.

Per Reference [12], the dissolved oxygen (DO) calculation shows the overall hydrogen water chemistry (HWC) availability is 47%. This means the time ratio under normal water chemistry (NWC, or pre-HWC) is 53%.

For the safe end location, the environmental fatigue factors for post-HWC and pre-HWC are 15.35 and 8.36 from Table 5 of Reference [12]. These result in an EAF adjusted CUF of $(15.35 \times 47\% + 8.36 \times 53\%) \times 0.0015 = 0.0175$ for 60 years, which is acceptable (i.e., less than the allowable value of 1.0). The overall environmental multiplier is 11.6453.

For the blend radius location, the environmental fatigue factors for post-HWC and pre-HWC are 2.45 and 12.43 from Table 5 of Reference [12]. These result in an EAF adjusted CUF of $(2.45 \times 47\% + 12.43 \times 53\%) \times 0.0108 = 0.08358$ for 60 years, which is acceptable (i.e., less than the allowable value of 1.0). The overall environmental multiplier is 7.739.



7.0 REFERENCES

1. Structural Integrity Associates Calculation No. VY-16Q-305, Revision 0, "Recirculation Outlet Nozzle Green's Functions."
2. Entergy Design Input Record (DIR), Rev. 1, EC No. 1773, Rev. 0, "Environmental Fatigue Analysis for Vermont Yankee Nuclear Power Station," 7/26/07, SI File No. VY-16Q-209.
3. CB&I, RPV Stress Report Section: S9 "Stress Analysis Recirculation Outlet Nozzle Vermont Yankee Reactor Vessel." 9-6201, SI document, VY-16Q-204.
4. CB&I, RPV Stress Report Section: T9 "Thermal Analysis Recirculation Outlet Nozzle Vermont Yankee Reactor Vessel." 9-6201, SI document, VY-16Q-204.
5. Structural Integrity Associates Calculation (Generic) No. SW-SPVF-01Q-301, Revision 0, "STRESS.EXE, P-V.EXE, and FATIGUE.EXE Software Verification."
6. VY Drawing, 5920-06623 Rev. 0, (Hitachi, Ltd. Drawing No IOR290-127, Revision 0), "Recirc. Outlet Safe End," SI File No. VY-16Q-204.
7. GE. Stress Report No. 23A4316, Revision 0, "Reactor Vessel Recirculation Outlet Safe End," SI File No. VY-16Q-204.
8. VY Drawing 5920-00238 Rev. 4, (Chicago Bridge & Iron Company, Contract No. 9-6201, Drawing No. 21, Revision 4), "36"x28" Nozzles Mk N1A/B," SI File No. VY-16Q-204.
9. ASME Boiler and Pressure Vessel Code, Section II, Materials, Part D, Properties, 1998 Edition with 2000 Addenda.
10. ANSYS, Release 8.1 (w/Service Pack 1), ANSYS, Inc., June 2004.
11. Structural Integrity Associates Calculation No. VY-16Q-304, Revision 0, "Recirculation Outlet Nozzle Finite Element Model."
12. Structural Integrity Associates Calculation No. VY-16Q-303, Revision 0, "Environmental Fatigue Evaluation of Reactor Recirculation Inlet Nozzle and Vessel Shell/Bottom Head."
13. American Society of Mechanical Engineers, Boiler and Pressure Vessel Code, Section III, Subsection NB, 1998 Edition, 2000 Addenda.
14. NUREG-1801, Revision 1, "Generic Aging Lessons Learned (GALL) Report," U. S. Nuclear Regulatory Commission, September 2005.
15. Kuo, A. Y., Tang, S. S., and Riccardella, P. C., "An On-Line Fatigue Monitoring System for Power Plants, Part I - Direct Calculation of Transient Peak Stress Through Transfer Matrices and Green's Functions," ASME PVP Conference, Chicago, 1986.
16. EPRI Report No. TR-105759, "An Environmental Factor Approach to Account for Reactor Water Effects in Light Water Reactor Pressure Vessel and Piping Fatigue Evaluations," December 1995.
17. American Society of Mechanical Engineers, Boiler and Pressure Vessel Code, Section III, Subsection A, 1965 Edition with Winter 1966 Addenda.

Table 1: Maximum Piping Stress Intensity Calculations

Blend Radius External Piping Loads			Safe End External Piping Loads		
Parameters			Parameters		
$F_x =$	20.00	kips	$F_x =$	20.00	kips
$F_y =$	20.00	kips	$F_y =$	20.00	kips
$F_z =$	30.00	kips	$F_z =$	30.00	kips
$M_x =$	2004.00	in-kips	$M_x =$	2004.00	in-kips
$M_y =$	3000.00	in-kips	$M_y =$	3000.00	in-kips
$M_z =$	2004.00	in-kips	$M_z =$	2004.00	in-kips
OD=	55.88	in	OD=	28.38	in
ID=	37.368	in	ID=	25.938	in
$R_N =$	23.31	in	$R_N =$	13.58	in
L =	42.77	in	L =	4.25	in
$t_N =$	9.25	in	$t_N =$	1.22	in
$(M_x)_2 =$	1148.54	in-kips	$(M_x)_1 =$	1919.00	in-kips
$(M_y)_2 =$	3855.46	in-kips	$(M_y)_1 =$	3085.00	in-kips
$M_{xy} =$	4022.90	in-kips	$M_{xy} =$	3633.15	in-kips
$F_{xy} =$	28.28	kips	$F_{xy} =$	28.28	kips
$N_z =$	2.56	kips/in	$N_z =$	6.62	kips/in
$q_N =$	-0.20	kips/in	$q_N =$	-1.07	kips/in
Primary Membrane Stress Intensity			Primary Membrane Stress Intensity		
$PM_z =$	0.28	ksi	$PM_z =$	5.43	ksi
$\tau =$	-0.02	ksi	$\tau =$	-0.88	ksi
$SI_{max} =$	0.28	ksi	$SI_{max} =$	5.71	ksi
$SI_{max} =$	280.16	psi	$SI_{max} =$	5708.89	psi

Note: The locations for Cut I and Cut II were defined in Reference [1] for safe end and blend radius paths, respectively.

Table 2: Blend Radius Transients

Transient Number	Time (s)	Temp (°F)	Time Step (s)	Pressure (psig)	Flow Rate (GPM)	Transient Number	Time (s)	Temp (°F)	Time Step (s)	Pressure (psig)	Flow Rate (GPM)	
1. Normal Startup with Heatup at 100°F/hr 300 Cycles	0	100		0	14147.0	6. Reactor Overpressure 1 Cycle	0	526		1010	28294	
	16164	549	16164	1010	(50%)		2	526	2	1375	(100%)	
	22164	549	6000	1010			32	526	30	940		
2. Turbine Roll and Increase to Rated Power 300 Cycles	0	549		1010	28294		1832	526	1800	940		
	1	542	1	1010	(100%)		2252	549	420	1010		
	601	542	600	1010			2312	549	60	1010		
	602	526	1	1010			2313	542	1	1010		
	6602	526	6000	1010			2913	542	600	1010		
3. Loss of Feedwater Heaters Turbine Trip 25% Power 10 Cycles	0	526		1010	28294		2914	526	1	1010		
	1800	542	1800	1010	(100%)		8914	526	6000	1010		
	2100	542	300	1010			7. SRV Blowdown 1 Cycle	0	526		1010	28294
	2460	526	360	1010				600	375	600	170	(100%)
	3060	526	600	1010		11580		70	10980	50		
	3960	542	900	1010		17580		70	6000	50		
	4260	542	300	1010		8. SCRAM Other 228 Cycles		0	526		1010	28294
6060	526	1800	1010		15		526	15	940	(100%)		
12060	526	6000	1010		1815		526	1800	940			
4. Loss of Feedwater Pumps 10 Cycles	0	526		1010	0		2235	549	420	1010		
	3	526	3	1190	(0%)		2295	549	60	1010		
	13	526	10	1135		2296	542	1	1010			
	233	300	220	1135		2356	542	60	1010			
	2213	500	1980	1136		2357	526	1	1010			
	2393	300	180	885		8357	526	6000	1010			
	6773	500	4380	1135		9. Improper Startup 1 Cycle	0	526		1010	3395	
	7193	300	420	675			1	268 ⁽³⁾	1	1010	(12%)	
	7493	300	300	675	14147		27	268 ⁽³⁾	26	1010		
	11093	400	3600	240	(50%)		28	526	1	1010		
	16457	549	5364	1010			6028	526	6000	1010		
	16517	549	60	1010		10. Shutdown 300 Cycles	0	549		1010	14147	
	16518	542	1	1010	28294		6264	375	6264	170	(50%)	
17118	542	600	1010	(100%)	6864		330	600	88			
17119	526	1	1010		16224		70	9360	50			
23119	526	6000	1010		22224		70	6000	50			
5. Turbine Generator Trip 60 Cycles	0	526		1010	28294	11. Design Hydrostatic Test 120 cycles	---	100	---	50	1981	
	10	526	10	1135	(100%)		---	100	---	1563	(7%)	
	15	526	5	1135		---	100	---	50			
	30	526	15	940		12. Hydrostatic Test 1 Cycle	---	100	---	0	1981	
	1830	526	1800	940			---	100	---	1100	(7%)	
	2250	549	420	1010			---	100	---	50		
	2310	549	60	1010								
	2311	542	1	1010								
	2911	542	600	1010								
	2912	526	1	1010								
8912	526	6000	1010									

- Notes:
1. The instant temperature change is assumed as 1 second time step.
 2. The number of cycles is for 60 years [2].
 3. 268°F is the blend radius temperature for this transient. The safe end has a different temperature for Transient 9. [2]

Table 3: Safe End Transients

Transient Number	Time (s)	Temp (°F)	Time Step (s)	Pressure (psig)	Flow Rate (GPM)	Transient Number	Time (s)	Temp (°F)	Time Step (s)	Pressure (psig)	Flow Rate (GPM)					
1. Normal Startup with Heatup at 100°F/hr 300 Cycles	0	100		0	14147.0	6. Reactor Overpressure 1 Cycle	0	526		1010	28294					
	16164	549	16164	1010	(50%)'		2	526	2	1375	(100%)'					
	16864	549	700	1010			32	526	30	940						
2. Turbine Roll and Increase to Rated Power 300 Cycles	0	549		1010	28294		1832	526	1800	940						
	1	542	1	1010	(100%)'		2252	549	420	1010						
	601	542	600	1010			2312	549	60	1010						
	602	526	1	1010			2313	542	1	1010						
	1302	526	700	1010			2913	542	600	1010						
3. Loss of Feedwater Heaters Turbine Trip 25% Power 10 Cycles	0	526		1010	28294		2914	526	1	1010		7. SRV Blowdown 1 Cycle	0	526	1010	28294
	1800	542	1800	1010	(100%)'		600	375	600	170			(100%)'			
	2100	542	300	1010			11580	70	10980	50						
	2460	526	360	1010			12280	70	700	50						
	3060	526	600	1010		8. SCRAM Other 228 Cycles	0	526		1010	28294					
	3960	542	900	1010			15	526	15	940	(100%)'					
	4260	542	300	1010			1815	526	1800	940						
	6060	526	1800	1010			2235	549	420	1010						
6760	526	700	1010		2295		549	60	1010							
4. Loss of Feedwater Pumps 10 Cycles	0	526		1010	0		2296	542	1	1010	9. Improper Startup 1 Cycle		0	526	1010	3395
	3	526	3	1190	(0%)'	2356	542	60	1010	10. Shutdown 300 Cycles		0	549	1010	14147	
	13	526	10	1135		2357	526	1	1010			(50%)'				
	233	300	220	1135		27	130 ⁽¹⁾	26	1010							
	2213	500	1980	1135		28	526	1	1010							
	2393	300	180	885		728	526	700	1010							
	6773	500	4380	1135		11. Design Hydrostatic Test 120 Cycles	0	100				0	1981			
	7193	300	420	675	14147							1100	(7%)'			
	7493	300	300	675	(50%)'							50				
	11093	400	3600	240								50	1981			
	16457	549	5364	1010								1563	(7%)'			
	16517	549	60	1010								50				
	16518	542	1	1010	28294	12. Hydrostatic Test 1 Cycle	0	100		50	1981					
	17118	542	600	1010	(100%)'					50						
17119	526	1	1010						50							
17819	526	700	1010						50							
5. Turbine Generator Trip 60 Cycles	0	526		1010	28294					50						
	10	526	10	1135	(100%)'					50						
	15	526	5	1135					50							
	30	526	15	940					50							
	1830	526	1800	940					50							
	2250	549	420	1010					50							
	2310	549	60	1010					50							
	2311	542	1	1010					50							
	2911	542	600	1010					50							
	2912	526	1	1010					50							
	3612	526	700	1010					50							

- Notes:
1. The instant temperature change is assumed as 1 second time step.
 2. The number of cycles is for 60 years [2].
 3. 130°F is the safe end temperature for this transient. The blend radius has a different temperature for Transient 9. [2]

Note: These transients are the same as in Table 2 with the exception of the 700 second steady state time increment that is used. The transients in Table 2 are plotted using a 6000 second steady state increment. The difference is due to the length of the Green's Function for the safe end which is shorter compared to the blend Radius.

Table 4: Blend Radius Stress Summary

1	2	3	4	5	6	7	8	9	10	11	12	13
Transient Number	Time (s)	Total Stress (psi)	M+B Stress (psi)	Temperature F	Pressure (psig)	Total Pressure Stress (psi)	M+B Pressure Stress (psi)	Total Piping Stress (psi)	M+B Piping Stress (psi)	Total Total Stress (psi)	Total M+B Stress (psi)	Number of Cycles (60 years)
1	0	459	388	100.00	0	0	0	16.64312	16.64312	475.64	404.64	300
	4303	-3417	-1594	219.53	1010	31613	33976.4	-82.95209	-82.95209	28113.05	32299.45	300
	22164	2713	2306	549.00	1010	31613	33976.4	265.7352	265.7352	34591.74	36548.14	300
2	0.00	3094	1934	549	1010	31613	33976.4	265.7352	265.7352	34972.74	36176.14	300
	94.30	4079	2481	542	1010	31613	33976.4	261.8518	261.8518	35953.85	36719.25	300
	601.70	3683	2435	538.8	1010	31613	33976.4	260.0765	260.0765	35556.08	36671.48	300
3	680.10	5891	3489	526	1010	31613	33976.4	252.9754	252.9754	37756.98	37718.38	300
	6602.00	2977	1859	526	1010	31613	33976.4	252.9754	252.9754	34842.98	36088.38	300
	0.00	2959	1849	526	1010	31613	33976.4	252.9754	252.9754	34824.98	36078.38	10
4	1807.20	1834	1043	542	1010	31613	33976.4	261.8518	261.8518	33708.85	35281.25	10
	2491.50	4425	2667	526	1010	31613	33976.4	252.9754	252.9754	36290.98	36896.38	10
	3974.40	1706	1060	542	1010	31613	33976.4	261.8518	261.8518	33590.85	35298.25	10
5	6070.80	3971	2551	526	1010	31613	33976.4	252.9754	252.9754	35836.98	36780.38	10
	12060.00	2965	1852	526	1010	31613	33976.4	252.9754	252.9754	34830.98	36081.38	10
	0	2465	-703	526.00	1010	31613	33976.4	252.9754	252.9754	34330.98	33020.42	10
6	3	2465	-703	526.00	1190	37247	40031.6	252.9754	252.9754	39964.98	39075.62	10
	13	2465	-703	526.00	1135	35525.5	38181.4	252.9754	252.9754	38243.48	37225.42	10
	435.6	18138	9690	356.38	1135	35525.5	38181.4	158.8774	158.8774	53622.38	48030.28	10
7	2222.5	-1169	-2598	489.44	1135	35525.5	38181.4	-232.6952	-232.6952	34123.80	35350.70	10
	2665.5	12763	6695	328.40	885	27700.5	29771.4	143.3539	143.3539	40606.85	36609.75	10
	6779.2	-4008	-2829	497.05	1010	31613	33976.4	-236.9137	-236.9137	27368.09	30910.49	10
8	7243.8	19275	9965	302.91	1010	31613	33976.4	129.2122	129.2122	51017.21	44070.61	10
	13996	-2135	34	542.00	1010	31613	33976.4	-261.8518	-261.8518	29216.15	34272.25	10
	17247	-3413	2074	526.00	1010	31613	33976.4	252.9754	252.9754	35278.98	36303.38	10
9	23119	2971	1855	526.00	1010	31613	33976.4	252.9754	252.9754	34836.98	36084.38	10
	0.00	2959	1849	526	1010	31613	33976.4	252.9754	252.9754	34824.98	36078.38	60
	10.00	2959	1849	526	1135	35525.5	38181.4	252.9754	252.9754	38737.48	40283.38	60
10	15.00	2959	1849	526	940	29422	31621.6	252.9754	252.9754	32633.98	33723.58	60
	2269.50	111	295	549	1010	31613	33976.4	265.7352	265.7352	31989.74	34537.14	60
	3010.10	4407	2579	526	1010	31613	33976.4	252.9754	252.9754	36272.98	36808.38	60
11	8912.00	2968	1854	526	1010	31613	33976.4	252.9754	252.9754	34833.98	36083.38	60
	0.00	2959	1849	526.00	1010	31613	33976.4	252.9754	252.9754	34824.98	36078.38	1
	2.00	2959	1849	526.00	1375	43037.5	46255	252.9754	252.9754	46249.48	48356.98	1
12	32.00	2959	1849	526.00	940	29422	31621.6	252.9754	252.9754	32633.98	33723.58	1
	2271.50	111	295	549.00	1010	31613	33976.4	265.7352	265.7352	31989.74	34537.14	1
	3022.00	4407	2579	526.00	1010	31613	33976.4	252.9754	252.9754	36272.98	36808.38	1
13	8914.00	2968	1854	526.00	1010	31613	33976.4	252.9754	252.9754	34833.98	36083.38	1
	0.00	2959	1849	526	1010	31613	33976.4	252.9754	252.9754	34824.98	36078.38	1
	615.10	20280	12980	374.581	170	5321	5718.8	168.9726	168.9726	25749.97	18867.77	1
14	17580.00	279	179	70	50	1565	1682	0	0	1844.00	1861.00	1
	0.00	2959	1849	526	1010	31613	33976.4	252.9754	252.9754	34824.98	36078.38	228
	15.00	2959	1849	526	940	29422	31621.6	252.9754	252.9754	32633.98	33723.58	228
15	2254.50	111	295	549	1010	31613	33976.4	265.7352	265.7352	31989.74	34537.14	228
	2491.20	3792	2234	526	1010	31613	33976.4	252.9754	252.9754	35657.98	36463.38	228
	8357.00	2963	1851	526	1010	31613	33976.4	252.9754	252.9754	34828.98	36080.38	228
16	0	2058	961	525.8	1010	31613	33976.4	252.8645	252.8645	33923.86	35190.26	1
	0.52	1956	734	525.6	1010	31613	33976.4	252.7535	252.7535	33821.75	34963.15	1
	28	23747	3188	504.5	1010	31613	33976.4	241.0479	241.0479	55601.05	37405.45	1
17	425	1520	611	525.5	1010	31613	33976.4	252.698	252.698	33385.70	34840.10	1
	12400	2058	879	525.8	1010	31613	33976.4	252.8645	252.8645	33923.86	35108.26	1
	0	2767	2176	549	1010	31613	33976.4	265.7352	265.7352	34645.74	36416.14	300
18	4240.8	6643	4158	445.775	441	13803.3	14835.24	208.469	208.469	20654.77	19201.71	300
	6268	6498	3675	374.7	170	5321	5718.8	169.0386	169.0386	11988.04	9562.84	300
	6891.8	9282	5241	329.228	88	2754.4	2960.32	143.8121	143.8121	12180.21	8345.13	300
19	22224	361	120	70	50	1565	1682	0	0	1926.00	1802.00	300
	0	0	0	100	0	0	0	16.64312	16.64312	16.64	16.64	120
	0	0	0	100	1100	34430	37004	16.64312	16.64312	34446.64	37020.64	120
20	0	0	0	100	50	1565	1682	16.64312	16.64312	1581.64	1698.64	120
	0	0	0	100	50	1565	1682	16.64312	16.64312	1581.64	1698.64	1
	0	0	0	100	1563	48921.9	52579.32	16.64312	16.64312	48938.54	52595.96	1
21	0	0	0	100	50	1565	1682	16.64312	16.64312	1581.64	1698.64	1

- NOTES: Column 1: Transient number identification.
 Column 2: Time during transient where a maxima or minima stress intensity occurs from P-V.OUT output file.
 Column 3: Maxima or minima total stress intensity from P-V.OUT output file.
 Column 4: Maxima or minima membrane plus bending stress intensity from P-V.OUT output file.
 Column 5: Temperature per total stress intensity.
 Column 6: Pressure per Table 2.
 Column 7: Total pressure stress intensity from the quantity (Column 6 x 31300)/1000.
 Column 8: Membrane plus bending pressure stress intensity from the quantity (Column 6 x 33640)/1000.
 Column 9: Total external stress from calculation in Table 1, 280.16 psi*(Column 5-70°F)/(575°F-70°F).
 Column 10: Same as Column 9, but for M+B stress.
 Column 11: Sum of total stresses (Columns 3, 7, and 9).
 Column 12: Sum of membrane plus bending stresses (Columns 4, 8, and 10).
 Column 13: Number of cycles for the transient (60 years).

Table 5: Safe End Stress Summary

1	2	3	4	5	6	7	8	9	10	11	12	13
Transient Number	Time (s)	Total Stress (psi)	M+B Stress (psi)	Temperature F	Pressure (psig)	Total Pressure Stress (psi)	M+B Pressure Stress (psi)	Total Piping Stress (psi)	M+B Piping Stress (psi)	Total Total Stress (psi)	Total M+B Stress (psi)	Number of Cycles (60 years)
1	0	-925	-949	100.00	0	0	0	-339.1419	-339.1419	-1264.14	-1288.14	300
	16164	-4814	-4433	549.00	1010	11604.9	11463.5	-5414.966	-5414.966	1375.93	1615.53	300
	16864	-3749	-3705	549.00	1010	11604.9	11463.5	-5414.966	-5414.966	2440.93	2343.53	300
2	0	-3838	-3665	549	1010	11604.9	11463.5	-5414.966	-5414.966	2351.93	2383.53	300
	6	-1664	-2263	542	1010	11604.9	11463.5	-5335.833	-5335.833	4605.07	3864.67	300
	601	-3773	-3607	542	1010	11604.9	11463.5	-5335.833	-5335.833	2496.07	2520.67	300
3	606.6	1196	-403	526	1010	11604.9	11463.5	5154.958	-5154.958	17955.86	5905.54	300
	1302	-3670	-3509	526	1010	11604.9	11463.5	-5154.958	-5154.958	2779.94	2799.54	300
	0	-3688	-3522	526	1010	11604.9	11463.5	-5154.958	-5154.958	2761.94	2786.54	10
4	1800.1	-4165	-3904	542	1010	11604.9	11463.5	-5335.833	-5335.833	2104.07	2223.67	10
	2460.2	-1932	-2200	526	1010	11604.9	11463.5	-5154.958	-5154.958	4517.94	4108.54	10
	3960.2	-4537	-4185	542	1010	11604.9	11463.5	-5335.833	-5335.833	1732.07	1942.67	10
5	6060.2	-3315	-3241	526	1010	11604.9	11463.5	-5154.958	-5154.958	3134.94	3067.54	10
	6760	-3687	-3522	526	1010	11604.9	11463.5	-5154.958	-5154.958	2762.94	2786.54	10
	0.00	-3756	-3716	526	1020	11719.8	11577	-5154.958	-5154.958	2808.84	2706.04	10
6	3.00	-3756	-3716	526	1190	13673.1	13506.5	-5154.958	-5154.958	4762.14	4635.54	10
	13.00	-3756	-3716	526	1135	13041.15	12882.25	-5154.958	-5154.958	4130.19	4011.29	10
	242.30	15878	10049	302.374	1135	13041.15	12882.25	2626.926	2626.926	31546.08	2558.18	10
7	2213.10	-6388	-5428	499.889	1135	13041.15	12882.25	-4859.78	-4859.78	1793.37	2504.47	10
	2408.60	13203	8265	301.443	885	10168.65	10044.75	2616.401	2616.401	25988.05	20926.15	10
	6773.40	-4763	-4312	499.809	1135	13041.15	12882.25	-4858.875	-4858.875	3419.27	3711.37	10
8	7193.10	15374	9801	300	675	7755.75	7661.25	2600.088	2600.088	25728.84	20062.34	10
	16457.50	-4812	-5032	549	240	2757.6	2724	-5414.966	-5414.966	-7469.37	-7722.97	10
	16524.70	-2358	-2725	542	1010	11604.9	11463.5	-5335.833	-5335.833	3911.07	3402.67	10
9	17118.00	-3778	-3610	541.996	1010	11604.9	11463.5	-5335.822	-5335.822	2491.11	2517.71	10
	17123.60	1192	-406	526	1010	11604.9	11463.5	5154.958	-5154.958	17951.86	5902.54	10
	17819.00	-3670	-3509	526	1010	11604.9	11463.5	-5154.958	-5154.958	2779.94	2799.54	10
10	0.00	-3688	-3522	526	1010	11604.9	11463.5	-5154.958	-5154.958	2761.94	2786.54	60
	10.00	-3688	-3522	526	1135	13041.15	12882.25	-5154.958	-5154.958	4198.19	4205.29	60
	30.00	-3688	-3522	526	940	10800.6	10669	-5154.958	-5154.958	1957.64	1957.64	60
11	2250.10	-6054	-5337	549	1010	11604.9	11463.5	-5414.966	-5414.966	135.93	711.53	60
	2319.90	-2977	-3123	542	1010	11604.9	11463.5	-5335.833	-5335.833	3292.07	3004.67	60
	2911.00	-3782	-3613	541.999	1010	11604.9	11463.5	-5335.822	-5335.822	2487.08	2514.68	60
12	2916.70	1188	-408	526	1010	11604.9	11463.5	5154.958	-5154.958	17947.86	5900.54	60
	3612.00	-3670	-3509	526	1010	11604.9	11463.5	-5154.958	-5154.958	2779.94	2799.54	60
	0.00	-3688	-3522	526	1010	11604.9	11463.5	-5154.958	-5154.958	2761.94	2786.54	1
13	2.00	-3688	-3522	5.26E+02	1375	15798.75	15606.25	-5154.958	-5154.958	6955.79	6929.29	1
	32.00	-3688	-3522	5.26E+02	940	10800.6	10669	-5154.958	-5154.958	1957.64	1957.64	1
	2252.10	-6054	-5337	5.49E+02	1010	11604.9	11463.5	-5414.966	-5414.966	135.93	711.53	1
14	2322.20	-2977	-3123	5.42E+02	1010	11604.9	11463.5	-5335.833	-5335.833	3292.07	3004.67	1
	2913.00	-3782	-3613	5.42E+02	1010	11604.9	11463.5	-5335.822	-5335.822	2487.08	2514.68	1
	2918.70	1188	-408	5.26E+02	1010	11604.9	11463.5	5154.958	-5154.958	17947.86	5900.54	1
15	3614.00	-3670	-3509	5.26E+02	1010	11604.9	11463.5	-5154.958	-5154.958	2779.94	2799.54	1
	0	-3688	-3522	526	1010	11604.9	11463.5	-5154.958	-5154.958	2761.94	2786.54	1
	600	7773	5336	375	170	1953.3	1929.5	3447.943	3447.943	13174.24	10713.44	1
16	1367.90	-1390	-1567	354.172	162	1861.38	1838.7	-3212.488	-3212.488	-2741.11	-2940.79	1
	11580.1	454	190	70	50	574.5	567.5	0	0	1028.50	757.50	1
	12280	-707	-689	70	50	574.5	567.5	0	0	-132.50	-121.50	1
17	0.00	-3688	-3522	526	1010	11604.9	11463.5	-5154.958	-5154.958	2761.94	2786.54	228
	15.00	-3688	-3522	526	940	10800.6	10669	-5154.958	-5154.958	1957.64	1957.64	228
	2235.10	-6054	-5337	549	1010	11604.9	11463.5	-5414.966	-5414.966	135.93	711.53	228
18	2305.20	-2977	-3123	542	1010	11604.9	11463.5	-5335.833	-5335.833	3292.07	3004.67	228
	2356.00	-3183	-3151	541.999	1010	11604.9	11463.5	-5335.822	-5335.822	3086.08	2976.68	228
	2361.50	1761	-28	526	1010	11604.9	11463.5	5154.958	-5154.958	18520.86	6280.54	228
19	3057.00	-3667	-3506	526	1010	11604.9	11463.5	-5154.958	-5154.958	2782.94	2802.54	228
	0	-2968	-2837	525.7	1010	11604.9	11463.5	-5151.566	-5151.566	3485.22	3474.82	1
	27	68473	45303	291.3	1010	11604.9	11463.5	2501.737	2501.737	82579.74	59268.34	1
20	80.7	-11546	-8877	518.4	1010	11604.9	11463.5	-5069.042	-5069.042	-5010.04	-2482.14	1
	5200	-2967	-2832	525.7	1010	11604.9	11463.5	-5151.566	-5151.566	3486.21	3479.78	1
	0	-3745	-3709	549	1010	11604.9	11463.5	-5414.966	-5414.966	2444.93	2339.53	300
21	6864.2	501	-405	329.994	170	1953.3	1929.5	2939.162	2939.162	5393.46	-1414.66	300
	7455.5	-1183	-1528	314.325	88	1011.12	998.8	-2762.029	-2762.029	-2933.91	-3291.23	300
	16224.1	334	-35	70	50	574.5	567.5	0	0	908.50	532.50	300
22	16924	-731	-763	70	50	574.5	567.5	0	0	-156.50	-195.50	300
	0	0	0	100	0	0	0	339.1419	339.1419	339.14	339.14	120
	0	0	0	100	1100	12639	12485	339.1419	339.1419	12978.14	12824.14	120
23	0	0	0	100	50	574.5	567.5	339.1419	339.1419	913.64	906.64	120
	0	0	0	100	50	574.5	567.5	339.1419	339.1419	913.64	906.64	1
	0	0	0	100	1563	17958.87	17740.05	339.1419	339.1419	18298.01	18079.19	1
24	0	0	0	100	50	574.5	567.5	339.1419	339.1419	913.64	906.64	1

NOTES: Column 1: Transient number identification.

- Column 2: Time during transient where a maxima or minima stress intensity occurs from P-V.OUT output file.
- Column 3: Maxima or minima total stress intensity from P-V.OUT output file.
- Column 4: Maxima or minima membrane plus bending stress intensity from P-V.OUT output file.
- Column 5: Temperature per total stress intensity.
- Column 6: Pressure per Table 3.
- Column 7: Total pressure stress intensity from the quantity (Column 6 x 11490)/1000.
- Column 8: Membrane plus bending pressure stress intensity from the quantity (Column 6 x 11350)/1000.
- Column 9: Total external stress from calculation in Table 1, 5708.89 psi*(Column 5-70°F)/(575°F -70°F).
- Column 10: Same as Column 9, but for M+B stress.
- Column 11: Sum of total stresses (Columns 3, 7, and 9).
- Column 12: Sum of membrane plus bending stresses (Columns 4, 8, and 10).
- Column 13: Number of cycles for the transient (60 years).

Table 6: Fatigue Results for Blend Radius (60 Years)

LOCATION = LOCATION NO. 2 -- BLEND RADIUS
 FATIGUE CURVE = 1 (1 = CARBON/LOW ALLOY, 2 = STAINLESS STEEL)
 m = 2.0
 n = .2
 Sm = 26700. psi
 Ecurve = 3.000E+07 psi
 Eanalysis = 2.670E+07 psi
 Kt = 1.00

MAX	MIN	RANGE	MEM+BEND	Ke	Salt	Napplied	Nallowed	U
55601.	17.	55584.	37389.	1.000	31227.	1.000E+00	1.951E+04	.0001
53822.	17.	53806.	48014.	1.000	30228.	1.000E+01	2.161E+04	.0005
51017.	17.	51001.	44054.	1.000	28652.	1.000E+01	2.547E+04	.0004
48939.	17.	48922.	52579.	1.000	27484.	1.000E+00	2.894E+04	.0000
46249.	17.	46233.	48340.	1.000	25974.	1.000E+00	3.443E+04	.0000
40607.	17.	40590.	36593.	1.000	22803.	1.000E+01	5.217E+04	.0002
39965.	17.	39948.	39059.	1.000	22443.	1.000E+01	5.647E+04	.0002
38737.	17.	38721.	40267.	1.000	21753.	6.000E+01	6.592E+04	.0009
38243.	17.	38227.	37209.	1.000	21476.	1.000E+01	7.025E+04	.0001
37757.	17.	37740.	37702.	1.000	21202.	7.000E+00	7.486E+04	.0001
37757.	476.	37281.	37314.	1.000	20945.	2.930E+02	7.954E+04	.0037
36291.	476.	35815.	36492.	1.000	20121.	7.000E+00	9.705E+04	.0001
36291.	1582.	34709.	35198.	1.000	19500.	3.000E+00	1.096E+05	.0000
36273.	1582.	34691.	35110.	1.000	19490.	6.000E+01	1.098E+05	.0005
36273.	1582.	34691.	35110.	1.000	19490.	1.000E+00	1.098E+05	.0000
35954.	1582.	34372.	35021.	1.000	19310.	5.600E+01	1.135E+05	.0005
35954.	1582.	34372.	35021.	1.000	19310.	1.000E+00	1.135E+05	.0000
35954.	1582.	34372.	35021.	1.000	19310.	1.000E+00	1.135E+05	.0000
35954.	1844.	34110.	34858.	1.000	19163.	1.000E+00	1.167E+05	.0000
35954.	1926.	34028.	34917.	1.000	19117.	2.410E+02	1.177E+05	.0020
35837.	1926.	33911.	34978.	1.000	19051.	1.000E+01	1.191E+05	.0001
35658.	1926.	33732.	34661.	1.000	18951.	4.900E+01	1.214E+05	.0004
35658.	11988.	23670.	26901.	1.000	13298.	1.790E+02	5.728E+05	.0003
35556.	11988.	23568.	27109.	1.000	13240.	1.210E+02	5.955E+05	.0002
35556.	12180.	23376.	28326.	1.000	13133.	1.790E+02	6.411E+05	.0003
35279.	12180.	23099.	27958.	1.000	12977.	1.000E+01	7.138E+05	.0000
34973.	12180.	22793.	27831.	1.000	12805.	1.110E+02	8.050E+05	.0001
34973.	20655.	14318.	16974.	1.000	8044.	1.890E+02	7.421E+07	.0000
34843.	20655.	14188.	16887.	1.000	7971.	1.110E+02	7.983E+07	.0000
34843.	25750.	9093.	17221.	1.000	5108.	1.000E+00	1.000E+20	.0000
34843.	27368.	7475.	5178.	1.000	4199.	1.000E+01	1.000E+20	.0000
34843.	28113.	6730.	3789.	1.000	3781.	1.780E+02	1.000E+20	.0000
34837.	28113.	6724.	3785.	1.000	3777.	1.000E+01	1.000E+20	.0000
34834.	28113.	6721.	3784.	1.000	3776.	6.000E+01	1.000E+20	.0000
34834.	28113.	6721.	3784.	1.000	3776.	1.000E+00	1.000E+20	.0000
34831.	28113.	6718.	3782.	1.000	3774.	1.000E+01	1.000E+20	.0000
34829.	28113.	6716.	3781.	1.000	3773.	4.100E+01	1.000E+20	.0000
34829.	29216.	5613.	1808.	1.000	3153.	1.000E+01	1.000E+20	.0000
34829.	31990.	2839.	1543.	1.000	1595.	6.000E+01	1.000E+20	.0000
34829.	31990.	2839.	1543.	1.000	1595.	1.000E+00	1.000E+20	.0000
34829.	31990.	2839.	1543.	1.000	1595.	1.160E+02	1.000E+20	.0000
34825.	31990.	2835.	1541.	1.000	1593.	1.000E+01	1.000E+20	.0000
34825.	31990.	2835.	1541.	1.000	1593.	6.000E+01	1.000E+20	.0000



34825.	31990.	2835.	1541.	1.000	1593.	1.000E+00	1.000E+20	.0000
34825.	31990.	2835.	1541.	1.000	1593.	1.000E+00	1.000E+20	.0000
34825.	31990.	2835.	1541.	1.000	1593.	4.000E+01	1.000E+20	.0000
34825.	32634.	2191.	2355.	1.000	1231.	6.000E+01	1.000E+20	.0000
34825.	32634.	2191.	2355.	1.000	1231.	1.000E+00	1.000E+20	.0000
34825.	32634.	2191.	2355.	1.000	1231.	1.270E+02	1.000E+20	.0000
34646.	32634.	2012.	2695.	1.000	1130.	1.010E+02	1.000E+20	.0000
34646.	33386.	1260.	1578.	1.000	708.	1.000E+00	1.000E+20	.0000
34646.	33581.	1065.	1120.	1.000	598.	1.000E+01	1.000E+20	.0000
34646.	33709.	937.	1137.	1.000	526.	1.000E+01	1.000E+20	.0000
34646.	33822.	824.	1455.	1.000	463.	1.000E+00	1.000E+20	.0000
34646.	33924.	722.	1228.	1.000	406.	1.000E+00	1.000E+20	.0000
34646.	33924.	722.	1310.	1.000	406.	1.000E+00	1.000E+20	.0000
34646.	34124.	522.	1067.	1.000	293.	1.000E+01	1.000E+20	.0000
34646.	34331.	315.	3398.	1.000	177.	1.000E+01	1.000E+20	.0000
34646.	34447.	199.	-603.	1.000	112.	1.200E+02	1.000E+20	.0000
34646.	34592.	54.	-130.	1.000	30.	3.500E+01	1.000E+20	.0000

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TOTAL USAGE FACTOR = .0108

Table 7: Fatigue Results for Safe End (60 Years)

LOCATION = LOCATION NO. 1 -- SAFE END
 FATIGUE CURVE = 2 (1 = CARBON/LOW ALLOY, 2 = STAINLESS STEEL)
 m = 1.7
 n = .3
 Sm = 17000. psi
 Ecurve = 2.830E+07 psi
 Eanalysis = 2.700E+07 psi
 Kt = 1.53

MAX	MIN	RANGE	MEM+BEND	Ke	Salt	Napplied	Nallowed	U
82580.	-7469.	90049.	66991.	2.045	134573.	1.000E+00	6.765E+02	.0015
31546.	-7469.	39015.	33281.	1.000	29691.	9.000E+00	6.857E+05	.0000
31546.	-5010.	36556.	28040.	1.000	26947.	1.000E+00	1.160E+06	.0000
25988.	-2934.	28922.	24217.	1.000	21884.	1.000E+01	2.383E+06	.0000
25730.	-2934.	28664.	23354.	1.000	21509.	1.000E+01	2.566E+06	.0000
18521.	-2934.	21455.	9572.	1.000	13903.	2.280E+02	9.710E+08	.0000
18298.	-2934.	21232.	21370.	1.000	17063.	1.000E+00	7.876E+06	.0000
17956.	-2934.	20890.	9197.	1.000	13502.	5.100E+01	1.000E+20	.0000
17956.	-2741.	20697.	8846.	1.000	13304.	1.000E+00	1.000E+20	.0000
17956.	-1264.	19220.	7194.	1.000	12071.	2.480E+02	1.000E+20	.0000
17952.	-1264.	19216.	7191.	1.000	12068.	1.000E+01	1.000E+20	.0000
17948.	-1264.	19212.	7189.	1.000	12065.	4.200E+01	1.000E+20	.0000
17948.	-157.	18104.	6096.	1.000	11181.	1.800E+01	1.000E+20	.0000
17948.	-157.	18104.	6096.	1.000	11181.	1.000E+00	1.000E+20	.0000
13174.	-157.	13331.	10909.	1.000	10016.	1.000E+00	1.000E+20	.0000
12978.	-157.	13135.	13020.	1.000	10500.	1.200E+02	1.000E+20	.0000
6956.	-157.	7112.	7125.	1.000	5706.	1.000E+00	1.000E+20	.0000
5393.	-157.	5550.	-1219.	1.000	2570.	1.590E+02	1.000E+20	.0000
5393.	-133.	5526.	-1293.	1.000	2537.	1.000E+00	1.000E+20	.0000
5393.	136.	5258.	-2126.	1.000	2165.	6.000E+01	1.000E+20	.0000
5393.	136.	5258.	-2126.	1.000	2165.	1.000E+00	1.000E+20	.0000
5393.	136.	5258.	-2126.	1.000	2165.	7.900E+01	1.000E+20	.0000
4762.	136.	4626.	3924.	1.000	3514.	1.000E+01	1.000E+20	.0000
4605.	136.	4469.	3153.	1.000	3218.	1.390E+02	1.000E+20	.0000
4605.	339.	4266.	3526.	1.000	3215.	1.200E+02	1.000E+20	.0000
4605.	909.	3697.	3332.	1.000	2863.	4.100E+01	1.000E+20	.0000
4518.	909.	3609.	3576.	1.000	2885.	1.000E+01	1.000E+20	.0000
4198.	909.	3290.	3673.	1.000	2744.	6.000E+01	1.000E+20	.0000
4130.	909.	3222.	3479.	1.000	2655.	1.000E+01	1.000E+20	.0000
3911.	909.	3003.	2870.	1.000	2371.	1.000E+01	1.000E+20	.0000
3486.	909.	2578.	2947.	1.000	2170.	1.000E+00	1.000E+20	.0000
3485.	909.	2577.	2942.	1.000	2168.	1.000E+00	1.000E+20	.0000
3419.	909.	2511.	3179.	1.000	2199.	1.000E+01	1.000E+20	.0000
3292.	909.	2384.	2472.	1.000	1936.	6.000E+01	1.000E+20	.0000
3292.	909.	2384.	2472.	1.000	1936.	1.000E+00	1.000E+20	.0000
3292.	909.	2384.	2472.	1.000	1936.	9.600E+01	1.000E+20	.0000
3292.	914.	2378.	2098.	1.000	1829.	1.200E+02	1.000E+20	.0000
3292.	914.	2378.	2098.	1.000	1829.	1.000E+00	1.000E+20	.0000
3292.	914.	2378.	2098.	1.000	1829.	1.000E+00	1.000E+20	.0000
3292.	1029.	2264.	2247.	1.000	1810.	1.000E+00	1.000E+20	.0000
3292.	1376.	1916.	1389.	1.000	1390.	9.000E+00	1.000E+20	.0000
3135.	1376.	1759.	1452.	1.000	1325.	1.000E+01	1.000E+20	.0000
3086.	1376.	1710.	1361.	1.000	1274.	2.280E+02	1.000E+20	.0000



2809.	1376.	1433.	1091.	1.000	1054.	1.000E+01	1.000E+20	.0000
2783.	1376.	1407.	1187.	1.000	1067.	4.300E+01	1.000E+20	.0000
2783.	1732.	1051.	860.	1.000	790.	1.000E+01	1.000E+20	.0000
2783.	1793.	990.	208.	1.000	576.	1.000E+01	1.000E+20	.0000
2783.	1958.	825.	811.	1.000	658.	6.000E+01	1.000E+20	.0000
2783.	1958.	825.	811.	1.000	658.	1.000E+00	1.000E+20	.0000
2783.	1958.	825.	811.	1.000	658.	1.040E+02	1.000E+20	.0000
2780.	1958.	822.	808.	1.000	655.	1.240E+02	1.000E+20	.0000
2780.	2104.	676.	576.	1.000	514.	1.000E+01	1.000E+20	.0000
2780.	2352.	428.	416.	1.000	340.	1.660E+02	1.000E+20	.0000
2780.	2352.	428.	416.	1.000	340.	1.000E+01	1.000E+20	.0000
2780.	2352.	428.	416.	1.000	340.	6.000E+01	1.000E+20	.0000
2780.	2352.	428.	416.	1.000	340.	1.000E+00	1.000E+20	.0000
2763.	2352.	411.	403.	1.000	327.	1.000E+01	1.000E+20	.0000
2762.	2352.	410.	403.	1.000	327.	1.000E+01	1.000E+20	.0000
2762.	2352.	410.	403.	1.000	327.	4.300E+01	1.000E+20	.0000
2762.	2441.	321.	443.	1.000	291.	1.700E+01	1.000E+20	.0000
2762.	2441.	321.	443.	1.000	291.	1.000E+00	1.000E+20	.0000
2762.	2441.	321.	443.	1.000	291.	1.000E+00	1.000E+20	.0000
2762.	2441.	321.	443.	1.000	291.	2.280E+02	1.000E+20	.0000
2496.	2441.	55.	177.	1.000	78.	5.300E+01	1.000E+20	.0000
2496.	2445.	51.	181.	1.000	77.	2.470E+02	1.000E+20	.0000
2491.	2445.	46.	178.	1.000	74.	1.000E+01	1.000E+20	.0000
2487.	2445.	42.	175.	1.000	71.	4.300E+01	1.000E+20	.0000
2487.	2487.	0.	0.	1.000	0.	1.700E+01	1.000E+20	.0000

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TOTAL USAGE FACTOR = .0015

Table 8: Material Properties (For Transient 9)⁽¹⁾

Material	SA-533 Gr B @400 °F (Mn-1/2Mo- 1/2Ni)	SA-508 Cl 2 @400 °F (3/4Ni-1/2Mo- 1/3Cr-V)	SA-240 Type 304 @400 °F (18Cr-8Ni)	SA-182 F316 @300 °F (16Cr-12Ni- 2Mo)
Modulus of Elasticity, e^{-6} psi	27.4	26.1	26.5	27.0
Coefficient of Thermal Expansion, e^{-6} , in/in/°F	8.0	7.7	10.2	9.8
Thermal Conductivity, Btu/hr-ft-°F	23.1	23.1	10.4	9.3
Thermal Diffusivity, ft ² /hr	0.378	0.378	0.165	0.150
Specific Heat, Btu/lb-°F ⁽²⁾	0.125	0.125	0.129	0.127
Density, lb/in ³	0.283	0.283	0.283	0.283
Poisson's Ratio	0.3	0.3	0.3	0.3

Notes: ⁽¹⁾ Material Properties are evaluated at 400°F from the 1998 ASME Code, Section II, Part D, with 2000 Addenda, except for density and Poisson's ratio, which are assumed typical values. This is consistent with information provided in the Design Input Record (page 13 of VY EC No. 1773, SI File No. VY-16Q-209). The use of a later code edition than that used for the original design code is acceptable since later editions typically reflect more accurate material properties than was published in prior Code editions. The safe end material properties were used for 300°F, the Code table values closest to the average temperature for the safe end for transient 9.

⁽²⁾ Calculated as $[k/(\rho d)]/12^3$.

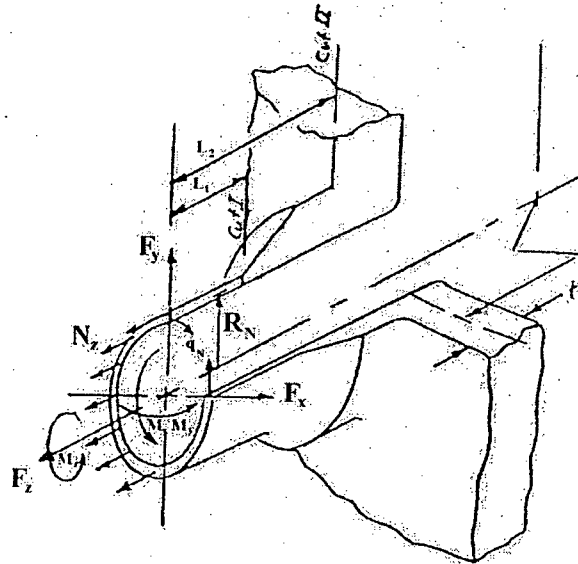


Figure 1: External Forces and Moments on the Recirculation Outlet Nozzle

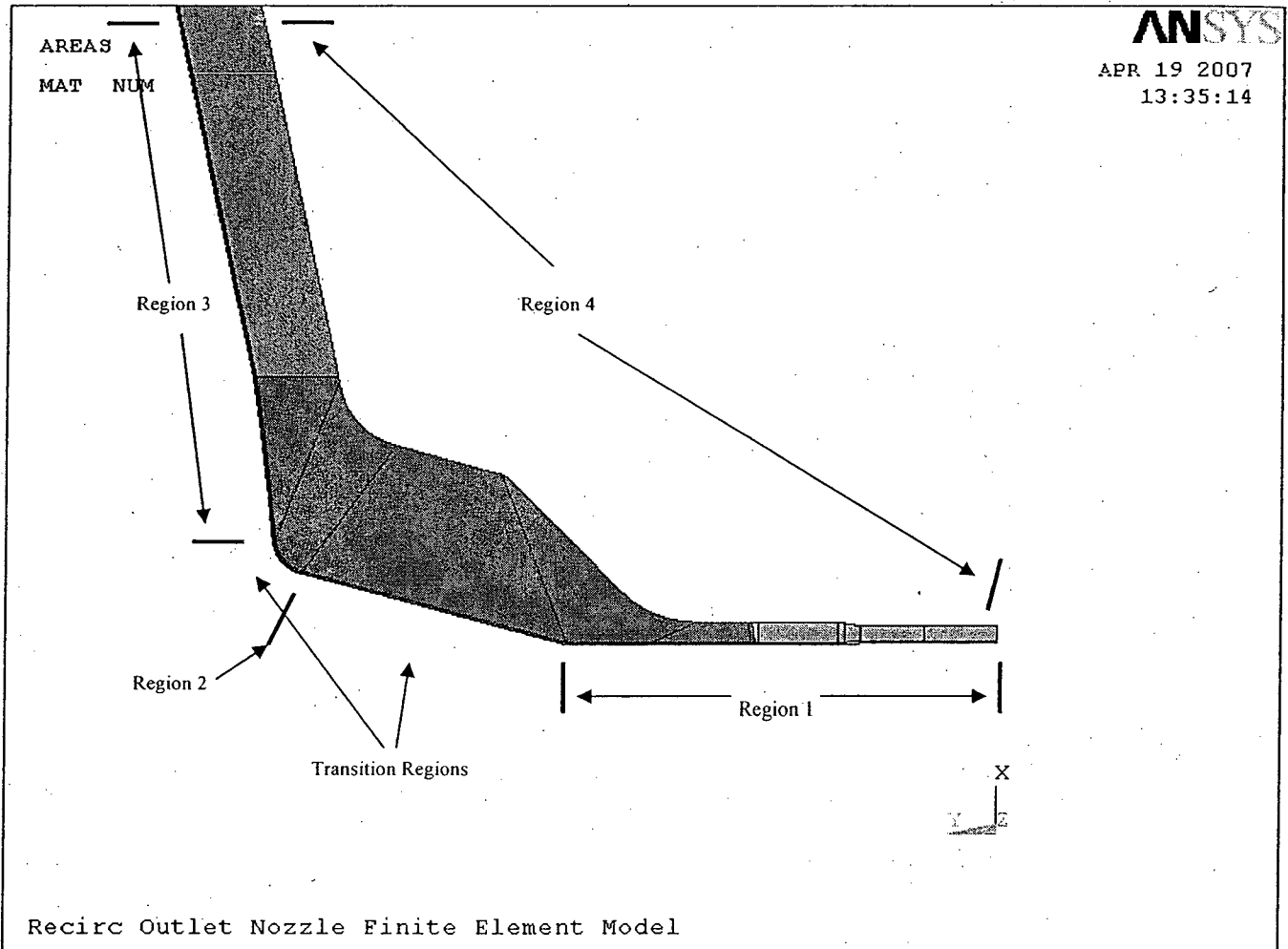


Figure 2: Nozzle and Vessel Wall Thermal and Heat Transfer Boundaries for Transient 9

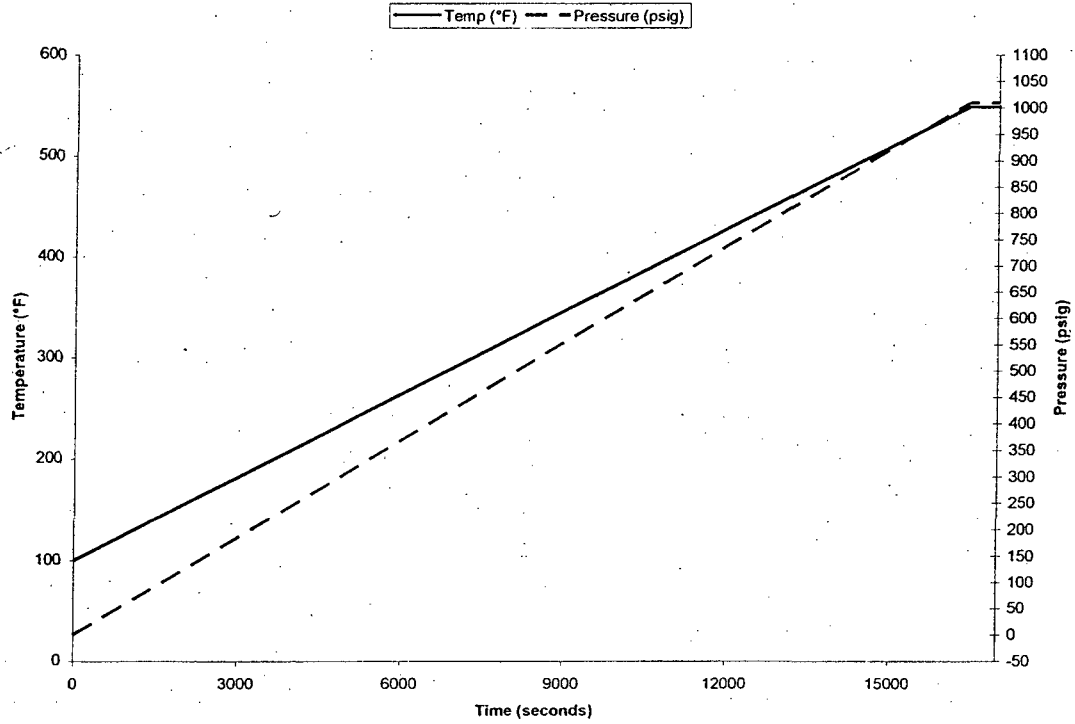


Figure 3: Transient 1 – Normal Startup at 100°F/hr

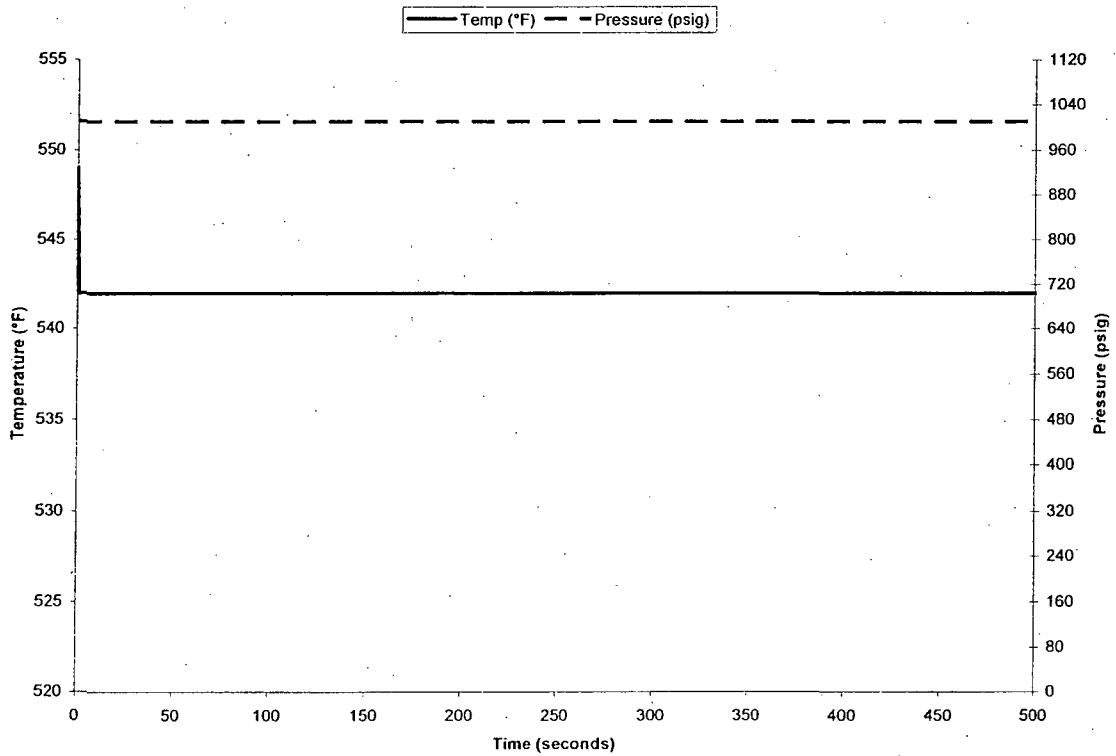


Figure 4: Transient 2 – Turbine Roll and Increase to Rated Power

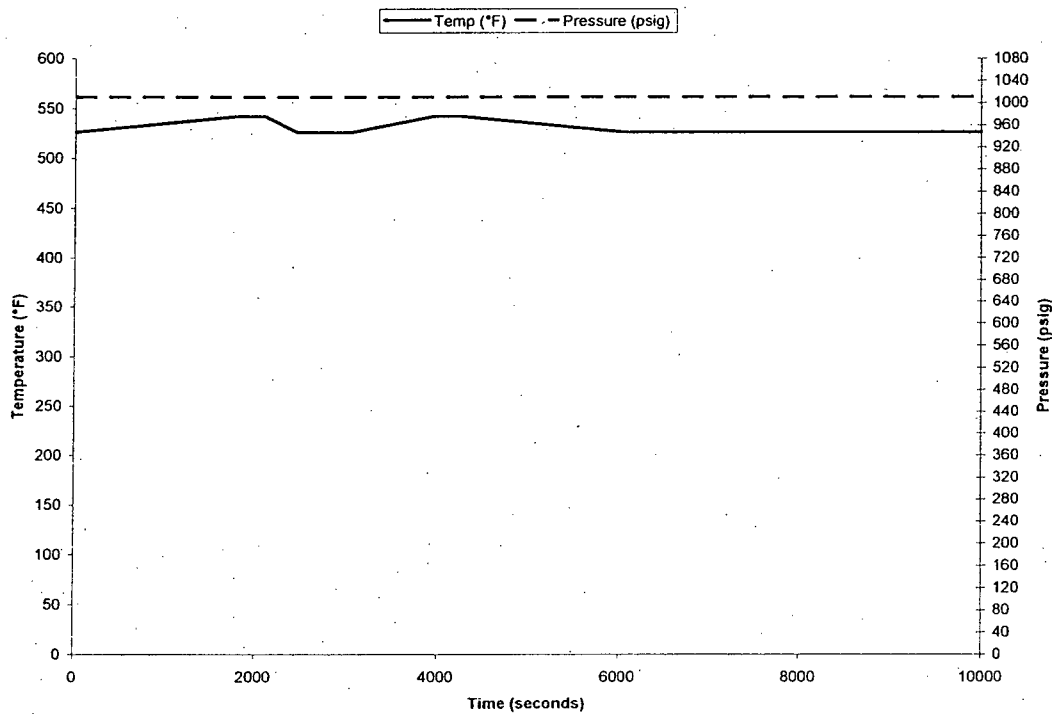


Figure 5: Transient 3 – Loss of Feedwater Heaters and Turbine Trip 25% Power

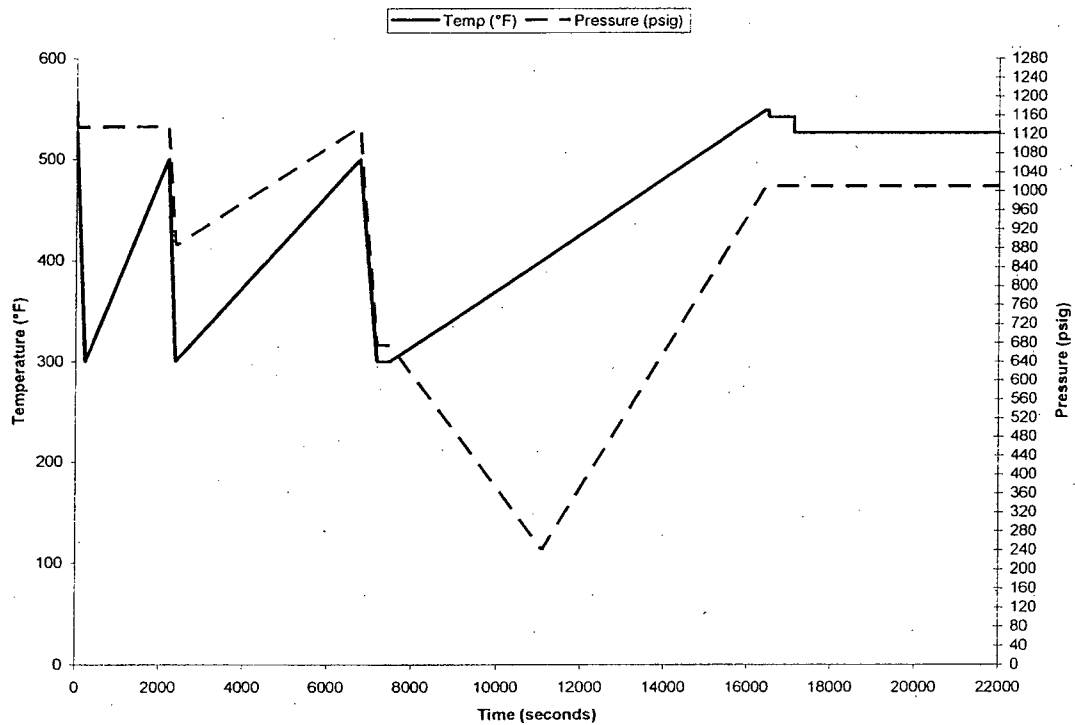


Figure 6: Transient 4 – Loss of Feedwater Pumps

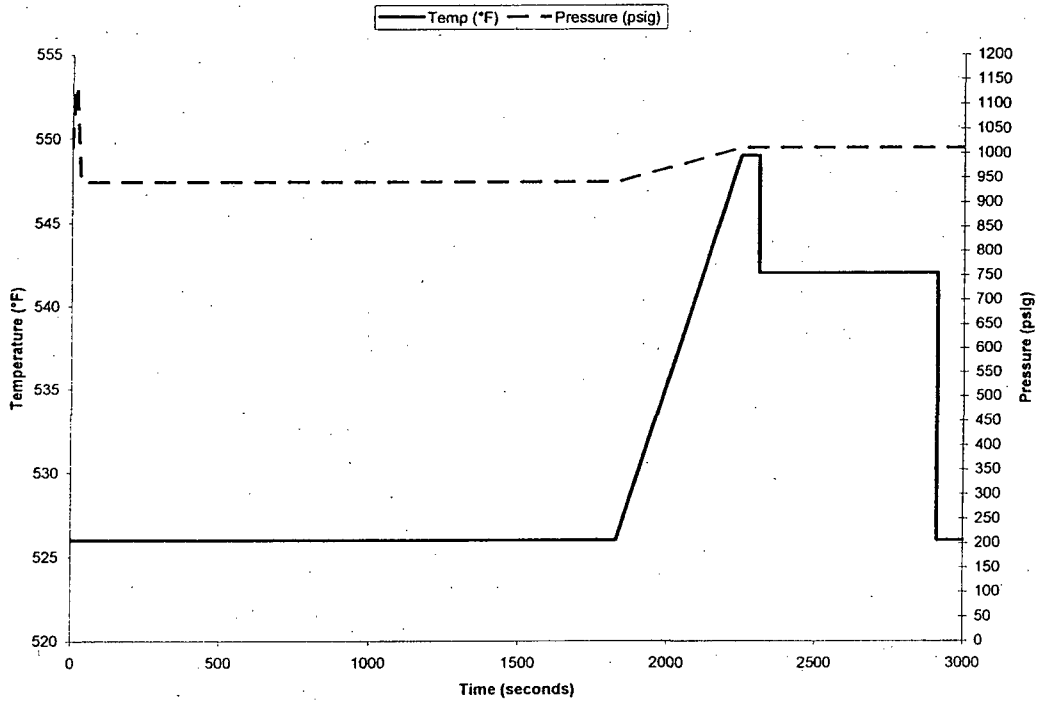


Figure 7: Transient 5 – Turbine Generator Trip

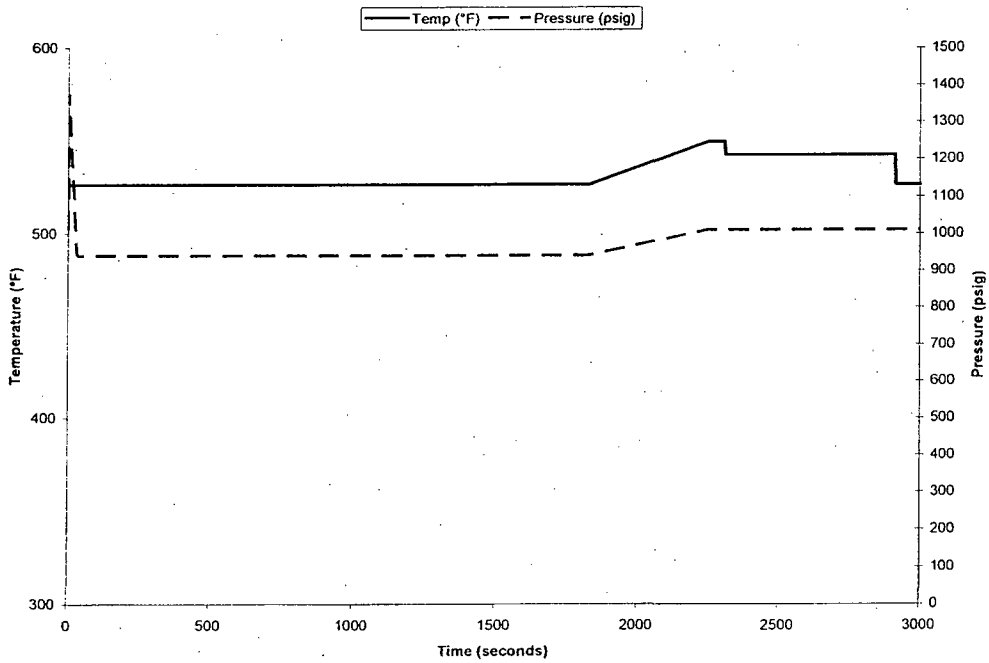


Figure 8: Transient 6 – Reactor Overpressure

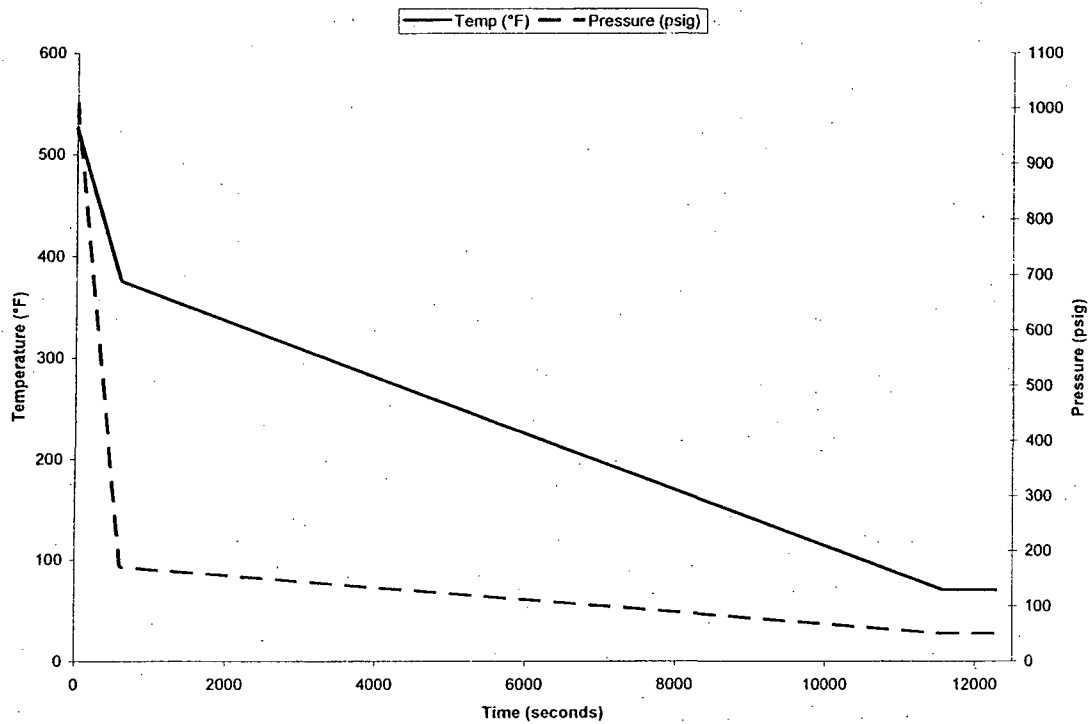


Figure 9: Transient 7 - SRV Blowdown

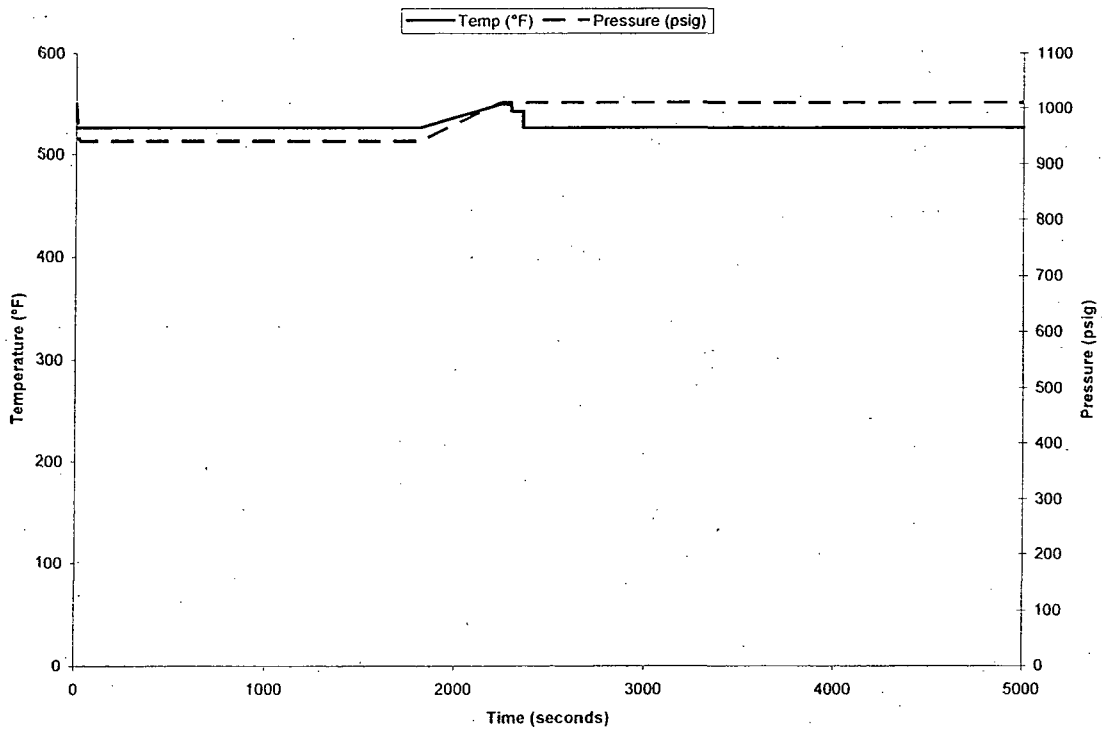


Figure 10: Transient 8 - SCRAM Other

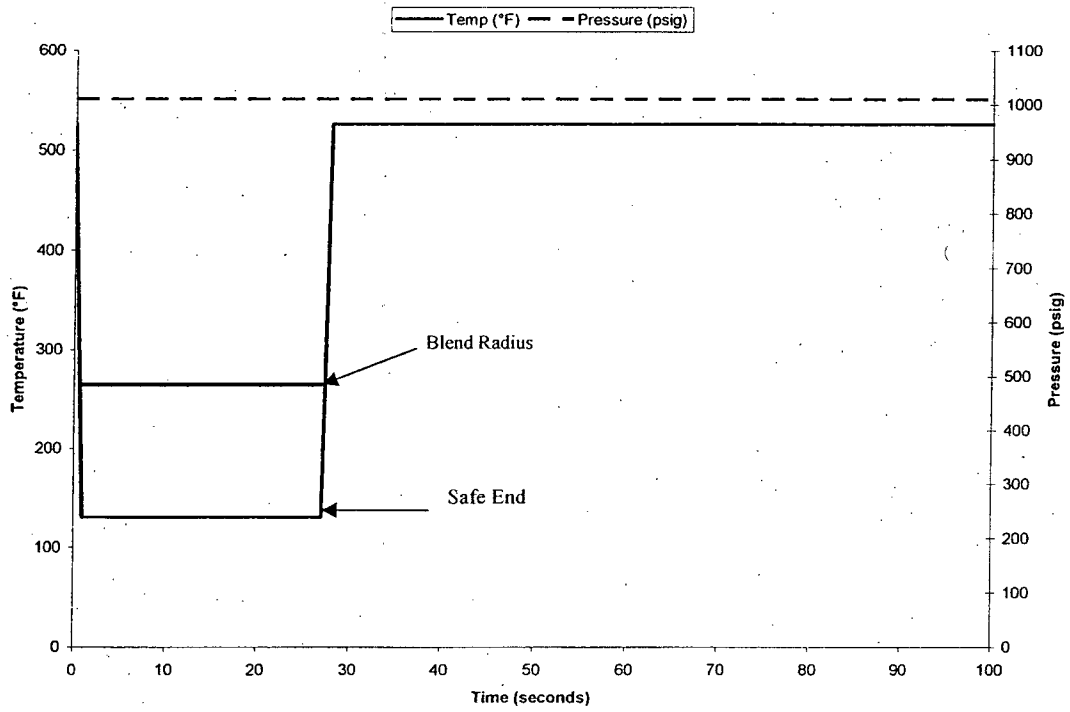


Figure 11: Transient 9 – Improper Startup

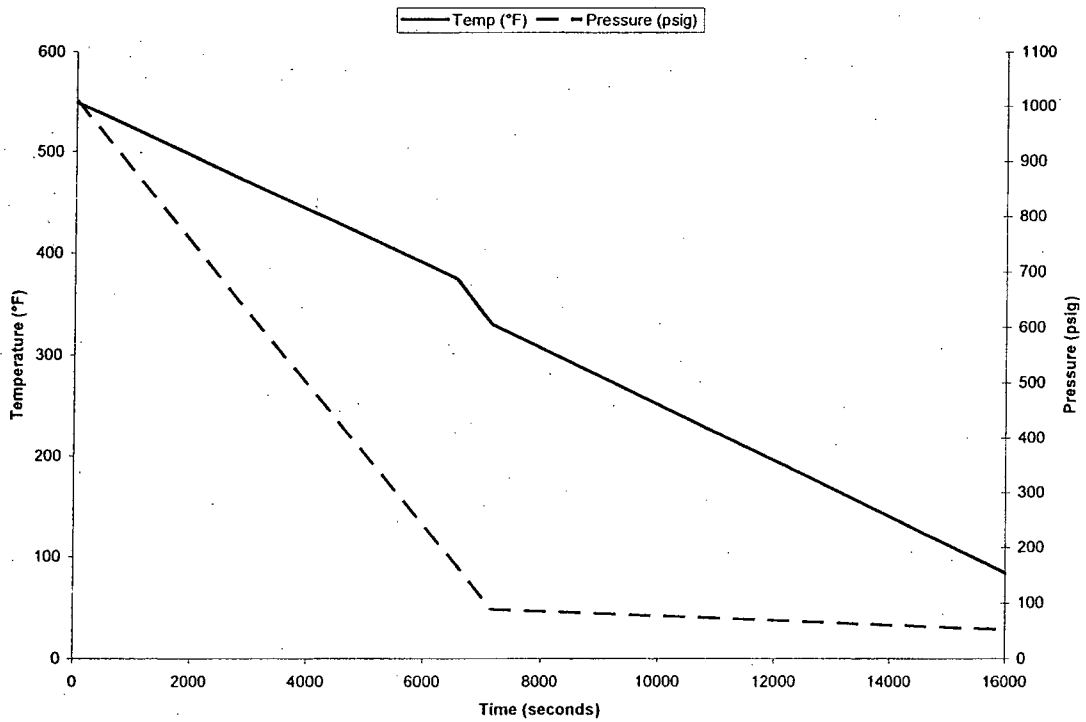
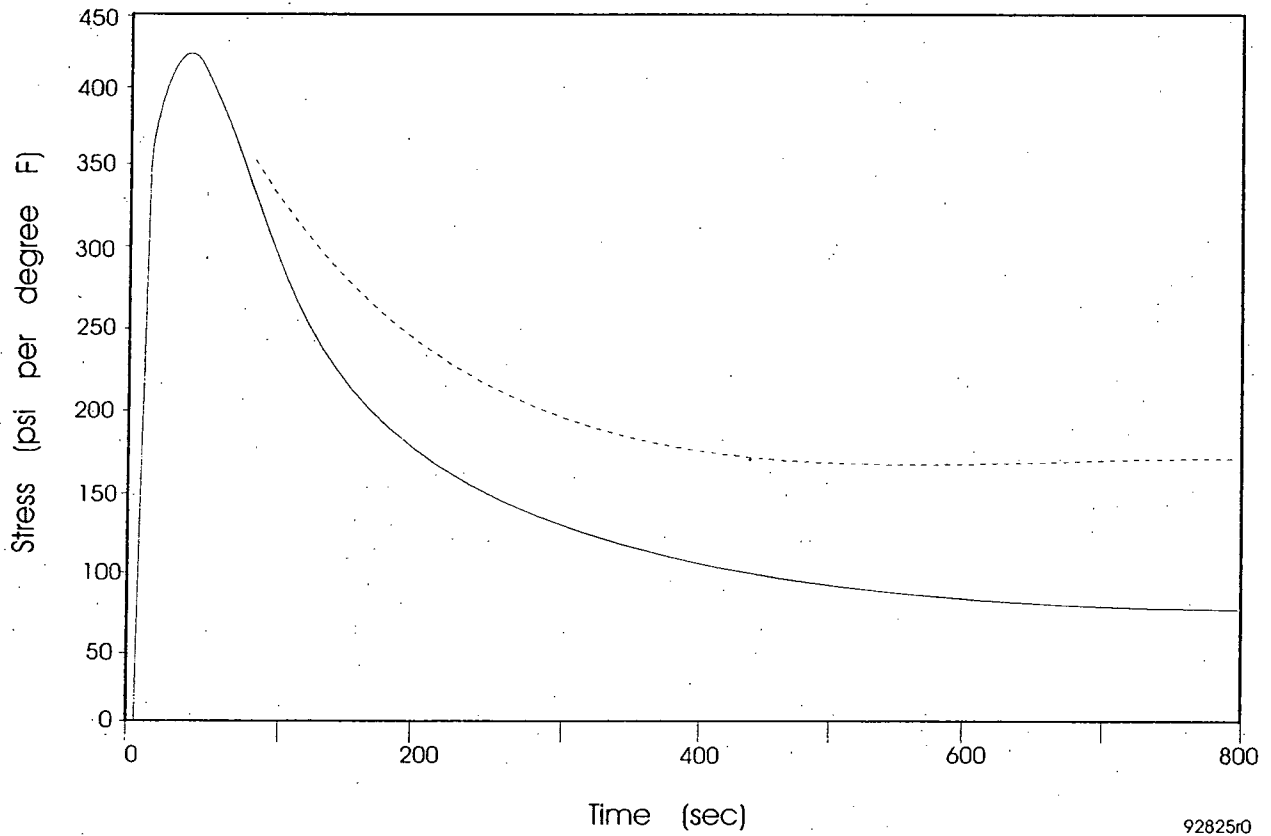


Figure 12: Transient 10 – Shutdown



92825r0

Note: A typical set of two Green's Functions is shown, each for a different set of heat transfer coefficients (representing different flow rate conditions).

Figure 13: Typical Green's Functions for Thermal Transient Stress

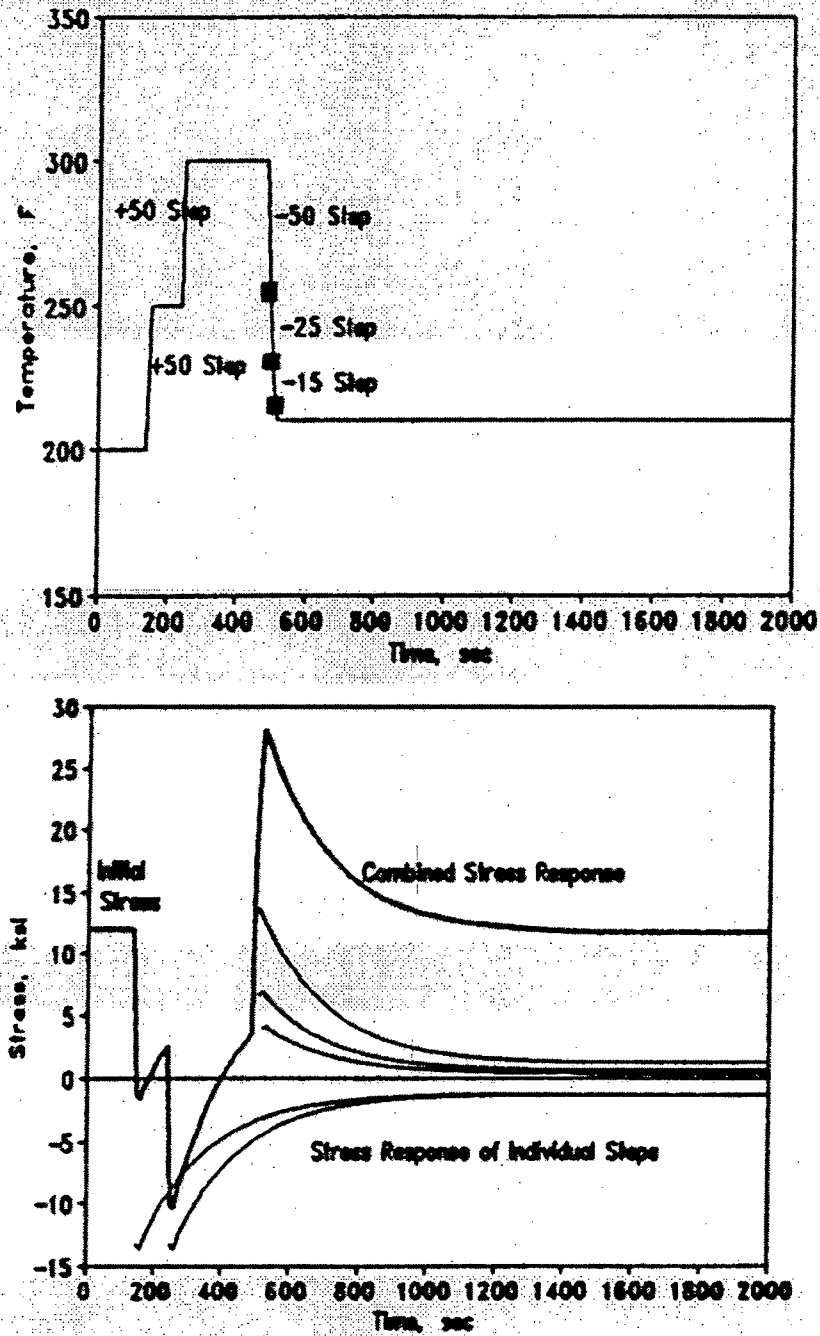


Figure 14: Typical Stress Response Using Green's Functions

APPENDIX A

SUMMARY OF OUTPUT FILES

VY RON T T9.INP	Input File for Transient 9 Thermal Analysis	In Computer files
VY RON S T9.INP	Input File for Transient 9 Stress Analysis	In Computer files
LFSE.OUT	Stress Output at Safe End	In Computer files
LFBR.OUT	Stress Output at Blend Radius	In Computer files
LFSE INSIDE.RED	Stress Extracted at Safe End	In Computer files
LFBR INSIDE.RED	Stress Extracted at Blend Radius	In Computer files
LFSE T.XLS	Stress Results with Total Stress at Safe End	In Computer files
LFSE_M+B.XLS	Stress Results with Membrane plus Bending Stress at Safe End	In Computer files
LFBR T.XLS	Stress Results with Total Stress at Blend Radius	In Computer files
LFBR_M+B.XLS	Stress Results with Membrane plus Bending Stress at Blend Radius	In Computer files
T9SE.OUT	Transient 9 Safe End stress output	In Computer files
T9BR.OUT	Transient 9 Blend Radius stress output	In Computer files
T9SE Inside.RED	Transient 9 Stress Extracted at Safe End	In Computer files
T9BR Inside.RED	Transient 9 Stress Extracted at Blend Radius	In Computer files
T9BR_M+B.xls	Transient 9 Stress Results with Membrane plus Bending Stress at Blend Radius	In Computer files
T9BR T.xls	Transient 9 Stress Results with Total Stress at Blend Radius	In Computer files
T9SE_M+B.xls	Transient 9 Stress Results with Membrane plus Bending Stress at Safe End	In Computer files
T9SE T.xls	Transient 9 Stress Results with Total Stress at Safe End	In Computer files
FATIGUE.OUT	Output file from FATIGUE.EXE	In Computer files
FATIGUE.inp	Input file for FATIGUE.EXE	In Computer files
TRANSNT_XX.inp	Input files for STRESS.EXE	In Computer files
P-V_XX.OUT	Output file from P-V.EXE	In Computer files



Structural Integrity Associates, Inc.

File No.: VY-16Q-307

CALCULATION PACKAGE

Project No.: VY-16Q

NEC-JH_10

PROJECT NAME:

Environmental Fatigue Analysis of VYNPS

CONTRACT NO.:

10150394

CLIENT:

Entergy Nuclear Operations, Inc.

PLANT:

Vermont Yankee Nuclear Power Station

CALCULATION TITLE:

Recirculation Class 1 Piping Fatigue and EAF Analysis

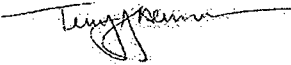
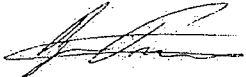

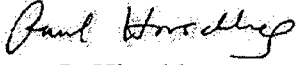
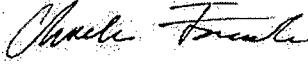
Document Revision	Affected Pages	Revision Description	Project Manager Approval Signature & Date	Preparer(s) & Checker(s) Signatures & Date
0	1-16 A1-A51 B1-B5 Computer Files	Initial Issue	 T. J. Herrmann 07/27/2007	 R.V. Perry 07/27/2007  Keith R. Cron K.R. Evon 07/27/2007  Paul Hirschberg P. Hirschberg 07/27/2007  C.J. Fourcade C.J. Fourcade 07/27/2007

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1.0 OBJECTIVE

The purpose of this calculation is to perform an ASME Section III, NB-3600 fatigue usage calculation (including environmental fatigue) for the Loop A NUREG/CR-6260 locations in the Reactor Recirculation (RR) and Residual Heat Removal (RHR) piping.

The fatigue calculation performed herein is not a certified ASME Code NB-3600 stress and fatigue analysis. Rather, it is an evaluation for the purposes of establishing fatigue usage to accommodate fatigue monitoring of the subject B31.1 piping. Although the PIPESTRESS program implements all ASME Code NB-3600 equations, only the fatigue usage results are utilized. All stress limit checks, although calculated by the program, are ignored since satisfactory stress limit checks were performed as a part of the already existing governing B31.1 stress analyses for all piping systems.

2.0 METHODOLOGY

2.1 Background

Since ASME Section III Design Specifications do not exist for the subject piping systems, SI developed transient definitions and expected number of cycles for the subject piping in a previous evaluation. These definitions are based on SI's experience in piping analysis at other BWR plants, as well as review of VY-specific operating procedures, and are appropriate for BWR-4 plants and tailored specifically to VY. Those transient definitions will reflect current plant operating conditions as shown in references [7 through 10]. Using the PIPESTRESS computer code [5], heat transfer analysis will be performed for the transients defined to establish the necessary parameters for use in an NB-3600 fatigue evaluation. This will result in a detailed usage factor calculation for the RR and RHR NUREG/CR-6260 locations from which to base the environmental fatigue evaluation.



2.2 Design Transients and Fatigue Analysis

The temperature time histories are obtained from the reactor thermal cycle diagrams [7] [8]. These diagrams also provide the changes in flow rate and system pressures. These temperatures and pressures were updated to account for EPU [9].

The computer program PIPESTRESS [5] was used, which is a full function, verified piping analysis package. The ASME Code methodology for fatigue analysis of Class 1 piping systems requires determination of the through-wall thermal gradient terms ΔT_1 (linear gradient), ΔT_2 (nonlinear gradient), and $T_a - T_b$ (transition gradient) for each transient containing a non-trivial ramp rate. PIPESTRESS calculates these terms for each thermal transient. Load sets were then developed for the critical time points of the transients, that include loads due to pressure, thermal expansion, OBE seismic, and thermal gradient stresses. PIPESTRESS was then used to determine the range of primary plus secondary plus peak stresses for each load set pair, and calculate the cumulative fatigue usage for the design numbers of cycles.

3.0 ASSUMPTIONS/DESIGN INPUTS

The Code of construction for VY is ANSI B31.1, 1967 Edition [3, 10]. In order to take advantage of improvements in the ASME Code that result in a lower calculated fatigue usage, this evaluation is done to the ASME Boiler and Pressure Vessel Code, Section III, 1998 Edition with 2000 Addenda [1]. The 1998 Edition of Section III (with 2000 Addenda) has been accepted by the US NRC for use in design analyses. Although there are a few restrictions on the application of this Edition, they involve the use of optional increased allowables that are not being used in this calculation.

The piping analysis input information was based on references provided by VY. The ADLPIPE input file [6] was the source for the piping geometry, and pipe support locations and types. Additional piping support information was obtained from plant drawings [15]. The pipe size, schedule, insulation, and weight per foot, were obtained from [3] (page 10). The flow element located between the pump and RHR return tee was not included in the model. The weight of the element would have no significant impact on the analysis and the element is remote from any areas of severe thermal transients such as the RHR return tee. The weight of the contents was automatically added by the PIPESTRESS program. The design temperature and piping material was obtained from reference [3] (page 9). Table 1 summarizes the material properties used in this analysis.

Reference [6] contains an SSE response spectrum. This spectrum was conservatively used as the OBE spectrum in this analysis. Code case N-411 damping is utilized and directional loading is combined by SRSS [3] (page 20).



Per Reference [9] (Item 14, section 3.2.1), the normal recirculation flow per loop, post EPU, is 12.3Mlbm/hr (at 526°F). Flow is converted to gpm as follows:

$$Q = 12,300,000 \frac{\text{lbm}}{\text{hr}} \left(\frac{1 \text{ ft}^3}{47.45 \text{ lbm}} \right) \left(\frac{7.48 \text{ gal}}{\text{ft}^3} \right) \left(\frac{1 \text{ hr}}{60 \text{ min}} \right) = 32,316 \text{ gpm}$$

Where flow is stopped, a flow rate that gives an equivalent natural convection heat transfer coefficient is calculated.

The applicable transients to consider for the RR and RHR systems are shown in the thermal cycle diagrams [7] and [8]. Level C transients are not required to be included in the fatigue analysis per NB-3224.4. Reference [3] describes which transients are considered level C. Note that a transient for RHR initiation is not accounted for on these diagrams. In order to account for this transient, RHR temperature data from RFO 25 [11] was used to conservatively determine an appropriate temperature change while reference [12] was used to determine flow rates and pressures. Table 2 describes each section and Figure 1 shows the piping model with node numbers. Table 3 contains a list of applicable transients. (Note that the transient RHR initiation contains a section 3B. This section accounts for the portion of the recirculation pump discharge piping that is affected by this transient.) OBE cycles are not listed in Table 3 but are included as Load Set 26 for +OBE and Load Set 27 for -OBE. A review of shutdown cooling mode operation since the recirculation piping was replaced in 1986 was performed by the station and the number of cycles per loop was conservatively estimated to be 150 through year 60 [10]. Based on this, the cycle counts for the Recirculation piping were reduced by a factor of 150/300 (50%) for all transients with the exception of transients that have fewer than 10 transient cycles.

To ensure this cycle reduction adequately considered the potential impact on carbon steel RHR piping, the full number of transient cycles [7] was initially applied to the PIPESTRESS model and the highest CUF for the carbon steel portion of the RHR piping, which has not been replaced, was lower than the value obtained for the recirculation piping with reduced cycles. The Recirculation and RHR line sizes are specified in reference [3] and are shown in Table 4.

Table 1: Material Properties [1] [3]

ASTM A-106 Grade B (C-Si)							
Temperature (°F)	Young's Modulus (x10 ⁶ psi)	Thermal Conductivity (Btu/hr-ft-°F)	Thermal Diffusivity (ft ² /hr)	Coefficient of Linear Thermal Expansion (in/100 ft)	Mean Coefficient of Thermal Expansion (10-6 /in/in/°F)	Design Stress Intensity (ksi)	Yield Strength (ksi)
70	29.5	27.5	0.529	0.00	6.40	20.0	35.0
100	29.3	27.6	0.512	0.20		20.0	35.0
200	28.8	27.6	0.486	1.00		20.0	32.1
300	28.3	27.2	0.453	1.90		20.0	31.0
400	27.7	26.7	0.428	2.80		20.0	29.9
500	27.3	25.9	0.398	3.70		18.9	28.5
600	26.7	25.0	0.374	4.70		17.3	26.8

ASME SA-376 TP 316 (16Cr-12Ni-2Mo)							
Temperature (°F)	Young's Modulus (x10 ⁶ psi)	Thermal Conductivity (Btu/hr-ft-°F)	Thermal Diffusivity (ft ² /hr)	Coefficient of Linear Thermal Expansion (in/100 ft)	Mean Coefficient of Thermal Expansion (10-6 /in/in/°F)	Design Stress Intensity (ksi)	Yield Strength (ksi)
70	28.3	8.2	0.139	0.00	8.50	20.0	30.0
100	28.1	8.3	0.140	0.30		20.0	30.0
200	27.6	8.8	0.145	1.40		20.0	25.9
300	27.0	9.3	0.150	2.50		20.0	23.4
400	26.5	9.8	0.155	3.70		19.3	21.4
500	25.8	10.2	0.160	5.00		18.0	20.0
600	25.3	10.7	0.165	6.30		17.0	18.9

ASME SA-403 WP 316 (16Cr-12Ni-2Mo)							
Temperature (°F)	Young's Modulus (x10 ⁶ psi)	Thermal Conductivity (Btu/hr-ft-°F)	Thermal Diffusivity (ft ² /hr)	Coefficient of Linear Thermal Expansion (in/100 ft)	Mean Coefficient of Thermal Expansion (10-6 /in/in/°F)	Design Stress Intensity (ksi)	Yield Strength (ksi)
70	28.3	8.2	0.139	0.00	8.50	20.0	30.0
100	28.1	8.3	0.140	0.30		20.0	30.0
200	27.6	8.8	0.145	1.40		20.0	25.9
300	27	9.3	0.150	2.50		20.0	23.4
400	26.5	9.8	0.155	3.70		18.7	21.4
500	25.8	10.2	0.160	5.00		17.5	20.0
600	25.3	10.7	0.165	6.30		16.4	18.9

The material properties applied in the analyses are taken from ASME Section II Part D 1998 Edition with 2000 Addenda. This is consistent with information provided in the Design Input Record (page 13 of VY EC No. 1773, SI File No. VY-16Q-209). The use of a later code edition than that used for the original design code is acceptable since later editions typically reflect more accurate material properties than was published in prior Code editions.



Table 2: Recirculation and RHR Piping Segment Numbers

Piping Region	Node Points		Description
	Start	End	
1	3	500	Outlet
2	500	50	Pump suction
3	150	210	Pump discharge
3B*	188	210	Down Stream of RHR Return
4	210	340	Inlet Header
	210	320	Inlet Header
5A	340	365	Riser
5B	340	345	Riser
5C	210	334	Riser
5D	320	325	Riser
5E	320	315	Riser
6A	365	366	Inlet Nozzle
6B	345	346	Inlet Nozzle
6C	334	336	Inlet Nozzle
6D	325	326	Inlet Nozzle
6E	315	316	Inlet Nozzle
7A	500	550	RHR Supply; tee to valve
7B	550	565	RHR Supply; valve to penetration
8	152	176	4" Bypass
9A	600	660	RHR Return; valve to tee
9B	660	675	RHR Return; penetration to valve

*Only applicable for RHR initiation

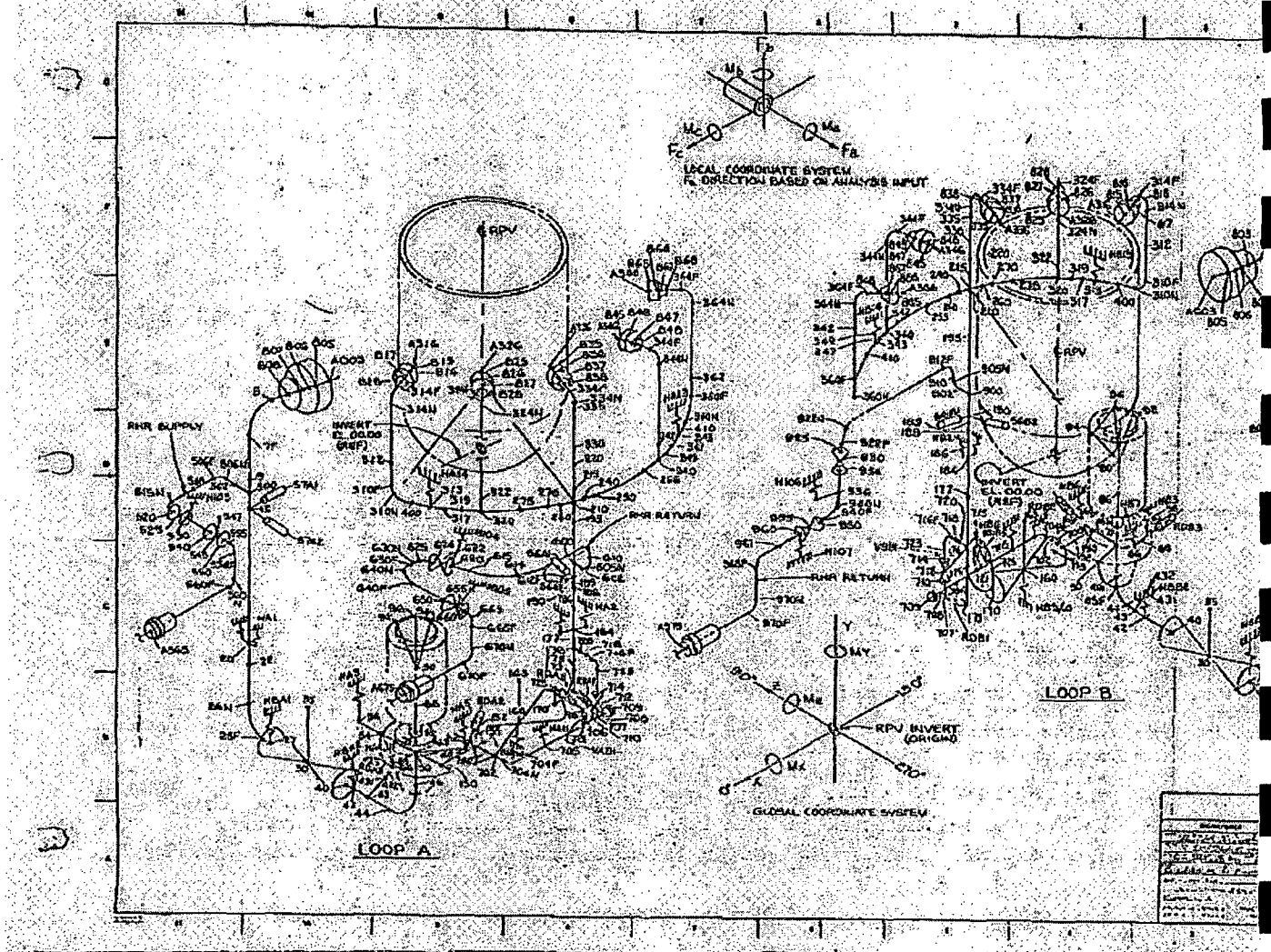


Figure 1. Recirculation and RHR Piping Diagram

File No.: VY-16Q-307

Revision: 0

Table 3: VY Thermal Transients

Transient Case	Description	Piping Region	Thermal Conditions [4] [7] [8] [10]						Pressure		No. of Cycles [10]
			Oper. Temp. (°F)	T _{max} (°F)	T _{min} (°F)	Time (sec.)	Rate (°F/hr)	Flow [9] (gpm)	Initial (psig)	Final (psig)	
1	Design Hydrotest (Leak Test) +	1	100	70	100	1800	60	2,262	0	1,100	60
		2	100	70	100	1800	60	2,262	0	1,100	
		3	100	70	100	1800	60	2,262	0	1,100	
		4	100	70	100	1800	60	905	0	1,100	
		5	100	70	100	1800	60	452	0	1,100	
		6	100	70	100	1800	60	452	0	1,100	
		7A	100	70	100	1800	60	0	0	1,100	
		7B	100	70	100	1800	60	0	0	120	
		8	100	70	100	1800	60	0	0	1,100	
		9A	100	70	100	1800	60	0	0	1,100	
9B	100	70	100	1800	60	0	0	1,100			
2	Design Hydrotest (Leak Test) -	1	100	100	100	1800	0	2,262	1,100	50	60
		2	100	100	100	1800	0	-2,262	1,100	50	
		3	100	100	100	1800	0	2,262	1,100	50	
		4	100	100	100	1800	0	905	1,100	50	
		5	100	100	100	1800	0	452	1,100	50	
		6	100	100	100	1800	0	452	1,100	50	
		7A	100	100	100	1800	0	0	1,100	50	
		7B	100	100	100	1800	0	0	120	50	
		8	100	100	100	1800	0	0	1,100	50	
		9A	100	100	100	1800	0	0	1,100	50	
9B	100	100	100	1800	0	0	1,100	50			
3	Startup	1	549	100	549	16164	100	16,158	50	1,010	150
		2	549	100	549	16164	100	16,158	50	1,010	
		3	549	100	549	16164	100	16,158	50	1,035	
		4	549	100	549	16164	100	6,463	50	1,035	
		5	549	100	549	16164	100	3,232	50	1,035	
		6	549	100	549	16164	100	3,232	50	1,035	
		7A	549	100	549	16164	100	300	50	1,010	
		7B	150	100	150	16164	0	0	50	120	
		8	549	100	549	16164	100	168	50	1,035	
		9A	549	100	549	16164	100	0	50	1,035	
9B	150	100	150	16164	11	0	50	1,035			
4	Turbine Roll & Increase to Rated Power + SCRAM - 1	1	542	549	542	0	STEP	32,316	1,010	1,010	290
		2	542	549	542	0	STEP	32,316	1,010	1,010	
		3	542	549	542	0	STEP	32,316	1,035	1,035	
		4	542	549	542	0	STEP	12,926	1,035	1,035	
		5	542	549	542	0	STEP	6,463	1,035	1,035	
		6	542	549	542	0	STEP	6,463	1,035	1,035	
		7A	542	549	542	0	STEP	364	1,010	1,010	
		7B	150	150	150	0	STEP	0	120	120	
		8	542	549	542	0	STEP	335	1,035	1,035	
		9A	542	549	542	0	STEP	520	1,035	1,035	
9B	150	150	150	0	STEP	0	1,035	1,035			
5	Turbine Roll & Increase to Rated Power + SCRAM - 2	1	526	542	526	0	STEP	32,316	1,010	1,010	290
		2	526	542	526	0	STEP	32,316	1,010	1,010	
		3	526	542	526	0	STEP	32,316	1,035	1,035	
		4	526	542	526	0	STEP	12,926	1,035	1,035	
		5	526	542	526	0	STEP	6,463	1,035	1,035	
		6	526	542	526	0	STEP	6,463	1,035	1,035	
		7A	526	542	526	0	STEP	358	1,010	1,010	
		7B	150	150	150	0	STEP	0	120	120	
		8	526	542	526	0	STEP	335	1,035	1,035	
		9A	526	542	526	0	STEP	511	1,035	1,035	
9B	150	150	150	0	STEP	0	1,035	1,035			
6	Loss of Feedwater Heater, Turbine Trip (+)	1	542	526	542	900	64	32,316	1,010	1,010	5 x 2
		2	542	526	542	900	64	32,316	1,010	1,010	
		3	542	526	542	900	64	32,316	1,035	1,035	
		4	542	526	542	900	64	12,926	1,035	1,035	
		5	542	526	542	900	64	6,463	1,035	1,035	
		6	542	526	542	900	64	6,463	1,035	1,035	
		7A	542	526	542	900	64	358	1,010	1,010	
		7B	150	150	150	900	0	0	120	120	
		8	542	526	542	900	64	335	1,035	1,035	
		9A	542	526	542	900	64	511	1,035	1,035	
9B	150	150	150	900	0	0	1,035	1,035			
7	Loss of Feedwater Heater, Turbine Trip (-)	1	526	542	526	360	160	32,316	1,010	1,010	5 x 2
		2	526	542	526	360	160	32,316	1,010	1,010	
		3	526	542	526	360	160	32,316	1,035	1,035	
		4	526	542	526	360	160	12,926	1,035	1,035	
		5	526	542	526	360	160	6,463	1,035	1,035	
		6	526	542	526	360	160	6,463	1,035	1,035	
		7A	526	542	526	360	160	358	1,010	1,010	
		7B	150	150	150	360	0	0	120	120	
		8	526	542	526	360	160	335	1,035	1,035	
		9A	526	542	526	360	160	511	1,035	1,035	
9B	150	150	150	360	0	0	1,035	1,035			

Table 3: VY Thermal Transients (continued)

Transient Case	Description	Piping Region	Thermal Conditions [4] [7] [8] [10]						Pressure		No. of Cycles [10]
			Oper. Temp. (°F)	T _{min} (°F)	T _{max} (°F)	Time (sec.)	Rate (°F/hr)	Flow [9] (gpm)	Initial (psig)	Final (psig)	
8	Loss of Feedwater Heater, Partial FW Heater Bypass (-)	1	516	526	516	0	STEP	32,316	1,010	1,010	35
		2	516	526	516	0	STEP	32,316	1,010	1,010	
		3	516	526	516	0	STEP	32,316	1,035	1,035	
		4	516	526	516	0	STEP	12,926	1,035	1,035	
		5	516	526	516	0	STEP	6,463	1,035	1,035	
		6	516	526	516	0	STEP	6,463	1,035	1,035	
		7A	516	526	516	0	STEP	351	1,010	1,010	
		7B	150	150	150	0	STEP	0	120	120	
		8	516	526	516	0	STEP	335	1,035	1,035	
		9A	516	526	516	0	STEP	502	1,035	1,035	
9B	150	150	150	0	STEP	0	1,035	1,035			
9	Loss of Feedwater Heater, Partial FW Heater Bypass (+)	1	526	516	526	0	STEP	32,316	1,010	1,010	35
		2	526	516	526	0	STEP	32,316	1,010	1,010	
		3	526	516	526	0	STEP	32,316	1,035	1,035	
		4	526	516	526	0	STEP	12,926	1,035	1,035	
		5	526	516	526	0	STEP	6,463	1,035	1,035	
		6	526	516	526	0	STEP	6,463	1,035	1,035	
		7A	526	516	526	0	STEP	351	1,010	1,010	
		7B	150	150	150	0	STEP	0	120	120	
		8	526	516	526	0	STEP	335	1,035	1,035	
		9A	526	516	526	0	STEP	502	1,035	1,035	
9B	150	150	150	0	STEP	0	1,035	1,035			
10	Loss of Feedwater Pumps (Isolation Valves Close) 1st step down	1	300	526	300	220	3698	600	1,190	1,135	5
		2	300	526	300	220	3698	600	1,190	1,135	
		3	300	526	300	220	3698	600	1,215	1,160	
		4	300	526	300	220	3698	400	1,215	1,160	
		5	300	526	300	220	3698	200	1,215	1,160	
		6	300	526	300	220	3698	200	1,215	1,160	
		7A	300	526	300	220	3698	306	1,190	1,135	
		7B	150	150	150	0.01	0	0	120	120	
		8	300	526	300	220	3698	6	1,215	1,160	
		9A	300	526	300	220	3698	437	1,215	1,160	
9B	150	150	150	0.01	0	0	1,215	1,160			
11	Loss of Feedwater Pumps (Isolation Valves Close) 1st & 2nd step up	1	500	300	500	1980	364	600	885	1,135	5 X 2
		2	500	300	500	1980	364	600	885	1,135	
		3	500	300	500	1980	364	600	910	1,160	
		4	500	300	500	1980	364	400	910	1,160	
		5	500	300	500	1980	364	200	910	1,160	
		6	500	300	500	1980	364	200	910	1,160	
		7A	500	300	500	1980	364	301	885	1,135	
		7B	150	150	150	0.01	0	0	120	120	
		8	500	300	500	1980	364	6	910	1,160	
		9A	500	300	500	1980	364	429	910	1,160	
9B	150	150	150	0.01	0	0	910	1,160			
12	Loss of Feedwater Pumps (Isolation Valves Close) 2nd & 3rd step down	1	300	500	300	180	4000	600	1,135	675	5 X 2
		2	300	500	300	180	4000	600	1,135	675	
		3	300	500	300	180	4000	600	1,160	700	
		4	300	500	300	180	4000	400	1,160	700	
		5	300	500	300	180	4000	200	1,160	700	
		6	300	500	300	180	4000	200	1,160	700	
		7A	300	500	300	180	4000	301	1,135	675	
		7B	150	150	150	0.01	0	0	120	121	
		8	300	500	300	180	4000	6	1,160	700	
		9A	300	500	300	180	4000	429	1,160	700	
9B	150	150	150	0.01	0	0	1,160	700			
13	Loss of Feedwater Pumps (Isolation Valves Close) last step up	1	549	300	549	8964	100	16,158	240	1,010	5
		2	549	300	549	8964	100	16,158	240	1,010	
		3	549	300	549	8964	100	16,158	265	1,035	
		4	549	300	549	8964	100	6,463	265	1,035	
		5	549	300	549	8964	100	3,232	265	1,035	
		6	549	300	549	8964	100	3,232	265	1,035	
		7A	549	300	549	8964	100	310	240	1,010	
		7B	150	150	150	8964	100	0	120	120	
		8	549	300	549	8964	100	168	265	1,035	
		9A	549	300	549	8964	100	443	265	1,035	
9B	150	150	150	8967	100	0	265	1,035			
14	Reduction to 0% Power	1	549	526	549	0	STEP	32,316	1,010	1,010	150
		2	549	526	549	0	STEP	32,316	1,010	1,010	
		3	549	526	549	0	STEP	32,316	1,035	1,035	
		4	549	526	549	0	STEP	12,926	1,035	1,035	
		5	549	526	549	0	STEP	6,463	1,035	1,035	
		6	549	526	549	0	STEP	6,463	1,035	1,035	
		7A	549	526	549	0	STEP	360	1,010	1,010	
		7B	150	150	150	0	STEP	0	120	120	
		8	549	526	549	0	STEP	335	1,035	1,035	
		9A	549	526	549	0	STEP	514	1,035	1,035	
9B	150	150	150	0	STEP	0	1,035	1,035			

Table 3: VY Thermal Transients (continued)

Transient Case	Description	Piping Region	Thermal Conditions [4] [7] [8] [10]						Pressure		No. of Cycles [10]
			Oper. Temp. (°F)	T _{max} (°F)	T _{min} (°F)	Time (sec.)	Rate (°F/hr)	Flow [9] (gpm)	Initial (psig)	Final (psig)	
15	Shutdown 1	1	375	549	375	6264	100	16,158	1,010	170	150
		2	375	549	375	6264	100	16,158	1,010	170	
		3	375	549	375	6264	100	16,158	1,035	195	
		4	375	549	375	6264	100	6,463	1,035	195	
		5	375	549	375	6264	100	3,232	1,035	195	
		6	375	549	375	6264	100	3,232	1,035	195	
		7A	375	549	375	6264	100	320	1,035	170	
		7B	150	150	150	0.01	0	0	120	120	
		8	375	549	375	6264	100	168	1,035	195	
		9A	375	549	375	6264	100	458	1,035	195	
9B	150	150	150	0.01	0	0	1,035	195			
16	Shutdown 2	1	330	375	330	600	270	16,158	170	90	150
		2	330	375	330	600	270	16,158	170	90	
		3	330	375	330	600	270	16,158	195	115	
		4	330	375	330	600	270	6,463	195	115	
		5	330	375	330	600	270	3,232	195	115	
		6	330	375	330	600	270	3,232	195	115	
		7A	330	375	330	600	270	282	170	90	
		7B	150	150	150	600	0	0	120	90	
		8	330	375	330	600	270	168	195	115	
		9A	330	375	330	600	270	403	195	115	
9B	150	150	150	600	0	0	195	115			
17	Shutdown 3	1	225	330	225	3780	100	16,158	90	0	150
		2	225	330	225	3780	100	16,158	90	0	
		3	225	330	225	3780	100	16,158	115	25	
		4	225	330	225	3780	100	6,463	115	25	
		5	225	330	225	3780	100	3,232	115	25	
		6	225	330	225	3780	100	3,232	115	25	
		7A	225	330	225	3780	100	260	90	0	
		7B	150	150	150	0	0	0	90	0	
		8	225	330	225	3780	100	168	115	25	
		9A	225	330	225	3780	100	260	115	25	
9B	150	150	150	0	0	0	115	25			
18	Shutdown 4 (RHR SDC Flow is on)	1	100	225	100	4500	100	22,858	0	0	150
		2	100	225	100	4500	100	16,158	0	0	
		3	100	225	100	4500	100	16,158	25	25	
		3B	100	225	100	4500	100	22,858	25	25	
		4	100	225	100	4500	100	9,143	25	25	
		5	100	225	100	4500	100	4,572	25	25	
		6	100	225	100	4500	100	4,572	25	25	
		7A	100	225	100	4500	100	6,700	0	0	
		7B	100	225	100	4500	100	6,700	0	0	
		8	100	225	100	4500	100	168	25	25	
9A	100	225	100	4500	100	6,700	100	100			
9B	100	225	100	4500	100	6,700	100	100			
19	Code Hydro	1	100	100	100	0.01	0	2,262	25	1,563	1
		2	100	100	100	0.01	0	2,262	25	1,563	
		3	100	100	100	0.01	0	2,262	25	1,563	
		4	100	100	100	0.01	0	905	25	1,563	
		5	100	100	100	0.01	0	452	25	1,563	
		6	100	100	100	0.01	0	452	25	1,563	
		7A	100	100	100	0.01	0	158	25	1,563	
		7B	100	100	100	0.01	0	0	0	450	
		8	100	100	100	0.01	0	23	25	1,563	
		9A	100	100	100	0.01	0	226	25	1,563	
9B	100	100	100	0.01	0	0	25	1,563			
20	RHR Initiation (+)	1	225	225	225	0.01	0	22,858	0	0	150
		2	225	225	225	0.01	0	16,158	0	0	
		3	225	225	225	0.01	0	16,158	25	25	
		3B	225	180	225	60	2700	22,858	25	25	
		4	225	180	225	60	2700	9,143	25	25	
		5	225	180	225	60	2700	4,572	25	25	
		6	225	180	225	60	2700	4,572	25	25	
		7A	225	225	225	60	0	6,700	0	0	
		7B	225	150	225	60	4500	6,700	0	0	
		8	225	225	225	0.01	0	237	25	25	
9A	225	70	225	60	9300	6,700	25	25			
9B	225	70	225	60	9300	6,700	25	25			
21	RHR Initiation (-)	1	225	225	225	0.01	0	22,858	0	0	150
		2	225	225	225	0.01	0	16,158	0	0	
		3	225	225	225	0.01	0	16,158	25	25	
		3B	180	225	180	60	2700	22,858	25	25	
		4	180	225	180	60	2700	9,143	25	25	
		5	180	225	180	60	2700	4,572	25	25	
		6	180	225	180	60	2700	4,572	25	25	
		7A	225	225	225	60	0	6,700	0	0	
		7B	150	150	150	0.01	0	6,700	0	0	
		8	225	225	225	0.01	0	237	25	25	
9A	70	225	70	60	9300	6,700	25	25			
9B	70	150	70	60	9290	6,700	25	25			

Table 4: Recirc/RHR Piping Size Information [3]

Regions	1, 2	3	4	5	6	7A, 7B	8	9A, 9B
Piping Nom. O.D. (in.)	28.169	28.339	21.878	12.748	14.17	20	4.5	24
Piping Nom. Wall (in.)	1.244	1.339	1.043	0.685	1.395	1.031	0.3385	1.217
Pipe Weight ¹ (lb/ft)	386.1	415.1	257.2	103.4	207.5	221.9	23.2	316.5

Note:

1. Weight of contents automatically added by the PIPESTRESS Program.

4.0 ANALYSIS

Through-wall thermal gradient terms were calculated by the PIPESTRESS program for all of the transients. Thermal transient cases were modeled for each transient, as shown in Table 3. Some transients were similar in nature and were lumped together and the number of cycles added together. Listings of the PIPESTRESS input files are included as Appendix A.

The forces and moments due to thermal expansion need to be included in the fatigue evaluation. The thermal expansion cases as analyzed by the piping program, PIPESTRESS, correspond to the end temperature and pressure of the transient. Table 5 lists the thermal expansion cases.

The material properties were obtained from the ASME Code Section III, 1998 Edition, Appendix I, with 2000 Addenda [1]. E and α are taken at 70°F, and k, ρ , and c_p are taken at the average temperature over the range of the individual transients.

The internal heat transfer coefficient h for the transients with flow occurring in the pipe is calculated based on the following relation for forced convection [13]:

$$h = 0.023 \text{ Re}^{0.8} \text{ Pr}^{0.4} k/D$$

Where Re = Reynolds number
 Pr = Prandtl number
 k = Thermal conductivity
 D = Pipe diameter



The heat transfer coefficients were calculated by PIPESTRESS using the above relation. The flow rates described for each transient in Section 3 were used. For the transients where flow is stopped, the natural convection heat transfer coefficient was used. The formula for h is [13]:

$$h = 0.55 (Gr Pr)^{0.25} k/L$$

Where Gr = Grashof Number
L = Pipe diameter

PIPESTRESS only has the forced convection heat transfer formula built in, so an equivalent flow rate was determined that would give the same heat transfer coefficient as the free convection coefficient.

Since the replacement of the Recirculation piping [10], HWC conditions exist for 39% of the time, and NWC conditions exist for 61% of the time. This is based on 17.5 years of operation with NWC between March and July 1986 when the piping was replaced and November 2003 when HWC was implemented and the 46 years from March 1986 to the end of the period of extended operation in March 2032. Using the bounding EAF multipliers (8.36 for HWC and 15.35 for NWC) [14], an overall multiplier may be calculated as follows:

$$(15.35)0.61 + (8.36)0.39 = 12.62$$

Table 5: Thermal Cycle Load Sets

Load Set	Transient Case	Region Temperatures (°F)											Region Prestures (psig)				
		1	2	3	3B	4	5	6	7A	7B	8	9A	9B	1, 2, 7A	3, 4, 5, 6, 8, 9A, 9B	7B	3B
1	1	100	100	100	-	100	100	100	100	100	100	100	100	1,100	1,100	120	-
2	2	100	100	100	-	100	100	100	100	100	100	100	100	50	50	50	-
3	3	549	549	549	-	549	549	549	549	150	549	549	150	1,010	1,035	120	-
4	4	542	542	542	-	542	542	542	542	150	542	542	150	1,010	1,035	120	-
5	5	526	526	526	-	526	526	526	526	150	526	526	150	1,010	1,035	120	-
6	6	542	542	542	-	542	542	542	542	150	542	542	150	1,010	1,035	120	-
7	7	526	526	526	-	526	526	526	526	150	526	526	150	1,010	1,035	120	-
8	8	516	516	516	-	516	516	516	516	150	516	516	150	1,010	1,035	120	-
9	9	526	526	526	-	526	526	526	526	150	526	526	150	1,010	1,035	120	-
10	10	300	300	300	-	300	300	300	300	150	300	300	150	1,135	1,160	120	-
11	11	500	500	500	-	500	500	500	500	150	500	500	150	1,135	1,160	120	-
12	12	300	300	300	-	300	300	300	300	150	300	300	150	675	700	121	-
13	13	549	549	549	-	549	549	549	549	150	549	549	150	1,010	1,035	120	-
14	14	549	549	549	-	549	549	549	549	150	549	549	150	1,010	1,035	120	-
15	15	375	375	375	-	375	375	375	375	150	375	375	150	170	195	120	-
16	16	330	330	330	-	330	330	330	330	150	330	330	150	90	115	90	-
17	17	225	225	225	-	225	225	225	225	150	225	225	150	0	25	0	-
18	18	100	100	100	100	100	100	100	100	100	100	100	100	0	25	0	25
19	19	100	100	100	-	100	100	100	100	100	100	100	100	1,563	1,563	450	-
20	20	225	225	225	225	225	225	225	225	225	225	225	225	0	25	0	25
21	21	225	225	225	180	180	180	180	225	150	225	70	70	0	25	0	25



5.0 RESULTS OF ANALYSIS

To perform the fatigue analysis, program PIPESTRESS [5] was used. PIPESTRESS calculates the thermal expansion and seismic moments, the ASME Code Equation 10, 12, and 13 stresses, performs the thermal stress ratchet check, and performs fatigue analysis per Equation 11 and 14. For each operating state of the recirculation/RHR piping, load sets are created. A load set includes the coincident pressure, thermal expansion moment, through-wall thermal gradient terms, number of cycles, and temperature at which the allowable S_m is taken. In general, the pressures and thermal expansion moments are taken at the end point of the transient, the thermal gradients taken at the point of maximum total thermal gradient stress during the transient, and the S_m allowable is initially conservatively taken at the highest temperature of the transient. Table 5 lists the inputs to the load sets.

In calculating fatigue, the range of stress in going from one load set to another is determined. Since the Code assumes that any transient could follow any other, all pairs of load sets are evaluated to determine the range of stresses for the Code stress equations. The number of allowable cycles for each load set pair is determined. The incremental fatigue usage is obtained by dividing the number of design cycles by the allowable cycles. The incremental fatigue usages for all load set pairs are then summed to obtain the total fatigue usage.

The cumulative fatigue usage for the Loop A recirculation RHR return isolation valve-to-pipe location (Node 641), prior to considering environmental effects, is 0.0128. Taking into account environmental effects, the bounding multiplier for stainless steel is 12.62. This results in a total fatigue usage of 0.1615. (Note that since the RHR carbon steel piping has not been replaced, these results represent the full projected 60 year cycle count.)

The cumulative fatigue usage for the RHR return tee (Node 600), prior to considering environmental effects, is 0.0590. Taking into account environmental effects, the bounding multiplier for stainless steel is 12.62. This results in a total fatigue usage of 0.7446.

Appendix A contains the PIPESTRESS input files. Appendix B contains the fatigue usage summary for both locations.



6.0 REFERENCES

1. ASME Boiler and Pressure Vessel Code, Section III, 1998 Edition with 2000 Addenda.
2. ASME Boiler and Pressure Vessel Code, Section XI, 1998 Edition.
3. Vermont Yankee Calculation 23A5569, "Recirculation System Stress Analysis Loop A", Revision 0, SI File No. VY-05Q-227.
4. Email from Jim Fitzpatrick (Entergy) to Terry Herrmann (SI), "RE: RHR Thermal Transients," dated: June 29, 2007 11:19AM, SI File Number VY-09Q-209.
5. Program PIPESTRESS, Version 3.5.1+26, DST Computer Services, S.A., June 2004.
6. ADLPIPE Model Input Listing, Vermont Yankee Calculation VYC-2030, Rev. 0, "Temporary Shielding Recirculation & RHR Piping Loop A," File c2030n2, SI File No. W-VY-05Q-227.
7. "Reactor Thermal Cycles for 60 Years of Operation," Attachment 1 of Entergy Design Input Record (DIR) Revision 1, EC No. 1773, Revision 0, "Environmental Fatigue Analysis for Vermont Yankee Nuclear Power Station," SI File No. VY-16Q-209.
8. "Nozzle Thermal Cycles (Recirculation Outlet)," Attachment 1, page 4, of Entergy Design Input Record (DIR) Revision 1, EC No. 1773, Revision 0, "Environmental Fatigue Analysis for Vermont Yankee Nuclear Power Station," SI File No. VY-16Q-209.
9. Entergy Nuclear Report VY-RPT-05-00022, "Task T0100 Reactor Heat Balance EPU Task Report for ER-0401409", Revision 0, SI File No. VY-16Q-205.
10. Design Input Record (DIR) Revision 1, EC No. 1773, Revision 0, "Environmental Fatigue Analysis for Vermont Yankee Nuclear Power Station," SI File No. VY-16Q-209.
11. "RHR Shutdown Cooling Temperature Data," page 8, of Entergy Design Input Record (DIR) EC No. 1773, Revision 1, "Environmental Fatigue Analysis for Vermont Yankee Nuclear Power Station," SI File No. VY-16Q-209.
12. "RHR Shutdown Cooling Flow Rate and Pressure Data," page 9, of Entergy Design Input Record (DIR) Revision 1, EC No. 1773, Revision 0, "Environmental Fatigue Analysis for Vermont Yankee Nuclear Power Station," SI File No. VY-16Q-209.
13. Holman, J.P., *Heat Transfer*, Fifth Edition, McGraw-Hill, 1981.



14. SI Calculation, "Environmental Fatigue Evaluation of Reactor Recirculation Inlet Nozzle and Vessel Shell/Bottom Head," Revision 0, SI File Number VY-16Q-303.

15. VY Drawings, SI File No. VY-16Q-205:

- a. 5920-6801, Sheet 1, Revision 1.
- b. 5920-6802, Sheet 1, Revision 2, Sheet 2, Revision 2, Sheet 3, Revision 3, Sheet 4, Revision 2, Sheet 5, Revision 2, Sheet 6, Revision 2.
- c. 5920-6808 Sheet 1, Revision 0.



APPENDIX A

PIPESTRESS Input Files

Input File	Description
Recirc_15.fre	Piping model and general input for reduced cycle count
RHR_15.fre	Piping model and general input for 60 year cycle count
Reg1.inp	Region 1 transient definitions
Reg2.inp	Region 2 transient definitions
Reg3.inp	Region 3 transient definitions
Reg4.inp	Region 4 transient definitions
Reg5.inp	Region 5 transient definitions
Reg6.inp	Region 6 transient definitions
Reg7A.inp	Region 7A transient definitions
Reg7B.inp	Region 7B transient definitions
Reg8.inp	Region 8 transient definitions
Reg9A.inp	Region 9A transient definitions
Reg9B.inp	Region 9B transient definitions

Recirc 15.fre

```

IDEN JB=3 *Job number (1 to 9999)
CD=1 *1=ASME Class 1
GR=-Y *Direction of gravity
VA=0 *0=Calculate 2=Verify
IU=1 *Input units 1=USA
OU=1 *Output units 1=USA
CH=$ *Delimiter character
AB=T *FREE errors = abort
PL=$Vermont Yankee$
EN=$RVPS$

TITL BL=3 *Modeling option:
* 3 = uniform mass for static analysis
* lumped mass for dynamic analysis
* rotational inertia ignored
GL=1 *Report forces/moment 0=Global 1=Local 2=G et L
SU=1 *Support summary 0=No 1=Yes
CV=15 *Code version - See Manual
HS=1 *Highest 20 stress ratios for each case
MD=1 *Hot modulus
J6=1 *File generated by program
TI=$Vermont Yankee Recirculation $
$Fatigue Analysis$

FREQ RF=1 RP=8 FR=36 MP=20 RC=0 MX=70 TI=$SEISMIC$

```

```

*****
**** THERMAL CYCLE LOAD CASES****
*****
LCAS RF=0 CA=1 TY=0 TI=$LC-1$ *TC-1
LCAS RF=0 CA=2 TY=0 TI=$LC-2$ *TC-2
LCAS RF=0 CA=3 TY=0 TI=$LC-3$ *TC-3
LCAS RF=0 CA=4 TY=0 TI=$LC-4$ *TC-4
LCAS RF=0 CA=5 TY=0 TI=$LC-5$ *TC-5
LCAS RF=0 CA=6 TY=0 TI=$LC-6$ *TC-6
LCAS RF=0 CA=7 TY=0 TI=$LC-7$ *TC-7
LCAS RF=0 CA=8 TY=0 TI=$LC-8$ *TC-8
LCAS RF=0 CA=9 TY=0 TI=$LC-9$ *TC-9
LCAS RF=0 CA=10 TY=0 TI=$LC-10$ *TC-10
LCAS RF=0 CA=11 TY=0 TI=$LC-11$ *TC-11
LCAS RF=0 CA=12 TY=0 TI=$LC-12$ *TC-12
LCAS RF=0 CA=13 TY=0 TI=$LC-13$ *TC-13
LCAS RF=0 CA=14 TY=0 TI=$LC-14$ *TC-14
LCAS RF=0 CA=15 TY=0 TI=$LC-15$ *TC-15
LCAS RF=0 CA=16 TY=0 TI=$LC-16$ *TC-16
LCAS RF=0 CA=17 TY=0 TI=$LC-17$ *TC-17
LCAS RF=0 CA=18 TY=0 TI=$LC-18$ *TC-18
LCAS RF=0 CA=19 TY=0 TI=$LC-19$ *TC-19
LCAS RF=0 CA=20 TY=0 TI=$LC-20$ *TC-20
LCAS RF=0 CA=21 TY=0 TI=$LC-21$ *TC-21
LCAS RF=0 CA=22 TY=0 TI=$LC-22$ *TC-22
LCAS RF=0 CA=23 TY=0 TI=$LC-23$ *TC-23
LCAS RF=0 CA=24 TY=0 TI=$LC-24$ *TC-24
LCAS RF=0 CA=25 TY=0 TI=$LC-25$ *TC-25

```

```

*****
**** WEIGHT CASES****
*****
LCAS CA=101 RF=1 TY=3 TI=$OPERATING WEIGHT$
LCAS CA=102 RF=2 TY=4 TI=$HYDROTEST WEIGHT$

```

 **** THERMAL TRANSIENT CASES****

TCAS CA=201 TI=\$Design Hydrotest (+) \$
 TCAS CA=202 TI=\$Design Hydrotest (-) \$
 TCAS CA=203 TI=\$Startup \$
 TCAS CA=204 TI=\$TRoll & Inc. PWR1 \$
 TCAS CA=205 TI=\$TRoll & Inc. PWR2 \$
 TCAS CA=206 TI=\$LOFWH+TT PWR1 \$
 TCAS CA=207 TI=\$LOFWH+TT PWR2 \$
 TCAS CA=208 TI=\$LOFWH+PFWHTR Byp1 \$
 TCAS CA=209 TI=\$LOFWH+PFWHTR Byp2 \$
 TCAS CA=210 TI=\$LOFWP, ISO C1 DN 1 \$
 TCAS CA=211 TI=\$LOFWP, ISO C1 UP 1 \$
 TCAS CA=212 TI=\$LOFWP, ISO C1 DN 2 \$
 TCAS CA=213 TI=\$LOFWP, ISO C1 UP 2 \$
 TCAS CA=214 TI=\$Reduction to 0% PWR \$
 TCAS CA=215 TI=\$Shutdown1 \$
 TCAS CA=216 TI=\$Shutdown2 \$
 TCAS CA=217 TI=\$Shutdown3 \$
 TCAS CA=218 TI=\$Shutdown4 \$
 TCAS CA=219 TI=\$Code Hydrotest \$
 TCAS CA=220 TI=\$RHR Initiation UP \$
 TCAS CA=221 TI=\$RHR Initiation DN \$
 TCAS CA=222 TI=\$Inadvert. Inj. DOWN \$
 TCAS CA=223 TI=\$Inadvert. Inj. UP \$
 TCAS CA=224 TI=\$Single Relief BD DN \$
 TCAS CA=225 TI=\$Single Relief BD UP \$

 **** SEISMIC CASES****

RCAS CA=103 EQ=3 EV=1 TY=1 SU=1 LO=1 FX=1 FY=1 FZ=1 TI=\$OBE INERTIAS

** *****
 **** LOAD COMBINATION CASES *
 ** *****

CCAS RF=1 CA=104 ME=1 FL=1 C1=103 CY=10 TI=\$OBE\$
 CCAS RF=1 CA=401 SS=1 ME=1 EQ=3 C1=101 C2=103 TI=\$EQUATION 9 LEVEL B\$
 CCAS RF=1 CA=402 SS=1 ME=3 F1=1 C1=103 C2=1 TI=\$NORMAL+OBE\$
 CCAS RF=1 CA=403 SS=1 ME=3 F1=-1 C1=103 C2=1 TI=\$NORMAL-OBE\$

 **** LOAD SETS****

LSET RF=1 FC=0 RP=1 CY=60 PR=1 MO=1 TR=201 TI=\$Design Hydrotest (+)LS-1\$
 LSET RF=2 FC=0 RP=1 CY=60 PR=2 MO=2 TR=-202 TI=\$Design Hydrotest (-)LS-2\$
 LSET RF=3 FC=0 RP=1 CY=150 PR=3 MO=3 TR=203 TI=\$Startup LS-3\$
 LSET RF=3 FC=0 RP=1 CY=290 PR=4 MO=4 TR=-204 TI=\$TRoll & Inc. PWR1 LS-4\$
 LSET RF=4 FC=0 RP=1 CY=290 PR=5 MO=5 TR=-205 TI=\$TRoll & Inc. PWR2 LS-5\$
 LSET RF=4 FC=0 RP=1 CY=10 PR=6 MO=6 TR=206 TI=\$LOFWH+TT PWR1 LS-6\$
 LSET RF=4 FC=0 RP=1 CY=10 PR=7 MO=7 TR=-207 TI=\$LOFWH+TT PWR2 LS-7\$
 LSET RF=5 FC=0 RP=1 CY=35 PR=8 MO=8 TR=-208 TI=\$LOFWH+PFWHTR Byp1 LS-8\$
 LSET RF=5 FC=0 RP=1 CY=35 PR=9 MO=9 TR=209 TI=\$LOFWH+PFWHTR Byp2 LS-9\$
 LSET RF=5 FC=0 RP=1 CY=5 PR=10 MO=10 TR=-210 TI=\$LOFWP, ISO C1 DN 1 LS-10\$
 LSET RF=11 FC=0 RP=1 CY=10 PR=11 MO=11 TR=211 TI=\$LOFWP, ISO C1 UP 1 LS-11\$
 LSET RF=11 FC=0 RP=1 CY=10 PR=12 MO=12 TR=-212 TI=\$LOFWP, ISO C1 DN 2 LS-12\$
 LSET RF=3 FC=0 RP=1 CY=5 PR=13 MO=13 TR=213 TI=\$LOFWP, ISO C1 UP 2 LS-13\$
 LSET RF=3 FC=0 RP=1 CY=150 PR=14 MO=14 TR=214 TI=\$Reduction to 0% PWR LS-14\$



LSET RF=5 FC=0 RP=1 CY=150 PR=15 MO=15 TR=-215 TI=\$Shutdown1 LS-15\$
 LSET RF=15 FC=0 RP=1 CY=150 PR=16 MO=16 TR=-216 TI=\$Shutdown2 LS-16\$
 LSET RF=16 FC=0 RP=1 CY=150 PR=17 MO=17 TR=-217 TI=\$Shutdown3 LS-17\$
 LSET RF=20 FC=0 RP=1 CY=150 PR=18 MO=18 TR=-218 TI=\$Shutdown4 LS-18\$
 LSET RF=19 FC=0 RP=1 CY=1 PR=19 MO=19 TR=219 TI=\$Code Hydrotest LS-19\$
 LSET RF=20 FC=0 RP=1 CY=150 PR=20 MO=20 TR=220 TI=\$RHR Initiation UP LS-20\$
 LSET RF=20 FC=0 RP=1 CY=150 PR=21 MO=21 TR=-221 TI=\$RHR Initiation DN LS-21\$
 LSET RF=5 FC=0 RP=1 CY=0 PR=22 MO=22 TR=-222 TI=\$Inadvert. Inj. DOWN LS-22\$
 LSET RF=5 FC=0 RP=1 CY=0 PR=23 MO=23 TR=223 TI=\$Inadvert. Inj. UP LS-23\$
 LSET RF=23 FC=0 RP=1 CY=0 PR=24 MO=24 TR=-224 TI=\$Single Relief BD DN LS-24\$
 LSET RF=24 FC=0 RP=1 CY=0 PR=25 MO=25 TR=225 TI=\$Single Relief BD UP LS-25\$

LSET RF=2 FC=0 CY=5 FL=1 PR=2 MO=402 TI=\$NORMAL+OBE LS-26\$
 LSET RF=2 FC=0 CY=5 FL=1 PR=2 MO=403 TI=\$NORMAL-OBE LS-27\$

 *FATG AT=500 AF=502
 *FATG AT=600 AF=602
 *

 **** RESPONSE SPECTRA****

SPEC FS=OBE EV=1 ME=3 FP=0 TI=\$RESPONSE\$

LV=1 DX=1 DY=1 DZ=1

DI=X

0.30/0.100	0.40/0.100	0.90/0.200	1.25/0.400	2.25/0.450	2.30/0.700
3.30/0.700	4.40/0.750	4.41/0.900	4.75/1.100	5.20/1.100	5.80/1.600
8.70/1.600	12.00/0.650	17.00/0.400	20.00/0.350	30.00/0.350	36.00/0.350

DI=Y

0.30/0.030	0.40/0.030	0.50/0.050	0.60/0.075	1.00/0.075	1.20/0.100
2.00/0.220	2.40/0.350	3.50/0.350	3.60/0.300	5.30/0.300	5.75/0.330
8.25/0.330	8.75/0.250	17.50/0.250	25.00/0.120	30.00/0.120	36.00/0.120

DI=Z

0.30/0.100	0.40/0.100	0.50/0.130	0.90/0.150	1.00/0.250	1.60/0.250
1.90/0.600	3.50/0.600	3.75/0.700	4.40/0.700	4.50/0.800	6.25/1.500
8.50/1.500	12.50/0.500	20.00/0.350	30.00/0.350	36.00/0.350	

 **** MATERIAL PROPERTIES ****

* ASTM A-106 Grade B, PIPE *

MATH CD=106	EX=0	TY=1	*C-Si	
MATD TE=70	EH=29.5	EX=0.0	SM=20.0	SY=35
MATD TE=100	EH=29.3	EX=0.20	SM=20.0	SY=35
MATD TE=200	EH=28.8	EX=1.00	SM=20.0	SY=32.1
MATD TE=300	EH=28.3	EX=1.90	SM=20.0	SY=31
MATD TE=400	EH=27.7	EX=2.80	SM=20.0	SY=29.9
MATD TE=500	EH=27.3	EX=3.70	SM=18.9	SY=28.5
MATD TE=600	EH=26.7	EX=4.70	SM=17.3	SY=26.8

* ASME SA-376 Grade TP316, PIPE *

MATH CD=376.316	EX=0	TY=4	*16Cr-12Ni-2Mo	
MATD TE=70	EH=28.3	EX=0.0	SM=20.0	SY=30.0
MATD TE=100	EH=28.1	EX=0.30	SM=20.0	SY=30.0
MATD TE=200	EH=27.6	EX=1.40	SM=20.0	SY=25.9
MATD TE=300	EH=27.0	EX=2.50	SM=20.0	SY=23.4
MATD TE=400	EH=26.5	EX=3.70	SM=19.3	SY=21.4
MATD TE=500	EH=25.8	EX=5.00	SM=18.0	SY=20.0
MATD TE=600	EH=25.3	EX=6.30	SM=17.0	SY=18.9

* ASME SA-403 Grade WP316, ELBOWS *

MATH CD=403.316	EX=0	TY=4	*16Cr-12Ni-2Mo	
MATD TE=70	EH=28.3	EX=0.0	SM=20.0	SY=30.0



MATD TE=100	EH=28.1	EX=0.30	SM=20.0	SY=30.0
MATD TE=200	EH=27.6	EX=1.40	SM=20.0	SY=25.9
MATD TE=300	EH=27.0	EX=2.50	SM=20.0	SY=23.4
MATD TE=400	EH=26.5	EX=3.70	SM=18.7	SY=21.4
MATD TE=500	EH=25.8	EX=5.00	SM=17.5	SY=20.0
MATD TE=600	EH=25.3	EX=6.30	SM=16.4	SY=18.9

*** Cross Sectional Properties

CROS CD=1	OD=50.0	WT=8.87	MA=3977.2	*CALC. PER GE SPEC. NO. 23A5569 [3]
	SO=1	ST=1.0		*RECIRCULATION OUTLET NOZZLE
CROS CD=2	OD=37.85	WT=6.1	MA=2122.2	*CALC. PER GE SPEC. NO. 23A5569 [3]
	SO=1	ST=1.0		
CROS CD=3	OD=28.875	WT=1.56	MA=484.9	*CALC. PER GE SPEC. NO. 23A5569 [3]
	SO=1	ST=1.0		
CROS CD=4	OD=28.638	WT=1.45	MA=450.4	*CALC. PER GE SPEC. NO. 23A5569 [3]
	SO=1	ST=1.0		
CROS CD=5	OD=28.169	WT=1.244	MA=386.1	*CALC. PER GE SPEC. NO. 23A5569 [3]
	SO=1	ST=1.0		
CROS CD=7	OD=28.166	WT=2.125	MA=0.001	*VALVE
	SO=1	ST=1.0	KL=1	
CROS CD=8	OD=42.507	WT=2.486	MA=0.001	*PUMP
	SO=0.001	ST=0.001	KL=1	
CROS CD=11	OD=6.625	WT=0.432	MA=0.001	*PUMP RIGID STRUTS
	SO=0.001	ST=0.001	KL=1	
CROS CD=13	OD=28.339	WT=1.339	MA=415.1	*CALC. PER GE SPEC. NO. 23A5569 [3]
	SO=1	ST=1		
CROS CD=14	OD=28.339	WT=2.67	MA=0.001	*VALVE
	SO=1	ST=1.0	KL=1	
CROS CD=15	OD=12.748	WT=0.685	MA=103.4	*CALC. PER GE SPEC. NO. 23A5569 [3]
	SO=1	ST=1.0		
CROS CD=16	OD=14.17	WT=1.395	MA=207.5	*CALC. PER GE SPEC. NO. 23A5569 [3]
	SO=1	ST=1.0		
CROS CD=17	OD=15.5	WT=2	MA=307.7	*CALC. PER GE SPEC. NO. 23A5569 [3]
	SO=1	ST=1.0		
CROS CD=18	OD=21.88	WT=4.06	MA=803.2	*CALC. PER GE SPEC. NO. 23A5569 [3]
	SO=1	ST=1.0		
CROS CD=19	OD=28.25	WT=7.25	MA=1673.1	*CALC. PER GE SPEC. NO. 23A5569 [3]
	SO=1	ST=1.0		
CROS CD=20	OD=21.878	WT=1.043	MA=257.2	*CALC. PER GE SPEC. NO. 23A5569 [3]
	SO=1	ST=1.0		
CROS CD=25	OD=20	WT=1.031	MA=221.9	*CALC. PER GE SPEC. NO. 23A5569 [3]
	SO=1	ST=1		
CROS CD=26	OD=20	WT=1.875	MA=0.001	*VALVE
	SO=1	ST=1	KL=1	
CROS CD=27	OD=4.5	WT=0.3385	MA=23.2	*CALC. PER GE SPEC. NO. 23A5569 [3]
	SO=1	ST=1	KL=1	*4 inch bypass line
CROS CD=28	OD=4.5	WT=0.67	MA=0.001	*VALVE V2-54A
	SO=1	ST=1	KL=1	
CROS CD=29	OD=24	WT=1.217	MA=316.5	*CALC. PER GE SPEC. NO. 23A5569 [3]
	SO=1	ST=1		
CROS CD=30	OD=24	WT=2.43	MA=0.001	*VALVE
	SO=1	ST=1	KL=1	
CROS CD=40	OD=4.5	WT=0.3385	MA=0.001	*4 inch bypass STRUTS
	SO=0.001	ST=0.001	KL=1	
CROS CD=41	OD=2.875	WT=0.276	MA=0.001	*STRUT RDA1, RDA5, & VBA1
	SO=0.001	ST=0.001	KL=1	
CROS CD=42	OD=28.339	WT=1.339	MA=0.001	*RIGID FROM RECIRC ELBOW TO RDA1 STRUT
	SO=0.001	ST=0.001	KL=1	

* STRUCTURE AND LOADS

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-----
DESN  TE=575.0  PR=1250.0 *Reference 12 GE Design Requirements Rpt VY-05Q-227
-----
*
*BEGIN REGION 1 TRANSIENT CARDS & GEOMETRY FROM RHR SUPPLY TO TEE
-----
*
INCL FN=Z:\SISJ-PROJECTS\VY-16Q\Rev0\REG1.INP
*
* RUN 1 FROM ANCHOR TO REACTOR VESSEL N3B
*GROUP 1 FROM ANCHOR TO REACTOR VESSEL N3B
*NOTE
*NOTE NODE 003 - RECIRC SUCTION NOZZLE N1A (EL. 279'5 INCH)
*NOTE NODE 003 IS AT THE SAFE END TO VESSEL NOZZLE CONNECTION
*NOTE
*NOTE SAFE END FROM NODES 003 TO 808
*NOTE CONNECTION TO VESSEL AT NODE 003
*NOTE OD AND WALL THICKNESS FOR SAFE END TAKEN FROM GE CALC
*NOTE WEIGHT FOR SAFE END BASED ON THICKNESS
*NOTE
MATL CD=376.316
CROS CD=1
COOR PT=3 AX=0 AY=0 AZ=0
ANCH PT=3
AMVT CA=1 PT=3 DX=0.0000 DY=0.0176 DZ=-0.0201
AMVT CA=2 PT=3 DX=0.0000 DY=0.3141 DZ=-0.3602
AMVT CA=3 PT=3 DX=0.0000 DY=0.3112 DZ=-0.3568
AMVT CA=4 PT=3 DX=0.0000 DY=0.2995 DZ=-0.3434
AMVT CA=5 PT=3 DX=0.0000 DY=0.3112 DZ=-0.3568
AMVT CA=6 PT=3 DX=0.0000 DY=0.2995 DZ=-0.3434
AMVT CA=7 PT=3 DX=0.0000 DY=0.2922 DZ=-0.3350
AMVT CA=8 PT=3 DX=0.0000 DY=0.2995 DZ=-0.3434
AMVT CA=9 PT=3 DX=0.0000 DY=0.1422 DZ=-0.1630
AMVT CA=10 PT=3 DX=0.0000 DY=0.2807 DZ=-0.3218
AMVT CA=11 PT=3 DX=0.0000 DY=0.1422 DZ=-0.1630
AMVT CA=12 PT=3 DX=0.0000 DY=0.3141 DZ=-0.3602
AMVT CA=13 PT=3 DX=0.0000 DY=0.3141 DZ=-0.3602
AMVT CA=14 PT=3 DX=0.0000 DY=0.1928 DZ=-0.2521
AMVT CA=15 PT=3 DX=0.0000 DY=0.1624 DZ=-0.1986
AMVT CA=16 PT=3 DX=0.0000 DY=0.0946 DZ=-0.1084
AMVT CA=17 PT=3 DX=0.0000 DY=0.0176 DZ=-0.0201
AMVT CA=18 PT=3 DX=0.0000 DY=0.0176 DZ=-0.0201
AMVT CA=19 PT=3 DX=0.0000 DY=0.0946 DZ=-0.1084
AMVT CA=20 PT=3 DX=0.0000 DY=0.0946 DZ=-0.1084
AMVT CA=21 PT=3 DX=0.0000 DY=0.0361 DZ=-0.0413
AMVT CA=22 PT=3 DX=0.0000 DY=0.2995 DZ=-0.3434
AMVT CA=23 PT=3 DX=0.0000 DY=0.1928 DZ=-0.2521
AMVT CA=24 PT=3 DX=0.0000 DY=0.0176 DZ=-0.0201

*
TANG PT=805 DZ=-1.017 EW=1
CROS CD=2
TANG PT=806 DZ=-0.823 EW=1
CROS CD=3
TANG PT=807 DZ=-0.58 EW=1
CROS CD=4
TANG PT=808 DZ=-0.47
CROS CD=5
TANG PT=5 DZ=-5.59 EW=1

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MATL CD=403.316
BRAD PT=7 RA=3.5 EW=1
MATL CD=376.316
TANG PT=9 DY=-6.69 EW=1
TANG PT=500 DY=-2.31

*-----
*END REGION 1 GEOMETRY FROM RHR SUPPLY TO TEE
*-----
*

*-----
*BEGIN REGION 2 TRANSIENT CARDS & GEOMETRY FROM RHR SUPPLY TEE TO PUMP
*-----
*

*GROUP 2 RHR SUPPLY TEE TO PUMP
INCL FN=Z:\SISJ-PROJECTS\VY-16Q\Rev0\REG2.INP
*

TANG PT=11 DY=-2.22 EW=1
CROS CD=5
TANG PT=12 DY=-1.78
TANG PT=20 DY=-6.77
TANG PT=22 DY=-3.25
TANG PT=25 DY=-15.49 EW=1
MATL CD=403.316
BRAD PT=26 RA=3.5 EW=1
MATL CD=376.316
TANG PT=27 DX=-3.3 DZ=1.27 EW=1
CROS CD=7
VALV PT=30 DX=-2.28 DZ=0.89 MA=10.368 PL=1
JUNC PT=30
VALV PT=40 DX=-2.31 DZ=0.9 PL=2 EW=1
JUNC PT=30
RIGD PT=35 DY=7
LUMP PT=35 MA=1.132
JUNC PT=40
CROS CD=5
TANG PT=42 DX=-1.18 DZ=0.46
TANG PT=43 DX=-0.55 DZ=0.21
TANG PT=44 DX=-3.31 DZ=1.28 EW=1
MATL CD=403.316
BRAD PT=46 RA=2.33 EW=1
MATL CD=376.316
CROS CD=8
TANG PT=50 DY=4.33 EW=0
LUMP PT=50 MA=28 *NOTE WEIGHT OF PUMP FLOODED 28K (EXCLUDING MOTOR)
TANG PT=75 DY=0.5
TANG PT=83 DY=2.13
TANG PT=86 DY=3.38
LUMP PT=86 MA=32 *NOTE TOTAL WEIGHT OF PUMP MOTOR 32000 LBS
TANG PT=90 DY=4.08 *TOP OF PUMP
*NOTE SNUBBERS ON TOP OF PUMPS WERE DELETED DURING
*NOTE THE RECIRC PIPE REPLACEMENT PROJECT
*NOTE - RIGID LINKS FOR CONSTANT SUPPORTS AT PUMP FOLLOW

*-----
*END REGION 2 GEOMETRY FROM RHR SUPPLY TEE TO PUMP
*-----
*

*-----
*BEGIN REGION 3 TRANSIENT CARDS & GEOMETRY FROM PUMP DISCHARGE TO HEADER
*-----
*

*GROUP 3 FROM PUMP DISCHARGE TO HEADER
INCL FN=Z:\SISJ-PROJECTS\VY-16Q\Rev0\REG3.INP

```

*
JUNC PT=50
CROS CD=8
RIGD PT=54 DX=1.06 DZ=1.06
RIGD PT=56 DX=1.06 DY=0.75 DZ=1.06 *NOTE CONSTANT SUPPORT HA3 AT NODE 56
JUNC PT=50
RIGD PT=66 DZ=-3.83
RIGD PT=69 DY=1 *NOTE CONSTANT SUPPORT HA4 AT NODE 69
JUNC PT=50
CROS CD=8
RIGD PT=60 DX=-3.83
RIGD PT=63 DY=1 *CONSTANT SUPPORT HA5 AT NODE 63
*
* *** CODING FOR PUMP RIGID STRUTS FOLLOW ***
* CODED FROM PUMP CENTERLINE
CROS CD=11
JUNC PT=66
RIGD PT=15 DY=0.7071 DZ=-0.7071
*
JUNC PT=60
RIGD PT=16 DX=-0.7071 DY=0.7071
* *** END OF CODING FOR PUMP SUPPORTS ***
*PUMP INLET
CROS CD=8
JUNC PT=50
TANG PT=150 DX=-2.17
BRAN PT=151 DZ=2.333 TE=1
*NOTE PUMP DISCHARGE CONNECTION TO PIPE AT NODE 151
CROS CD=13
TANG PT=152 DZ=1.25
TANG PT=155 DZ=1 EW=1
CROS CD=14
VALV PT=160 PL=1 DX=0.0 DY=0.0 DZ=2.52 MA=6.8285
JUNC PT=160
RIGD PT=163 DX=0.0 DY=7.12 DZ=0.0
LUMP PT=163 MA=0.9715
JUNC PT=160
VALV PT=170 PL=2 DX=0.0 DY=0.0 DZ=6.18 EW=1
CROS CD=13
MATL CD=403.316
BRAD PT=175 RA=3.5 EW=1
MATL CD=376.316
TANG PT=176 DY=5.95
TANG PT=177 DY=4.42
*NOTE ***WEIGHT OF FLOW ELEMENT NOT INCLUDED***
*NOTE ***REF. DWG. 5920-6800 FOR DIMENSIONS***
TANG PT=184 DY=4.42
TANG PT=186 DY=3.02
TANG PT=188 DY=1.51
TANG PT=189 DY=0.74
TANG PT=190 DY=1.15 EW=1
TANG PT=600 DY=1.06

***INPUT FILE TO INCLUDE EFFECTS OF RHR INITIATION ON LINE NEAR RHR RETURN TO HEADER
INCL FN=Z:\SISJ-PROJECTS\VY-16Q\Rev0\REG3B.INP

JUNC PT=600
TANG PT=195 DY=2.08 EW=1
TANG PT=210 DX=0.0 DY=1.83 DZ=0.0 KL=1 *CENTER OF CROSS, RECIRC HEADER
*MUST HAVE INDI CARD FOR EACH MEMBER CONNECTED TO CROSS CENTER

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*-----
*END REGION 3 GEOMETRY FROM PUMP DISCHARGE TO HEADER
*-----
*
*-----
*BEGIN REGION 5 TRANSIENT CARDS & GEOMETRY RISER TO NOZZLE NODE 336
*-----
*GROUP 5 RISER TO NOZZLE NODE 336
INCL FN=Z:\SISJ-PROJECTS\VY-16Q\Rev0\REG5.INP
*
*NOTE CROSS AND REDUCER DIMENSIONS TAKEN FROM 5920-6632 SHT.3
CROS CD=13
MATL CD=376.316
TANG PT=215 DX=0.0 DY=2.59 DZ=0.0 EW=0
CRED PT=220 DY=1.29 AN=30 EW=1 *AL=$CONC. REDUCER$
CROS CD=15
TANG PT=330 DY=4.58
TANG PT=335 DY=3.29 EW=1
MATL CD=403.316
BRAD PT=334 RA=1.5 EW=1
*-----
*END REGION 5 GEOMETRY RISER TO NOZZLE NODE 336
*-----
*
*-----
*BEGIN REGION 6 TRANSIENT CARDS & GEOMETRY TO NOZZLE NODE 336
*-----
*GROUP 6 TO NOZZLE NODE 336
INCL FN=Z:\SISJ-PROJECTS\VY-16Q\Rev0\REG6.INP
*
MATL CD=376.316
TANG PT=838 DX=3.875
CROS CD=16
TANG PT=837 DX=0.875 EW=1
CROS CD=17
TANG PT=836 DX=0.37 EW=1
CROS CD=18
TANG PT=835 DX=0.53 EW=1
CROS CD=19
TANG PT=336 DX=0.704 EW=1
NOZZ PT=336
AMVT CA=1 PT=336 DX=-0.0201 DY=0.0246 DZ=0.0000
AMVT CA=2 PT=336 DX=-0.3602 DY=0.4398 DZ=0.0000
AMVT CA=3 PT=336 DX=-0.3568 DY=0.4316 DZ=0.0000
AMVT CA=4 PT=336 DX=-0.3434 DY=0.4152 DZ=0.0000
AMVT CA=5 PT=336 DX=-0.3568 DY=0.4050 DZ=0.0000
AMVT CA=6 PT=336 DX=-0.3434 DY=0.2940 DZ=0.0000
AMVT CA=7 PT=336 DX=-0.3350 DY=0.3229 DZ=0.0000
AMVT CA=8 PT=336 DX=-0.3434 DY=0.2700 DZ=0.0000
AMVT CA=9 PT=336 DX=-0.1630 DY=0.1991 DZ=0.0000
AMVT CA=10 PT=336 DX=-0.3218 DY=0.1626 DZ=0.0000
AMVT CA=11 PT=336 DX=-0.1630 DY=0.0246 DZ=0.0000
AMVT CA=12 PT=336 DX=-0.3602 DY=0.4398 DZ=0.0000
AMVT CA=13 PT=336 DX=-0.3602 DY=0.4316 DZ=0.0000
AMVT CA=14 PT=336 DX=-0.2193 DY=0.4152 DZ=0.0000
AMVT CA=15 PT=336 DX=-0.1862 DY=0.4050 DZ=0.0000
AMVT CA=16 PT=336 DX=-0.1084 DY=0.2940 DZ=0.0000
AMVT CA=17 PT=336 DX=-0.0201 DY=0.3229 DZ=0.0000
AMVT CA=18 PT=336 DX=-0.0201 DY=0.2700 DZ=0.0000
AMVT CA=19 PT=336 DX=-0.1084 DY=0.1991 DZ=0.0000

```

AMVT	CA=20	PT=336	DX=-0.0201	DY=0.1626	DZ=0.0000
AMVT	CA=21	PT=336	DX=-0.0413	DY=0.3229	DZ=0.0000
AMVT	CA=22	PT=336	DX=-0.3434	DY=0.2700	DZ=0.0000
AMVT	CA=23	PT=336	DX=-0.2211	DY=0.1991	DZ=0.0000
AMVT	CA=24	PT=336	DX=-0.0201	DY=0.1626	DZ=0.0000

*NOTE SAFE END FROM NODES 838 TO 336
*NOTE CONNECTION TO VESSEL AT NODE 336
*NOTE OD AND WALL THICKNESS FOR SAFE END TAKEN FROM GE CALC
*NOTE WEIGHT BASED ON THICKNESS

*-----
*END REGION 6 GEOMETRY TO NOZZLE NODE 336
*-----
*

*-----
*BEGIN REGION 4 TRANSIENT CARDS & GEOMETRY HEADER TO NOZZLE NODE 366
*-----
*

*GROUP 4 HEADER TO NOZZLE NODE 366
INCL FN=Z:\SISJ-PROJECTS\VY-16Q\Rev0\REG4.INP
*

JUNC PT=210
CROS CD=20
BRAN PT=240 DX=0.1786 DY=0.0 DZ=1.7
TANG PT=250 DX=0.3 DZ=2.853 EW=0
BRAD PT=255 RA=4.578 EW=0 *NOTE BEND RADIUS IS 4.578 FEET
TANG PT=340 DX=1.799 DZ=3.108
*-----
*

*END REGION 4 GEOMETRY HEADER TO NOZZLE NODE 366
*-----
*

*-----
*BEGIN REGION 5 TRANSIENT CARDS & GEOMETRY RISER TO NOZZLE NODE 366
*-----
*

*GROUP 5 RISER TO NOZZLE NODE 366
INCL FN=Z:\SISJ-PROJECTS\VY-16Q\Rev0\REG5.INP
*

TANG PT=349 DX=0.71 DZ=1.23 EW=0
CRED PT=347 DX=0.75 DZ=1.3 AN=30
CROS CD=15
TANG PT=343 DX=0.5525 DZ=0.957 EW=1
BRAD PT=410 RA=1.5 EW=1
TANG PT=360 DX=3.483 DZ=2.011 EW=1
MATL CD=403.316
BRAD PT=361 RA=1.5 EW=1
MATL CD=376.316
CROS CD=15
TANG PT=362 DY=3.18
TANG PT=364 DY=8.56 EW=1
MATL CD=403.316
BRAD PT=365 RA=1.5 EW=1
*-----
*

*END REGION 5 GEOMETRY RISER TO NOZZLE NODE 366
*-----
*

*-----
*BEGIN REGION 6 TRANSIENT CARDS & GEOMETRY TO NOZZLE NODE 366
*-----
*

*GROUP 6 TO NOZZLE NODE 366
INCL FN=Z:\SISJ-PROJECTS\VY-16Q\Rev0\REG6.INP



*
MATL CD=376.316
TANG PT=868 DX=1.8 DZ=-3.1
CROS CD=16
TANG PT=867 DX=0.4375 DZ=-0.76 EW=1
CROS CD=17
TANG PT=866 DX=0.185 DZ=-0.32 EW=1
CROS CD=18
TANG PT=865 DX=0.265 DZ=-0.46 EW=1
CROS CD=19
TANG PT=366 DX=0.352 DZ=-0.61 EW=1
NOZZ PT=366
AMVT CA=1 PT=366 DX=-0.0101 DY=0.0246 DZ=0.0174
AMVT CA=2 PT=366 DX=-0.1800 DY=0.4398 DZ=0.3120
AMVT CA=3 PT=366 DX=-0.1783 DY=0.4357 DZ=0.3091
AMVT CA=4 PT=366 DX=-0.1716 DY=0.4193 DZ=0.2974
AMVT CA=5 PT=366 DX=-0.1783 DY=0.4357 DZ=0.3091
AMVT CA=6 PT=366 DX=-0.1716 DY=0.4193 DZ=0.2974
AMVT CA=7 PT=366 DX=-0.1674 DY=0.4091 DZ=0.2902
AMVT CA=8 PT=366 DX=-0.1716 DY=0.4193 DZ=0.2974
AMVT CA=9 PT=366 DX=-0.0815 DY=0.1991 DZ=0.1412
AMVT CA=10 PT=366 DX=-0.1609 DY=0.3930 DZ=0.2788
AMVT CA=11 PT=366 DX=-0.0815 DY=0.1991 DZ=0.1412
AMVT CA=12 PT=366 DX=-0.1800 DY=0.4398 DZ=0.3120
AMVT CA=13 PT=366 DX=-0.1800 DY=0.4398 DZ=0.3120
AMVT CA=14 PT=366 DX=-0.1097 DY=0.2678 DZ=0.1899
AMVT CA=15 PT=366 DX=-0.0931 DY=0.2275 DZ=0.1613
AMVT CA=16 PT=366 DX=-0.0542 DY=0.1324 DZ=0.0939
AMVT CA=17 PT=366 DX=-0.0101 DY=0.0246 DZ=0.0174
AMVT CA=18 PT=366 DX=-0.0101 DY=0.0246 DZ=0.0174
AMVT CA=19 PT=366 DX=-0.0542 DY=0.1324 DZ=0.0939
AMVT CA=20 PT=366 DX=-0.0101 DY=0.0246 DZ=0.0174
AMVT CA=21 PT=366 DX=-0.0207 DY=0.0505 DZ=0.0358
AMVT CA=22 PT=366 DX=-0.1716 DY=0.4193 DZ=0.2974
AMVT CA=23 PT=366 DX=-0.1105 DY=0.2700 DZ=0.1915
AMVT CA=24 PT=366 DX=-0.0101 DY=0.0246 DZ=0.0174

*-----
*END REGION 6 GEOMETRY TO NOZZLE NODE 366
*-----

*-----
*BEGIN REGION 4 TRANSIENT CARDS & GEOMETRY HEADER TO NOZZLES NODE 326 & 316
*-----

*GROUP 4 HEADER TO NOZZLES NODE 326 & 316
INCL FN=Z:\SISJ-PROJECTS\VY-16Q\Rev0\REG4.INP
*

JUNC PT=210
CROS CD=20
BRAN PT=260 DX=0.1786 DY=0.0 DZ=-1.7 TE=2
TANG PT=270 DX=0.3 DZ=-2.853 EW=0
BRAD PT=275 RA=4.578 EW=0
TANG PT=320 DX=1.799 DZ=-3.108
*-----

*END REGION 4 GEOMETRY HEADER TO NOZZLES NODE 326 & 316
*-----

*-----
*BEGIN REGION 5 TRANSIENT CARDS & GEOMETRY RISER TO NOZZLE NODE 316
*-----



*GROUP 5 RISER TO NOZZLE NODE 316
 INCL FN=Z:\SISJ-PROJECTS\VY-16Q\Rev0\REG5.INP

*
 TANG PT=319 DX=0.71 DZ=-1.23 EW=1
 CRED PT=317 DX=0.75 DZ=-1.3 AN=30
 CROS CD=15
 TANG PT=313 DX=0.5525 DZ=-0.957 EW=1
 BRAD PT=400 RA=1.5 EW=1
 TANG PT=310 DX=3.483 DZ=-2.011 EW=1
 MATL CD=403.316
 BRAD PT=311 RA=1.5 EW=1
 MATL CD=376.316
 CROS CD=15
 TANG PT=312 DY=4.74
 TANG PT=314 DY=6.99 EW=1
 MATL CD=403.316
 BRAD PT=315 RA=1.5 EW=1

*-----
 *END REGION 5 GEOMETRY RISER TO NOZZLE NODE 316
 *-----

*-----
 *BEGIN REGION 6 TRANSIENT CARDS & GEOMETRY TO NOZZLE NODE 316
 *-----

*GROUP 6 TO NOZZLE NODE 316
 INCL FN=Z:\SISJ-PROJECTS\VY-16Q\Rev0\REG6.INP

*
 MATL CD=376.316
 TANG PT=818 DX=1.84 DZ=3.19
 CROS CD=16
 TANG PT=817 DX=0.4375 DZ=0.76 EW=1
 CROS CD=17
 TANG PT=816 DX=0.185 DZ=0.32 EW=1
 CROS CD=18
 TANG PT=815 DX=0.265 DZ=0.46 EW=1
 CROS CD=19
 TANG PT=316 DX=0.352 DZ=0.61 EW=1
 NOZZ PT=316

AMVT	CA=1	PT=316	DX=-0.0101	DY=0.0246	DZ=-0.0174
AMVT	CA=2	PT=316	DX=-0.1800	DY=0.4398	DZ=-0.3120
AMVT	CA=3	PT=316	DX=-0.1783	DY=0.4357	DZ=-0.3091
AMVT	CA=4	PT=316	DX=-0.1716	DY=0.4193	DZ=-0.2974
AMVT	CA=5	PT=316	DX=-0.1783	DY=0.4357	DZ=-0.3091
AMVT	CA=6	PT=316	DX=-0.1716	DY=0.4193	DZ=-0.2974
AMVT	CA=7	PT=316	DX=-0.1674	DY=0.4091	DZ=-0.2902
AMVT	CA=8	PT=316	DX=-0.1716	DY=0.4193	DZ=-0.2974
AMVT	CA=9	PT=316	DX=-0.0815	DY=0.1991	DZ=-0.1412
AMVT	CA=10	PT=316	DX=-0.1609	DY=0.3930	DZ=-0.2788
AMVT	CA=11	PT=316	DX=-0.0815	DY=0.1991	DZ=-0.1412
AMVT	CA=12	PT=316	DX=-0.1800	DY=0.4398	DZ=-0.3120
AMVT	CA=13	PT=316	DX=-0.1800	DY=0.4398	DZ=-0.3120
AMVT	CA=14	PT=316	DX=-0.1097	DY=0.2678	DZ=-0.1899
AMVT	CA=15	PT=316	DX=-0.0931	DY=0.2275	DZ=-0.1613
AMVT	CA=16	PT=316	DX=-0.0542	DY=0.1324	DZ=-0.0939
AMVT	CA=17	PT=316	DX=-0.0101	DY=0.0246	DZ=-0.0174
AMVT	CA=18	PT=316	DX=-0.0101	DY=0.0246	DZ=-0.0174
AMVT	CA=19	PT=316	DX=-0.0542	DY=0.1324	DZ=-0.0939
AMVT	CA=20	PT=316	DX=-0.0101	DY=0.0246	DZ=-0.0174
AMVT	CA=21	PT=316	DX=-0.0207	DY=0.0505	DZ=-0.0358
AMVT	CA=22	PT=316	DX=-0.1716	DY=0.4193	DZ=-0.2974

AMVT CA=23 PT=316 DX=-0.1105 DY=0.2700 DZ=-0.1915
 AMVT CA=24 PT=316 DX=-0.0101 DY=0.0246 DZ=-0.0174

*-----
 *END REGION 6 GEOMETRY TO NOZZLE NODE 316
 *-----
 *

*-----
 *BEGIN REGION 5 TRANSIENT CARDS & GEOMETRY RISER TO NOZZLE NODE 346
 *-----
 *

*GROUP 5 RISER TO NOZZLE NODE 346
 INCL FN=Z:\SISJ-PROJECTS\VY-16Q\Rev0\REG5.INP
 *

JUNC PT=340
 CROS CD=15
 BRAN PT=342 DY=1.36 TE=2
 TANG PT=344 DY=10.39 EW=0
 MATL CD=403.316
 BRAD PT=345 RA=1.5 EW=1
 *

*-----
 *END REGION 5 GEOMETRY RISER TO NOZZLE NODE 346
 *-----
 *

*-----
 *BEGIN REGION 6 TRANSIENT CARDS & GEOMETRY TO NOZZLE NODE 346
 *-----
 *

*GROUP 6 TO NOZZLE NODE 346
 INCL FN=Z:\SISJ-PROJECTS\VY-16Q\Rev0\REG6.INP
 *

MATL CD=376.316
 TANG PT=848 DX=3.17 DZ=-1.83
 CROS CD=16
 TANG PT=847 DX=0.758 DZ=-0.4375 EW=1
 CROS CD=17
 TANG PT=846 DX=0.32 DZ=-0.185 EW=1
 CROS CD=18
 TANG PT=845 DX=0.46 DZ=-0.265 EW=1
 CROS CD=19
 TANG PT=346 DX=0.61 DZ=-0.352 EW=1
 NOZZ PT=346

AMVT	CA=1	PT=346	DX=-0.0174	DY=0.0246	DZ=0.0101
AMVT	CA=2	PT=346	DX=-0.3120	DY=0.4398	DZ=0.1800
AMVT	CA=3	PT=346	DX=-0.3091	DY=0.4357	DZ=0.1783
AMVT	CA=4	PT=346	DX=-0.2974	DY=0.4193	DZ=0.1716
AMVT	CA=5	PT=346	DX=-0.3091	DY=0.4357	DZ=0.1783
AMVT	CA=6	PT=346	DX=-0.2974	DY=0.4193	DZ=0.1716
AMVT	CA=7	PT=346	DX=-0.2902	DY=0.4091	DZ=0.1674
AMVT	CA=8	PT=346	DX=-0.2974	DY=0.4193	DZ=0.1716
AMVT	CA=9	PT=346	DX=-0.1412	DY=0.1991	DZ=0.0815
AMVT	CA=10	PT=346	DX=-0.2788	DY=0.3930	DZ=0.1609
AMVT	CA=11	PT=346	DX=-0.1412	DY=0.1991	DZ=0.0815
AMVT	CA=12	PT=346	DX=-0.3120	DY=0.4398	DZ=0.1800
AMVT	CA=13	PT=346	DX=-0.3120	DY=0.4398	DZ=0.1800
AMVT	CA=14	PT=346	DX=-0.1899	DY=0.2678	DZ=0.1097
AMVT	CA=15	PT=346	DX=-0.1613	DY=0.2275	DZ=0.0931
AMVT	CA=16	PT=346	DX=-0.0939	DY=0.1324	DZ=0.0542
AMVT	CA=17	PT=346	DX=-0.0174	DY=0.0246	DZ=0.0101
AMVT	CA=18	PT=346	DX=-0.0174	DY=0.0246	DZ=0.0101
AMVT	CA=19	PT=346	DX=-0.0939	DY=0.1324	DZ=0.0542
AMVT	CA=20	PT=346	DX=-0.0174	DY=0.0246	DZ=0.0101
AMVT	CA=21	PT=346	DX=-0.0358	DY=0.0505	DZ=0.0207

```

AMVT   CA=22   PT=346   DX=-0.2974   DY=0.4193   DZ=0.1716
AMVT   CA=23   PT=346   DX=-0.1915   DY=0.2700   DZ=0.1105
AMVT   CA=24   PT=346   DX=-0.0174   DY=0.0246   DZ=0.0101
*-----
*END REGION 6 GEOMETRY TO NOZZLE NODE 346
*-----
*
*-----
*BEGIN REGION 5 TRANSIENT CARDS & GEOMETRY RISER TO NOZZLE NODE 326
*-----
*GROUP 5 RISER TO NOZZLE NODE 326
INCL FN=Z:\SISJ-PROJECTS\VY-16Q\Rev0\REG5.INP
*
JUNC PT=320
CROS CD=15
BRAN PT=322 DY=1.42 TE=2
TANG PT=324 DY=10.33 EW=1
MATL CD=403.316
BRAD PT=325 RA=1.5 EW=1
*-----
*END REGION 5 GEOMETRY RISER TO NOZZLE NODE 326
*-----
*
*-----
*BEGIN REGION 6 TRANSIENT CARDS & GEOMETRY TO NOZZLE NODE 326
*-----
*GROUP 6 TO NOZZLE NODE 326
INCL FN=Z:\SISJ-PROJECTS\VY-16Q\Rev0\REG6.INP
*
MATL CD=376.316
TANG PT=828 DX=3.18 DZ=1.84
CROS CD=16
TANG PT=827 DX=0.758 DZ=0.4375 EW=1
CROS CD=17
TANG PT=826 DX=0.32 DZ=0.185 EW=1
CROS CD=18
TANG PT=825 DX=0.46 DZ=0.265 EW=1
CROS CD=19
TANG PT=326 DX=0.61 DZ=0.352 EW=1
NOZZ PT=326
AMVT   CA=1     PT=326   DX=-0.0174   DY=0.0246   DZ=-0.0101
AMVT   CA=2     PT=326   DX=-0.3120   DY=0.4398   DZ=-0.1800
AMVT   CA=3     PT=326   DX=-0.3091   DY=0.4357   DZ=-0.1783
AMVT   CA=4     PT=326   DX=-0.2974   DY=0.4193   DZ=-0.1716
AMVT   CA=5     PT=326   DX=-0.3091   DY=0.4357   DZ=-0.1783
AMVT   CA=6     PT=326   DX=-0.2974   DY=0.4193   DZ=-0.1716
AMVT   CA=7     PT=326   DX=-0.2902   DY=0.4091   DZ=-0.1674
AMVT   CA=8     PT=326   DX=-0.2974   DY=0.4193   DZ=-0.1716
AMVT   CA=9     PT=326   DX=-0.1412   DY=0.1991   DZ=-0.0815
AMVT   CA=10    PT=326   DX=-0.2788   DY=0.3930   DZ=-0.1609
AMVT   CA=11    PT=326   DX=-0.1412   DY=0.1991   DZ=-0.0815
AMVT   CA=12    PT=326   DX=-0.3120   DY=0.4398   DZ=-0.1800
AMVT   CA=13    PT=326   DX=-0.3120   DY=0.4398   DZ=-0.1800
AMVT   CA=14    PT=326   DX=-0.1899   DY=0.2678   DZ=-0.1097
AMVT   CA=15    PT=326   DX=-0.1613   DY=0.2275   DZ=-0.0931
AMVT   CA=16    PT=326   DX=-0.0939   DY=0.1324   DZ=-0.0542
AMVT   CA=17    PT=326   DX=-0.0174   DY=0.0246   DZ=-0.0101
AMVT   CA=18    PT=326   DX=-0.0174   DY=0.0246   DZ=-0.0101
AMVT   CA=19    PT=326   DX=-0.0939   DY=0.1324   DZ=-0.0542
AMVT   CA=20    PT=326   DX=-0.0174   DY=0.0246   DZ=-0.0101

```

AMVT CA=21 PT=326 DX=-0.0358 DY=0.0505 DZ=-0.0207
AMVT CA=22 PT=326 DX=-0.2974 DY=0.4193 DZ=-0.1716
AMVT CA=23 PT=326 DX=-0.1915 DY=0.2700 DZ=-0.1105
AMVT CA=24 PT=326 DX=-0.0174 DY=0.0246 DZ=-0.0101

*-----
*END REGION 6 GEOMETRY TO NOZZLE NODE 326
*-----

*-----
*BEGIN REGION 7A TRANSIENT CARDS & GEOMETRY TO RHR SUPPLY VALVE NODE 550
*-----

*GROUP 7 TO RHR SUPPLY VALVE NODE 550

INCL FN=Z:\SISJ-PROJECTS\VY-16Q\Rev0\REG7A.INP

*
MATL CD=376.316
JUNC PT=500
CROS CD=25
BRAN PT=502 DX=1.67 EW=0 TE=1
TANG PT=506 DX=2.53 EW=0
MATL CD=403.316
BRAD PT=507 RA=1.67 EW=1
MATL CD=376.316
TANG PT=508 DZ=-4.01
TANG PT=515 DZ=-4.53 EW=1
MATL CD=403.316
BRAD PT=520 RA=1.67 EW=1
MATL CD=376.316
CROS CD=26
VALV PT=525 DX=-3.34 PL=1
JUNC PT=525
VALV PT=530 DX=-1.99 PL=2 EW=1
JUNC PT=525
RIGD PT=526 DY=2.5
LUMP PT=526 MA=7.569
JUNC PT=530
CROS CD=25
TANG PT=540 DX=-1.13 EW=1
CROS CD=26
VALV PT=545 DX=-1.97 PL=1
JUNC PT=545
RIGD PT=547 DY=2.5
LUMP PT=547 MA=7.355
JUNC PT=545
VALV PT=550 DX=-1.98 PL=2 EW=1

*-----
*END REGION 7A GEOMETRY TO RHR SUPPLY VALVE NODE 550
*-----

*-----
*BEGIN REGION 7B TRANSIENT CARDS & GEOMETRY FROM RHR SUPPLY VALVE TO PENET. NODE 565
*-----

*GROUP 17 FROM RHR SUPPLY VALVE TO PENET. NODE 565

INCL FN=Z:\SISJ-PROJECTS\VY-16Q\Rev0\REG7B.INP

*
CROS CD=25
MATL CD=106
TANG PT=555 DX=-3.36 EW=1
BRAD PT=556 RA=1.67 EW=1
TANG PT=560 DY=-10.17 EW=1
BRAD PT=561 RA=1.67 EW=1



TANG PT=563 DZ=-6.92

TANG PT=565 DZ=-6.92

*-----
*END REGION 7B GEOMETRY FROM RHR SUPPLY VALVE TO PENET. NODE 565
*-----

*-----
*BEGIN REGION 8 TRANSIENT CARDS & GEOMETRY FOR 4 INCH BYPASS
*-----

*GROUP 8 4 INCH BYPASS

INCL FN=Z:\SISJ-PROJECTS\VY-16Q\Rev0\REG8.INP

*NOTE CODING FOR 4 INCH BYPASS STARTS HERE

JUNC PT=152

CROS CD=27

MATL CD=376.316

BRAN PT=700 DX=-1.19 TE=4

TANG PT=702 DX=-0.61

TANG PT=703 DX=-1.43 EW=0

MATL CD=403.316

BRAD PT=704 RA=0.5 EW=0

MATL CD=376.316

TANG PT=705 DZ=5.08

*NOTE CONSTANT SUPPORT HA11 AT NODE 705

TANG PT=721 DZ=1.12

TANG PT=706 DZ=2.47

TANG PT=707 DZ=1.03

TANG PT=708 DZ=0.34

TANG PT=709 DZ=0.38

JUNC PT=707

BRAN PT=710 DY=0.34 TE=1

CROS CD=28

VALV PT=712 DY=0.71 MA=0.3669 PL=1 *AL=\$VALVE V2-54A\$

VALV PT=715 DZ=-3.5 MA=0.1831 PL=3

JUNC PT=712

VALV PT=714 DY=0.71 PL=2

CROS CD=27

TANG PT=723 DY=4.19

MATL CD=403.316

BRAD PT=716 RA=0.5

MATL CD=376.316

TANG PT=718 DX=1.48

TANG PT=720 DX=0.56

BRAN PT=176 DX=1.19 TE=4

*****CODING FOR STRUTS RDA5 AND VAB1 FOLLOW

JUNC PT=170

CROS CD=40 *OD=4.5 inch

RIGD PT=725 DP=0 DX=-0.583 DY=1.84 *AL=\$RDA5\$

CROS CD=41 *OD=2.875 inch

RIGD PT=715 DP=0 DX=-2.67 DY=-0.79

RIGD PT=721 DP=0 DY=-1.05 *AL=\$VAB1\$

*****CODING FOR RDA1 STRUT FOLLOWS

CROS CD=42 *OD=28.339 inch

JUNC PT=175

RIGD PT=173 DP=0 DY=-3.5 DZ=0.34

CROS CD=41 *OD=2.875 inch

RIGD PT=708 DP=0 DX=-3.21 *AL=\$RDA1\$

*-----
*END REGION 8 GEOMETRY FOR 4 INCH BYPASS
*-----



```
*
*-----
*BEGIN REGION 9A TRANSIENT CARDS & GEOMETRY FOR RHR RETURN FROM TEE TO VALVE NODE 660
*-----
*GROUP 9 RHR RETURN FROM TEE TO VALVE NODE 660
INCL FN=Z:\SISJ-PROJECTS\VY-16Q\Rev0\REG9A.INP
*
*NOTE CODING FOR RHR RETURN STARTS HERE
CROS CD=29
JUNC PT=600
MATL CD=376.316
BRAN PT=602 DX=-3.8123 TE=1
MATL CD=403.316
BRAD PT=610 RA=2 EW=1
TANP DY=4
BRAD PT=612 RA=2 EW=1
MATL CD=376.316
TANG PT=614 DZ=-10.38 EW=1
MATL CD=403.316
BRAD PT=615 RA=10 EW=1
MATL CD=376.316
TANG PT=620 DX=5.98 DZ=-3.45 EW=1
*NOTE
*NOTE VARIABLE SPRING H104 AT NODE 620
*NOTE
*NOTE VALVE V10-81A DATA FROM 5920-4590 WEIGHT - 6845.#
*NOTE WEIGHT APPLIED AT ESTIMATED CENTER OF GRAVITY (NODE 623)
CROS CD=30
VALV PT=622 DX=1.98 DZ=-1.15 PL=1 *AL=$VALVE V10-81A$
JUNC PT=622
VALV PT=624 DX=1.98 DZ=-1.15 PL=2 EW=1
JUNC PT=622
RIGD PT=623 DY=2.5
LUMP PT=623 MA=7.32 *VALVE ACTUATOR
CROS CD=29
JUNC PT=624
TANG PT=625 DX=1.867 DZ=-1.078
TANG PT=630 DX=2.598 DZ=-1.5 EW=1
MATL CD=403.316
BRAD PT=631 RA=3 EW=1
MATL CD=376.316
TANG PT=640 DZ=-4.54 EW=1
MATL CD=403.316
BRAD PT=641 RA=2 EW=1
MATL CD=376.316
*NOTE VALVE V10-46A DATA FROM 5920-4718 WEIGHT - 5295.#
CROS CD=30
VALV PT=655 DX=-3.79 PL=1 TA=2 *AL=$VALVE V10-46A$
LUMP PT=655 MA=5.77
*-----
*END REGION 9A GEOMETRY FOR RHR RETURN FROM TEE TO VALVE NODE 660
*-----
*
*-----
*BEGIN REGION 9B TRANSIENT CARDS & GEOMETRY FOR RHR RETURN FROM VALVE NODE 660 TO PENET. NODE
675
*-----
*GROUP 19 RHR RETURN FROM VALVE NODE 660 TO PENET. NODE 675
INCL FN=Z:\SISJ-PROJECTS\VY-16Q\Rev0\REG9B.INP
```



*

*NOTE

*NOTE VARIABLE SPRING H105 AT NODE 655

*NOTE

VALV PT=660 DX=-1.79 PL=2 EW=1

*NOTE SPEC CHANGE TO CARBON STEEL

MATL CD=106

CROS CD=29

TANG PT=661 DX=-1

TANG PT=663 DX=-3.31 EW=1

BRAD PT=665 RA=2 EW=1

TANG PT=670 DY=-10.5 DZ=0.38 EW=1

BRAD PT=671 RA=2 EW=1

TANG PT=673 DZ=-7.74

TANG PT=675 DZ=-7.74

*

*END REGION 9B GEOMETRY FOR RHR RETURN FROM VALVE NODE 660 TO PENET. NODE 675

*

*

***STRESS INDICES AT CROSS POINT

*

*

INDI AT=210 AF=195 B1=0.5 C1=1 K1=4 B2=2.256 C2=3.024 K2=1 C3=1 K3=1 CP=0.5

INDI AT=210 AF=215 B1=0.5 C1=1 K1=4 B2=2.256 C2=3.024 K2=1 C3=1 K3=1 CP=0.5

INDI AT=210 AF=240 B1=0.5 C1=1 K1=4 B2=1.805 C2=3.024 K2=1 C3=1 K3=1 CP=0.5

INDI AT=210 AF=260 B1=0.5 C1=1 K1=4 B2=1.805 C2=3.024 K2=1 C3=1 K3=1 CP=0.5

*

*** SUPPORTS

*

RSTN PT=675 DX=1 SP=16000

*RHR SUPPLY PENET.

RSTN PT=675 DY=1 SP=16000

*RHR SUPPLY PENET.

RSTN PT=675 DZ=1 SP=23000

*RHR SUPPLY PENET.

ROTR PT=675 RX=1 SP=300000

*RHR SUPPLY PENET.

ROTR PT=675 RY=1 SP=300000

*RHR SUPPLY PENET.

ROTR PT=675 RZ=1 SP=340000

*RHR SUPPLY PENET.

RSTN PT=565 DX=1 SP=16000

*RHR SUPPLY PENET.

RSTN PT=565 DY=1 SP=16000

*RHR SUPPLY PENET.

RSTN PT=565 DZ=1 SP=23000

*RHR SUPPLY PENET.

ROTR PT=565 RX=1 SP=300000

*RHR SUPPLY PENET.

ROTR PT=565 RY=1 SP=300000

*RHR SUPPLY PENET.

ROTR PT=565 RZ=1 SP=340000

*RHR SUPPLY PENET.

*

SNUB PT=12 DZ=-1 SP=1000

*AL=\$SNUBBER SS-7A-1\$

SNUB PT=12 DX=1 SP=1000

*AL=\$SNUBBER SS-7A-2\$

SNUB PT=190 DX=-1 SP=1000

*AL=\$SNUBBER SS-6-A1\$

SNUB PT=190 DZ=1 SP=1000

*AL=\$SNUBBER SS-6-A2\$

*

VSUP PT=20 DY=1 FO=24.8 SP=2.664

*AL=\$VARI. SUPT. HA-1\$

*

CSUP PT=27 DY=1 FO=8.3 KP=0.01

*AL=\$CONST. SUPT. H-8-A1\$

CSUP PT=42 DY=1 FO=8.3 KP=0.01

*AL=\$CONST. SUPT. H-8-A2\$

CSUP PT=56 DY=1 FO=18.05 KP=0.01

*AL=\$CONST. SUPT. HA3 FOR PUMP\$

CSUP PT=69 DY=1 FO=18.0 KP=0.01

*AL=\$CONST. SUPT. HA4 FOR PUMP\$

CSUP PT=63 DY=1 FO=18.02 KP=0.01

*AL=\$CONST. SUPT. HA5 FOR PUMP\$

CSUP PT=160 DY=-1 FO=11.8 KP=0.01

*AL=\$CONST. SUPT. HA-9 & HA-10\$

CSUP PT=705 DY=1 FO=0.960 KP=0.01

*AL=\$CONST. SUPT. HA-11 ON 4 INCH BYPASS\$

*

VSUP PT=184 DY=1 FO=36.0 SP=3.542

*AL=\$VARI. SUPT. HA-2\$

VSUP PT=343 DY=1 FO=7.1 SP=3.014

*AL=\$VARI. SUPT. HA13\$

```

VSUP PT=313  DY=1  FO=7.1  SP=3.014  *AL=$VARI. SUPT. HA14$
*
VSUP PT=530  DY=1  SP=9.420  FO=26.0  *AL=$HANGER H109 RHR SUPPLY VALVE$
VSUP PT=620  DY=1  SP=7.084  FO=14.9  *AL=$HANGER H104 RHR RETURN VALVE$
VSUP PT=655  DY=1  SP=4.710  FO=22.0  *AL=$HANGER H105 RHR RETURN VALVE$
*
RSTN PT=15   DY=0.7071  DZ=-0.7071  SP=6000  *RECIRC PUMP
RSTN PT=16   DX=-0.7071  DY=0.7071  SP=6000  *RECIRC PUMP
ENDP

```

RHR 15.inp

```

IDEN JB=3  *Job number (1 to 9999)
CD=1  *1=ASME Class 1
GR=-Y  *Direction of gravity
VA=0  *0=Calculate 2=Verify
IU=1  *Input units 1=USA
OU=1  *Output units 1=USA
CH=$  *Delimiter character
AB=T  *FREE errors = abort
PL=$Vermont Yankee$
EN=$RVPS$
TITL BL=3  *Modeling option:
* 3 = uniform mass for static analysis
* lumped mass for dynamic analysis
* rotational inertia ignored
GL=1  *Report forces/moment 0=Global 1=Local 2=G et L
SU=1  *Support summary 0=No 1=Yes
CV=15 *Code version - See Manual
HS=1  *Highest 20 stress ratios for each case
MD=1  *Hot modulus
J6=1  *File generated by program
TI=$Vermont Yankee Recirculation $
$Fatigue Analysis$
FREQ RF=1 RP=8 FR=36 MP=20 RC=0 MX=70 TI=$SEISMIC$
*
*****
**** THERMAL CYCLE LOAD CASES****
*****
LCAS RF=0 CA=1 TY=0 TI=$LC-1$ *TC-1
LCAS RF=0 CA=2 TY=0 TI=$LC-2$ *TC-2
LCAS RF=0 CA=3 TY=0 TI=$LC-3$ *TC-3
LCAS RF=0 CA=4 TY=0 TI=$LC-4$ *TC-4
LCAS RF=0 CA=5 TY=0 TI=$LC-5$ *TC-5
LCAS RF=0 CA=6 TY=0 TI=$LC-6$ *TC-6
LCAS RF=0 CA=7 TY=0 TI=$LC-7$ *TC-7
LCAS RF=0 CA=8 TY=0 TI=$LC-8$ *TC-8
LCAS RF=0 CA=9 TY=0 TI=$LC-9$ *TC-9
LCAS RF=0 CA=10 TY=0 TI=$LC-10$ *TC-10
LCAS RF=0 CA=11 TY=0 TI=$LC-11$ *TC-11
LCAS RF=0 CA=12 TY=0 TI=$LC-12$ *TC-12
LCAS RF=0 CA=13 TY=0 TI=$LC-13$ *TC-13
LCAS RF=0 CA=14 TY=0 TI=$LC-14$ *TC-14
LCAS RF=0 CA=15 TY=0 TI=$LC-15$ *TC-15
LCAS RF=0 CA=16 TY=0 TI=$LC-16$ *TC-16
LCAS RF=0 CA=17 TY=0 TI=$LC-17$ *TC-17
LCAS RF=0 CA=18 TY=0 TI=$LC-18$ *TC-18
LCAS RF=0 CA=19 TY=0 TI=$LC-19$ *TC-19
LCAS RF=0 CA=20 TY=0 TI=$LC-20$ *TC-20
LCAS RF=0 CA=21 TY=0 TI=$LC-21$ *TC-21
LCAS RF=0 CA=22 TY=0 TI=$LC-22$ *TC-22

```

LCAS RF=0 CA=23 TY=0 TI=\$LC-23\$ *TC-23
 LCAS RF=0 CA=24 TY=0 TI=\$LC-24\$ *TC-24
 LCAS RF=0 CA=25 TY=0 TI=\$LC-25\$ *TC-25

*

 **** WEIGHT CASES****

LCAS CA=101 RF=1 TY=3 TI=\$OPERATING WEIGHTS\$
 LCAS CA=102 RF=2 TY=4 TI=\$HYDROTEST WEIGHTS\$

*

 **** THERMAL TRANSIENT CASES****

TCAS CA=201 TI=\$Design Hydrotest (+) \$
 TCAS CA=202 TI=\$Design Hydrotest (-) \$
 TCAS CA=203 TI=\$Startup \$
 TCAS CA=204 TI=\$TRoll & Inc. PWR1 \$
 TCAS CA=205 TI=\$TRoll & Inc. PWR2 \$
 TCAS CA=206 TI=\$LOFWH+TT PWR1 \$
 TCAS CA=207 TI=\$LOFWH+TT PWR2 \$
 TCAS CA=208 TI=\$LOFWH+PFWHTR Byp1 \$
 TCAS CA=209 TI=\$LOFWH+PFWHTR Byp2 \$
 TCAS CA=210 TI=\$LOFWP, ISO C1 DN 1 \$
 TCAS CA=211 TI=\$LOFWP, ISO C1 UP 1 \$
 TCAS CA=212 TI=\$LOFWP, ISO C1 DN 2 \$
 TCAS CA=213 TI=\$LOFWP, ISO C1 UP 2 \$
 TCAS CA=214 TI=\$Reduction to 0% PWR \$
 TCAS CA=215 TI=\$Shutdown1 \$
 TCAS CA=216 TI=\$Shutdown2 \$
 TCAS CA=217 TI=\$Shutdown3 \$
 TCAS CA=218 TI=\$Shutdown4 \$
 TCAS CA=219 TI=\$Code Hydrotest \$
 TCAS CA=220 TI=\$RHR Initiation UP \$
 TCAS CA=221 TI=\$RHR Initiation DN \$
 TCAS CA=222 TI=\$Inadvert. Inj. DOWN \$
 TCAS CA=223 TI=\$Inadvert. Inj. UP \$
 TCAS CA=224 TI=\$Single Relief BD DN \$
 TCAS CA=225 TI=\$Single Relief BD UP \$

*

 **** SEISMIC CASES****

RCAS CA=103 EQ=3 EV=1 TY=1 SU=1 LO=1 FX=1 FY=1 FZ=1 TI=\$OBE INERTIAS\$

*
 ** *****
 **** LOAD COMBINATION CASES *
 ** *****

CCAS RF=1 CA=104 ME=1 FL=1 C1=103 CY=10 TI=\$OBE\$
 CCAS RF=1 CA=401 SS=1 ME=1 EQ=3 C1=101 C2=103 TI=\$EQUATION 9 LEVEL B\$
 CCAS RF=1 CA=402 SS=1 ME=3 F1=1 C1=103 C2=1 TI=\$NORMAL+OBE\$
 CCAS RF=1 CA=403 SS=1 ME=3 F1=-1 C1=103 C2=1 TI=\$NORMAL-OBE\$

*

 **** LOAD SETS****

*
 LSET RF=1 FC=0 RP=1 CY=120 PR=1 MO=1 TR=201 TI=\$Design Hydrotest (+)LS-1\$
 LSET RF=2 FC=0 RP=1 CY=120 PR=2 MO=2 TR=-202 TI=\$Design Hydrotest (-)LS-2\$
 LSET RF=3 FC=0 RP=1 CY=300 PR=3 MO=3 TR=203 TI=\$Startup LS-3\$
 LSET RF=3 FC=0 RP=1 CY=579 PR=4 MO=4 TR=-204 TI=\$TRoll & Inc. PWR1 LS-4\$

```

LSET RF=4 FC=0 RP=1 CY=579 PR=5 MO=5 TR=-205 TI=$TRoll & Inc. PWR2 LS-5$
LSET RF=4 FC=0 RP=1 CY=20 PR=6 MO=6 TR=206 TI=$LOFWH+TT PWR1 LS-6$
LSET RF=4 FC=0 RP=1 CY=20 PR=7 MO=7 TR=-207 TI=$LOFWH+TT PWR2 LS-7$
LSET RF=5 FC=0 RP=1 CY=70 PR=8 MO=8 TR=-208 TI=$LOFWH+PFWHTR Byp1 LS-8$
LSET RF=5 FC=0 RP=1 CY=70 PR=9 MO=9 TR=209 TI=$LOFWH+PFWHTR Byp2 LS-9$
LSET RF=5 FC=0 RP=1 CY=10 PR=10 MO=10 TR=-210 TI=$LOFWP, ISO Cl DN 1 LS-10$
LSET RF=11 FC=0 RP=1 CY=20 PR=11 MO=11 TR=211 TI=$LOFWP, ISO Cl UP 1 LS-11$
LSET RF=11 FC=0 RP=1 CY=20 PR=12 MO=12 TR=-212 TI=$LOFWP, ISO Cl DN 2 LS-12$
LSET RF=3 FC=0 RP=1 CY=10 PR=13 MO=13 TR=213 TI=$LOFWP, ISO Cl UP 2 LS-13$
LSET RF=3 FC=0 RP=1 CY=300 PR=14 MO=14 TR=214 TI=$Reduction to 0% PWR LS-14$
LSET RF=5 FC=0 RP=1 CY=300 PR=15 MO=15 TR=-215 TI=$Shutdown1 LS-15$
LSET RF=15 FC=0 RP=1 CY=300 PR=16 MO=16 TR=-216 TI=$Shutdown2 LS-16$
LSET RF=16 FC=0 RP=1 CY=300 PR=17 MO=17 TR=-217 TI=$Shutdown3 LS-17$
LSET RF=20 FC=0 RP=1 CY=300 PR=18 MO=18 TR=-218 TI=$Shutdown4 LS-18$
LSET RF=19 FC=0 RP=1 CY=1 PR=19 MO=19 TR=219 TI=$Code Hydrotest LS-19$
LSET RF=20 FC=0 RP=1 CY=300 PR=20 MO=20 TR=220 TI=$RHR Initiation UP LS-20$
LSET RF=20 FC=0 RP=1 CY=300 PR=21 MO=21 TR=-221 TI=$RHR Initiation DN LS-21$
LSET RF=5 FC=0 RP=1 CY=0 PR=22 MO=22 TR=-222 TI=$Inadvert. Inj. DOWN LS-22$
LSET RF=5 FC=0 RP=1 CY=0 PR=23 MO=23 TR=223 TI=$Inadvert. Inj. UP LS-23$
LSET RF=23 FC=0 RP=1 CY=0 PR=24 MO=24 TR=-224 TI=$Single Relief BD DN LS-24$
LSET RF=24 FC=0 RP=1 CY=0 PR=25 MO=25 TR=225 TI=$Single Relief BD UP LS-25$

```

```

*
LSET RF=2 FC=0 CY=5 FL=1 PR=2 MO=402 TI=$NORMAL+OBE LS-26$
LSET RF=2 FC=0 CY=5 FL=1 PR=2 MO=403 TI=$NORMAL-OBE LS-27$

```

```

*FATG AT=500 AF=502
*FATG AT=600 AF=602

```

*

**** RESPONSE SPECTRA****

```

SPEC FS=OBE EV=1 ME=3 FP=0 TI=$RESPONSE$

```

```

LV=1 DX=1 DY=1 DZ=1

```

```

DI=X

```

```

0.30/0.100 0.40/0.100 0.90/0.200 1.25/0.400 2.25/0.450 2.30/0.700
3.30/0.700 4.40/0.750 4.41/0.900 4.75/1.100 5.20/1.100 5.80/1.600
8.70/1.600 12.00/0.650 17.00/0.400 20.00/0.350 30.00/0.350 36.00/0.350

```

```

DI=Y

```

```

0.30/0.030 0.40/0.030 0.50/0.050 0.60/0.075 1.00/0.075 1.20/0.100
2.00/0.220 2.40/0.350 3.50/0.350 3.60/0.300 5.30/0.300 5.75/0.330
8.25/0.330 8.75/0.250 17.50/0.250 25.00/0.120 30.00/0.120 36.00/0.120

```

```

DI=Z

```

```

0.30/0.100 0.40/0.100 0.50/0.130 0.90/0.150 1.00/0.250 1.60/0.250
1.90/0.600 3.50/0.600 3.75/0.700 4.40/0.700 4.50/0.800 6.25/1.500
8.50/1.500 12.50/0.500 20.00/0.350 30.00/0.350 36.00/0.350

```

*

**** MATERIAL PROPERTIES ****

```

* ASTM A-106 Grade B, PIPE *

```

```

MATH CD=106 EX=0 TY=1 *C-Si
MATD TE=70 EH=29.5 EX=0.0 SM=20.0 SY=35
MATD TE=100 EH=29.3 EX=0.20 SM=20.0 SY=35
MATD TE=200 EH=28.8 EX=1.00 SM=20.0 SY=32.1
MATD TE=300 EH=28.3 EX=1.90 SM=20.0 SY=31
MATD TE=400 EH=27.7 EX=2.80 SM=20.0 SY=29.9
MATD TE=500 EH=27.3 EX=3.70 SM=18.9 SY=28.5
MATD TE=600 EH=26.7 EX=4.70 SM=17.3 SY=26.8

```

```

* ASME SA-376 Grade TP316, PIPE *

```

```

MATH CD=376.316 EX=0 TY=4 *16Cr-12Ni-2Mo

```

MATD TE=70	EH=28.3	EX=0.0	SM=20.0	SY=30.0
MATD TE=100	EH=28.1	EX=0.30	SM=20.0	SY=30.0
MATD TE=200	EH=27.6	EX=1.40	SM=20.0	SY=25.9
MATD TE=300	EH=27.0	EX=2.50	SM=20.0	SY=23.4
MATD TE=400	EH=26.5	EX=3.70	SM=19.3	SY=21.4
MATD TE=500	EH=25.8	EX=5.00	SM=18.0	SY=20.0
MATD TE=600	EH=25.3	EX=6.30	SM=17.0	SY=18.9

* ASME SA-403 Grade WP316, ELBOWS *

MATH CD=403.316	EX=0	TY=4	*16Cr-12Ni-2Mo	
MATD TE=70	EH=28.3	EX=0.0	SM=20.0	SY=30.0
MATD TE=100	EH=28.1	EX=0.30	SM=20.0	SY=30.0
MATD TE=200	EH=27.6	EX=1.40	SM=20.0	SY=25.9
MATD TE=300	EH=27.0	EX=2.50	SM=20.0	SY=23.4
MATD TE=400	EH=26.5	EX=3.70	SM=18.7	SY=21.4
MATD TE=500	EH=25.8	EX=5.00	SM=17.5	SY=20.0
MATD TE=600	EH=25.3	EX=6.30	SM=16.4	SY=18.9

*** Cross Sectional Properties

CROS CD=1	OD=50.0	WT=8.87	MA=3977.2	*CALC. PER GE SPEC. NO. 23A5569 [3]
	SO=1	ST=1.0		*RECIRCULATION OUTLET NOZZLE
CROS CD=2	OD=37.85	WT=6.1	MA=2122.2	*CALC. PER GE SPEC. NO. 23A5569 [3]
	SO=1	ST=1.0		
CROS CD=3	OD=28.875	WT=1.56	MA=484.9	*CALC. PER GE SPEC. NO. 23A5569 [3]
	SO=1	ST=1.0		
CROS CD=4	OD=28.638	WT=1.45	MA=450.4	*CALC. PER GE SPEC. NO. 23A5569 [3]
	SO=1	ST=1.0		
CROS CD=5	OD=28.169	WT=1.244	MA=386.1	*CALC. PER GE SPEC. NO. 23A5569 [3]
	SO=1	ST=1.0		
CROS CD=7	OD=28.166	WT=2.125	MA=0.001	*VALVE
	SO=1	ST=1.0	KL=1	
CROS CD=8	OD=42.507	WT=2.486	MA=0.001	*PUMP
	SO=0.001	ST=0.001	KL=1	
CROS CD=11	OD=6.625	WT=0.432	MA=0.001	*PUMP RIGID STRUTS
	SO=0.001	ST=0.001	KL=1	
CROS CD=13	OD=28.339	WT=1.339	MA=415.1	*CALC. PER GE SPEC. NO. 23A5569 [3]
	SO=1	ST=1		
CROS CD=14	OD=28.339	WT=2.67	MA=0.001	*VALVE
	SO=1	ST=1.0	KL=1	
CROS CD=15	OD=12.748	WT=0.685	MA=103.4	*CALC. PER GE SPEC. NO. 23A5569 [3]
	SO=1	ST=1.0		
CROS CD=16	OD=14.17	WT=1.395	MA=207.5	*CALC. PER GE SPEC. NO. 23A5569 [3]
	SO=1	ST=1.0		
CROS CD=17	OD=15.5	WT=2	MA=307.7	*CALC. PER GE SPEC. NO. 23A5569 [3]
	SO=1	ST=1.0		
CROS CD=18	OD=21.88	WT=4.06	MA=803.2	*CALC. PER GE SPEC. NO. 23A5569 [3]
	SO=1	ST=1.0		
CROS CD=19	OD=28.25	WT=7.25	MA=1673.1	*CALC. PER GE SPEC. NO. 23A5569 [3]
	SO=1	ST=1.0		
CROS CD=20	OD=21.878	WT=1.043	MA=257.2	*CALC. PER GE SPEC. NO. 23A5569 [3]
	SO=1	ST=1.0		
CROS CD=25	OD=20	WT=1.031	MA=221.9	*CALC. PER GE SPEC. NO. 23A5569 [3]
	SO=1	ST=1		
CROS CD=26	OD=20	WT=1.875	MA=0.001	*VALVE
	SO=1	ST=1	KL=1	
CROS CD=27	OD=4.5	WT=0.3385	MA=23.2	*CALC. PER GE SPEC. NO. 23A5569 [3]
	SO=1	ST=1	KL=1	*4 inch bypass line
CROS CD=28	OD=4.5	WT=0.67	MA=0.001	*VALVE V2-54A
	SO=1	ST=1	KL=1	
CROS CD=29	OD=24	WT=1.217	MA=316.5	*CALC. PER GE SPEC. NO. 23A5569 [3]
	SO=1	ST=1		
CROS CD=30	OD=24	WT=2.43	MA=0.001	*VALVE



SO=1 ST=1 KL=1
 CROS CD=40 OD=4.5 WT=0.3385 MA=0.001 *4 inch bypass STRUTS
 SO=0.001 ST=0.001 KL=1
 CROS CD=41 OD=2.875 WT=0.276 MA=0.001 *STRUT RDA1, RDA5, & VBA1
 SO=0.001 ST=0.001 KL=1
 CROS CD=42 OD=28.339 WT=1.339 MA=0.001 *RIGID FROM RECIRC ELBOW TO RDA1 STRUT
 SO=0.001 ST=0.001 KL=1

 * STRUCTURE AND LOADS

 DESN TE=575.0 PR=1250.0 *Reference 12 GE Design Requirements Rpt VY-05Q-227

*-----
 *BEGIN REGION 1 TRANSIENT CARDS & GEOMETRY FROM RHR SUPPLY TO TEE
 *-----

*
 INCL FN=Z:\SISJ-PROJECTS\VY-16Q\Rev0\REG1.INP
 *

* RUN 1 FROM ANCHOR TO REACTOR VESSEL N3B
 *GROUP 1 FROM ANCHOR TO REACTOR VESSEL N3B
 *NOTE
 *NOTE NODE 003 - RECIRC SUCTION NOZZLE N1A (EL. 279'5 INCH)
 *NOTE NODE 003 IS AT THE SAFE END TO VESSEL NOZZLE CONNECTION
 *NOTE
 *NOTE SAFE END FROM NODES 003 TO 808
 *NOTE CONNECTION TO VESSEL AT NODE 003
 *NOTE OD AND WALL THICKNESS FOR SAFE END TAKEN FROM GE CALC
 *NOTE WEIGHT FOR SAFE END BASED ON THICKNESS
 *NOTE

MATL CD=376.316

CROS CD=1

COOR PT=3 AX=0 AY=0 AZ=0

ANCH PT=3

AMVT	CA=1	PT=3	DX=0.0000	DY=0.0176	DZ=-0.0201
AMVT	CA=2	PT=3	DX=0.0000	DY=0.3141	DZ=-0.3602
AMVT	CA=3	PT=3	DX=0.0000	DY=0.3112	DZ=-0.3568
AMVT	CA=4	PT=3	DX=0.0000	DY=0.2995	DZ=-0.3434
AMVT	CA=5	PT=3	DX=0.0000	DY=0.3112	DZ=-0.3568
AMVT	CA=6	PT=3	DX=0.0000	DY=0.2995	DZ=-0.3434
AMVT	CA=7	PT=3	DX=0.0000	DY=0.2922	DZ=-0.3350
AMVT	CA=8	PT=3	DX=0.0000	DY=0.2995	DZ=-0.3434
AMVT	CA=9	PT=3	DX=0.0000	DY=0.1422	DZ=-0.1630
AMVT	CA=10	PT=3	DX=0.0000	DY=0.2807	DZ=-0.3218
AMVT	CA=11	PT=3	DX=0.0000	DY=0.1422	DZ=-0.1630
AMVT	CA=12	PT=3	DX=0.0000	DY=0.3141	DZ=-0.3602
AMVT	CA=13	PT=3	DX=0.0000	DY=0.3141	DZ=-0.3602
AMVT	CA=14	PT=3	DX=0.0000	DY=0.1928	DZ=-0.2521
AMVT	CA=15	PT=3	DX=0.0000	DY=0.1624	DZ=-0.1986
AMVT	CA=16	PT=3	DX=0.0000	DY=0.0946	DZ=-0.1084
AMVT	CA=17	PT=3	DX=0.0000	DY=0.0176	DZ=-0.0201
AMVT	CA=18	PT=3	DX=0.0000	DY=0.0176	DZ=-0.0201
AMVT	CA=19	PT=3	DX=0.0000	DY=0.0946	DZ=-0.1084
AMVT	CA=20	PT=3	DX=0.0000	DY=0.0946	DZ=-0.1084
AMVT	CA=21	PT=3	DX=0.0000	DY=0.0361	DZ=-0.0413
AMVT	CA=22	PT=3	DX=0.0000	DY=0.2995	DZ=-0.3434
AMVT	CA=23	PT=3	DX=0.0000	DY=0.1928	DZ=-0.2521
AMVT	CA=24	PT=3	DX=0.0000	DY=0.0176	DZ=-0.0201

*
TANG PT=805 DZ=-1.017 EW=1
CROS CD=2
TANG PT=806 DZ=-0.823 EW=1
CROS CD=3
TANG PT=807 DZ=-0.58 EW=1
CROS CD=4
TANG PT=808 DZ=-0.47
CROS CD=5
TANG PT=5 DZ=-5.59 EW=1
MATL CD=403.316
BRAD PT=7 RA=3.5 EW=1
MATL CD=376.316
TANG PT=9 DY=-6.69 EW=1
TANG PT=500 DY=-2.31
*-----
*END REGION 1 GEOMETRY FROM RHR SUPPLY TO TEE
*-----
*-----
*BEGIN REGION 2 TRANSIENT CARDS & GEOMETRY FROM RHR SUPPLY TEE TO PUMP
*-----
*GROUP 2 RHR SUPPLY TEE TO PUMP
INCL FN=Z:\SISJ-PROJECTS\VY-16Q\Rev0\REG2.INP
*
TANG PT=11 DY=-2.22 EW=1
CROS CD=5
TANG PT=12 DY=-1.78
TANG PT=20 DY=-6.77
TANG PT=22 DY=-3.25
TANG PT=25 DY=-15.49 EW=1
MATL CD=403.316
BRAD PT=26 RA=3.5 EW=1
MATL CD=376.316
TANG PT=27 DX=-3.3 DZ=1.27 EW=1
CROS CD=7
VALV PT=30 DX=-2.28 DZ=0.89 MA=10.368 PL=1
JUNC PT=30
VALV PT=40 DX=-2.31 DZ=0.9 PL=2 EW=1
JUNC PT=30
RIGD PT=35 DY=7
LUMP PT=35 MA=1.132
JUNC PT=40
CROS CD=5
TANG PT=42 DX=-1.18 DZ=0.46
TANG PT=43 DX=-0.55 DZ=0.21
TANG PT=44 DX=-3.31 DZ=1.28 EW=1
MATL CD=403.316
BRAD PT=46 RA=2.33 EW=1
MATL CD=376.316
CROS CD=8
TANG PT=50 DY=4.33 EW=0
LUMP PT=50 MA=28 *NOTE WEIGHT OF PUMP FLOODED 28K (EXCLUDING MOTOR)
TANG PT=75 DY=0.5
TANG PT=83 DY=2.13
TANG PT=86 DY=3.38
LUMP PT=86 MA=32 *NOTE TOTAL WEIGHT OF PUMP MOTOR 32000 LBS
TANG PT=90 DY=4.08 *TOP OF PUMP
*NOTE SNUBBERS ON TOP OF PUMPS WERE DELETED DURING
*NOTE THE RECIRC PIPE REPLACEMENT PROJECT



*NOTE - RIGID LINKS FOR CONSTANT SUPPORTS AT PUMP FOLLOW

*-----
*END REGION 2 GEOMETRY FROM RHR SUPPLY TEE TO PUMP
*-----

*-----
*BEGIN REGION 3 TRANSIENT CARDS & GEOMETRY FROM PUMP DISCHARGE TO HEADER
*-----

*GROUP 3 FROM PUMP DISCHARGE TO HEADER

INCL FN=Z:\SISJ-PROJECTS\VY-16Q\Rev0\REG3.INP

*
JUNC PT=50.

CROS CD=8

RIGD PT=54 DX=1.06 DZ=1.06

RIGD PT=56 DX=1.06 DY=0.75 DZ=1.06 *NOTE CONSTANT SUPPORT HA3 AT NODE 56

JUNC PT=50

RIGD PT=66 DZ=-3.83

RIGD PT=69 DY=1

*NOTE CONSTANT SUPPORT HA4 AT NODE 69

JUNC PT=50

CROS CD=8

RIGD PT=60 DX=-3.83

RIGD PT=63 DY=1

*CONSTANT SUPPORT HA5 AT NODE 63

*

* *** CODING FOR PUMP RIGID STRUTS FOLLOW ***

* CODED FROM PUMP CENTERLINE

CROS CD=11

JUNC PT=66

RIGD PT=15 DY=0.7071 DZ=-0.7071

*

JUNC PT=60

RIGD PT=16 DX=-0.7071 DY=0.7071

* *** END OF CODING FOR PUMP SUPPORTS ***

*PUMP INLET

CROS CD=8

JUNC PT=50

TANG PT=150 DX=-2.17

BRAN PT=151 DZ=2.333 TE=1

*NOTE PUMP DISCHARGE CONNECTION TO PIPE AT NODE 151.

CROS CD=13

TANG PT=152 DZ=1.25

TANG PT=155 DZ=1 EW=1

CROS CD=14

VALV PT=160 PL=1 DX=0.0 DY=0.0 DZ=2.52 MA=6.8285

JUNC PT=160

RIGD PT=163 DX=0.0 DY=7.12 DZ=0.0

LUMP PT=163 MA=0.9715

JUNC PT=160

VALV PT=170 PL=2 DX=0.0 DY=0.0 DZ=6.18 EW=1

CROS CD=13

MATL CD=403.316

BRAD PT=175 RA=3.5 EW=1

MATL CD=376.316

TANG PT=176 DY=5.95

TANG PT=177 DY=4.42

*NOTE ***WEIGHT OF FLOW ELEMENT NOT INCLUDED***

*NOTE ***REF. DWG. 5920-6800 FOR DIMENSIONS***

TANG PT=184 DY=4.42

TANG PT=186 DY=3.02

TANG PT=188 DY=1.51

TANG PT=189 DY=0.74



AMVT	CA=10	PT=336	DX=-0.3218	DY=0.1626	DZ=0.0000
AMVT	CA=11	PT=336	DX=-0.1630	DY=0.0246	DZ=0.0000
AMVT	CA=12	PT=336	DX=-0.3602	DY=0.4398	DZ=0.0000
AMVT	CA=13	PT=336	DX=-0.3602	DY=0.4316	DZ=0.0000
AMVT	CA=14	PT=336	DX=-0.2193	DY=0.4152	DZ=0.0000
AMVT	CA=15	PT=336	DX=-0.1862	DY=0.4050	DZ=0.0000
AMVT	CA=16	PT=336	DX=-0.1084	DY=0.2940	DZ=0.0000
AMVT	CA=17	PT=336	DX=-0.0201	DY=0.3229	DZ=0.0000
AMVT	CA=18	PT=336	DX=-0.0201	DY=0.2700	DZ=0.0000
AMVT	CA=19	PT=336	DX=-0.1084	DY=0.1991	DZ=0.0000
AMVT	CA=20	PT=336	DX=-0.0201	DY=0.1626	DZ=0.0000
AMVT	CA=21	PT=336	DX=-0.0413	DY=0.3229	DZ=0.0000
AMVT	CA=22	PT=336	DX=-0.3434	DY=0.2700	DZ=0.0000
AMVT	CA=23	PT=336	DX=-0.2211	DY=0.1991	DZ=0.0000
AMVT	CA=24	PT=336	DX=-0.0201	DY=0.1626	DZ=0.0000

*NOTE SAFE END FROM NODES 838 TO 336
*NOTE CONNECTION TO VESSEL AT NODE 336
*NOTE OD AND WALL THICKNESS FOR SAFE END TAKEN FROM GE CALC
*NOTE WEIGHT BASED ON THICKNESS

*-----
*END REGION 6 GEOMETRY TO NOZZLE NODE 336
*-----
*

*-----
*BEGIN REGION 4 TRANSIENT CARDS & GEOMETRY HEADER TO NOZZLE NODE 366
*-----
*

*GROUP 4 HEADER TO NOZZLE NODE 366
INCL FN=Z:\SISJ-PROJECTS\VY-16Q\Rev0\REG4.INP
*

JUNC PT=210
CROS CD=20
BRAN PT=240 DX=0.1786 DY=0.0 DZ=1.7
TANG PT=250 DX=0.3 DZ=2.853 EW=0
BRAD PT=255 RA=4.578 EW=0 *NOTE BEND RADIUS IS 4.578 FEET
TANG PT=340 DX=1.799 DZ=3.108
*-----
*

*END REGION 4 GEOMETRY HEADER TO NOZZLE NODE 366
*-----
*

*-----
*BEGIN REGION 5 TRANSIENT CARDS & GEOMETRY RISER TO NOZZLE NODE 366
*-----
*

*GROUP 5 RISER TO NOZZLE NODE 366
INCL FN=Z:\SISJ-PROJECTS\VY-16Q\Rev0\REG5.INP
*

TANG PT=349 DX=0.71 DZ=1.23 EW=0
CRED PT=347 DX=0.75 DZ=1.3 AN=30
CROS CD=15
TANG PT=343 DX=0.5525 DZ=0.957 EW=1
BRAD PT=410 RA=1.5 EW=1
TANG PT=360 DX=3.483 DZ=2.011 EW=1
MATL CD=403.316
BRAD PT=361 RA=1.5 EW=1
MATL CD=376.316
CROS CD=15
TANG PT=362 DY=3.18
TANG PT=364 DY=8.56 EW=1
MATL CD=403.316

BRAD PT=365 RA=1.5 EW=1

*END REGION 5 GEOMETRY RISER TO NOZZLE NODE 366

*BEGIN REGION 6 TRANSIENT CARDS & GEOMETRY TO NOZZLE NODE 366

*GROUP 6 TO NOZZLE NODE 366

INCL FN=Z:\SISJ-PROJECTS\VY-16Q\Rev0\REG6.INP

*

MATL CD=376.316

TANG PT=868 DX=1.8 DZ=-3.1

CROS CD=16

TANG PT=867 DX=0.4375 DZ=-0.76 EW=1

CROS CD=17

TANG PT=866 DX=0.185 DZ=-0.32 EW=1

CROS CD=18

TANG PT=865 DX=0.265 DZ=-0.46 EW=1

CROS CD=19

TANG PT=366 DX=0.352 DZ=-0.61 EW=1

NOZZ PT=366

AMVT	CA=1	PT=366	DX=-0.0101	DY=0.0246	DZ=0.0174
AMVT	CA=2	PT=366	DX=-0.1800	DY=0.4398	DZ=0.3120
AMVT	CA=3	PT=366	DX=-0.1783	DY=0.4357	DZ=0.3091
AMVT	CA=4	PT=366	DX=-0.1716	DY=0.4193	DZ=0.2974
AMVT	CA=5	PT=366	DX=-0.1783	DY=0.4357	DZ=0.3091
AMVT	CA=6	PT=366	DX=-0.1716	DY=0.4193	DZ=0.2974
AMVT	CA=7	PT=366	DX=-0.1674	DY=0.4091	DZ=0.2902
AMVT	CA=8	PT=366	DX=-0.1716	DY=0.4193	DZ=0.2974
AMVT	CA=9	PT=366	DX=-0.0815	DY=0.1991	DZ=0.1412
AMVT	CA=10	PT=366	DX=-0.1609	DY=0.3930	DZ=0.2788
AMVT	CA=11	PT=366	DX=-0.0815	DY=0.1991	DZ=0.1412
AMVT	CA=12	PT=366	DX=-0.1800	DY=0.4398	DZ=0.3120
AMVT	CA=13	PT=366	DX=-0.1800	DY=0.4398	DZ=0.3120
AMVT	CA=14	PT=366	DX=-0.1097	DY=0.2678	DZ=0.1899
AMVT	CA=15	PT=366	DX=-0.0931	DY=0.2275	DZ=0.1613
AMVT	CA=16	PT=366	DX=-0.0542	DY=0.1324	DZ=0.0939
AMVT	CA=17	PT=366	DX=-0.0101	DY=0.0246	DZ=0.0174
AMVT	CA=18	PT=366	DX=-0.0101	DY=0.0246	DZ=0.0174
AMVT	CA=19	PT=366	DX=-0.0542	DY=0.1324	DZ=0.0939
AMVT	CA=20	PT=366	DX=-0.0101	DY=0.0246	DZ=0.0174
AMVT	CA=21	PT=366	DX=-0.0207	DY=0.0505	DZ=0.0358
AMVT	CA=22	PT=366	DX=-0.1716	DY=0.4193	DZ=0.2974
AMVT	CA=23	PT=366	DX=-0.1105	DY=0.2700	DZ=0.1915
AMVT	CA=24	PT=366	DX=-0.0101	DY=0.0246	DZ=0.0174

*END REGION 6 GEOMETRY TO NOZZLE NODE 366

*BEGIN REGION 4 TRANSIENT CARDS & GEOMETRY HEADER TO NOZZLES NODE 326 & 316

*GROUP 4 HEADER TO NOZZLES NODE 326 & 316

INCL FN=Z:\SISJ-PROJECTS\VY-16Q\Rev0\REG4.INP

*

JUNC PT=210

CROS CD=20

BRAN PT=260 DX=0.1786 DY=0.0 DZ=-1.7 TE=2

TANG PT=270 DX=0.3 DZ=-2.853 EW=0
 BRAD PT=275 RA=4.578 EW=0
 TANG PT=320 DX=1.799 DZ=-3.108

*-----
 *END REGION 4 GEOMETRY HEADER TO NOZZLES NODE 326 & 316
 *-----

*-----
 *BEGIN REGION 5 TRANSIENT CARDS & GEOMETRY RISER TO NOZZLE NODE 316
 *-----

*GROUP 5 RISER TO NOZZLE NODE 316
 INCL FN=Z:\SISJ-PROJECTS\VY-16Q\Rev0\REG5.INP

*
 TANG PT=319 DX=0.71 DZ=-1.23 EW=1
 CRED PT=317 DX=0.75 DZ=-1.3 AN=30
 CROS CD=15
 TANG PT=313 DX=0.5525 DZ=-0.957 EW=1
 BRAD PT=400 RA=1.5 EW=1
 TANG PT=310 DX=3.483 DZ=-2.011 EW=1
 MATL CD=403.316
 BRAD PT=311 RA=1.5 EW=1
 MATL CD=376.316
 CROS CD=15
 TANG PT=312 DY=4.74
 TANG PT=314 DY=6.99 EW=1
 MATL CD=403.316
 BRAD PT=315 RA=1.5 EW=1

*-----
 *END REGION 5 GEOMETRY RISER TO NOZZLE NODE 316
 *-----

*-----
 *BEGIN REGION 6 TRANSIENT CARDS & GEOMETRY TO NOZZLE NODE 316
 *-----

*GROUP 6 TO NOZZLE NODE 316
 INCL FN=Z:\SISJ-PROJECTS\VY-16Q\Rev0\REG6.INP

*
 MATL CD=376.316
 TANG PT=818 DX=1.84 DZ=3.19
 CROS CD=16
 TANG PT=817 DX=0.4375 DZ=0.76 EW=1
 CROS CD=17
 TANG PT=816 DX=0.185 DZ=0.32 EW=1
 CROS CD=18
 TANG PT=815 DX=0.265 DZ=0.46 EW=1
 CROS CD=19
 TANG PT=316 DX=0.352 DZ=0.61 EW=1
 NOZZ PT=316

AMVT	CA=1	PT=316	DX=-0.0101	DY=0.0246	DZ=-0.0174
AMVT	CA=2	PT=316	DX=-0.1800	DY=0.4398	DZ=-0.3120
AMVT	CA=3	PT=316	DX=-0.1783	DY=0.4357	DZ=-0.3091
AMVT	CA=4	PT=316	DX=-0.1716	DY=0.4193	DZ=-0.2974
AMVT	CA=5	PT=316	DX=-0.1783	DY=0.4357	DZ=-0.3091
AMVT	CA=6	PT=316	DX=-0.1716	DY=0.4193	DZ=-0.2974
AMVT	CA=7	PT=316	DX=-0.1674	DY=0.4091	DZ=-0.2902
AMVT	CA=8	PT=316	DX=-0.1716	DY=0.4193	DZ=-0.2974
AMVT	CA=9	PT=316	DX=-0.0815	DY=0.1991	DZ=-0.1412
AMVT	CA=10	PT=316	DX=-0.1609	DY=0.3930	DZ=-0.2788
AMVT	CA=11	PT=316	DX=-0.0815	DY=0.1991	DZ=-0.1412
AMVT	CA=12	PT=316	DX=-0.1800	DY=0.4398	DZ=-0.3120



```

AMVT CA=13 PT=316 DX=-0.1800 DY=0.4398 DZ=-0.3120
AMVT CA=14 PT=316 DX=-0.1097 DY=0.2678 DZ=-0.1899
AMVT CA=15 PT=316 DX=-0.0931 DY=0.2275 DZ=-0.1613
AMVT CA=16 PT=316 DX=-0.0542 DY=0.1324 DZ=-0.0939
AMVT CA=17 PT=316 DX=-0.0101 DY=0.0246 DZ=-0.0174
AMVT CA=18 PT=316 DX=-0.0101 DY=0.0246 DZ=-0.0174
AMVT CA=19 PT=316 DX=-0.0542 DY=0.1324 DZ=-0.0939
AMVT CA=20 PT=316 DX=-0.0101 DY=0.0246 DZ=-0.0174
AMVT CA=21 PT=316 DX=-0.0207 DY=0.0505 DZ=-0.0358
AMVT CA=22 PT=316 DX=-0.1716 DY=0.4193 DZ=-0.2974
AMVT CA=23 PT=316 DX=-0.1105 DY=0.2700 DZ=-0.1915
AMVT CA=24 PT=316 DX=-0.0101 DY=0.0246 DZ=-0.0174

```

*-----
*END REGION 6 GEOMETRY TO NOZZLE NODE 316
*-----

*-----
*BEGIN REGION 5 TRANSIENT CARDS & GEOMETRY RISER TO NOZZLE NODE 346
*-----

*GROUP 5 RISER TO NOZZLE NODE 346
INCL FN=Z:\SISJ-PROJECTS\VY-16Q\Rev0\REG5.INP

```

*
JUNC PT=340
CROS CD=15
BRAN PT=342 DY=1.36 TE=2
TANG PT=344 DY=10.39 EW=0
MATL CD=403.316
BRAD PT=345 RA=1.5 EW=1

```

*-----
*END REGION 5 GEOMETRY RISER TO NOZZLE NODE 346
*-----

*-----
*BEGIN REGION 6 TRANSIENT CARDS & GEOMETRY TO NOZZLE NODE 346
*-----

*GROUP 6 TO NOZZLE NODE 346
INCL FN=Z:\SISJ-PROJECTS\VY-16Q\Rev0\REG6.INP

```

*
MATL CD=376.316
TANG PT=848 DX=3.17 DZ=-1.83
CROS CD=16
TANG PT=847 DX=0.758 DZ=-0.4375 EW=1
CROS CD=17
TANG PT=846 DX=0.32 DZ=-0.185 EW=1
CROS CD=18
TANG PT=845 DX=0.46 DZ=-0.265 EW=1
CROS CD=19
TANG PT=346 DX=0.61 DZ=-0.352 EW=1
NOZZ PT=346

```

```

AMVT CA=1 PT=346 DX=-0.0174 DY=0.0246 DZ=0.0101
AMVT CA=2 PT=346 DX=-0.3120 DY=0.4398 DZ=0.1800
AMVT CA=3 PT=346 DX=-0.3091 DY=0.4357 DZ=0.1783
AMVT CA=4 PT=346 DX=-0.2974 DY=0.4193 DZ=0.1716
AMVT CA=5 PT=346 DX=-0.3091 DY=0.4357 DZ=0.1783
AMVT CA=6 PT=346 DX=-0.2974 DY=0.4193 DZ=0.1716
AMVT CA=7 PT=346 DX=-0.2902 DY=0.4091 DZ=0.1674
AMVT CA=8 PT=346 DX=-0.2974 DY=0.4193 DZ=0.1716
AMVT CA=9 PT=346 DX=-0.1412 DY=0.1991 DZ=0.0815
AMVT CA=10 PT=346 DX=-0.2788 DY=0.3930 DZ=0.1609
AMVT CA=11 PT=346 DX=-0.1412 DY=0.1991 DZ=0.0815

```



```

AMVT CA=12 PT=346 DX=-0.3120 DY=0.4398 DZ=0.1800
AMVT CA=13 PT=346 DX=-0.3120 DY=0.4398 DZ=0.1800
AMVT CA=14 PT=346 DX=-0.1899 DY=0.2678 DZ=0.1097
AMVT CA=15 PT=346 DX=-0.1613 DY=0.2275 DZ=0.0931
AMVT CA=16 PT=346 DX=-0.0939 DY=0.1324 DZ=0.0542
AMVT CA=17 PT=346 DX=-0.0174 DY=0.0246 DZ=0.0101
AMVT CA=18 PT=346 DX=-0.0174 DY=0.0246 DZ=0.0101
AMVT CA=19 PT=346 DX=-0.0939 DY=0.1324 DZ=0.0542
AMVT CA=20 PT=346 DX=-0.0174 DY=0.0246 DZ=0.0101
AMVT CA=21 PT=346 DX=-0.0358 DY=0.0505 DZ=0.0207
AMVT CA=22 PT=346 DX=-0.2974 DY=0.4193 DZ=0.1716
AMVT CA=23 PT=346 DX=-0.1915 DY=0.2700 DZ=0.1105
AMVT CA=24 PT=346 DX=-0.0174 DY=0.0246 DZ=0.0101

```

*-----
*END REGION 6 GEOMETRY TO NOZZLE NODE 346
*-----
*

*-----
*BEGIN REGION 5 TRANSIENT CARDS & GEOMETRY RISER TO NOZZLE NODE 326
*-----
*

```

*GROUP 5 RISER TO NOZZLE NODE 326
INCL FN=Z:\SISJ-PROJECTS\VY-16Q\Rev0\REG5.INP
*
JUNC PT=320
CROS CD=15
BRAN PT=322 DY=1.42 TE=2
TANG PT=324 DY=10.33 EW=1
MATL CD=403.316
BRAD PT=325 RA=1.5 EW=1

```

*-----
*END REGION 5 GEOMETRY RISER TO NOZZLE NODE 326
*-----
*

*-----
*BEGIN REGION 6 TRANSIENT CARDS & GEOMETRY TO NOZZLE NODE 326
*-----
*

```

*GROUP 6 TO NOZZLE NODE 326
INCL FN=Z:\SISJ-PROJECTS\VY-16Q\Rev0\REG6.INP
*
MATL CD=376.316
TANG PT=828 DX=3.18 DZ=1.84
CROS CD=16
TANG PT=827 DX=0.758 DZ=0.4375 EW=1
CROS CD=17
TANG PT=826 DX=0.32 DZ=0.185 EW=1
CROS CD=18
TANG PT=825 DX=0.46 DZ=0.265 EW=1
CROS CD=19
TANG PT=326 DX=0.61 DZ=0.352 EW=1
NOZZ PT=326

```

```

AMVT CA=1 PT=326 DX=-0.0174 DY=0.0246 DZ=-0.0101
AMVT CA=2 PT=326 DX=-0.3120 DY=0.4398 DZ=-0.1800
AMVT CA=3 PT=326 DX=-0.3091 DY=0.4357 DZ=-0.1783
AMVT CA=4 PT=326 DX=-0.2974 DY=0.4193 DZ=-0.1716
AMVT CA=5 PT=326 DX=-0.3091 DY=0.4357 DZ=-0.1783
AMVT CA=6 PT=326 DX=-0.2974 DY=0.4193 DZ=-0.1716
AMVT CA=7 PT=326 DX=-0.2902 DY=0.4091 DZ=-0.1674
AMVT CA=8 PT=326 DX=-0.2974 DY=0.4193 DZ=-0.1716
AMVT CA=9 PT=326 DX=-0.1412 DY=0.1991 DZ=-0.0815
AMVT CA=10 PT=326 DX=-0.2788 DY=0.3930 DZ=-0.1609

```

```

AMVT  CA=11  PT=326  DX=-0.1412  DY=0.1991  DZ=-0.0815
AMVT  CA=12  PT=326  DX=-0.3120  DY=0.4398  DZ=-0.1800
AMVT  CA=13  PT=326  DX=-0.3120  DY=0.4398  DZ=-0.1800
AMVT  CA=14  PT=326  DX=-0.1899  DY=0.2678  DZ=-0.1097
AMVT  CA=15  PT=326  DX=-0.1613  DY=0.2275  DZ=-0.0931
AMVT  CA=16  PT=326  DX=-0.0939  DY=0.1324  DZ=-0.0542
AMVT  CA=17  PT=326  DX=-0.0174  DY=0.0246  DZ=-0.0101
AMVT  CA=18  PT=326  DX=-0.0174  DY=0.0246  DZ=-0.0101
AMVT  CA=19  PT=326  DX=-0.0939  DY=0.1324  DZ=-0.0542
AMVT  CA=20  PT=326  DX=-0.0174  DY=0.0246  DZ=-0.0101
AMVT  CA=21  PT=326  DX=-0.0358  DY=0.0505  DZ=-0.0207
AMVT  CA=22  PT=326  DX=-0.2974  DY=0.4193  DZ=-0.1716
AMVT  CA=23  PT=326  DX=-0.1915  DY=0.2700  DZ=-0.1105
AMVT  CA=24  PT=326  DX=-0.0174  DY=0.0246  DZ=-0.0101

```

*-----
*END REGION 6 GEOMETRY TO NOZZLE NODE 326
*-----

*-----
*BEGIN REGION 7A TRANSIENT CARDS & GEOMETRY TO RHR SUPPLY VALVE NODE 550
*-----

*GROUP 7 TO RHR SUPPLY VALVE NODE 550
INCL FN=Z:\SISJ-PROJECTS\VY-16Q\Rev0\REG7A.INP
*

```

MATL CD=376.316
JUNC PT=500
CROS CD=25
BRAN PT=502 DX=1.67  EW=0  TE=1
TANG PT=506 DX=2.53  EW=0
MATL CD=403.316
BRAD PT=507 RA=1.67  EW=1
MATL CD=376.316
TANG PT=508 DZ=-4.01
TANG PT=515 DZ=-4.53  EW=1
MATL CD=403.316
BRAD PT=520 RA=1.67  EW=1
MATL CD=376.316
CROS CD=26
VALV PT=525 DX=-3.34  PL=1
JUNC PT=525
VALV PT=530 DX=-1.99  PL=2  EW=1
JUNC PT=525
RIGD PT=526 DY=2.5
LUMP PT=526 MA=7.569
JUNC PT=530
CROS CD=25
TANG PT=540 DX=-1.13  EW=1
CROS CD=26
VALV PT=545 DX=-1.97  PL=1
JUNC PT=545
RIGD PT=547 DY=2.5
LUMP PT=547 MA=7.355
JUNC PT=545
VALV PT=550 DX=-1.98  PL=2  EW=1

```

*-----
*END REGION 7A GEOMETRY TO RHR SUPPLY VALVE NODE 550
*-----

*-----
*BEGIN REGION 7B TRANSIENT CARDS & GEOMETRY FROM RHR SUPPLY VALVE TO PENET. NODE 565
*-----



*-----
*GROUP 17 FROM RHR SUPPLY VALVE TO PENET. NODE 565
INCL FN=Z:\SISJ-PROJECTS\VY-16Q\Rev0\REG7B.INP
*

CROS CD=25
MATL CD=106
TANG PT=555 DX=-3.36 EW=1
BRAD PT=556 RA=1.67 EW=1
TANG PT=560 DY=-10.17 EW=1
BRAD PT=561 RA=1.67 EW=1
TANG PT=563 DZ=-6.92
TANG PT=565 DZ=-6.92

*-----
*END REGION 7B GEOMETRY FROM RHR SUPPLY VALVE TO PENET. NODE 565
*-----
*

*-----
*BEGIN REGION 8 TRANSIENT CARDS & GEOMETRY FOR 4 INCH BYPASS
*-----
*

*GROUP 8 4 INCH BYPASS
INCL FN=Z:\SISJ-PROJECTS\VY-16Q\Rev0\REG8.INP
*

*NOTE CODING FOR 4 INCH BYPASS STARTS HERE

JUNC PT=152
CROS CD=27
MATL CD=376.316
BRAN PT=700 DX=-1.19 TE=4
TANG PT=702 DX=-0.61
TANG PT=703 DX=-1.43 EW=0
MATL CD=403.316
BRAD PT=704 RA=0.5 EW=0
MATL CD=376.316
TANG PT=705 DZ=5.08
*NOTE CONSTANT SUPPORT HA11 AT NODE 705
TANG PT=721 DZ=1.12
TANG PT=706 DZ=2.47
TANG PT=707 DZ=1.03
TANG PT=708 DZ=0.34
TANG PT=709 DZ=0.38
JUNC PT=707
BRAN PT=710 DY=0.34 TE=1
CROS CD=28
VALV PT=712 DY=0.71 MA=0.3669 PL=1 *AL=\$VALVE V2-54AS
VALV PT=715 DZ=-3.5 MA=0.1831 PL=3
JUNC PT=712
VALV PT=714 DY=0.71 PL=2
CROS CD=27
TANG PT=723 DY=4.19
MATL CD=403.316
BRAD PT=716 RA=0.5
MATL CD=376.316
TANG PT=718 DX=1.48
TANG PT=720 DX=0.56
BRAN PT=176 DX=1.19 TE=4
*****CODING FOR STRUTS RDA5 AND VAB1 FOLLOW
JUNC PT=170
CROS CD=40 *OD=4.5 inch
RIGD PT=725 DP=0 DX=-0.583 DY=1.84 *AL=\$RDA5\$
CROS CD=41 *OD=2.875 inch
RIGD PT=715 DP=0 DX=-2.67 DY=-0.79



```
RIGD PT=721 DP=0 DY=-1.05 *AL=$VAB1$
*****CODING FOR RDA1 STRUT FOLLOWS
CROS CD=42 *OD=28.339 inch
JUNC PT=175
RIGD PT=173 DP=0 DY=-3.5 DZ=0.34
CROS CD=41 *OD=2.875 inch
RIGD PT=708 DP=0 DX=-3.21 *AL=$RDA1$
*-----
*END REGION 8 GEOMETRY FOR 4 INCH BYPASS
*-----
*
*-----
*BEGIN REGION 9A TRANSIENT CARDS & GEOMETRY FOR RHR RETURN FROM TEE TO VALVE NODE 660
*-----
*GROUP 9 RHR RETURN FROM TEE TO VALVE NODE 660
INCL FN=Z:\SISJ-PROJECTS\VY-16Q\Rev0\REG9A.INP
*
*NOTE CODING FOR RHR RETURN STARTS HERE
CROS CD=29
JUNC PT=600
MATL CD=376.316
BRAN PT=602 DX=-3.8123 TE=1
MATL CD=403.316
BRAD PT=610 RA=2 EW=1
TANP DY=4
BRAD PT=612 RA=2 EW=1
MATL CD=376.316
TANG PT=614 DZ=-10.38 EW=1
MATL CD=403.316
BRAD PT=615 RA=10 EW=1
MATL CD=376.316
TANG PT=620 DX=5.98 DZ=-3.45 EW=1
*NOTE
*NOTE VARIABLE SPRING H104 AT NODE 620
*NOTE
*NOTE VALVE V10-81A DATA FROM 5920-4590 WEIGHT - 6845.#
*NOTE WEIGHT APPLIED AT ESTIMATED CENTER OF GRAVITY (NODE 623)
CROS CD=30
VALV PT=622 DX=1.98 DZ=-1.15 PL=1 *AL=$VALVE V10-81A$
JUNC PT=622
VALV PT=624 DX=1.98 DZ=-1.15 PL=2 EW=1
JUNC PT=622
RIGD PT=623 DY=2.5
LUMP PT=623 MA=7.32 *VALVE ACTUATOR
CROS CD=29
JUNC PT=624
TANG PT=625 DX=1.867 DZ=-1.078
TANG PT=630 DX=2.598 DZ=-1.5 EW=1
MATL CD=403.316
BRAD PT=631 RA=3 EW=1
MATL CD=376.316
TANG PT=640 DZ=-4.54 EW=1
MATL CD=403.316
BRAD PT=641 RA=2 EW=1
MATL CD=376.316
*NOTE VALVE V10-46A DATA FROM 5920-4718 WEIGHT - 5295.#
CROS CD=30
VALV PT=655 DX=-3.79 PL=1 TA=2 *AL=$VALVE V10-46A$
LUMP PT=655 MA=5.77
```



*-----
 *END REGION 9A GEOMETRY FOR RHR RETURN FROM TEE TO VALVE NODE 660
 *-----
 *

*-----
 *BEGIN REGION 9B TRANSIENT CARDS & GEOMETRY FOR RHR RETURN FROM VALVE NODE 660 TO PENET. NODE 675
 *-----
 *

*GROUP 19 RHR RETURN FROM VALVE NODE 660 TO PENET. NODE 675
 INCL FN=Z:\SISJ-PROJECTS\VY-16Q\Rev0\REG9B.INP
 *

*NOTE
 *NOTE VARIABLE SPRING H105 AT NODE 655
 *NOTE

VALV PT=660 DX=-1.79 PL=2 EW=1
 *NOTE SPEC CHANGE TO CARBON STEEL
 MATL CD=106
 CROS CD=29
 TANG PT=661 DX=-1
 TANG PT=663 DX=-3.31 EW=1
 BRAD PT=665 RA=2 EW=1
 TANG PT=670 DY=-10.5 DZ=0.38 EW=1
 BRAD PT=671 RA=2 EW=1
 TANG PT=673 DZ=-7.74
 TANG PT=675 DZ=-7.74

*-----
 *END REGION 9B GEOMETRY FOR RHR RETURN FROM VALVE NODE 660 TO PENET. NODE 675
 *-----
 *

*-----
 ***STRESS INDICES AT CROSS POINT
 *-----
 *

INDI AT=210 AF=195 B1=0.5 C1=1 K1=4 B2=2.256 C2=3.024 K2=1 C3=1 K3=1 CP=0.5
 INDI AT=210 AF=215 B1=0.5 C1=1 K1=4 B2=2.256 C2=3.024 K2=1 C3=1 K3=1 CP=0.5
 INDI AT=210 AF=240 B1=0.5 C1=1 K1=4 B2=1.805 C2=3.024 K2=1 C3=1 K3=1 CP=0.5
 INDI AT=210 AF=260 B1=0.5 C1=1 K1=4 B2=1.805 C2=3.024 K2=1 C3=1 K3=1 CP=0.5

*-----
 *** SUPPORTS
 *-----
 *

RSTN PT=675 DX=1 SP=16000	*RHR SUPPLY PENET.
RSTN PT=675 DY=1 SP=16000	*RHR SUPPLY PENET.
RSTN PT=675 DZ=1 SP=23000	*RHR SUPPLY PENET.
ROTR PT=675 RX=1 SP=300000	*RHR SUPPLY PENET.
ROTR PT=675 RY=1 SP=300000	*RHR SUPPLY PENET.
ROTR PT=675 RZ=1 SP=340000	*RHR SUPPLY PENET.
RSTN PT=565 DX=1 SP=16000	*RHR SUPPLY PENET.
RSTN PT=565 DY=1 SP=16000	*RHR SUPPLY PENET.
RSTN PT=565 DZ=1 SP=23000	*RHR SUPPLY PENET.
ROTR PT=565 RX=1 SP=300000	*RHR SUPPLY PENET.
ROTR PT=565 RY=1 SP=300000	*RHR SUPPLY PENET.
ROTR PT=565 RZ=1 SP=340000	*RHR SUPPLY PENET.

*
 SNUB PT=12 DZ=-1 SP=1000 *AL=\$SNUBBER SS-7A-1\$
 SNUB PT=12 DX=1 SP=1000 *AL=\$SNUBBER SS-7A-2\$
 SNUB PT=190 DZ=-1 SP=1000 *AL=\$SNUBBER SS-6-A1\$
 SNUB PT=190 DX=1 SP=1000 *AL=\$SNUBBER SS-6-A2\$
 *

VSUP PT=20 DY=1 FO=24.8 SP=2.664 *AL=\$VARI. SUPT. HA-1\$
 *



```

CSUP PT=27   DY=1   FO=8.3   KP=0.01   *AL=$CONST. SUPT. H-8-A1$
CSUP PT=42   DY=1   FO=8.3   KP=0.01   *AL=$CONST. SUPT. H-8-A2$
CSUP PT=56   DY=1   FO=18.05  KP=0.01   *AL=$CONST. SUPT. HA3 FOR PUMP$
CSUP PT=69   DY=1   FO=18.0   KP=0.01   *AL=$CONST. SUPT. HA4 FOR PUMP$
CSUP PT=63   DY=1   FO=18.02  KP=0.01   *AL=$CONST. SUPT. HA5 FOR PUMP$
CSUP PT=160  DY=-1  FO=11.8   KP=0.01   *AL=$CONST. SUPT. HA-9 & HA-10$
CSUP PT=705  DY=1   FO=0.960  KP=0.01   *AL=$CONST. SUPT. HA-11 ON 4 INCH BYPASS$
*
VSUP PT=184  DY=1   FO=36.0   SP=3.542  *AL=$VARI. SUPT. HA-2$
VSUP PT=343  DY=1   FO=7.1    SP=3.014  *AL=$VARI. SUPT. HA13$
VSUP PT=313  DY=1   FO=7.1    SP=3.014  *AL=$VARI. SUPT. HA14$
*
VSUP PT=530  DY=1   SP=9.420  FO=26.0   *AL=$HANGER H109 RHR SUPPLY VALVES$
VSUP PT=620  DY=1   SP=7.084  FO=14.9   *AL=$HANGER H104 RHR RETURN VALVES$
VSUP PT=655  DY=1   SP=4.710  FO=22.0   *AL=$HANGER H105 RHR RETURN VALVES$
*
RSTN PT=15   DY=0.7071 DZ=-0.7071 SP=6000   *RECIRC PUMP
RSTN PT=16   DX=-0.7071 DY=0.7071 SP=6000   *RECIRC PUMP
ENDP

```

Reg1.inp

*BEGIN REGION 1 TRANSIENT CARDS & GEOMETRY FROM RHR SUPPLY TO TEE

```

*
OPER CA=1   TE=100   PR=1100
OPER CA=2   TE=100   PR=50
OPER CA=3   TE=549   PR=1010
OPER CA=4   TE=542   PR=1010
OPER CA=5   TE=526   PR=1010
OPER CA=6   TE=542   PR=1010
OPER CA=7   TE=526   PR=1010
OPER CA=8   TE=516   PR=1010
OPER CA=9   TE=526   PR=1010
OPER CA=10  TE=300   PR=1135
OPER CA=11  TE=500   PR=1135
OPER CA=12  TE=300   PR=675
OPER CA=13  TE=549   PR=1010
OPER CA=14  TE=549   PR=1010
OPER CA=15  TE=375   PR=170
OPER CA=16  TE=330   PR=90
OPER CA=17  TE=225   PR=0
OPER CA=18  TE=100   PR=0
OPER CA=19  TE=100   PR=1563
OPER CA=20  TE=225   PR=0
OPER CA=21  TE=225   PR=0
OPER CA=22  TE=130   PR=1010
OPER CA=23  TE=526   PR=1010
OPER CA=24  TE=375   PR=200
OPER CA=25  TE=100   PR=0

```

```

*
TRAN CA=201  IS=1  FS=1  IT=70  FT=100  TT=1800  FL=2262  IP=15  FP=1115  TP=0
TRAN CA=202  IS=1  FS=1  IT=100  FT=100  TT=1800  FL=2262  IP=1115  FP=65  TP=0
TRAN CA=203  IS=1  FS=1  IT=100  FT=549  TT=16164  FL=16158  IP=65  FP=1025  TP=0
TRAN CA=204  IS=1  FS=1  IT=549  FT=542  TT=0      FL=32316  IP=1025  FP=1025  TP=0
TRAN CA=205  IS=1  FS=1  IT=542  FT=526  TT=0      FL=32316  IP=1025  FP=1025  TP=0
TRAN CA=206  IS=1  FS=1  IT=526  FT=542  TT=900    FL=32316  IP=1025  FP=1025  TP=0
TRAN CA=207  IS=1  FS=1  IT=542  FT=526  TT=360    FL=32316  IP=1025  FP=1025  TP=0
TRAN CA=208  IS=1  FS=1  IT=526  FT=516  TT=0      FL=32316  IP=1025  FP=1025  TP=0
TRAN CA=209  IS=1  FS=1  IT=516  FT=526  TT=0      FL=32316  IP=1025  FP=1025  TP=0

```



```

TRAN  CA=210  IS=1 FS=1 IT=526 FT=300 TT=220  FL=600  IP=1205 FP=1150 TP=0
TRAN  CA=211  IS=1 FS=1 IT=300 FT=500 TT=1980 FL=600  IP=900  FP=1150 TP=0
TRAN  CA=212  IS=1 FS=1 IT=500 FT=300 TT=180  FL=600  IP=1150 FP=690 TP=0
TRAN  CA=213  IS=1 FS=1 IT=300 FT=549 TT=8964  FL=16158 IP=255  FP=1025 TP=0
TRAN  CA=214  IS=1 FS=1 IT=526 FT=549 TT=0      FL=32316 IP=1025 FP=1025 TP=0
TRAN  CA=215  IS=1 FS=1 IT=549 FT=375 TT=6264  FL=16158 IP=1025 FP=185 TP=0
TRAN  CA=216  IS=1 FS=1 IT=375 FT=330 TT=600    FL=16158 IP=185  FP=105 TP=0
TRAN  CA=217  IS=1 FS=1 IT=330 FT=225 TT=3780  FL=16158 IP=105  FP=15 TP=0
TRAN  CA=218  IS=1 FS=1 IT=225 FT=100 TT=4500  FL=22858 IP=15   FP=15 TP=0
TRAN  CA=219  IS=1 FS=1 IT=100 FT=100 TT=0      FL=2262  IP=40   FP=1578 TP=0
TRAN  CA=220
TRAN  CA=221
TRAN  CA=222  IS=1 FS=1 IT=526 FT=130 TT=0      FL=32316 IP=1025 FP=1025 TP=0
TRAN  CA=223  IS=1 FS=1 IT=130 FT=526 TT=0      FL=32316 IP=1025 FP=1025 TP=0
TRAN  CA=224  IS=1 FS=1 IT=526 FT=375 TT=600    FL=32316 IP=1025 FP=215 TP=0
TRAN  CA=225  IS=1 FS=1 IT=375 FT=100 TT=9900  FL=32316 IP=215  FP=15 TP=0

```

```

*
PAIR  CA=201  CO=8.3  DI=0.140  EX=8.5 *Tavg=85.0
PAIR  CA=202  CO=8.3  DI=0.140  EX=8.5 *Tavg=100.0
PAIR  CA=203  CO=9.4  DI=0.151  EX=8.5 *Tavg=324.5
PAIR  CA=204  CO=10.5 DI=0.162  EX=8.5 *Tavg=545.5
PAIR  CA=205  CO=10.4 DI=0.161  EX=8.5 *Tavg=534.0
PAIR  CA=206  CO=10.4 DI=0.161  EX=8.5 *Tavg=534.0
PAIR  CA=207  CO=10.4 DI=0.161  EX=8.5 *Tavg=534.0
PAIR  CA=208  CO=10.3 DI=0.161  EX=8.5 *Tavg=521.0
PAIR  CA=209  CO=10.3 DI=0.161  EX=8.5 *Tavg=521.0
PAIR  CA=210  CO=9.9  DI=0.156  EX=8.5 *Tavg=413.0
PAIR  CA=211  CO=9.8  DI=0.155  EX=8.5 *Tavg=400.0
PAIR  CA=212  CO=9.8  DI=0.155  EX=8.5 *Tavg=400.0
PAIR  CA=213  CO=9.9  DI=0.156  EX=8.5 *Tavg=424.5
PAIR  CA=214  CO=10.4 DI=0.162  EX=8.5 *Tavg=537.5
PAIR  CA=215  CO=10.0 DI=0.158  EX=8.5 *Tavg=462.0
PAIR  CA=216  CO=9.5  DI=0.152  EX=8.5 *Tavg=352.5
PAIR  CA=217  CO=9.2  DI=0.149  EX=8.5 *Tavg=277.5
PAIR  CA=218  CO=8.7  DI=0.143  EX=8.5 *Tavg=162.5
PAIR  CA=219  CO=8.3  DI=0.140  EX=8.5 *Tavg=100.0
*PAIR  CA=220  CO=9.0  DI=0.146  EX=8.5 *Tavg=225.0
*PAIR  CA=221  CO=9.0  DI=0.146  EX=8.5 *Tavg=225.0
PAIR  CA=222  CO=9.4  DI=0.151  EX=8.5 *Tavg=328.0
PAIR  CA=223  CO=9.4  DI=0.151  EX=8.5 *Tavg=328.0
PAIR  CA=224  CO=10.0 DI=0.157  EX=8.5 *Tavg=450.5
PAIR  CA=225  CO=9.0  DI=0.147  EX=8.5 *Tavg=237.5

```

Reg2.inp

*-----
 *BEGIN REGION 2 TRANSIENT CARDS & GEOMETRY FROM RHR SUPPLY TEE TO PUMP
 *-----

```

*
OPER  CA=1    TE=100  PR=1100
OPER  CA=2    TE=100  PR=50
OPER  CA=3    TE=549  PR=1010
OPER  CA=4    TE=542  PR=1010
OPER  CA=5    TE=526  PR=1010
OPER  CA=6    TE=542  PR=1010
OPER  CA=7    TE=526  PR=1010
OPER  CA=8    TE=516  PR=1010
OPER  CA=9    TE=526  PR=1010
OPER  CA=10   TE=300  PR=1135
OPER  CA=11   TE=500  PR=1135
OPER  CA=12   TE=300  PR=675

```

OPER CA=13 TE=549 PR=1010
 OPER CA=14 TE=549 PR=1010
 OPER CA=15 TE=375 PR=170
 OPER CA=16 TE=330 PR=90
 OPER CA=17 TE=225 PR=0
 OPER CA=18 TE=100 PR=0
 OPER CA=19 TE=100 PR=1563
 OPER CA=20 TE=225 PR=0
 OPER CA=21 TE=225 PR=0
 OPER CA=22 TE=130 PR=1010
 OPER CA=23 TE=526 PR=1010
 OPER CA=24 TE=375 PR=200
 OPER CA=25 TE=100 PR=0

*

TRAN	CA=201	IS=1	FS=1	IT=70	FT=100	TT=1800	FL=2262	IP=15	FP=1115	TP=0
TRAN	CA=202	IS=1	FS=1	IT=100	FT=100	TT=1800	FL=2262	IP=1115	FP=65	TP=0
TRAN	CA=203	IS=1	FS=1	IT=100	FT=549	TT=16164	FL=16158	IP=65	FP=1025	TP=0
TRAN	CA=204	IS=1	FS=1	IT=549	FT=542	TT=0	FL=32316	IP=1025	FP=1025	TP=0
TRAN	CA=205	IS=1	FS=1	IT=542	FT=526	TT=0	FL=32316	IP=1025	FP=1025	TP=0
TRAN	CA=206	IS=1	FS=1	IT=526	FT=542	TT=900	FL=32316	IP=1025	FP=1025	TP=0
TRAN	CA=207	IS=1	FS=1	IT=542	FT=526	TT=360	FL=32316	IP=1025	FP=1025	TP=0
TRAN	CA=208	IS=1	FS=1	IT=526	FT=516	TT=0	FL=32316	IP=1025	FP=1025	TP=0
TRAN	CA=209	IS=1	FS=1	IT=516	FT=526	TT=0	FL=32316	IP=1025	FP=1025	TP=0
TRAN	CA=210	IS=1	FS=1	IT=526	FT=300	TT=220	FL=600	IP=1205	FP=1150	TP=0
TRAN	CA=211	IS=1	FS=1	IT=300	FT=500	TT=1980	FL=600	IP=900	FP=1150	TP=0
TRAN	CA=212	IS=1	FS=1	IT=500	FT=300	TT=180	FL=600	IP=1150	FP=690	TP=0
TRAN	CA=213	IS=1	FS=1	IT=300	FT=549	TT=8964	FL=16158	IP=255	FP=1025	TP=0
TRAN	CA=214	IS=1	FS=1	IT=526	FT=549	TT=0	FL=32316	IP=1025	FP=1025	TP=0
TRAN	CA=215	IS=1	FS=1	IT=549	FT=375	TT=6264	FL=16158	IP=1025	FP=185	TP=0
TRAN	CA=216	IS=1	FS=1	IT=375	FT=330	TT=600	FL=16158	IP=185	FP=105	TP=0
TRAN	CA=217	IS=1	FS=1	IT=330	FT=225	TT=3780	FL=16158	IP=105	FP=15	TP=0
TRAN	CA=218	IS=1	FS=1	IT=225	FT=100	TT=4500	FL=16158	IP=15	FP=15	TP=0
TRAN	CA=219	IS=1	FS=1	IT=100	FT=100	TT=0	FL=2262	IP=40	FP=1578	TP=0
TRAN	CA=220									
TRAN	CA=221									
TRAN	CA=222	*IS=1	FS=1	IT=526	FT=130	TT=0	FL=32316	IP=1025	FP=1025	TP=0
TRAN	CA=223	IS=1	FS=1	IT=130	FT=526	TT=0	FL=32316	IP=1025	FP=1025	TP=0
TRAN	CA=224	IS=1	FS=1	IT=526	FT=375	TT=600	FL=32316	IP=1025	FP=215	TP=0
TRAN	CA=225	IS=1	FS=1	IT=375	FT=100	TT=9900	FL=32316	IP=215	FP=15	TP=0

*

PAIR	CA=201	CO=8.3	DI=0.140	EX=8.5	*Tavg=85.0
PAIR	CA=202	CO=8.3	DI=0.140	EX=8.5	*Tavg=100.0
PAIR	CA=203	CO=9.4	DI=0.151	EX=8.5	*Tavg=324.5
PAIR	CA=204	CO=10.5	DI=0.162	EX=8.5	*Tavg=545.5
PAIR	CA=205	CO=10.4	DI=0.161	EX=8.5	*Tavg=534.0
PAIR	CA=206	CO=10.4	DI=0.161	EX=8.5	*Tavg=534.0
PAIR	CA=207	CO=10.4	DI=0.161	EX=8.5	*Tavg=534.0
PAIR	CA=208	CO=10.3	DI=0.161	EX=8.5	*Tavg=521.0
PAIR	CA=209	CO=10.3	DI=0.161	EX=8.5	*Tavg=521.0
PAIR	CA=210	CO=9.9	DI=0.156	EX=8.5	*Tavg=413.0
PAIR	CA=211	CO=9.8	DI=0.155	EX=8.5	*Tavg=400.0
PAIR	CA=212	CO=9.8	DI=0.155	EX=8.5	*Tavg=400.0
PAIR	CA=213	CO=9.9	DI=0.156	EX=8.5	*Tavg=424.5
PAIR	CA=214	CO=10.4	DI=0.162	EX=8.5	*Tavg=537.5
PAIR	CA=215	CO=10.0	DI=0.158	EX=8.5	*Tavg=462.0
PAIR	CA=216	CO=9.5	DI=0.152	EX=8.5	*Tavg=352.5
PAIR	CA=217	CO=9.2	DI=0.149	EX=8.5	*Tavg=277.5
PAIR	CA=218	CO=8.7	DI=0.143	EX=8.5	*Tavg=162.5
PAIR	CA=219	CO=8.3	DI=0.140	EX=8.5	*Tavg=100.0
*PAIR	CA=220	CO=9.0	DI=0.146	EX=8.5	*Tavg=225.0

*PAIR CA=221 CO=9.0 DI=0.146 EX=8.5 *Tavg=225.0
 *PAIR CA=222 CO=9.4 DI=0.151 EX=8.5 *Tavg=328.0
 PAIR CA=223 CO=9.4 DI=0.151 EX=8.5 *Tavg=328.0
 PAIR CA=224 CO=10.0 DI=0.157 EX=8.5 *Tavg=450.5
 PAIR CA=225 CO=9.0 DI=0.147 EX=8.5 *Tavg=237.5

Reg3.inp

*-----
 *BEGIN REGION 3 TRANSIENT CARDS & GEOMETRY FROM PUMP DISCHARGE TO HEADER
 *-----

*
 OPER CA=1 TE=100 PR=1100
 OPER CA=2 TE=100 PR=50
 OPER CA=3 TE=549 PR=1035
 OPER CA=4 TE=542 PR=1035
 OPER CA=5 TE=526 PR=1035
 OPER CA=6 TE=542 PR=1035
 OPER CA=7 TE=526 PR=1035
 OPER CA=8 TE=516 PR=1035
 OPER CA=9 TE=526 PR=1035
 OPER CA=10 TE=300 PR=1160
 OPER CA=11 TE=500 PR=1160
 OPER CA=12 TE=300 PR=700
 OPER CA=13 TE=549 PR=1035
 OPER CA=14 TE=549 PR=1035
 OPER CA=15 TE=375 PR=195
 OPER CA=16 TE=330 PR=115
 OPER CA=17 TE=225 PR=25
 OPER CA=18 TE=100 PR=25
 OPER CA=19 TE=100 PR=1563
 OPER CA=20 TE=225 PR=25
 OPER CA=21 TE=225 PR=25
 OPER CA=22 TE=130 PR=1035
 OPER CA=23 TE=526 PR=1035
 OPER CA=24 TE=375 PR=225
 OPER CA=25 TE=100 PR=25

*
 TRAN CA=201 IS=1 FS=1 IT=70 FT=100 TT=1800 FL=2262 IP=15 FP=1115 TP=0
 TRAN CA=202 IS=1 FS=1 IT=70 FT=100 TT=1800 FL=2262 IP=1115 FP=65 TP=0
 TRAN CA=203 IS=1 FS=1 IT=100 FT=549 TT=16164 FL=16158 IP=65 FP=1050 TP=0
 TRAN CA=204 IS=1 FS=1 IT=549 FT=542 TT=0 FL=32316 IP=1050 FP=1050 TP=0
 TRAN CA=205 IS=1 FS=1 IT=542 FT=526 TT=0 FL=32316 IP=1050 FP=1050 TP=0
 TRAN CA=206 IS=1 FS=1 IT=526 FT=542 TT=900 FL=32316 IP=1050 FP=1050 TP=0
 TRAN CA=207 IS=1 FS=1 IT=542 FT=526 TT=360 FL=32316 IP=1050 FP=1050 TP=0
 TRAN CA=208 IS=1 FS=1 IT=526 FT=516 TT=0 FL=32316 IP=1050 FP=1050 TP=0
 TRAN CA=209 IS=1 FS=1 IT=516 FT=526 TT=0 FL=32316 IP=1050 FP=1050 TP=0
 TRAN CA=210 IS=1 FS=1 IT=526 FT=300 TT=220 FL=600 IP=1230 FP=1175 TP=0
 TRAN CA=211 IS=1 FS=1 IT=300 FT=500 TT=1980 FL=600 IP=925 FP=1175 TP=0
 TRAN CA=212 IS=1 FS=1 IT=500 FT=300 TT=180 FL=600 IP=1175 FP=715 TP=0
 TRAN CA=213 IS=1 FS=1 IT=300 FT=549 TT=8964 FL=16158 IP=280 FP=1050 TP=0
 TRAN CA=214 IS=1 FS=1 IT=526 FT=549 TT=0 FL=32316 IP=1050 FP=1050 TP=0
 TRAN CA=215 IS=1 FS=1 IT=549 FT=375 TT=6264 FL=16158 IP=1050 FP=210 TP=0
 TRAN CA=216 IS=1 FS=1 IT=375 FT=330 TT=600 FL=16158 IP=210 FP=130 TP=0
 TRAN CA=217 IS=1 FS=1 IT=330 FT=225 TT=3780 FL=16158 IP=130 FP=40 TP=0
 TRAN CA=218 IS=1 FS=1 IT=225 FT=100 TT=4500 FL=16158 IP=40 FP=40 TP=0
 TRAN CA=219 IS=1 FS=1 IT=100 FT=100 TT=0 FL=2262 IP=40 FP=1578 TP=0
 TRAN CA=220
 TRAN CA=221
 TRAN CA=222 *IS=1 FS=1 IT=526 FT=130 TT=0 FL=32316 IP=1050 FP=1050 TP=0
 TRAN CA=223 IS=1 FS=1 IT=130 FT=526 TT=0 FL=32316 IP=1050 FP=1050 TP=0

```

TRAN CA=224 IS=1 FS=1 IT=526 FT=375 TT=600 FL=32316 IP=1050 FP=240 TP=0
TRAN CA=225 IS=1 FS=1 IT=375 FT=100 TT=9900 FL=32316 IP=240 FP=40 TP=0
*
PAIR CA=201 CO=8.3 DI=0.140 EX=8.5 *Tavg=85.0
PAIR CA=202 CO=8.3 DI=0.140 EX=8.5 *Tavg=100.0
PAIR CA=203 CO=9.4 DI=0.151 EX=8.5 *Tavg=324.5
PAIR CA=204 CO=10.5 DI=0.162 EX=8.5 *Tavg=545.5
PAIR CA=205 CO=10.4 DI=0.161 EX=8.5 *Tavg=534.0
PAIR CA=206 CO=10.4 DI=0.161 EX=8.5 *Tavg=534.0
PAIR CA=207 CO=10.4 DI=0.161 EX=8.5 *Tavg=534.0
PAIR CA=208 CO=10.3 DI=0.161 EX=8.5 *Tavg=521.0
PAIR CA=209 CO=10.3 DI=0.161 EX=8.5 *Tavg=521.0
PAIR CA=210 CO=9.9 DI=0.156 EX=8.5 *Tavg=413.0
PAIR CA=211 CO=9.8 DI=0.155 EX=8.5 *Tavg=400.0
PAIR CA=212 CO=9.8 DI=0.155 EX=8.5 *Tavg=400.0
PAIR CA=213 CO=9.9 DI=0.156 EX=8.5 *Tavg=424.5
PAIR CA=214 CO=10.4 DI=0.162 EX=8.5 *Tavg=537.5
PAIR CA=215 CO=10.0 DI=0.158 EX=8.5 *Tavg=462.0
PAIR CA=216 CO=9.5 DI=0.152 EX=8.5 *Tavg=352.5
PAIR CA=217 CO=9.2 DI=0.149 EX=8.5 *Tavg=277.5
PAIR CA=218 CO=8.7 DI=0.143 EX=8.5 *Tavg=162.5
PAIR CA=219 CO=8.3 DI=0.140 EX=8.5 *Tavg=100.0
*PAIR CA=220 CO=9.0 DI=0.146 EX=8.5 *Tavg=225.0
*PAIR CA=221 CO=9.0 DI=0.146 EX=8.5 *Tavg=225.0
*PAIR CA=222 CO=9.4 DI=0.151 EX=8.5 *Tavg=328.0
PAIR CA=223 CO=9.4 DI=0.151 EX=8.5 *Tavg=328.0
PAIR CA=224 CO=10.0 DI=0.157 EX=8.5 *Tavg=450.5
PAIR CA=225 CO=9.0 DI=0.147 EX=8.5 *Tavg=237.5

```

Reg3B.inp

```

*-----*
*BEGIN REGION 3B TRANSIENT CARDS & GEOMETRY AFFECTED BY RHR INITIATION
*-----*

```

```

*
OPER CA=20 TE=225 PR=25
OPER CA=21 TE=180 PR=25

```

```

*
TRAN CA=220 IS=1 FS=1 IT=180 FT=225 TT=60 FL=22858 IP=40 FP=40
TRAN CA=221 IS=1 FS=1 IT=225 FT=180 TT=60 FL=22858 IP=40 FP=40
PAIR CA=220 CO=8.8 DI=0.145 EX=8.5 *Tavg=202.5
PAIR CA=221 CO=8.8 DI=0.145 EX=8.5 *Tavg=202.5

```

Reg4.inp

```

*-----*
*BEGIN REGION 4 TRANSIENT CARDS & GEOMETRY HEADER TO NOZZLE NODE 366
*-----*

```

```

*
OPER CA=1 TE=100 PR=1100
OPER CA=2 TE=100 PR=50
OPER CA=3 TE=549 PR=1035
OPER CA=4 TE=542 PR=1035
OPER CA=5 TE=526 PR=1035
OPER CA=6 TE=542 PR=1035
OPER CA=7 TE=526 PR=1035
OPER CA=8 TE=516 PR=1035
OPER CA=9 TE=526 PR=1035
OPER CA=10 TE=300 PR=1160
OPER CA=11 TE=500 PR=1160
OPER CA=12 TE=300 PR=700
OPER CA=13 TE=549 PR=1035

```


OPER CA=14 TE=549 PR=1035
 OPER CA=15 TE=375 PR=195
 OPER CA=16 TE=330 PR=115
 OPER CA=17 TE=225 PR=25
 OPER CA=18 TE=100 PR=25
 OPER CA=19 TE=100 PR=1563
 OPER CA=20 TE=225 PR=25
 OPER CA=21 TE=180 PR=25
 OPER CA=22 TE=130 PR=1035
 OPER CA=23 TE=526 PR=1035
 OPER CA=24 TE=375 PR=225
 OPER CA=25 TE=100 PR=25

*
 TRAN CA=201 IS=1 FS=1 IT=70 FT=100 TT=1800 FL=905 IP=15 FP=1115 TP=0
 TRAN CA=202 IS=1 FS=1 IT=100 FT=100 TT=1800 FL=905 IP=1115 FP=65 TP=0
 TRAN CA=203 IS=1 FS=1 IT=100 FT=549 TT=16164 FL=6463 IP=65 FP=1050 TP=0
 TRAN CA=204 IS=1 FS=1 IT=549 FT=542 TT=0 FL=12926 IP=1050 FP=1050 TP=0
 TRAN CA=205 IS=1 FS=1 IT=542 FT=526 TT=0 FL=12926 IP=1050 FP=1050 TP=0
 TRAN CA=206 IS=1 FS=1 IT=526 FT=542 TT=900 FL=12926 IP=1050 FP=1050 TP=0
 TRAN CA=207 IS=1 FS=1 IT=542 FT=526 TT=360 FL=12926 IP=1050 FP=1050 TP=0
 TRAN CA=208 IS=1 FS=1 IT=526 FT=516 TT=0 FL=12926 IP=1050 FP=1050 TP=0
 TRAN CA=209 IS=1 FS=1 IT=516 FT=526 TT=0 FL=12926 IP=1050 FP=1050 TP=0
 TRAN CA=210 IS=1 FS=1 IT=526 FT=300 TT=220 FL=400 IP=1230 FP=1175 TP=0
 TRAN CA=211 IS=1 FS=1 IT=300 FT=500 TT=1980 FL=400 IP=925 FP=1175 TP=0
 TRAN CA=212 IS=1 FS=1 IT=500 FT=300 TT=180 FL=400 IP=1175 FP=715 TP=0
 TRAN CA=213 IS=1 FS=1 IT=300 FT=549 TT=8964 FL=6463 IP=280 FP=1050 TP=0
 TRAN CA=214 IS=1 FS=1 IT=526 FT=549 TT=0 FL=12926 IP=1050 FP=1050 TP=0
 TRAN CA=215 IS=1 FS=1 IT=549 FT=375 TT=6264 FL=6463 IP=1050 FP=210 TP=0
 TRAN CA=216 IS=1 FS=1 IT=375 FT=330 TT=600 FL=6463 IP=210 FP=130 TP=0
 TRAN CA=217 IS=1 FS=1 IT=330 FT=225 TT=3780 FL=6463 IP=130 FP=40 TP=0
 TRAN CA=218 IS=1 FS=1 IT=225 FT=100 TT=4500 FL=9143 IP=40 FP=40 TP=0
 TRAN CA=219 IS=1 FS=1 IT=100 FT=100 TT=0 FL=905 IP=40 FP=1578 TP=0
 TRAN CA=220 IS=1 FS=1 IT=180 FT=225 TT=60 FL=9143 IP=40 FP=40 TP=0
 TRAN CA=221 IS=1 FS=1 IT=225 FT=180 TT=60 FL=9143 IP=40 FP=40 TP=0
 TRAN *CA=222 IS=1 FS=1 IT=526 FT=130 TT=0 FL=12926 IP=1050 FP=1050 TP=0
 TRAN CA=223 IS=1 FS=1 IT=130 FT=526 TT=0 FL=12926 IP=1050 FP=1050 TP=0
 TRAN CA=224 IS=1 FS=1 IT=526 FT=375 TT=600 FL=12926 IP=1050 FP=240 TP=0
 TRAN CA=225 IS=1 FS=1 IT=375 FT=100 TT=9900 FL=12926 IP=240 FP=40 TP=0

*
 PAIR CA=201 CO=8.3 DI=0.140 EX=8.5 *Tavg=85.0
 PAIR CA=202 CO=8.3 DI=0.140 EX=8.5 *Tavg=100.0
 PAIR CA=203 CO=9.4 DI=0.151 EX=8.5 *Tavg=324.5
 PAIR CA=204 CO=10.5 DI=0.162 EX=8.5 *Tavg=545.5
 PAIR CA=205 CO=10.4 DI=0.161 EX=8.5 *Tavg=534.0
 PAIR CA=206 CO=10.4 DI=0.161 EX=8.5 *Tavg=534.0
 PAIR CA=207 CO=10.4 DI=0.161 EX=8.5 *Tavg=534.0
 PAIR CA=208 CO=10.3 DI=0.161 EX=8.5 *Tavg=521.0
 PAIR CA=209 CO=10.3 DI=0.161 EX=8.5 *Tavg=521.0
 PAIR CA=210 CO=9.9 DI=0.156 EX=8.5 *Tavg=413.0
 PAIR CA=211 CO=9.8 DI=0.155 EX=8.5 *Tavg=400.0
 PAIR CA=212 CO=9.8 DI=0.155 EX=8.5 *Tavg=400.0
 PAIR CA=213 CO=9.9 DI=0.156 EX=8.5 *Tavg=424.5
 PAIR CA=214 CO=10.4 DI=0.162 EX=8.5 *Tavg=537.5
 PAIR CA=215 CO=10.0 DI=0.158 EX=8.5 *Tavg=462.0
 PAIR CA=216 CO=9.5 DI=0.152 EX=8.5 *Tavg=352.5
 PAIR CA=217 CO=9.2 DI=0.149 EX=8.5 *Tavg=277.5
 PAIR CA=218 CO=8.7 DI=0.143 EX=8.5 *Tavg=162.5
 PAIR CA=219 CO=8.3 DI=0.140 EX=8.5 *Tavg=100.0
 PAIR CA=220 CO=8.8 DI=0.145 EX=8.5 *Tavg=202.5
 PAIR CA=221 CO=8.8 DI=0.145 EX=8.5 *Tavg=202.5



*PAIR CA=222 CO=9.4 DI=0.151 EX=8.5 *Tavg=328.0
 PAIR CA=223 CO=9.4 DI=0.151 EX=8.5 *Tavg=328.0
 PAIR CA=224 CO=10.0 DI=0.157 EX=8.5 *Tavg=450.5
 PAIR CA=225 CO=9.0 DI=0.147 EX=8.5 *Tavg=237.5

Reg5.inp

*-----
 *BEGIN REGION 5 TRANSIENT CARDS & GEOMETRY RISER TO NOZZLE NODE 336
 *-----
 *

OPER CA=1 TE=100 PR=1100
 OPER CA=2 TE=100 PR=50
 OPER CA=3 TE=549 PR=1035
 OPER CA=4 TE=542 PR=1035
 OPER CA=5 TE=526 PR=1035
 OPER CA=6 TE=542 PR=1035
 OPER CA=7 TE=526 PR=1035
 OPER CA=8 TE=516 PR=1035
 OPER CA=9 TE=526 PR=1035
 OPER CA=10 TE=300 PR=1160
 OPER CA=11 TE=500 PR=1160
 OPER CA=12 TE=300 PR=700
 OPER CA=13 TE=549 PR=1035
 OPER CA=14 TE=549 PR=1035
 OPER CA=15 TE=375 PR=195
 OPER CA=16 TE=330 PR=115
 OPER CA=17 TE=225 PR=25
 OPER CA=18 TE=100 PR=25
 OPER CA=19 TE=100 PR=1563
 OPER CA=20 TE=225 PR=25
 OPER CA=21 TE=180 PR=25
 OPER CA=22 TE=130 PR=1035
 OPER CA=23 TE=526 PR=1035
 OPER CA=24 TE=375 PR=225
 OPER CA=25 TE=100 PR=25

*
 TRAN CA=201 IS=1 FS=1 IT=70 FT=100 TT=1800 FL=452 IP=15 FP=1115 TP=0
 TRAN CA=202 IS=1 FS=1 IT=100 FT=100 TT=1800 FL=452 IP=1115 FP=65 TP=0
 TRAN CA=203 IS=1 FS=1 IT=100 FT=549 TT=16164 FL=3232 IP=65 FP=1050 TP=0
 TRAN CA=204 IS=1 FS=1 IT=549 FT=542 TT=0 FL=6463 IP=1050 FP=1050 TP=0
 TRAN CA=205 IS=1 FS=1 IT=542 FT=526 TT=0 FL=6463 IP=1050 FP=1050 TP=0
 TRAN CA=206 IS=1 FS=1 IT=526 FT=542 TT=900 FL=6463 IP=1050 FP=1050 TP=0
 TRAN CA=207 IS=1 FS=1 IT=542 FT=526 TT=360 FL=6463 IP=1050 FP=1050 TP=0
 TRAN CA=208 IS=1 FS=1 IT=526 FT=516 TT=0 FL=6463 IP=1050 FP=1050 TP=0
 TRAN CA=209 IS=1 FS=1 IT=516 FT=526 TT=0 FL=6463 IP=1050 FP=1050 TP=0
 TRAN CA=210 IS=1 FS=1 IT=526 FT=300 TT=220 FL=200 IP=1230 FP=1175 TP=0
 TRAN CA=211 IS=1 FS=1 IT=300 FT=500 TT=1980 FL=200 IP=925 FP=1175 TP=0
 TRAN CA=212 IS=1 FS=1 IT=500 FT=300 TT=180 FL=200 IP=1175 FP=715 TP=0
 TRAN CA=213 IS=1 FS=1 IT=300 FT=549 TT=8964 FL=3232 IP=280 FP=1050 TP=0
 TRAN CA=214 IS=1 FS=1 IT=526 FT=549 TT=0 FL=6463 IP=1050 FP=1050 TP=0
 TRAN CA=215 IS=1 FS=1 IT=549 FT=375 TT=6264 FL=3232 IP=1050 FP=210 TP=0
 TRAN CA=216 IS=1 FS=1 IT=375 FT=330 TT=600 FL=3232 IP=210 FP=130 TP=0
 TRAN CA=217 IS=1 FS=1 IT=330 FT=225 TT=3780 FL=3232 IP=130 FP=40 TP=0
 TRAN CA=218 IS=1 FS=1 IT=225 FT=100 TT=4500 FL=4571 IP=40 FP=40 TP=0
 TRAN CA=219 IS=1 FS=1 IT=100 FT=100 TT=0 FL=452 IP=40 FP=1578 TP=0
 TRAN CA=220 IS=1 FS=1 IT=180 FT=225 TT=60 FL=4572 IP=40 FP=40 TP=0
 TRAN CA=221 IS=1 FS=1 IT=225 FT=180 TT=60 FL=4572 IP=40 FP=40 TP=0
 TRAN *CA=222 IS=1 FS=1 IT=526 FT=130 TT=0 FL=6463 IP=1050 FP=1050 TP=0
 TRAN CA=223 IS=1 FS=1 IT=130 FT=526 TT=0 FL=6463 IP=1050 FP=1050 TP=0
 TRAN CA=224 IS=1 FS=1 IT=526 FT=375 TT=600 FL=6463 IP=1050 FP=240 TP=0



```

TRAN CA=225 IS=1 FS=1 IT=375 FT=100 TT=9900 FL=6463 IP=240 FP=40 TP=0
*
PAIR CA=201 CO=8.3 DI=0.140 EX=8.5 *Tavg=85.0
PAIR CA=202 CO=8.3 DI=0.140 EX=8.5 *Tavg=100.0
PAIR CA=203 CO=9.4 DI=0.151 EX=8.5 *Tavg=324.5
PAIR CA=204 CO=10.5 DI=0.162 EX=8.5 *Tavg=545.5
PAIR CA=205 CO=10.4 DI=0.161 EX=8.5 *Tavg=534.0
PAIR CA=206 CO=10.4 DI=0.161 EX=8.5 *Tavg=534.0
PAIR CA=207 CO=10.4 DI=0.161 EX=8.5 *Tavg=534.0
PAIR CA=208 CO=10.3 DI=0.161 EX=8.5 *Tavg=521.0
PAIR CA=209 CO=10.3 DI=0.161 EX=8.5 *Tavg=521.0
PAIR CA=210 CO=9.9 DI=0.156 EX=8.5 *Tavg=413.0
PAIR CA=211 CO=9.8 DI=0.155 EX=8.5 *Tavg=400.0
PAIR CA=212 CO=9.8 DI=0.155 EX=8.5 *Tavg=400.0
PAIR CA=213 CO=9.9 DI=0.156 EX=8.5 *Tavg=424.5
PAIR CA=214 CO=10.4 DI=0.162 EX=8.5 *Tavg=537.5
PAIR CA=215 CO=10.0 DI=0.158 EX=8.5 *Tavg=462.0
PAIR CA=216 CO=9.5 DI=0.152 EX=8.5 *Tavg=352.5
PAIR CA=217 CO=9.2 DI=0.149 EX=8.5 *Tavg=277.5
PAIR CA=218 CO=8.7 DI=0.143 EX=8.5 *Tavg=162.5
PAIR CA=219 CO=8.3 DI=0.140 EX=8.5 *Tavg=100.0
PAIR CA=220 CO=8.8 DI=0.145 EX=8.5 *Tavg=202.5
PAIR CA=221 CO=8.8 DI=0.145 EX=8.5 *Tavg=202.5
*PAIR CA=222 CO=9.4 DI=0.151 EX=8.5 *Tavg=328.0
PAIR CA=223 CO=9.4 DI=0.151 EX=8.5 *Tavg=328.0
PAIR CA=224 CO=10.0 DI=0.157 EX=8.5 *Tavg=450.5
PAIR CA=225 CO=9.0 DI=0.147 EX=8.5 *Tavg=237.5

```

Reg6.inp

```

*-----
*BEGIN REGION 6 TRANSIENT CARDS & GEOMETRY TO NOZZLE NODE 336
*-----

```

```

*
OPER CA=1 TE=100 PR=1100
OPER CA=2 TE=100 PR=50
OPER CA=3 TE=549 PR=1035
OPER CA=4 TE=542 PR=1035
OPER CA=5 TE=526 PR=1035
OPER CA=6 TE=542 PR=1035
OPER CA=7 TE=526 PR=1035
OPER CA=8 TE=516 PR=1035
OPER CA=9 TE=526 PR=1035
OPER CA=10 TE=300 PR=1160
OPER CA=11 TE=500 PR=1160
OPER CA=12 TE=300 PR=700
OPER CA=13 TE=549 PR=1035
OPER CA=14 TE=549 PR=1035
OPER CA=15 TE=375 PR=195
OPER CA=16 TE=330 PR=115
OPER CA=17 TE=225 PR=25
OPER CA=18 TE=100 PR=25
OPER CA=19 TE=100 PR=1563
OPER CA=20 TE=225 PR=25
OPER CA=21 TE=180 PR=25
OPER CA=22 TE=130 PR=1035
OPER CA=23 TE=526 PR=1035
OPER CA=24 TE=375 PR=225
OPER CA=25 TE=100 PR=25

```

```

*
TRAN CA=201 IS=1 FS=1 IT=70 FT=100 TT=1800 FL=452 IP=15 FP=1115 TP=0

```

```

TRAN CA=202 IS=1 FS=1 IT=100 FT=100 TT=1800 FL=452 IP=1115 FP=65 TP=0
TRAN CA=203 IS=1 FS=1 IT=100 FT=549 TT=16164 FL=3232 IP=65 FP=1050 TP=0
TRAN CA=204 IS=1 FS=1 IT=549 FT=542 TT=0 FL=6463 IP=1050 FP=1050 TP=0
TRAN CA=205 IS=1 FS=1 IT=542 FT=526 TT=0 FL=6463 IP=1050 FP=1050 TP=0
TRAN CA=206 IS=1 FS=1 IT=526 FT=542 TT=900 FL=6463 IP=1050 FP=1050 TP=0
TRAN CA=207 IS=1 FS=1 IT=542 FT=526 TT=360 FL=6463 IP=1050 FP=1050 TP=0
TRAN CA=208 IS=1 FS=1 IT=526 FT=516 TT=0 FL=6463 IP=1050 FP=1050 TP=0
TRAN CA=209 IS=1 FS=1 IT=516 FT=526 TT=0 FL=6463 IP=1050 FP=1050 TP=0
TRAN CA=210 IS=1 FS=1 IT=526 FT=300 TT=220 FL=200 IP=1230 FP=1175 TP=0
TRAN CA=211 IS=1 FS=1 IT=300 FT=500 TT=1980 FL=200 IP=925 FP=1175 TP=0
TRAN CA=212 IS=1 FS=1 IT=500 FT=300 TT=180 FL=200 IP=1175 FP=715 TP=0
TRAN CA=213 IS=1 FS=1 IT=300 FT=549 TT=8964 FL=3232 IP=280 FP=1050 TP=0
TRAN CA=214 IS=1 FS=1 IT=526 FT=549 TT=0 FL=6463 IP=1050 FP=1050 TP=0
TRAN CA=215 IS=1 FS=1 IT=549 FT=375 TT=6264 FL=3232 IP=1050 FP=210 TP=0
TRAN CA=216 IS=1 FS=1 IT=375 FT=330 TT=600 FL=3232 IP=210 FP=130 TP=0
TRAN CA=217 IS=1 FS=1 IT=330 FT=225 TT=3780 FL=3232 IP=130 FP=40 TP=0
TRAN CA=218 IS=1 FS=1 IT=225 FT=100 TT=4500 FL=4572 IP=40 FP=40 TP=0
TRAN CA=219 IS=1 FS=1 IT=100 FT=100 TT=0 FL=452 IP=40 FP=1578 TP=0
TRAN CA=220 IS=1 FS=1 IT=180 FT=225 TT=60 FL=4572 IP=40 FP=40 TP=0
TRAN CA=221 IS=1 FS=1 IT=225 FT=180 TT=60 FL=4572 IP=40 FP=40 TP=0
TRAN CA=222 IS=1 FS=1 IT=526 FT=130 TT=0 FL=6463 IP=1050 FP=1050 TP=0
TRAN CA=223 IS=1 FS=1 IT=130 FT=526 TT=0 FL=6463 IP=1050 FP=1050 TP=0
TRAN CA=224 IS=1 FS=1 IT=526 FT=375 TT=600 FL=6463 IP=1050 FP=240 TP=0
TRAN CA=225 IS=1 FS=1 IT=375 FT=100 TT=9900 FL=6463 IP=240 FP=40 TP=0

```

```

*
PAIR CA=201 CO=8.3 DI=0.140 EX=8.5 *Tavg=85.0
PAIR CA=202 CO=8.3 DI=0.140 EX=8.5 *Tavg=100.0
PAIR CA=203 CO=9.4 DI=0.151 EX=8.5 *Tavg=324.5
PAIR CA=204 CO=10.5 DI=0.162 EX=8.5 *Tavg=545.5
PAIR CA=205 CO=10.4 DI=0.161 EX=8.5 *Tavg=534.0
PAIR CA=206 CO=10.4 DI=0.161 EX=8.5 *Tavg=534.0
PAIR CA=207 CO=10.4 DI=0.161 EX=8.5 *Tavg=534.0
PAIR CA=208 CO=10.3 DI=0.161 EX=8.5 *Tavg=521.0
PAIR CA=209 CO=10.3 DI=0.161 EX=8.5 *Tavg=521.0
PAIR CA=210 CO=9.9 DI=0.156 EX=8.5 *Tavg=413.0
PAIR CA=211 CO=9.8 DI=0.155 EX=8.5 *Tavg=400.0
PAIR CA=212 CO=9.8 DI=0.155 EX=8.5 *Tavg=400.0
PAIR CA=213 CO=9.9 DI=0.156 EX=8.5 *Tavg=424.5
PAIR CA=214 CO=10.4 DI=0.162 EX=8.5 *Tavg=537.5
PAIR CA=215 CO=10.0 DI=0.158 EX=8.5 *Tavg=462.0
PAIR CA=216 CO=9.5 DI=0.152 EX=8.5 *Tavg=352.5
PAIR CA=217 CO=9.2 DI=0.149 EX=8.5 *Tavg=277.5
PAIR CA=218 CO=8.7 DI=0.143 EX=8.5 *Tavg=162.5
PAIR CA=219 CO=8.3 DI=0.140 EX=8.5 *Tavg=100.0
PAIR CA=220 CO=8.8 DI=0.145 EX=8.5 *Tavg=202.5
PAIR CA=221 CO=8.8 DI=0.145 EX=8.5 *Tavg=202.5
PAIR CA=222 CO=9.4 DI=0.151 EX=8.5 *Tavg=328.0
PAIR CA=223 CO=9.4 DI=0.151 EX=8.5 *Tavg=328.0
PAIR CA=224 CO=10.0 DI=0.157 EX=8.5 *Tavg=450.5
PAIR CA=225 CO=9.0 DI=0.147 EX=8.5 *Tavg=237.5

```

Reg7A.inp

```

*-----
*BEGIN REGION 7A TRANSIENT CARDS & GEOMETRY TO RHR SUPPLY VALVE NODE 550
*-----

```

```

*
OPER CA=1 TE=100 PR=1100
OPER CA=2 TE=100 PR=50
OPER CA=3 TE=549 PR=1010
OPER CA=4 TE=542 PR=1010

```

OPER CA=5 TE=526 PR=1010
 OPER CA=6 TE=542 PR=1010
 OPER CA=7 TE=526 PR=1010
 OPER CA=8 TE=516 PR=1010
 OPER CA=9 TE=526 PR=1010
 OPER CA=10 TE=300 PR=1135
 OPER CA=11 TE=500 PR=1135
 OPER CA=12 TE=300 PR=675
 OPER CA=13 TE=549 PR=1010
 OPER CA=14 TE=549 PR=1010
 OPER CA=15 TE=375 PR=170
 OPER CA=16 TE=330 PR=90
 OPER CA=17 TE=225 PR=0
 OPER CA=18 TE=100 PR=0
 OPER CA=19 TE=100 PR=1563
 OPER CA=20 TE=225 PR=0
 OPER CA=21 TE=225 PR=0
 OPER CA=22 TE=130 PR=1010
 OPER CA=23 TE=526 PR=1010
 OPER CA=24 TE=375 PR=200
 OPER CA=25 TE=100 PR=0

*

TRAN CA=201 IS=1 FS=1 IT=70 FT=100 TT=1800 FL=143 IP=15 FP=1115 TP=0
 TRAN CA=202 IS=1 FS=1 IT=100 FT=100 TT=1800 FL=143 IP=1115 FP=65 TP=0
 TRAN CA=203 IS=1 FS=1 IT=100 FT=549 TT=16164 FL=300 IP=65 FP=1025 TP=0
 TRAN CA=204 IS=1 FS=1 IT=549 FT=542 TT=0 FL=364 IP=1025 FP=1025 TP=0
 TRAN CA=205 IS=1 FS=1 IT=542 FT=526 TT=0 FL=358 IP=1025 FP=1025 TP=0
 TRAN CA=206 IS=1 FS=1 IT=526 FT=542 TT=900 FL=358 IP=1025 FP=1025 TP=0
 TRAN CA=207 IS=1 FS=1 IT=542 FT=526 TT=360 FL=358 IP=1025 FP=1025 TP=0
 TRAN CA=208 IS=1 FS=1 IT=526 FT=516 TT=0 FL=351 IP=1025 FP=1025 TP=0
 TRAN CA=209 IS=1 FS=1 IT=516 FT=526 TT=0 FL=351 IP=1025 FP=1025 TP=0
 TRAN CA=210 IS=1 FS=1 IT=526 FT=300 TT=220 FL=306 IP=1205 FP=1150 TP=0
 TRAN CA=211 IS=1 FS=1 IT=300 FT=500 TT=1980 FL=301 IP=900 FP=1150 TP=0
 TRAN CA=212 IS=1 FS=1 IT=500 FT=300 TT=180 FL=301 IP=1150 FP=690 TP=0
 TRAN CA=213 IS=1 FS=1 IT=300 FT=549 TT=8964 FL=310 IP=255 FP=1025 TP=0
 TRAN CA=214 IS=1 FS=1 IT=526 FT=549 TT=0 FL=360 IP=1025 FP=1025 TP=0
 TRAN CA=215 IS=1 FS=1 IT=549 FT=375 TT=6264 FL=320 IP=1050 FP=185 TP=0
 TRAN CA=216 IS=1 FS=1 IT=375 FT=330 TT=600 FL=282 IP=185 FP=105 TP=0
 TRAN CA=217 IS=1 FS=1 IT=330 FT=225 TT=3780 FL=260 IP=105 FP=15 TP=0
 TRAN CA=218 IS=1 FS=1 IT=225 FT=100 TT=4500 FL=6700 IP=15 FP=15 TP=0
 TRAN CA=219 IS=1 FS=1 IT=100 FT=100 TT=0 FL=158 IP=40 FP=1578 TP=0
 TRAN CA=220 *IS=1 FS=1 IT=225 FT=225 TT=60 FL=6700 IP=15 FP=15 TP=0
 TRAN CA=221 *IS=1 FS=1 IT=225 FT=225 TT=60 FL=6700 IP=15 FP=15 TP=0
 TRAN CA=222 *IS=1 FS=1 IT=526 FT=130 TT=0 FL=272 IP=1025 FP=1025 TP=0
 TRAN CA=223 IS=1 FS=1 IT=130 FT=526 TT=0 FL=272 IP=1025 FP=1025 TP=0
 TRAN CA=224 IS=1 FS=1 IT=526 FT=375 TT=600 FL=320 IP=1025 FP=215 TP=0
 TRAN CA=225 IS=1 FS=1 IT=375 FT=100 TT=9900 FL=234 IP=215 FP=15 TP=0

*

PAIR CA=201 CO=8.3 DI=0.140 EX=8.5 *Tavg=85.0
 PAIR CA=202 CO=8.3 DI=0.140 EX=8.5 *Tavg=100.0
 PAIR CA=203 CO=9.4 DI=0.151 EX=8.5 *Tavg=324.5
 PAIR CA=204 CO=10.5 DI=0.162 EX=8.5 *Tavg=545.5
 PAIR CA=205 CO=10.4 DI=0.161 EX=8.5 *Tavg=534.0
 PAIR CA=206 CO=10.4 DI=0.161 EX=8.5 *Tavg=534.0
 PAIR CA=207 CO=10.4 DI=0.161 EX=8.5 *Tavg=534.0
 PAIR CA=208 CO=10.3 DI=0.161 EX=8.5 *Tavg=521.0
 PAIR CA=209 CO=10.3 DI=0.161 EX=8.5 *Tavg=521.0
 PAIR CA=210 CO=9.9 DI=0.156 EX=8.5 *Tavg=413.0
 PAIR CA=211 CO=9.8 DI=0.155 EX=8.5 *Tavg=400.0
 PAIR CA=212 CO=9.8 DI=0.155 EX=8.5 *Tavg=400.0



PAIR CA=213 CO=9.9 DI=0.156 EX=8.5 *Tavg=424.5
PAIR CA=214 CO=10.4 DI=0.162 EX=8.5 *Tavg=537.5
PAIR CA=215 CO=10.0 DI=0.158 EX=8.5 *Tavg=462.0
PAIR CA=216 CO=9.5 DI=0.152 EX=8.5 *Tavg=352.5
PAIR CA=217 CO=9.2 DI=0.149 EX=8.5 *Tavg=277.5
PAIR CA=218 CO=8.7 DI=0.143 EX=8.5 *Tavg=162.5
PAIR CA=219 CO=8.3 DI=0.140 EX=8.5 *Tavg=100.0
*PAIR CA=220 CO=9.0 DI=0.146 EX=8.5 *Tavg=225.0
*PAIR CA=221 CO=9.0 DI=0.146 EX=8.5 *Tavg=225.0
*PAIR CA=222 CO=9.4 DI=0.151 EX=8.5 *Tavg=328.0
PAIR CA=223 CO=9.4 DI=0.151 EX=8.5 *Tavg=328.0
PAIR CA=224 CO=10.0 DI=0.157 EX=8.5 *Tavg=450.5
PAIR CA=225 CO=9.0 DI=0.147 EX=8.5 *Tavg=237.5

Reg7B.inp

*-----
*BEGIN REGION 7B TRANSIENT CARDS & GEOMETRY FROM RHR SUPPLY VALVE TO PENET. NODE 565
*-----
*

OPER CA=1 TE=100 PR=120
OPER CA=2 TE=100 PR=50
OPER CA=3 TE=150 PR=120
OPER CA=4 TE=150 PR=120
OPER CA=5 TE=150 PR=120
OPER CA=6 TE=150 PR=120
OPER CA=7 TE=150 PR=120
OPER CA=8 TE=150 PR=120
OPER CA=9 TE=150 PR=120
OPER CA=10 TE=150 PR=120
OPER CA=11 TE=150 PR=120
OPER CA=12 TE=150 PR=120
OPER CA=13 TE=150 PR=120
OPER CA=14 TE=150 PR=120
OPER CA=15 TE=150 PR=120
OPER CA=16 TE=150 PR=120
OPER CA=17 TE=150 PR=100
OPER CA=18 TE=100 PR=0
OPER CA=19 TE=100 PR=450
OPER CA=20 TE=225 PR=25
OPER CA=21 TE=150 PR=25
OPER CA=22 TE=150 PR=100
OPER CA=23 TE=150 PR=1035
OPER CA=24 TE=150 PR=100
OPER CA=25 TE=150 PR=100
*

TRAN CA=201 IS=1 FS=1 IT=70 FT=100 TT=1800 FL=143 IP=15 FP=135 TP=0
TRAN CA=202 IS=1 FS=1 IT=100 FT=100 TT=1800 FL=143 IP=135 FP=65 TP=0
TRAN CA=203 IS=1 FS=1 IT=100 FT=150 TT=16164 FL=300 IP=65 FP=135 TP=0
TRAN CA=204
TRAN CA=205
TRAN CA=206
TRAN CA=207
TRAN CA=208
TRAN CA=209
TRAN CA=210
TRAN CA=211
TRAN CA=212
TRAN CA=213
TRAN CA=214
TRAN CA=215



TRAN CA=216
 TRAN CA=217
 TRAN CA=218 IS=1 FS=1 IT=225 FT=100 TT=4500 FL=6700 IP=15 FP=15 TP=0
 TRAN CA=219 IS=1 FS=1 IT=100 FT=100 TT=0 FL=143 IP=15 FP=465 TP=0
 TRAN CA=220 IS=1 FS=1 IT=150 FT=225 TT=60 FL=6700 IP=40 FP=40 TP=0
 TRAN CA=221 *IS=1 FS=1 IT=150 FT=150 TT=0 FL=6700 IP=40 FP=40 TP=0
 TRAN CA=222
 TRAN CA=223
 TRAN CA=224
 TRAN CA=225

*
 PAIR CA=201 CO=27.6 DI=0.529 EX=6.4 *Tavg=85.0
 PAIR CA=202 CO=27.6 DI=0.512 EX=6.4 *Tavg=100.0
 PAIR CA=203 CO=27.6 DI=0.506 EX=6.4 *Tavg=125.0
 *PAIR CA=204 CO=27.6 DI=0.499 EX=6.4 *Tavg=150.0
 *PAIR CA=205 CO=27.6 DI=0.499 EX=6.4 *Tavg=150.0
 *PAIR CA=206 CO=27.6 DI=0.499 EX=6.4 *Tavg=150.0
 *PAIR CA=207 CO=27.6 DI=0.499 EX=6.4 *Tavg=150.0
 *PAIR CA=208 CO=27.6 DI=0.499 EX=6.4 *Tavg=150.0
 *PAIR CA=209 CO=27.6 DI=0.499 EX=6.4 *Tavg=150.0
 *PAIR CA=210 CO=27.6 DI=0.499 EX=6.4 *Tavg=150.0
 *PAIR CA=211 CO=27.6 DI=0.499 EX=6.4 *Tavg=150.0
 *PAIR CA=212 CO=27.6 DI=0.499 EX=6.4 *Tavg=150.0
 *PAIR CA=213 CO=27.6 DI=0.499 EX=6.4 *Tavg=150.0
 *PAIR CA=214 CO=27.6 DI=0.499 EX=6.4 *Tavg=150.0
 *PAIR CA=215 CO=27.6 DI=0.499 EX=6.4 *Tavg=150.0
 *PAIR CA=216 CO=27.6 DI=0.499 EX=6.4 *Tavg=150.0
 *PAIR CA=217 CO=27.6 DI=0.499 EX=6.4 *Tavg=150.0
 PAIR CA=218 CO=27.6 DI=0.496 EX=6.4 *Tavg=162.5
 PAIR CA=219 CO=27.6 DI=0.512 EX=6.4 *Tavg=100.0
 PAIR CA=220 CO=27.6 DI=0.489 EX=6.4 *Tavg=187.5
 PAIR CA=221 CO=27.6 DI=0.499 EX=6.4 *Tavg=150.0
 *PAIR CA=222 CO=27.6 DI=0.499 EX=6.4 *Tavg=150.0
 *PAIR CA=223 CO=27.6 DI=0.499 EX=6.4 *Tavg=150.0
 *PAIR CA=224 CO=27.6 DI=0.499 EX=6.4 *Tavg=150.0
 *PAIR CA=225 CO=27.6 DI=0.499 EX=6.4 *Tavg=150.0

Reg8.inp

*-----
 *BEGIN REGION 8 TRANSIENT CARDS & GEOMETRY FOR 4 INCH BYPASS
 *-----
 *

OPER CA=1 TE=100 PR=1100
 OPER CA=2 TE=100 PR=50
 OPER CA=3 TE=549 PR=1035
 OPER CA=4 TE=542 PR=1035
 OPER CA=5 TE=526 PR=1035
 OPER CA=6 TE=542 PR=1035
 OPER CA=7 TE=526 PR=1035
 OPER CA=8 TE=516 PR=1035
 OPER CA=9 TE=526 PR=1035
 OPER CA=10 TE=300 PR=1160
 OPER CA=11 TE=500 PR=1160
 OPER CA=12 TE=300 PR=700
 OPER CA=13 TE=549 PR=1035
 OPER CA=14 TE=549 PR=1035
 OPER CA=15 TE=375 PR=195
 OPER CA=16 TE=330 PR=115
 OPER CA=17 TE=225 PR=25
 OPER CA=18 TE=100 PR=25

OPER CA=19 TE=100 PR=1563
 OPER CA=20 TE=225 PR=25
 OPER CA=21 TE=225 PR=25
 OPER CA=22 TE=130 PR=1035
 OPER CA=23 TE=526 PR=1035
 OPER CA=24 TE=375 PR=225
 OPER CA=25 TE=100 PR=25

*
 TRAN CA=201 IS=1 FS=1 IT=70 FT=100 TT=1800 FL=23.5 IP=15 FP=1115 TP=0
 TRAN CA=202 IS=1 FS=1 IT=100 FT=100 TT=1800 FL=23.5 IP=1115 FP=65 TP=0
 TRAN CA=203 IS=1 FS=1 IT=100 FT=549 TT=16164 FL=168 IP=65 FP=1050 TP=0
 TRAN CA=204 IS=1 FS=1 IT=549 FT=542 TT=0 FL=335 IP=1050 FP=1050 TP=0
 TRAN CA=205 IS=1 FS=1 IT=542 FT=526 TT=0 FL=335 IP=1050 FP=1050 TP=0
 TRAN CA=206 IS=1 FS=1 IT=526 FT=542 TT=900 FL=335 IP=1050 FP=1050 TP=0
 TRAN CA=207 IS=1 FS=1 IT=542 FT=526 TT=360 FL=335 IP=1050 FP=1050 TP=0
 TRAN CA=208 IS=1 FS=1 IT=526 FT=516 TT=0 FL=335 IP=1050 FP=1050 TP=0
 TRAN CA=209 IS=1 FS=1 IT=516 FT=526 TT=0 FL=335 IP=1050 FP=1050 TP=0
 TRAN CA=210 IS=1 FS=1 IT=526 FT=300 TT=220 FL=6 IP=1230 FP=1175 TP=0
 TRAN CA=211 IS=1 FS=1 IT=300 FT=500 TT=1980 FL=6 IP=925 FP=1175 TP=0
 TRAN CA=212 IS=1 FS=1 IT=500 FT=300 TT=180 FL=6 IP=1175 FP=715 TP=0
 TRAN CA=213 IS=1 FS=1 IT=300 FT=549 TT=8964 FL=167.5 IP=280 FP=1050 TP=0
 TRAN CA=214 IS=1 FS=1 IT=526 FT=549 TT=0 FL=335 IP=1050 FP=1050 TP=0
 TRAN CA=215 IS=1 FS=1 IT=549 FT=375 TT=6264 FL=167.5 IP=1050 FP=210 TP=0
 TRAN CA=216 IS=1 FS=1 IT=375 FT=330 TT=600 FL=167.5 IP=210 FP=130 TP=0
 TRAN CA=217 IS=1 FS=1 IT=330 FT=225 TT=3780 FL=167.5 IP=130 FP=40 TP=0
 TRAN CA=218 IS=1 FS=1 IT=225 FT=100 TT=4500 FL=167.5 IP=40 FP=40 TP=0
 TRAN CA=219 IS=1 FS=1 IT=100 FT=100 TT=0 FL=23.5 IP=40 FP=1578 TP=0
 TRAN CA=220
 TRAN CA=221
 TRAN CA=222 *IS=1 FS=1 IT=526 FT=130 TT=0 FL=335 IP=1050 FP=1050 TP=0
 TRAN CA=223 IS=1 FS=1 IT=130 FT=526 TT=0 FL=335 IP=1050 FP=1050 TP=0
 TRAN CA=224 IS=1 FS=1 IT=526 FT=375 TT=600 FL=335 IP=1050 FP=240 TP=0
 TRAN CA=225 IS=1 FS=1 IT=375 FT=100 TT=9900 FL=335 IP=240 FP=40 TP=0

*
 PAIR CA=201 CO=8.3 DI=0.140 EX=8.5 *Tavg=85.0
 PAIR CA=202 CO=8.3 DI=0.140 EX=8.5 *Tavg=100.0
 PAIR CA=203 CO=9.4 DI=0.151 EX=8.5 *Tavg=324.5
 PAIR CA=204 CO=10.5 DI=0.162 EX=8.5 *Tavg=545.5
 PAIR CA=205 CO=10.4 DI=0.161 EX=8.5 *Tavg=534.0
 PAIR CA=206 CO=10.4 DI=0.161 EX=8.5 *Tavg=534.0
 PAIR CA=207 CO=10.4 DI=0.161 EX=8.5 *Tavg=534.0
 PAIR CA=208 CO=10.3 DI=0.161 EX=8.5 *Tavg=521.0
 PAIR CA=209 CO=10.3 DI=0.161 EX=8.5 *Tavg=521.0
 PAIR CA=210 CO=9.9 DI=0.156 EX=8.5 *Tavg=413.0
 PAIR CA=211 CO=9.8 DI=0.155 EX=8.5 *Tavg=400.0
 PAIR CA=212 CO=9.8 DI=0.155 EX=8.5 *Tavg=400.0
 PAIR CA=213 CO=9.9 DI=0.156 EX=8.5 *Tavg=424.5
 PAIR CA=214 CO=10.4 DI=0.162 EX=8.5 *Tavg=537.5
 PAIR CA=215 CO=10.0 DI=0.158 EX=8.5 *Tavg=462.0
 PAIR CA=216 CO=9.5 DI=0.152 EX=8.5 *Tavg=352.5
 PAIR CA=217 CO=9.2 DI=0.149 EX=8.5 *Tavg=277.5
 PAIR CA=218 CO=8.7 DI=0.143 EX=8.5 *Tavg=162.5
 PAIR CA=219 CO=8.3 DI=0.140 EX=8.5 *Tavg=100.0
 *PAIR CA=220 CO=9.0 DI=0.146 EX=8.5 *Tavg=225.0
 *PAIR CA=221 CO=9.0 DI=0.146 EX=8.5 *Tavg=225.0
 *PAIR CA=222 CO=9.4 DI=0.151 EX=8.5 *Tavg=328.0
 PAIR CA=223 CO=9.4 DI=0.151 EX=8.5 *Tavg=328.0
 PAIR CA=224 CO=10.0 DI=0.157 EX=8.5 *Tavg=450.5
 PAIR CA=225 CO=9.0 DI=0.147 EX=8.5 *Tavg=237.5



Reg9A.inp

*BEGIN REGION 9A TRANSIENT CARDS & GEOMETRY FOR RHR RETURN FROM TEE TO VALVE NODE 660

OPER CA=1 TE=100 PR=1100
OPER CA=2 TE=100 PR=50
OPER CA=3 TE=549 PR=1035
OPER CA=4 TE=542 PR=1035
OPER CA=5 TE=526 PR=1035
OPER CA=6 TE=542 PR=1035
OPER CA=7 TE=526 PR=1035
OPER CA=8 TE=516 PR=1035
OPER CA=9 TE=526 PR=1035
OPER CA=10 TE=300 PR=1160
OPER CA=11 TE=500 PR=1160
OPER CA=12 TE=300 PR=700
OPER CA=13 TE=549 PR=1035
OPER CA=14 TE=549 PR=1035
OPER CA=15 TE=375 PR=195
OPER CA=16 TE=330 PR=115
OPER CA=17 TE=225 PR=25
OPER CA=18 TE=100 PR=100
OPER CA=19 TE=100 PR=1563
OPER CA=20 TE=225 PR=25
OPER CA=21 TE=70 PR=25
OPER CA=22 TE=130 PR=1035
OPER CA=23 TE=526 PR=1035
OPER CA=24 TE=375 PR=225
OPER CA=25 TE=100 PR=25

TRAN CA=201 IS=1 FS=1 IT=70 FT=100 TT=1800 FL=204 IP=15 FP=1115 TP=0
TRAN CA=202 IS=1 FS=1 IT=100 FT=100 TT=1800 FL=204 IP=1115 FP=65 TP=0
TRAN CA=203 IS=1 FS=1 IT=100 FT=549 TT=16164 FL=247 IP=65 FP=1050 TP=0
TRAN CA=204 IS=1 FS=1 IT=549 FT=542 TT=0 FL=520 IP=1050 FP=1050 TP=0
TRAN CA=205 IS=1 FS=1 IT=542 FT=526 TT=0 FL=511 IP=1050 FP=1050 TP=0
TRAN CA=206 IS=1 FS=1 IT=526 FT=542 TT=900 FL=511 IP=1050 FP=1050 TP=0
TRAN CA=207 IS=1 FS=1 IT=542 FT=526 TT=360 FL=511 IP=1050 FP=1050 TP=0
TRAN CA=208 IS=1 FS=1 IT=526 FT=516 TT=0 FL=502 IP=1050 FP=1050 TP=0
TRAN CA=209 IS=1 FS=1 IT=516 FT=526 TT=0 FL=502 IP=1050 FP=1050 TP=0
TRAN CA=210 IS=1 FS=1 IT=526 FT=300 TT=220 FL=437 IP=1230 FP=1175 TP=0
TRAN CA=211 IS=1 FS=1 IT=300 FT=500 TT=1980 FL=429 IP=925 FP=1175 TP=0
TRAN CA=212 IS=1 FS=1 IT=500 FT=300 TT=180 FL=429 IP=1175 FP=715 TP=0
TRAN CA=213 IS=1 FS=1 IT=300 FT=549 TT=8964 FL=443 IP=280 FP=1050 TP=0
TRAN CA=214 IS=1 FS=1 IT=526 FT=549 TT=0 FL=514 IP=1050 FP=1050 TP=0
TRAN CA=215 IS=1 FS=1 IT=549 FT=375 TT=6264 FL=458 IP=1050 FP=210 TP=0
TRAN CA=216 IS=1 FS=1 IT=375 FT=330 TT=600 FL=403 IP=210 FP=130 TP=0
TRAN CA=217 IS=1 FS=1 IT=330 FT=225 TT=3780 FL=260 IP=130 FP=40 TP=0
TRAN CA=218 IS=1 FS=1 IT=225 FT=100 TT=4500 FL=6700 IP=115 FP=115 TP=0
TRAN CA=219 IS=1 FS=1 IT=100 FT=100 TT=0 FL=226 IP=40 FP=1578 TP=0
TRAN CA=220 IS=1 FS=1 IT=70 FT=225 TT=60 FL=6700 IP=40 FP=40 TP=0
TRAN CA=221 IS=1 FS=1 IT=225 FT=70 TT=60 FL=6700 IP=40 FP=40 TP=0
TRAN CA=222 *IS=1 FS=1 IT=526 FT=130 TT=0 FL=389 IP=1050 FP=1050 TP=0
TRAN CA=223 IS=1 FS=1 IT=130 FT=526 TT=0 FL=389 IP=1050 FP=1050 TP=0
TRAN CA=224 IS=1 FS=1 IT=526 FT=375 TT=600 FL=458 IP=1050 FP=240 TP=0
TRAN CA=225 IS=1 FS=1 IT=375 FT=100 TT=9900 FL=334 IP=240 FP=40 TP=0

PAIR CA=201 CO=8.3 DI=0.140 EX=8.5 *Tavg=85.0
PAIR CA=202 CO=8.3 DI=0.140 EX=8.5 *Tavg=100.0
PAIR CA=203 CO=9.4 DI=0.151 EX=8.5 *Tavg=324.5

PAIR CA=204 CO=10.5 DI=0.162 EX=8.5 *Tavg=545.5
 PAIR CA=205 CO=10.4 DI=0.161 EX=8.5 *Tavg=534.0
 PAIR CA=206 CO=10.4 DI=0.161 EX=8.5 *Tavg=534.0
 PAIR CA=207 CO=10.4 DI=0.161 EX=8.5 *Tavg=534.0
 PAIR CA=208 CO=10.3 DI=0.161 EX=8.5 *Tavg=521.0
 PAIR CA=209 CO=10.3 DI=0.161 EX=8.5 *Tavg=521.0
 PAIR CA=210 CO=9.9 DI=0.156 EX=8.5 *Tavg=413.0
 PAIR CA=211 CO=9.8 DI=0.155 EX=8.5 *Tavg=400.0
 PAIR CA=212 CO=9.8 DI=0.155 EX=8.5 *Tavg=400.0
 PAIR CA=213 CO=9.9 DI=0.156 EX=8.5 *Tavg=424.5
 PAIR CA=214 CO=10.4 DI=0.162 EX=8.5 *Tavg=537.5
 PAIR CA=215 CO=10.0 DI=0.158 EX=8.5 *Tavg=462.0
 PAIR CA=216 CO=9.5 DI=0.152 EX=8.5 *Tavg=352.5
 PAIR CA=217 CO=9.2 DI=0.149 EX=8.5 *Tavg=277.5
 PAIR CA=218 CO=8.7 DI=0.143 EX=8.5 *Tavg=162.5
 PAIR CA=219 CO=8.3 DI=0.140 EX=8.5 *Tavg=100.0
 PAIR CA=220 CO=8.6 DI=0.142 EX=8.5 *Tavg=147.5
 PAIR CA=221 CO=8.6 DI=0.142 EX=8.5 *Tavg=147.5
 *PAIR CA=222 CO=9.4 DI=0.151 EX=8.5 *Tavg=328.0
 PAIR CA=223 CO=9.4 DI=0.151 EX=8.5 *Tavg=328.0
 PAIR CA=224 CO=10.0 DI=0.157 EX=8.5 *Tavg=450.5
 PAIR CA=225 CO=9.0 DI=0.147 EX=8.5 *Tavg=237.5

Reg9B.inp

*-----
 *BEGIN REGION 9B TRANSIENT CARDS & GEOMETRY FOR RHR RETURN FROM VALVE NODE 660 TO PENET. NODE 675
 *-----

*
 OPER CA=1 TE=100 PR=1100
 OPER CA=2 TE=100 PR=50
 OPER CA=3 TE=150 PR=1035
 OPER CA=4 TE=150 PR=1035
 OPER CA=5 TE=150 PR=1035
 OPER CA=6 TE=150 PR=1035
 OPER CA=7 TE=150 PR=1035
 OPER CA=8 TE=150 PR=1035
 OPER CA=9 TE=150 PR=1035
 OPER CA=10 TE=150 PR=1160
 OPER CA=11 TE=150 PR=1160
 OPER CA=12 TE=150 PR=700
 OPER CA=13 TE=150 PR=1035
 OPER CA=14 TE=150 PR=1035
 OPER CA=15 TE=150 PR=195
 OPER CA=16 TE=150 PR=115
 OPER CA=17 TE=150 PR=25
 OPER CA=18 TE=100 PR=100
 OPER CA=19 TE=100 PR=1563
 OPER CA=20 TE=225 PR=25
 OPER CA=21 TE=70 PR=25
 OPER CA=22 TE=150 PR=1035
 OPER CA=23 TE=150 PR=1035
 OPER CA=24 TE=150 PR=225
 OPER CA=25 TE=150 PR=25

*
 TRAN CA=201 IS=1 FS=1 IT=70 FT=100 TT=1800 FL=204 IP=15 FP=1115 TP=0
 TRAN CA=202 IS=1 FS=1 IT=100 FT=100 TT=1800 FL=204 IP=1115 FP=65 TP=0
 TRAN CA=203 IS=1 FS=1 IT=100 FT=150 TT=16164 FL=247 IP=65 FP=1050 TP=0
 TRAN CA=204
 TRAN CA=205

TRAN CA=206
 TRAN CA=207
 TRAN CA=208
 TRAN CA=209
 TRAN CA=210
 TRAN CA=211
 TRAN CA=212 IS=1 FS=1 IT=150 FT=150 TT=0 FL=429 IP=1175 FP=715 TP=0
 TRAN CA=213
 TRAN CA=214
 TRAN CA=215
 TRAN CA=216 IS=1 FS=1 IT=150 FT=150 TT=600 FL=403 IP=210 FP=130 TP=0
 TRAN CA=217
 TRAN CA=218 IS=1 FS=1 IT=225 FT=100 TT=4500 FL=6700 IP=115 FP=115 TP=0
 TRAN CA=219 IS=1 FS=1 IT=100 FT=100 TT=0 FL=247 IP=40 FP=1578 TP=0
 TRAN CA=220 IS=1 FS=1 IT=70 FT=225 TT=60 FL=6700 IP=40 FP=40 TP=0
 TRAN CA=221 IS=1 FS=1 IT=150 FT=70 TT=60 FL=6700 IP=40 FP=40 TP=0
 TRAN CA=222
 TRAN CA=223
 TRAN CA=224 IS=1 FS=1 IT=150 FT=150 TT=0 FL=458 IP=1040 FP=240 TP=0
 TRAN CA=225 IS=1 FS=1 IT=150 FT=150 TT=0 FL=334 IP=240 FP=40 TP=0
 *
 PAIR CA=201 CO=27.6 DI=0.521 EX=6.4 *Tavg=85.0
 PAIR CA=202 CO=27.6 DI=0.512 EX=6.4 *Tavg=100.0
 PAIR CA=203 CO=27.6 DI=0.506 EX=6.4 *Tavg=125.0
 *PAIR CA=204 CO=27.6 DI=0.499 EX=6.4 *Tavg=150.0
 *PAIR CA=205 CO=27.6 DI=0.499 EX=6.4 *Tavg=150.0
 *PAIR CA=206 CO=27.6 DI=0.499 EX=6.4 *Tavg=150.0
 *PAIR CA=207 CO=27.6 DI=0.499 EX=6.4 *Tavg=150.0
 *PAIR CA=208 CO=27.6 DI=0.499 EX=6.4 *Tavg=150.0
 *PAIR CA=209 CO=27.6 DI=0.499 EX=6.4 *Tavg=150.0
 *PAIR CA=210 CO=27.6 DI=0.499 EX=6.4 *Tavg=150.0
 *PAIR CA=211 CO=27.6 DI=0.499 EX=6.4 *Tavg=150.0
 PAIR CA=212 CO=27.6 DI=0.499 EX=6.4 *Tavg=150.0
 *PAIR CA=213 CO=27.6 DI=0.499 EX=6.4 *Tavg=150.0
 *PAIR CA=214 CO=27.6 DI=0.499 EX=6.4 *Tavg=150.0
 *PAIR CA=215 CO=27.6 DI=0.499 EX=6.4 *Tavg=150.0
 PAIR CA=216 CO=27.6 DI=0.499 EX=6.4 *Tavg=150.0
 *PAIR CA=217 CO=27.6 DI=0.499 EX=6.4 *Tavg=150.0
 PAIR CA=218 CO=27.6 DI=0.496 EX=6.4 *Tavg=162.5
 PAIR CA=219 CO=27.6 DI=0.512 EX=6.4 *Tavg=100.0
 PAIR CA=220 CO=27.6 DI=0.500 EX=6.4 *Tavg=147.5
 PAIR CA=221 CO=27.6 DI=0.509 EX=6.4 *Tavg=110.0
 *PAIR CA=222 CO=27.6 DI=0.499 EX=6.4 *Tavg=150.0
 *PAIR CA=223 CO=27.6 DI=0.499 EX=6.4 *Tavg=150.0
 PAIR CA=224 CO=27.6 DI=0.499 EX=6.4 *Tavg=150.0
 PAIR CA=225 CO=27.6 DI=0.499 EX=6.4 *Tavg=150.0

APPENDIX B

PIPESTRESS Output

Output File	Description
Recirc_15.prf	Fatigue results for reduced cycle count
RHR_15.prf	Fatigue results for full 60 year cycle count

Recirc 15.prj

DST COMPUTER SERVICES S. A.

F-4.2

PAGE NO. 3947

++ DST/PIPESTRESS ++

Vermont Yankee

Version 3.5.1+026 PC-EXE

Release: Jun

CALCULATION NUMBER : 3 CODE SECTION III CLASS 1 ASME-1998
 Vermont Yankee Recirculation Fatigue Analysis

RVP

2007/07/26 08:42:12 [42

FATIGUE ANALYSIS AT POINT 600, WELDING TEE 600 TO 602

 INDIVIDUAL STRESS RANGE CHECK

DELTA T1 IN DEGREES F
 STRESSES IN PSI

LOAD SET	PAIR	SALT EQN.14	OCCURENCES		NUMBER USED	SETS ELIMINATED DYNAM.	NO. CYCLES TO FAILURE	USAGE FACTOR	REMARKS
			NI	NJ					
11	21	88675.	10	150	10	11	2801.	0.0036	
			0	140					
14	21	84491.	150	140	140	21	3359.	0.0417	
			10	0					
19	20	67184.	1	150	1	19	8221.	0.0001	
			0	149					
14	27	65876.	10	5	5	27	8900.	0.0006	
			5	0	45				
10	20	62914.	5	149	5	10	10868.	0.0005	
			0	144					
14	20	55493.	5	144	5	14	20012.	0.0002	
			0	139					
9	20	54186.	35	139	35	9	22574.	0.0016	
			0	104					
5.	20	53739.	290	104	104	20	23541.	0.0044	
			186	0					
2	3	46216.	60	150	60	2	50502.	0.0012	
			0	90					
1	26	46169.	60	5	5	26	50782.	0.0001	
			55	0	45				
3	17	42799.	90	150	90	3	76939.	0.0012	
			0	60					
6.	17	42219.	10	60	10	6	82911.	0.0001	
			0	50					
5.	18	42196.	186	150	150	18	83156.	0.0018	
			36	0					
13	17	42131.	5	50	5	13	83864.	0.0001	
			0	45					
8.	17	41392.	35	45	35	8	92408.	0.0004	
			0	10					

File No.: VY-16Q-307

Revision: 0

++ DST/PIPESTRESS ++

Vermont Yankee

Version 3.5.1+026 PC-EXE

Release: Jun

 CALCULATION NUMBER 3 CODE SECTION III CLASS 1 ASME-1998
 Vermont Yankee Recirculation Fatigue Analysis

RVP

2007/07/26 08:42:12 [42

FATIGUE ANALYSIS AT POINT 600, WELDING TEE 600 TO 602

 INDIVIDUAL STRESS RANGE CHECK

 DELTA T1 IN DEGREES F
 STRESSES IN PSI

LOAD SET PAIR	SALT EQN.14	OCCURENCES		NUMBER USED	SETS ELIMINATED DYNAM.	NO. CYCLES TO FAILURE	USAGE FACTOR	REMARKS	
		NI	NJ						
7 17	41326.	10	10	10	7,17	93222.	0.0001		
1 15	38663.	55	150	55	1	133841.	0.0004		
4 16	35177.	290	150	150	16	227086.	0.0007		
4 15	34727.	140	95	95	15	246096.	0.0004		
4 12	25167.	45	10	10	12	1449206.	0.0000		
26 27	23758.	45	45	45	27 26	1748766.	0.0000	DYN. RANGE OF EVENT NO. 1	
4 5	2773.	35	36	35	4	>100000000000.	0.0000		
		0	1						
TOTAL USAGE FACTOR =							0.0590		

 Notes a: Fails
 f: Weld ISI
 j: Rupture Location

File No.: VY-16Q-307

Revision: 0

RHR 15.prf

DST COMPUTER SERVICES S. A.

F-4.2

PAGE NO. 4019

++ DST/PIPESTRESS ++

Vermont Yankee

Version 3.5.1+026 PC-EXE

Release: Jun

 CALCULATION NUMBER 3 CODE SECTION III CLASS 1 ASME-1998
 Vermont Yankee Recirculation Fatigue Analysis

RVP

2007/07/26 08:44:06 [43

FATIGUE ANALYSIS AT POINT 641, SR ELBOW 640 TO 641

 INDIVIDUAL STRESS RANGE CHECK

 DELTA T1 IN DEGREES F
 STRESSES IN PSI

LOAD SET	PAIR	SALT EQN.14	OCCURENCES		NUMBER USED	SETS ELIMINATED DYNAM.	NO. CYCLES TO FAILURE	USAGE FACTOR	REMARKS
			NI	NJ					
10	20	100252.	10	300	10	10	1788.	0.0056	
			0	290					
11	21	68706.	20	300	20	11	7511.	0.0027	
			0	280					
12	20	64893.	20	290	20	12	9456.	0.0021	
			0	270					
20	21	39947.	270	280	270	20	112120.	0.0024	
			0	10					
3	21	35368.	300	10	10	21	219528.	0.0000	
			290	0					
3	17	20439.	290	300	290	3	3190872.	0.0000	
			0	10					
13	17	19221.	10	10	10	13,17	4148766.	0.0000	
			0	0					
14	18	14925.	300	300	300	14,18	>100000000000.	0.0000	
			0	0					
15	19	14877.	300	1	1	19	>100000000000.	0.0000	
			299	0					
1	15	14434.	120	299	120	1	>100000000000.	0.0000	
			0	179					
6	15	12896.	20	179	20	6	>100000000000.	0.0000	
			0	159					
9	15	12506.	70	159	70	9	>100000000000.	0.0000	
			0	89					
4	15	11542.	579	89	89	15	>100000000000.	0.0000	
			490	0					
7	27	10017.	20	5	5	27	>100000000000.	0.0000	
			15	0					
4	16	9993.	490	300	300	16	>100000000000.	0.0000	
			190	0					

File No.: VY-16Q-307

Revision: 0

++ DST/PIPESTRESS ++

Vermont Yankee

Version 3.5.1+026 PC-EXE

Release: Jun

 CALCULATION NUMBER 3 CODE SECTION III CLASS 1 ASME-1998
 Vermont Yankee Recirculation Fatigue Analysis

RVP

2007/07/26 08:44:06 [4]

FATIGUE ANALYSIS AT POINT 641, SR ELBOW 640 TO 641

 INDIVIDUAL STRESS RANGE CHECK

 DELTA T1 IN DEGREES
 STRESSES IN PSI

LOAD SET PAIR	SALT EQN.14	OCCURENCES		NUMBER USED	SETS ELIMINATED	NO. CYCLES TO FAILURE	USAGE FACTOR	REMARKS
		NI	NJ					
2 7	9353.	120	15	15	7	>100000000000.	0.0000	
2 5	9353.	105	579	105	2	>100000000000.	0.0000	
5 26	8235.	474	5	5	26	>100000000000.	0.0000	
26 27	2019.	45	45	45	27 26	>100000000000.	0.0000	DYN. RANGE OF EVENT NO.
4 5	1570.	190	469	190	4	>100000000000.	0.0000	
5 8	1512.	279	70	70	8	>100000000000.	0.0000	
		209	0					
TOTAL USAGE FACTOR =							0.0128	

 Notes a: Fails
 f: Weld ISI
 j: Rupture Location

File No.: VY-16Q-307

Revision: 0



Structural Integrity Associates, Inc.

File No.: VY-16Q-308

NEC-JH_11

CALCULATION PACKAGE

Project No.: VY-16Q

PROJECT NAME:

Environmental Fatigue Analysis of VYNPS

CONTRACT NO.:

10150394

CLIENT:

Entergy Nuclear Operations, Inc.

PLANT:

Vermont Yankee Nuclear Power Station

CALCULATION TITLE:

Core Spray Nozzle Finite Element Model

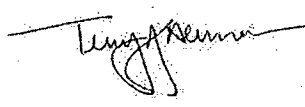

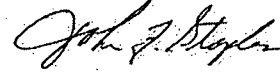
Document Revision	Affected Pages	Revision Description	Project Manager Approval Signature & Date	Preparer(s) & Checker(s) Signatures & Date
0	1-7, Appendix: A1-A17	Initial Issue	Terry J. Herrmann 07/19/2007 	Roland Horvath 07/12/2007  John Staples 07/12/2007 



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1.0 OBJECTIVE

The objective of this calculation is to create a finite element model of the Vermont Yankee Nuclear Power Station Core Spray Nozzle. This model will be used to develop a Green's Function to be used in a subsequent fatigue analysis.

2.0 GEOMETRY / MATERIAL PROPERTIES

A 2-D axisymmetric finite element model (FEM) of the nozzle was developed with element type PLANE82. The developed model includes the part of the pipe, the safe end, the nozzle forging, a portion of the vessel shell, and the cladding. The radius of the vessel in the finite element model was multiplied by a factor of 2 to account for the fact that the vessel portion of the 2D axisymmetric model is a sphere, but the true geometry is a cylinder. The equation for the membrane hoop stress for a sphere is:

$$\sigma = \frac{(\text{pressure}) \times (\text{radius})}{2 \times \text{thickness}}$$

The equation for the membrane hoop stress in a cylinder is:

$$\sigma = \frac{(\text{pressure}) \times (\text{radius})}{\text{thickness}}$$

The factor of two was verified in Reference [11] where actual stress results were compared to the results of this analytical form.

The 2-D axisymmetric FEM was constructed using the dimensions and information from References [1 -8] based on ANSYS [9] finite element software. Figure 1 shows the resulting finite element model.

The materials of the various components of the model are listed below:

- Safe End – SB 166 [1] (72Ni-15Cr-8Fe, N06600)
- 8Ø x 10Ø Conc. Reduction – SA312 TP304 [7] (18Cr-8Ni)
- Nozzle Forging – SA508 Class II [1] (¾ Ni-1/2Mo-1/3 Cr-V)
- Vessel – SA533 Grade B [7] (Mn-1/2Mo-1/2Ni)
- Cladding – SA240 TP 304 [7] (18Cr-8Ni)

Note: In the FEM, the 8Ø x 10Ø Conc. Reduction was modeled as a straight pipe with the material properties of the original design [7]. Later, this piping section was replaced by a new material (SA403 T316L) [10]. These two stainless steels have the same modulus of elasticity and thermal coefficient properties.



Material properties for these materials are based upon the 1998 ASME Code, Section II, Part D, with 2000 Addenda [8] and are shown in Table 1. The properties are taken at an average temperature of 300°F. This average temperature is based on a thermal shock of 500°F to 100°F, which will be applied to the FEM model for Green's Function development.

3.0 PROGRAM INPUT

The input file, VY_CSN_GEOM.inp (included in Appendix A), creates the finite element model for the core spray nozzle.



4.0 REFERENCES

1. Reactor 8 In. Dia. Nozzles Mk. N5A & B, 5920-00624 Rev. 8, SI File No. VY-16Q-207.
2. Core Spray Nozzle Weld Overlay Profile N5A & N5B, 5920-06813, Sh. 1 of 2 Rev. 0, SI File No. VY-16Q-206.
3. N5A/B Thermal Sleeve Details, 5920-00898, Rev 1, SI File No. VY-16Q-206.
4. Special Safe End Forging for Nozzles N2A/B & N5A/B, 5920-00655, Rev. 6, SI File No. VY-16Q-206.
5. Special Forgings for Nozzles N5A & N5B, 5920-00069, Rev 1, SI File No. VY-16Q-206.
6. Core Spray Nozzle Weld Overlay Profile N5A & N5B, 5920-06813, Sh. 2 of 2 Rev. 0, SI File No. VY-16Q-204.
7. CB&I RPV Stress Report, Section S7, "Stress Analysis Core Spray and Flooding Nozzle, Vermont Yankee Reactor Vessel, CB&I Contract 9-6201, SI File No. VY-16Q-206.
8. American Society of Mechanical Engineers, Boiler and Pressure Vessel Code, Section II, Part D, 1998 Edition, 2000 Addenda.
9. ANSYS, Release 8.1 (w/Service Pack 1), ANSYS, Inc., June 2004.
10. "10" x 8" SA403 T316L CONC REDUCER", Page 18 of Attachment 2 of Entergy Design Input Record (DIR) EC No. 1773, Revision 0, "Environmental Fatigue Analysis for Vermont Yankee Nuclear Power Station," 7/3/07, SI File No. VY-16Q-209.
11. SI Calculation No. VY-16Q-309, Revision 0, "Core Spray Nozzle Green's Functions".



Table 1: Material Properties @ 300°F ⁽¹⁾

Material ID	Part Description	Material		Modulus of Elasticity, e+6 psi [EX]	Coefficient of Thermal Expansion, e-6, in/in/°F [ALPX]	Thermal Conductivity, Btu/hr-ft-°F [KXX]	Thermal Diffusivity, ft ² /hr	Specific Heat, Btu/lb-°F [C] ⁽³⁾
2	Safe End	SB 166	72Ni-15Cr-8Fe N06600	29.8	7.9	9.6	0.160	0.1157
2	Weld Overlay	INCONEL 82	72Ni-15Cr-8Fe N06600	29.8	7.9	9.6	0.160	0.1157
1	Nozzle	SA508 Class II	¾ Ni-1/2Mo-1/3 Cr-V	26.7	7.3	23.4	0.401	0.1193
3	Vessel	SA533 Grade B	Mn-1/2Mo-1/2Ni	28.0	7.7	23.4	0.401	0.1193
4	3/16 Clad	SA240 TP 304	18Cr-8Ni	27.0	9.8	9.8	0.160	0.1252
4	8Ø x 10Ø Conc. Reduction ⁽²⁾	SA312 TP304	18Cr-8Ni	27.0	9.8	9.8	0.160	0.1252

Notes:

1. The material properties applied in the analyses are taken from ASME Code, Section II, Part D 1998 Edition, with 2000 information provided in the Design Input Record (page 13 of VY EC No. 1773, SI File No. VY-16Q-209). The use of a for the original design code is acceptable, since later editions typically reflect more accurate material properties than wa editions. Material Properties are evaluated at 300°F from the 1998 ASME Code, 2000 Addenda, Section II, Part D, except for densi assumed typical values [8].
2. In the FEM, the 8Ø x 10Ø Conc. Reduction was modeled as a straight pipe with the material properties of the original d was replaced by a new material (SA403 T316L). These two stainless steels have the same modulus of elasticity and thei
3. Calculated as $[k/(pd)]/12^3$.

File No.: VY-16Q-308

Revision: 0

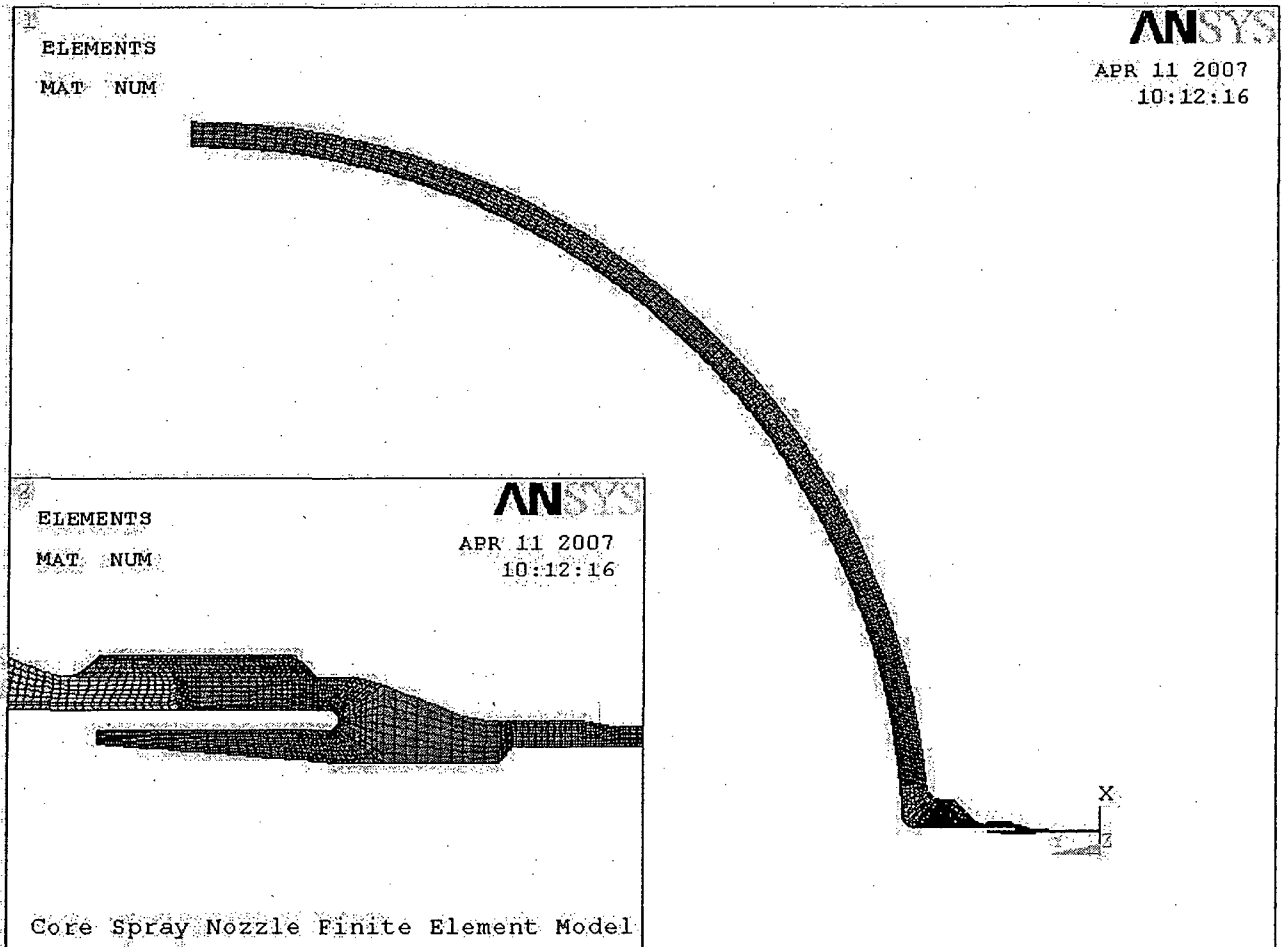


Figure 1: ANSYS Finite Element Model



APPENDIX A
VY_CSN_GEOM.INP

!finish
!/clear,start
!/prep7

et,1,PLANE182,,,1 !Axisymmetric
/com, *****
/com, Material Properties @T=300F
/com, *****
/COM, Material #1 (Nozzle: SA-508 Class II, 3/4Ni-1/2Mo-1/3Cr-V)
mp,ex ,1,26.7E+06
mp,alpx,1,7.3E-06
mp,kxx ,1,23.4 /3600/12
mp,c ,1,0.1193277
mp,nuxy,1,0.3
mp,dens,1,0.283

/COM, Material #2 (Safe End: N06600, Inconel 82 Weld Overlay)
mp,ex ,2,29.8E+06
mp,alpx,2,7.9E-06
mp,kxx ,2,9.6 /3600/12
mp,c ,2,0.1157407
mp,nuxy,2,0.29
mp,dens,2,0.3

/COM, Material #3 (Vessel: SA-533 Grade B, Mn-1/2Mo-1/2Ni)
mp,ex ,3,28.00E+06
mp,alpx,3,7.7E-06
mp,kxx ,3,23.4 /3600/12
mp,c ,3,0.1193277
mp,nuxy,3,0.3
mp,dens,3,0.283

/COM, Material #4 (3/16 Clad: SA-240 TP304, 8-10 Diam. Conc. Red.: SA-312 TP 304, Thermal Sleeve: SA-312 TP304)
mp,ex ,4,27.0E+06
mp,alpx,4,9.8E-06
mp,kxx ,4,9.8 /3600/12
mp,c ,4,0.1252495
mp,nuxy,4,0.3
mp,dens,4,0.283

/com, *****
/com, Geometric Parameters
/com, *****
*AFUN,deg



/com, pipe parameters

*set, pID, 9.834
*set, pOD, 10.815
*set, pL, 8

k, 1, pID/2, 0
k, 2, POD/2, 0
k, 3, POD/2, pL
k, 4, PID/2, pL

l, 1, 2
l, 2, 3
l, 3, 4
l, 4, 1

/com, *****

/com, Safe End Parameters

/com, *****

*set, seBX, pL
*set, seID01, 9.834
*set, seID02, 9
*set, seID03, $9 + 31/32$
*set, seID04, $11 + 3/4$

*set, seOD01, 10.815
*set, seOD02, $11 + 1/6$
*set, seOD03, $13 + 27/64$
*set, seOD04, $10 + 11/16$

*set, seL01, $3 + 1/32$
*set, seL02, $7/8$
*set, seL03, $1 + 11/16$
*set, seL04, $13/32$
*set, seL05, 4
*set, seL06, $3 + 1/2$
*set, seL07, $12 + 4 + 1/16$
*set, seL08, $seL07 - (seL01 + seL02 + seL03 + seL04 + seL05 + seL06)$
*set, seR01, 3
*set, seR02, $3/4$
*set, seR03, $1/4$
*set, seR04, $1/8$

k, 5, seOD01/2, seBX+seL01
k, 6, seOD02/2, seBX+seL01+seL02



k, 7, seOD02/2, seBX+seL01+seL02+seL03+seL04 +.496
k, 8, (seOD02+seOD03)/4, seBX+seL01+seL02+seL03+seL04+seL05/2
k, 9, seOD03/2, seBX+seL01+seL02+seL03+seL04+seL05
k, 10, seOD03/2, seBX+seL01+seL02+seL03+seL04+seL05+seL06
k, 11, seID04/2, seBX+seL01+seL02+seL03+seL04+seL05+seL06
k, 12, seID04/2, seBX+seL01+seL02+seL03+seL04+seL05
k, 13, seOD04/2, seBX+seL01+seL02+seL03+seL04+seL05
k, 14, seOD04/2, seBX+seL07
k, 15, seID03/2, seBX+seL07
k, 16, seID02/2, seBX+seL01+seL02+seL03+seL04+seL05
k, 17, seID02/2, seBX+seL01+seL02+seL03+seL04
k, 18, seID01/2, seBX+seL01+seL02+seL03

l, 3, 5
l, 5, 6
l, 6, 7
l, 9, 10
l, 10, 11
l, 11, 12
l, 12, 13
l, 13, 14
l, 14, 15
l, 15, 16
l, 16, 17
l, 17, 18
l, 18, 4

k, 19, seOD02/2+seR01, seBX+seL01+seL02+seL03+seL04 +.496
k, 8, seOD02/2+seR01, seBX+seL01+seL02+seL03+seL04+seR01 +.496
larc, 7, 8, 19, seR01

k, 20, seOD03/2-seR01, seBX+seL01+seL02+seL03+seL04+seL05
k, 21, seOD03/2-seR01, seBX+seL01+seL02+seL03+seL04+seL05-seR01
larc, 9, 21, 20, seR01
L2ANG,19,18,0,0,,,
ldele, 20, 21, ,1

lfillt, 5, 6, seR02
lfillt, 6, 7, seR02
lfillt, 10, 11, seR03
lfillt, 11, 12, seR03
lfillt, 15, 16, seR04
lfillt, 16, 17, seR04

/com, weld 1/8 gap
*set, wgap, 1/8



k, 40, seOD03/2, seBX+seL01+seL02+seL03+seL04+seL05+seL06 + wgap
k, 41, seID04/2, seBX+seL01+seL02+seL03+seL04+seL05+seL06 + wgap

l, 10, 40
l, 40, 41
l, 41, 11

/com, *****
/com, Nozzle
/com, *****

*set, nID01, seOD01
*set, nOD01, seOD03
*set, nOD02, 24+1/4
*set, nOD03, 2*12+7.25
*set, nOD04, 2*12+7.25-1-1/8

*set, nL01, 4+5/16
*set, nL02, 5+3/8
*set, nL03, 5+1/8+5+5/8
*set, nW01, 1/16
*set, wClad, 3/16
*set, wReactor, 5+5/8-wClad
*set, nL04, 7/16
*set, nR01, 1/4
*set, nR02, 3/16
*set, nR03, (8*12+7)*2
*set, nR04, 2.5
*set, nR05, nR04-wClad
*set, nR06, 3.5
*set, nR07, 3 +7/8
*set, nR08, 0.5

K, 42, KX(11) + nW01, KY(11)
K, 43, KX(41) + wClad, KY(41)+nL04+nR01
K, 44, KX(43) + nR01, KY(43)
K, 46, KX(44) + nR01*sin(15), KY(44)-nR01*cos(15)
K, 47, KX(46) + 10*nR01*cos(15), KY(46)+10*nR01*sin(15)
K, 48, KX(43), KY(43) + 24
K, 49, KX(41), KY(41) + 24
K, 50, KX(40), KY(40) + nL01
K, 51, KX(44) + (nR01+wClad)*sin(15), KY(44)-(nR01+wClad)*cos(15)
K, 52, KX(51) + 10*nR01*cos(15), KY(51)+10*nR01*sin(15)
K, 53, KX(51) - 10*nR01*cos(15), KY(51)-10*nR01*sin(15)
K, 54, KX(42), KY(42)+wClad*2
larc, 43, 46, 44, nR01



L, 46, 47
L, 43, 48
L, 41, 49
L, 40, 50
L, 53, 52
L, 42, 54

LOVLAP, 35,36
LDELE, 39, 40,,0

LOVLAP, 31,34,38
LDELE, 40,42,2,0

lfillt, 37, 35, nR02

K, 60, nOD02/2, KY(40) + nL01+nL02
K, 61, nOD02/2, KY(40) + nL01+nL02+nL03
K, 62, 0, KY(40) + nL01+nL02+nL03+nR03
K, 63, 0, KY(62) - nR03
K, 64, nR03, KY(62)
K, 65, 0, KY(63)-wClad
K, 66, nR03+wClad, KY(62)
K, 67, 0, KY(65)-wReactor
K, 68, nR03+wReactor, KY(62)

LARC, 63, 64, 62, nR03
LARC, 65, 66, 62, nR03+wClad
LARC, 67, 68, 62, nR03+wReactor

L, 64, 66
L, 66, 68

LOVLAP, 34, 33
LDELE, 46,47

LOVLAP, 32, 38
LDELE, 34
LDELE, 46

LFILLT,45,48,nR04, ,
LFILLT,33,47,nR05, ,

L, 50, 60
L, 60, 61

LOVLAP, 40, 46



LDELE, 50, 51
LFILLT, 49, 52, nR06
LFILLT, 38, 41, nR07
LFILLT, 49, 38, nR08

/com, Nozzle and Vessel border

K, 80, nOD03/2, KY(60)+2*nL03
K, 81, nOD03/2, KY(60)
K, 82, nOD04/2, KY(60)+2*nL03
K, 83, nOD04/2, KY(60)
L, 80, 81
L, 82, 83

LPTN, 53, 48.
LPTN, 51, 52
LDELE, 56, 59, 1, 0
LSTR, 76, 75
LPTN, 51, 47
KL, 40, 0.5, ,
KL, 34, 0.5, ,
KL, 32, 0.5, ,

LSTR, 78, 79
LSTR, 79, 84

K, 90, KX(73)+wReactor*2*cos(160), KY(73)+wReactor*2*sin(160)
L, 73, 90
LPTN, 59, 33
LPTN, 63, 45
LDELE, 65

K, 91, KX(71)+wReactor*2*cos(170), KY(71)+wReactor*2*sin(170)
L, 71, 91
LPTN, 45, 60
LPTN, 33, 67
LDELE, 69
KCENTER, KP, 69, 78, 70, 0
LSTR, 89, 58
LSTR, 89, 57

LPTN, 40, 33, 67
LDELE, 73, 74

L, 58, 56
L, 57, 55



/com, *****

/com, Weld Overlay

/com, *****

*set, woA, 3.100

*set, woB, 0.781

*set, woC, 2.500

*set, woD, 3.734

*set, woE, 3.480

*set, woF, 6.310

*set, woG, 8.313

*set, woH, 0.535

*set, woR01, 7/16

K, 80, KX(40), KY(40)-wgap/2-woA

K, 81, KX(80)+woH, KY(80)+woH

K, 83, KX(40), KY(40)-wgap/2+woB/2+woC

K, 82, KX(83)+woH, KY(83)-woH

L, 80, 81

L, 81, 82

L, 82, 83

LPTN, 74, 46

LDELE, 79

LFILLT, 78, 76, woR01, ,

LSTR, 94, 96

/com, *****

/com, Heat transfer coef. points

/com, *****

*set, tsL01, 2.25

*set, tsL02, 3.5

K, 100, KX(41), KY(11)+seL08+tsL01

K, 101, KX(41)+wClad, KY(11)+seL08+tsL01

K, 102, KX(41), KY(11)+seL08+tsL01+tsL02

K, 103, KX(41)+wClad, KY(11)+seL08+tsL01+tsL02

L, 100, 101

L, 102, 103

LDELE, 51

LDELE, 47



K, 104, $KX(103)+wReactor*\cos(-20)$, $KY(103)+wReactor*\sin(-20)$
K, 105, $KX(101)+wReactor*\cos(-10)$, $KY(101)+wReactor*\sin(-10)$
L, 103, 104
L, 101, 105
LPTN, 38, 47
LPTN, 76, 51

LDELE, 86
LDELE, 84
LDELE, 65
LDELE, 68

LDELE, 63
LDELE, 45
LDELE, 66
LDELE, 60

LSTR, 43, 101
LSTR, 101, 103
LSTR, 103, 85
LSTR, 86, 102
LSTR, 102, 100
LSTR, 100, 41

LDIV,30,0.5, ,2,0
K, 106, $KX(99)+wReactor*\cos(200)$, $KY(99)+wReactor*\sin(200)$
K, 107, $KX(38)+wReactor*\cos(160)$, $KY(38)+wReactor*\sin(160)$
L, 99, 106
L, 38, 107
LPTN, 66, 84
LPTN, 88, 76

LDELE, 89,90
LSTR, 99, 38
LDELE, 28
LSTR, 26, 9
LSTR, 29, 16

LCOMB,11,23,0
LCOMB,11,24,0
LDIV,11, , ,3,0

K, 110, $KX(22)+wReactor*\cos(180)$, $KY(22)+wReactor*\sin(180)$
L, 110, 22
LPTN, 15, 89
LDELE, 94



LSTR, 28, 111
 LSTR, 27, 22
 LSTR, 17, 7
 K, 112, KX(33)+wReactor, KY(33)
 L, 33, 112
 LPTN, 7, 95
 LDELE, 99
 K, 114, KX(25)+wReactor*cos(180), KY(25)+wReactor*sin(180)
 L, 114,25
 K, 115, KX(8)+wReactor*cos(180), KY(8)+wReactor*sin(180)
 L, 115, 8
 LPTN, 95,17
 LPTN, 7,101
 LDELE, 102,103

/com, *****
 /com, Creating Areas and Meshing
 /com, *****
 allsel,all,all

MSHKEY,1 ! MAPPED MESHING
 AL,1,2,3,4
 MAT,4 ! Pipe
 LESIZE,1,,,8
 LESIZE,3,,,8
 LESIZE,2,,,20
 LESIZE,4,,,20
 AMESH, 1

MAT,2 ! Safe End
 AL, 3, 5, 100, 99
 LESIZE,3,,,8
 LESIZE,100,,,8
 LESIZE,5,,,20
 LESIZE,99,,,20
 AMESH, 2

LCOMB, 20,6,0
 LCOMB, 6,21,0
 AL, 100, 6, 17, 104
 LESIZE,100,,,8
 LESIZE,17,,,8
 LESIZE,6,,,10
 LESIZE,104,,,10
 AMESH, 3

AL, 17, 97, 98, 95
LESIZE,17,,,8
LESIZE,98,,,8
LESIZE,97,,,10
LESIZE,95,,,10
AMESH, 4

LDELE, 94
LSTR, 7, 30
LCOMB, 26,16,0
LCOMB, 16,25,0
AL, 98, 96, 7, 16
LESIZE,98,,,8
LESIZE,7,,,8
LESIZE,96,,,8
LESIZE,16,,,8
AMESH, 5

LCOMB, 18, 22
AL, 7, 18, 92, 93
LESIZE,7,,,8
LESIZE,92,,,8
LESIZE,18,,,10
LESIZE,93,,,10
AMESH, 6

AL, 92, 89, 23, 15
LESIZE,92,,,8
LESIZE,23,,,8
LESIZE,89,,,8
LESIZE,15,,,8
AMESH, 7

AL, 15, 24, 88, 90
LESIZE,15,,,8
LESIZE,24,,,8
LESIZE,88,,,8
LESIZE,90,,,8
AMESH, 8

AL, 89, 19, 28, 11
LESIZE,89,,,8
LESIZE,19,,,8
LESIZE,28,,,8

LESIZE,11,,,8
AMESH, 9

AL, 88, 12, 13, 14
LESIZE,88,,,8
LESIZE,13,,,8
LESIZE,12, , ,28,5, , ,1
LESIZE,14, , ,28,0.2, , , ,1
AMESH, 10

K, 118, KX(80)+wReactor*cos(180), KY(80)+wReactor*sin(180)
L, 118, 80
LPTN, 10,20,8
LDELE, 101
AL, 28, 21, 94 , 26
LESIZE,28,,,8
LESIZE,94,,,8
LESIZE,21,,,6
LESIZE,26,,,6
AMESH, 11

LDELE, 9
LSTR, 42, 11
LSTR, 42, 10

LESIZE,8, , ,2, , , , ,1
LESIZE,9, , ,6, , , , ,1
LESIZE,22, , ,20,0.2, , , ,1
LESIZE,25, , ,20,0.2, , , ,1
AL, 94, 22, 9, 8, 25
AMAP,12,11,10,80,21

LCOMB, 37, 31
LCOMB, 27, 36
AL, 9, 27, 35, 31
LESIZE,35,,,6
LESIZE,27,,,4
LESIZE,31,,,4
AMESH, 13

MAT,4 ! Clad
LCOMB, 68, 39

LCOMB, 29, 87
AL, 8, 31, 86, 29
LESIZE, 8,,,2
LESIZE, 86,,,2
LESIZE, 31,,,4
LESIZE, 29,,,4
AMESH, 14

AL, 35, 43, 39, 76
LESIZE, 35,,,6
LESIZE, 39,,,6
LESIZE, 43,,,4
LESIZE, 76,,,4
AMESH, 15

AL, 86, 76, 66, 91
LESIZE, 86,,,2
LESIZE, 66,,,2
LESIZE, 76,,,4
LESIZE, 91,,,4
AMESH, 16

MAT, 1 ! Nozzle
LCOMB, 41, 77,
LCOMB, 41, 74,
LCOMB, 41, 47,
LDELE, 41
LDELE, 47

LESIZE, 45, , , 19, , , , 1
LESIZE, 30, , , 1, , , , 1
LESIZE, 10,,,20
LESIZE, 85,,,6
AL, 39, 10, 85, 45, 30
AMAP, 17, 101, 98, 36, 99

MAT, 4 ! Clad
LESIZE, 79,,,2
LESIZE, 84,,,20
AL, 66, 30, 45, 79, 84
AMAP, 18, 100, 101, 99, 109

MAT, 1 ! Nozzle
LCOMB, 38, 81
LESIZE, 38,,,14



LESIZE, 83,,,6
LESIZE, 51,,,14
AL, 85, 38, 83, 51
AMESH, 19

MAT,4 ! Clad
LESIZE, 80,,,2
LESIZE, 65,,,14
AL, 79, 51, 80, 65
AMESH, 20

MAT,1 ! Nozzle
LCOMB, 82, 50
LESIZE, 50,,,20
LESIZE, 62,,,6
LESIZE, 60,,,20
AL, 83, 50, 62, 60
AMESH, 21

MAT,4 ! Clad
LESIZE, 64,,,2
LESIZE, 63,,,20
AL, 80, 60, 64, 63
AMESH, 22

MAT,1 ! Nozzle
LCOMB, 49, 71
LESIZE, 49,,, 20
LESIZE, 69,,,6
LESIZE, 61,,,20
AL, 62,49,69,61
AMESH, 23

MAT,4 ! Clad
LESIZE, 40,,,2
LESIZE, 59,,,20
AL, 64, 61, 40, 59
AMESH, 24

MAT,1 ! Nozzle
LESIZE, 75,,,6
LESIZE, 70,,,6
LESIZE, 34,,,6
AL, 69, 75, 70, 34
AMESH, 25



MAT,4 ! Clad
LESIZE, 33,,,2
LESIZE, 32,,,6
AL, 40, 34, 33, 32
AMESH, 26

MAT,1 ! Nozzle
LCOMB, 53, 72
LESIZE, 53,,,8
LESIZE, 58,,,6
LESIZE, 52,,,8
AL, 70, 53, 58, 52
AMESH, 27

MAT,4 ! Clad
LESIZE, 57,,,2
LESIZE, 54,,,8
AL, 33, 52, 57, 54
AMESH, 28

MAT,3 ! Vessel
LESIZE, 57,,,2
LESIZE, 54,,,8
LESIZE, 48,,,100,0.2, , , 1
LESIZE, 55,,,100,0.2, , , 1
LESIZE, 56,,,100,0.2, , , 1
AL, 48, 44, 56, 58
AMESH, 29

MAT,4 ! Clad
LESIZE, 42,,,2
AL, 57, 56, 42, 55
AMESH, 30

MAT,1 ! Nozzle
ACLEAR, 17
ADELE, 17
LOVLAP,10, 46
NUMMRG,KP, , , ,LOW
LCOMB,37,41 ,0
AL, 20, 37, 85, 45, 30, 39
AMAP,17,101,98,36,99

MAT,2 ! Safe End
LCOMB, 36, 78
LESIZE, 67,,6
LESIZE, 36,,6, 0.2,,1
AL, 67, 73,36,20,43,27,22
AMAP,31,23,82,81,80

/COM,*****
/COM, HTC point of Region 3
/COM,*****
ACLEAR, 4
ADELE, 4
LDELE, 95
LDELE, 97
K, 120, KX(18), KY(18) - 3/8
K, 121, KX(25), KY(120)
L, 25, 121
L, 121, 113
L, 117, 120
L, 120, 33
L, 120, 121

MAT,2 ! Safe End
AL, 17, 10, 68, 46
LESIZE,10,,12
LESIZE,68,,8
LESIZE,46,,12
AMESH, 4

AL, 68, 41, 98, 47
LESIZE,41,,4
LESIZE,98,,8
LESIZE,47,,4
AMESH, 32

/COM,*****
/COM, HTC point of Region 7
/COM,*****
MAT,4 ! Clad
ACLEAR, 18
ADELE, 18
K, 122, KX(14)+wReactor, KY(14)
L, 14,122
LSBL, 84, 71

LESIZE, 72,,,10
LESIZE, 74,,,10
AL, 66, 30,45,79,72,74
AMAP,18,100,101,99,109

/COM,*****
/COM, HTC point of Region 3
/COM,*****
MAT,2 ! Safe End
ACLEAR, 2
ADELE, 2
K, 123, KX(120), KY(120)-3
K, 124, KX(4), KY(4)+1+1/16
LDELE, 99
L, 4, 124
L, 124, 123
L, 123, 116
AL, 3, 5, 100, 78, 77, 71
AMAP,2,116,8,3,4

/COM,*****
/COM, Define DOF constraints on lines
/COM,*****
DL,42, ,SYMM
DL,44, ,SYMM

FLST,4,9,1,ORDE,2
FITEM,4,1
FITEM,4,-9
CP,1,UY,P51X



Structural Integrity Associates, Inc.

File No.: VY-16Q-309

NEG-JH_12

CALCULATION PACKAGE

Project No.: VY-16Q

PROJECT NAME:

Environmental Fatigue Analysis of VYNPS

CONTRACT NO.:

10150394

CLIENT:

Entergy Nuclear Operations, Inc.

PLANT:

Vermont Yankee Nuclear Power Station

CALCULATION TITLE:

Core Spray Nozzle Green's Functions

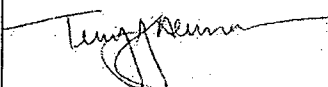
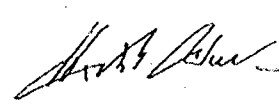
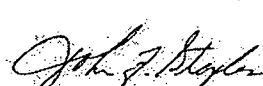
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1.0 OBJECTIVE

The objective of this calculation is to compute the pressure stresses, thermal stresses, and the Green's Functions for high (100%) and no (0%) flow thermal loading of the Vermont Yankee Nuclear Power Station Core Spray Nozzle.

2.0 CORE SPRAY NOZZLE MODEL DESCRIPTION

An axisymmetric finite element model of the core spray nozzle was developed in Reference [1] using ANSYS [2]. The geometry used in Reference [1] was utilized in this calculation. The material properties are taken at an average temperature of 300°F. This average temperature is based on a thermal shock of 500°F to 100°F, which will be applied to the FE model for Green's Function development. Table 1 lists the material properties at 300°F. The meshed model is shown in Figure 1.

Table 1: Material Properties @ 300°F ⁽¹⁾

Part Description	Material		Modulus of Elasticity, e+6 psi [EX]	Coefficient of Thermal Expansion, e-6, in/in/°F [ALPX]	Thermal Conductivity, Btu/hr-ft-°F [KXX]	Thermal Diffusivity, ft ² /hr	Specific Heat, Btu/lb-°F [C] ⁽³⁾	Poisson's Ratio [NU]
Safe End	SB 166	72Ni-15Cr-8Fe N06600	29.8	7.9	9.6	0.160	0.1157	0.2
Weld Overlay	INCONEL 82							
Nozzle	SA508 Class II	¾ Ni-1/2Mo-1/3 Cr-V	26.7	7.3	23.4	0.401	0.1193	0.2
Vessel	SA533 Grade B	Mn-1/2Mo-1/2Ni	28.0	7.7	23.4	0.401	0.1193	0.2
3/16 Clad	SA240 TP 304	18Cr-8Ni	27.0	9.8	9.8	0.160	0.1252	0.2
8Ø x 10Ø Conc. Reduction ⁽²⁾	SA312 TP304							
Thermal Sleeve	SA312 TP304							

Notes:

1. The material properties applied in the analyses are taken from ASME Code, Section II, Part D 1998 Edition, with 2000 Addenda information provided in the Design Input Record (page 13 of VY EC No. 1773, SI File No. VY-16Q-209). The use of a later edition for the original design code is acceptable, since later editions typically reflect more accurate material properties than was used. Material Properties are evaluated at 300°F from the 1998 ASME Code, 2000 Addenda, Section II, Part D, except for density and Poisson's ratio values [3].
2. In the FEM, the 8Ø x 10Ø Conc. Reduction was modeled as a straight pipe with the material properties of the original design. It was replaced by a new material (SA403 T316L). These two stainless steels have the same modulus of elasticity and thermal expansion.
3. Calculated as $[k/(pd)]/12^3$.

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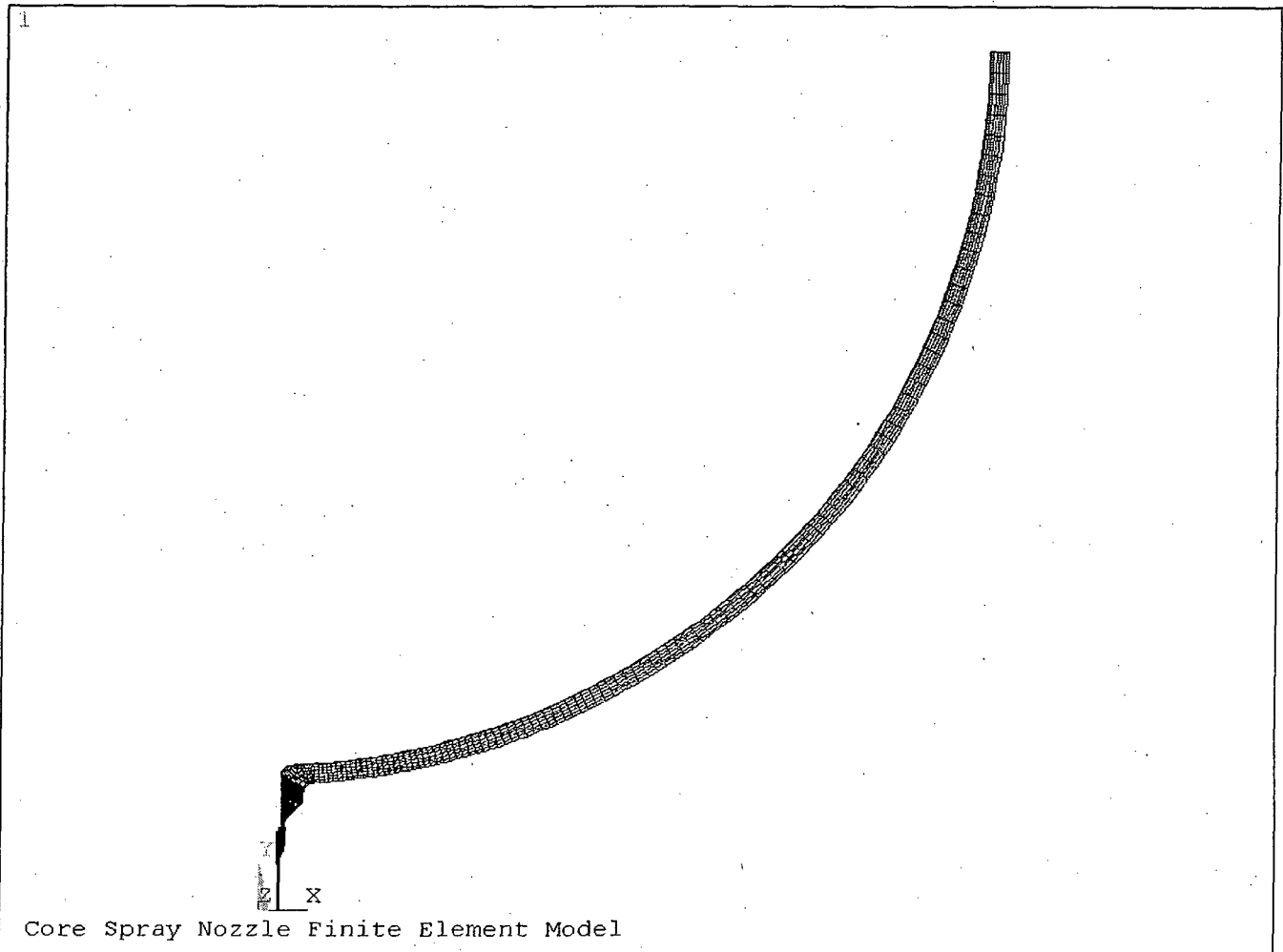


Figure 1: ANSYS Finite Element Model



3.0 APPLIED LOADS

Both pressure and thermal loads were applied to the finite element model.

3.1 Pressure Load

A uniform pressure of 1000 psi was applied along the inside surface of the core spray nozzle and the reactor vessel wall (Figure 2). A pressure load of 1000 psi was used because it is easily scaled up or down to account for different pressures that occur during transients. In addition, a cap load was applied to the piping at the end of the nozzle. This cap load was calculated as follows:

$$P_{CAP} = \frac{P \cdot D_i^2}{D_o^2 - D_i^2}$$

where:

- P = Pressure = 1,000 psi
- D_i = Inside Diameter = 9.834 in
- D_o = Outside Diameter = 10.815 in

Therefore, the cap load is 4,774 psi. The calculated value was given a negative sign in order for it to exert tension on the end of the model. The nodes on the end of the safe end are coupled in the axial direction (UY, Figure 4) to ensure mutual displacement of the end of the nozzle due to attached piping. The boundary conditions at the end of the modeled portion of the reactor pressure vessel wall constructed to be "symmetric" (Figure 3).

The ANSYS input file VY_16Q_P.inp generates the core spray nozzle geometry from VY_CSN_Geom.inp [1] and performs the internal pressure load case just described. Figure 2, 3 and 4 show the internal pressure distribution, cap load, and symmetry conditions applied to the vessel end of the model, respectively.

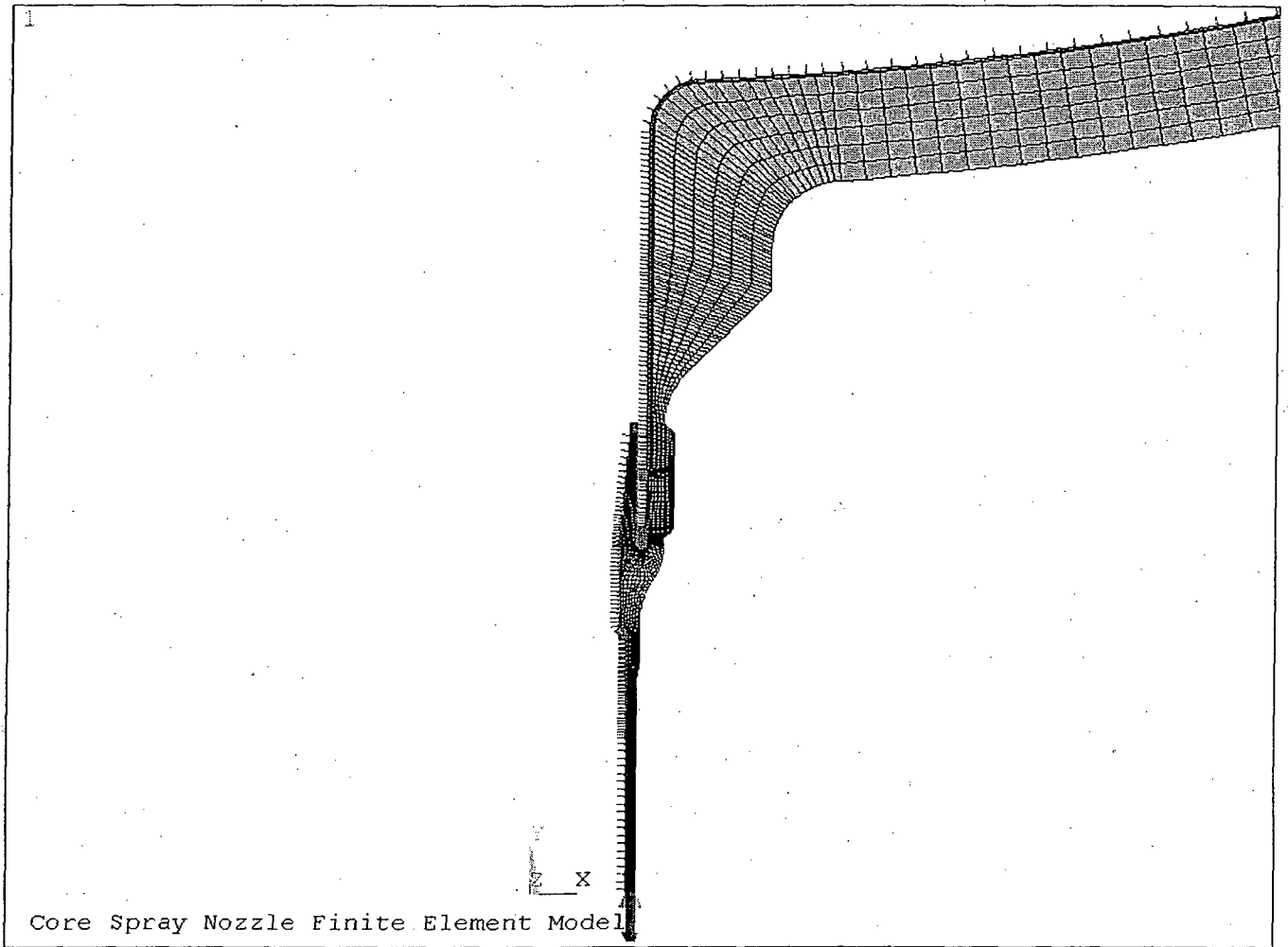


Figure 2: Core Spray Nozzle Internal Pressure Distribution

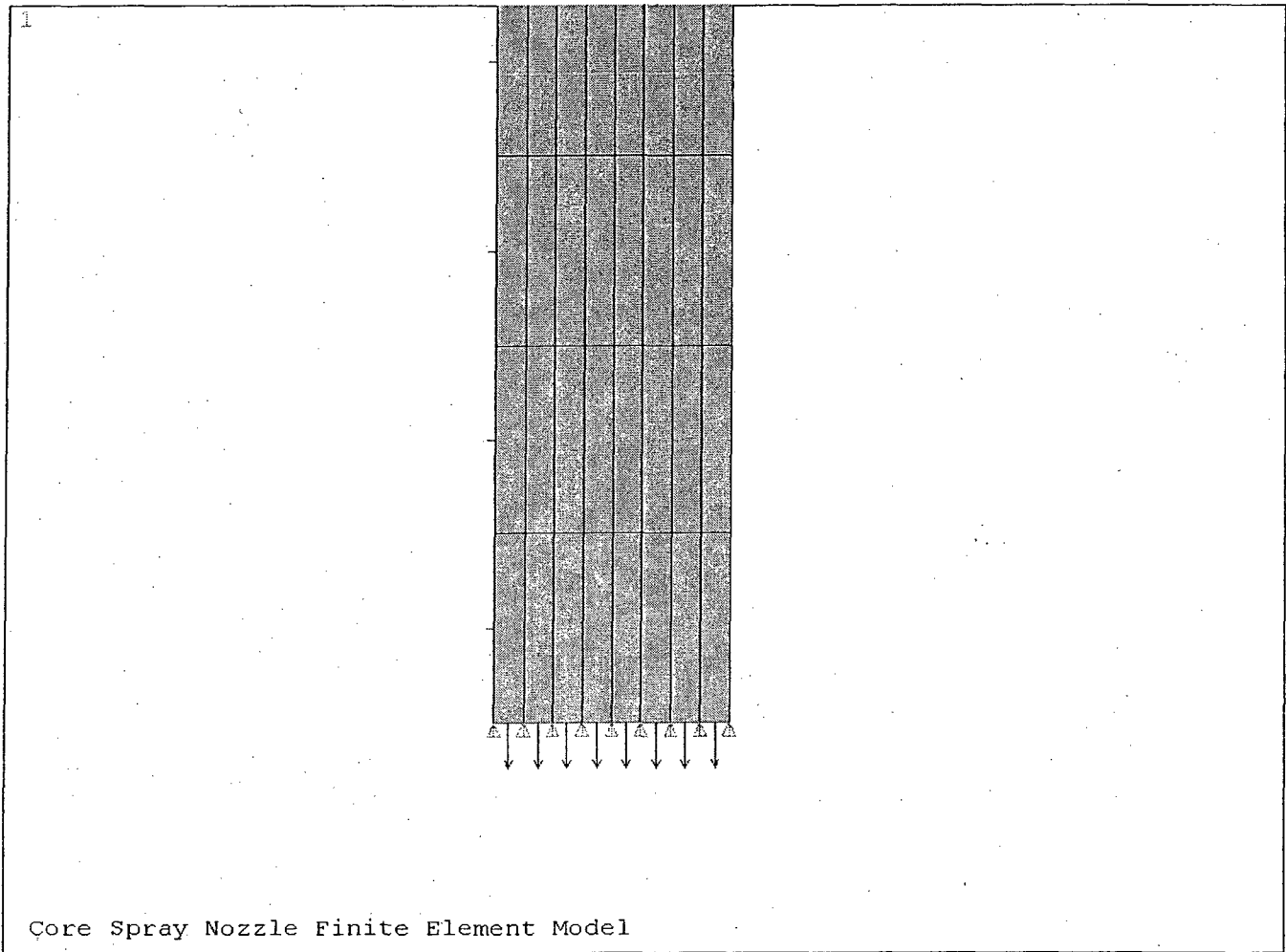


Figure 3: Core Spray Nozzle Pressure Cap Load

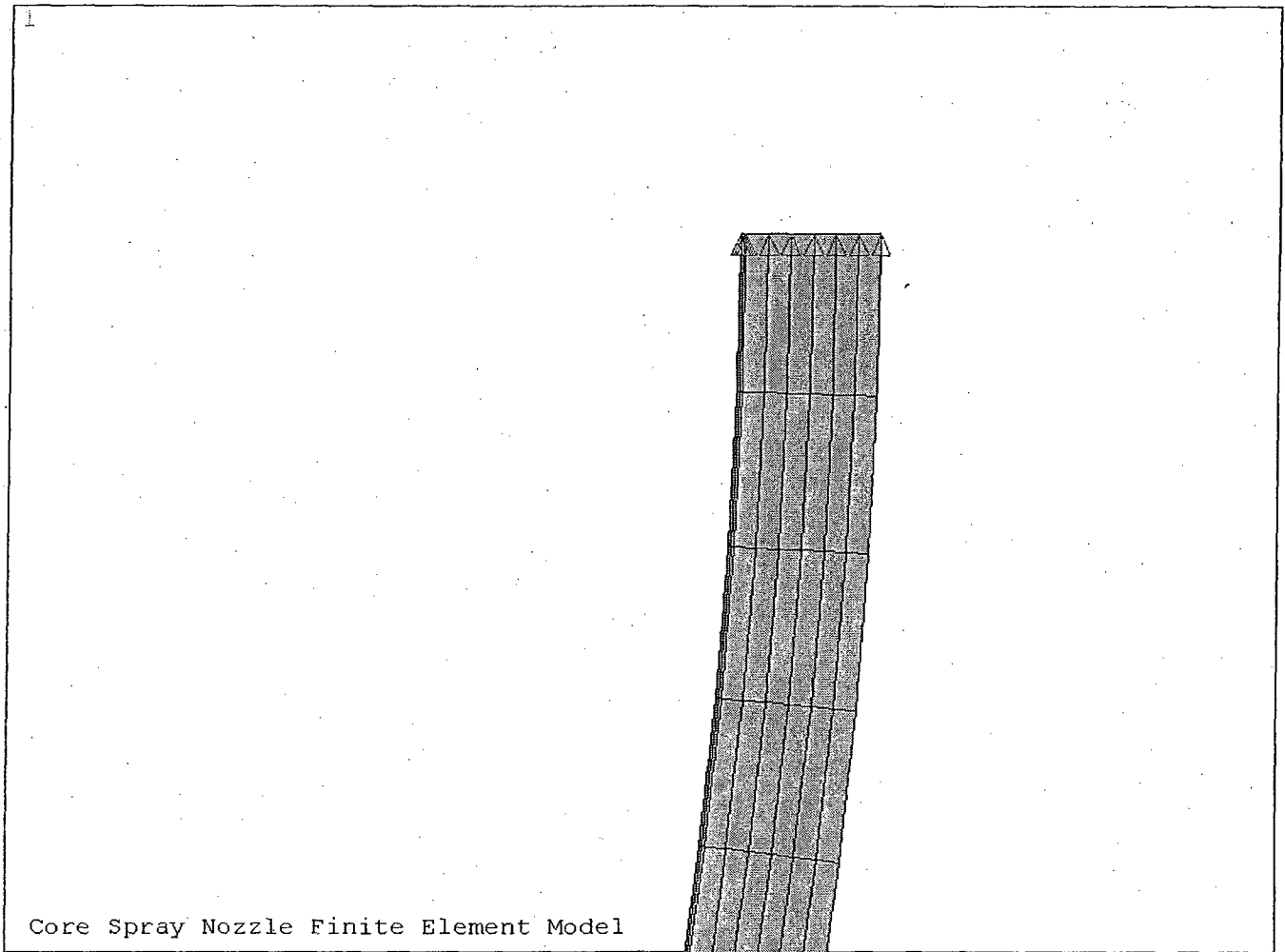


Figure 4: Core Spray Nozzle Vessel Wall Boundary Conditions



3.2 Thermal Load

Thermal loads are applied to the core spray nozzle model. The heat transfer coefficients (HTC) were determined using the methodology in the Excel spreadsheet "Heat Transfer Coefficients.xls", which is included in the project files. The HTCs were determined for various regions of the core spray FEM, (see Figure 5) for two different flow cases. The flow cases are for 100% (3200 gpm [6]) and 0% core spray flow through the nozzle.

The 0% flow case simulates a stagnant condition of the core spray nozzle when not in operation (i.e., the entire core spray nozzle is at the same temperature as the reactor pressure vessel due to reflooding). The HTCs for the no flow case are for free convection (stagnant) at the temperature of the reactor pressure vessel 500°F. The applied boundary fluid temperature is changed to simulate a thermal shock from 500°F to 100°F to develop the stress response on the core spray nozzle in the stagnant condition.

The 100% flow case simulates operational condition of the core spray nozzle (i.e., the entire core spray nozzle experiences 100°F water due to injection). The HTCs for the high flow case are for forced and free convection depending on the region of the FEM. The applied boundary fluid temperature is changed to simulate a thermal shock from 500°F to 100°F to develop the stress response on the core spray nozzle due to injection.

For both Green's Functions, a 500°F – 100°F thermal shock was run to determine the stress response. For the 0% flow case, the entire inside surface of the FEM was shocked. For the 100% flow case, only the nozzle flow path was shocked.

3.2.1 Boundary Fluid Temperatures

For the Green's Functions, a 500°F – 100°F thermal shock was run to determine the stress response to a degree change in temperature. The temperature on the exterior of the reactor, nozzle, safe end and the pipe is assumed to be 120 °F (ambient).

3.2.2 Heat Transfer Coefficients

Figure 5 shows where the heat transfer coefficients were applied to the FEM for the 0% (steady-state) and 100% core spray flow injection load case. For all the regions, the applied heat transfer coefficients and the initial temperatures are summarized in Table 2. The heat transfer coefficient for outside the reactor vessel wall is 0.2 BTU/hr-ft²-°F and the heat transfer coefficient for inside the reactor vessel wall is 500 BTU/Hr-ft²-°F, from page I-T7-5 of Reference [8].

Table 3 through Table 12 show the excel spreadsheets to calculate the HTC for regions 1, 3, 5, 7, and 9 respectively. These tables calculate the HTC for a certain part of the nozzle using the geometry of the bounding piping, the flow rate, and other physical fluid parameters. These tables calculate the Reynolds, Grashof and Rayleigh numbers in order to determine the HTC for inside surface/annulus forced and natural convection [4]. For several regions, the resultant HTCs had to be calculated from the partial heat transfer coefficients. These resultant HTCs are summarized in Table 13. In regions 2, 4, 6, 8, and 10 the HTCs are interpolated because of the complexity of the material profile.

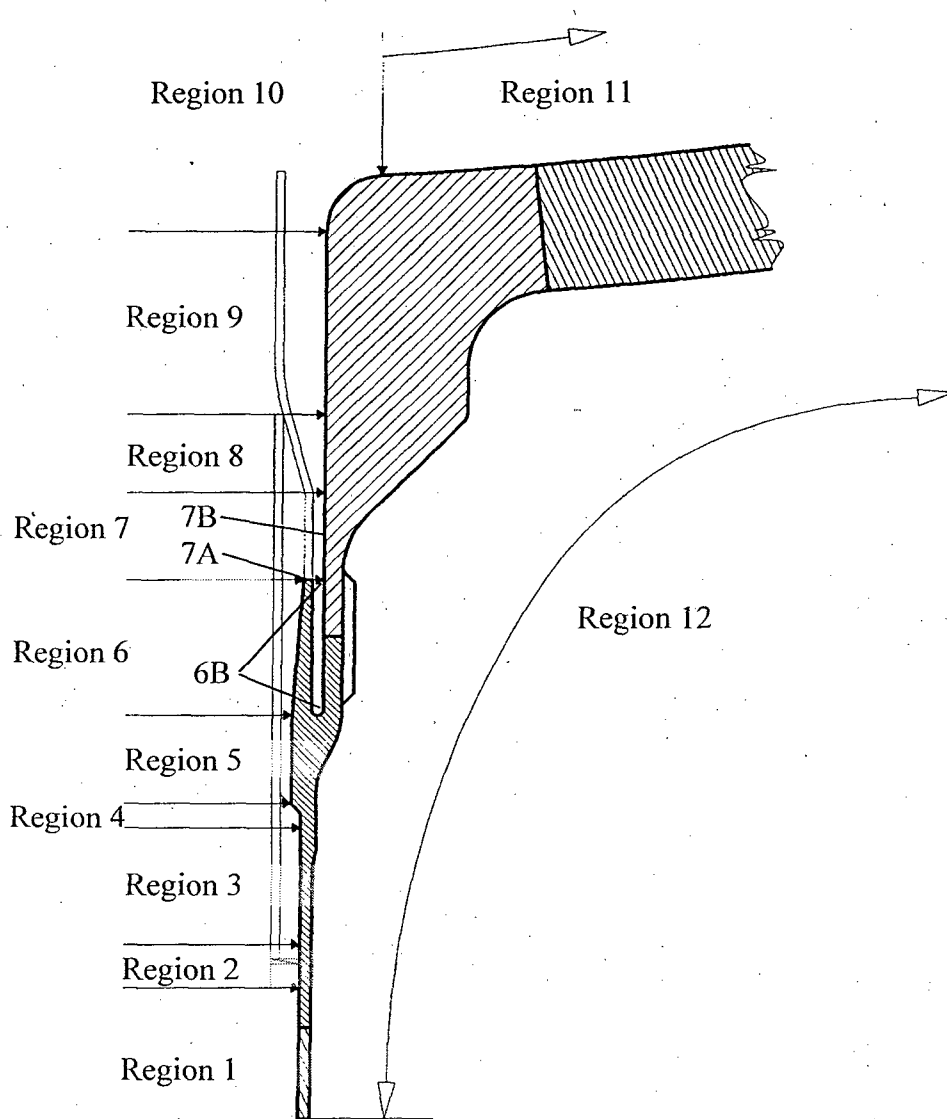


Figure 5: Nozzle and Vessel Wall Thermal and Heat Transfer Boundaries (not to scale)

Table 2: Heat Transfer Coefficients

Regions	0% Flow		100% Flow	
	Initial Temperature °F	HTC Btu/hr-ft ² -°F	Initial Temperature °F	HTC Btu/hr-ft ² -°F
R1	500	143	500	2693
R2	500	Interpolated	500	Interpolated
R3 ⁽¹⁾	500	39	500	52
R4	500	Interpolated	500	Interpolated
R5 ⁽¹⁾	500	47	500	66
R6	500	Interpolated	500	Interpolated
R6B	500	97	500	97
R7A ⁽¹⁾	500	38	500	50
R7B ⁽¹⁾	500	20	500	23
R8	500	Interpolated	500	Interpolated
R9 ⁽¹⁾	500	33	500	41
R10	500	Interpolated	500	Interpolated
R11	500	500	500	500
R12	120	0.20	120	0.2

(1) See Table 13

Table 3: Heat Transfer Coefficients for Region 1

Pipe Inside Diameter, D =		9.834	inches =	0.820	ft	100% rated flow =		3,200	gpm
Flow, % of rated =		100%		0.250	m	@ T =		549	°F
Fluid Velocity, V =		13.517	ft/sec =	3,200.0	gpm =	Density, ρ =		48.087	lbm/ft ³
Characteristic Length, L = D =		0.820	ft =	0.250	m	1.234236214		Mib/hr	
T _{fluid} - T _{surface} , ΔT = assumed to be 12% of fluid temperature =		8.40	12.00	24.00	36.00	48.00	60.00	72.00	°F
Note: The above assumption is based on experience with past RPV heat transfer analyses.		4.67	6.67	13.33	20.00	26.67	33.33	40.00	°C
		Value at Fluid Temperature, T [7]							
Water Property	Conversion Factor [4]	70	100	200	300	400	500	600	Units
k	1.7307	21.11	37.78	93.33	148.89	204.44	260.00	315.56	°F °C
(Thermal Conductivity)		0.5997	0.6300	0.6784	0.6836	0.6611	0.6040	0.5071	W/m-°C Btu/hr-ft-°F
C _p	4.1869	4.185	4.179	4.229	4.313	4.522	4.982	6.322	kJ/kg-°C
(Specific Heat)		1.000	0.998	1.010	1.030	1.080	1.190	1.510	Btu/lbm-°F
ρ	16.018	997.1	994.7	962.7	917.8	858.6	784.9	679.2	kg/m ³
(Density)		62.3	62.1	60.1	57.3	53.6	49.0	42.4	lbm/ft ³
β	1.8	1.89E-04	3.24E-04	6.66E-04	1.01E-03	1.40E-03	1.98E-03	3.15E-03	m ³ /m ³ -°C
(Volumetric Rate of Expansion)		1.05E-04	1.80E-04	3.70E-04	5.60E-04	7.80E-04	1.10E-03	1.75E-03	ft ³ /ft ³ -°F
g	0.3048	9.806	9.806	9.806	9.806	9.806	9.806	9.806	m/s ²
(Gravitational Constant)		32.17	32.17	32.17	32.17	32.17	32.17	32.17	ft/s ²
μ	1.4881	9.96E-04	6.82E-04	3.07E-04	1.93E-04	1.38E-04	1.04E-04	8.62E-05	kg/m-s
(Dynamic Viscosity)		6.69E-04	4.58E-04	2.06E-04	1.30E-04	9.30E-05	7.00E-05	5.79E-05	lbm/ft-s
Pr		6.980	4.510	1.910	1.220	0.950	0.859	1.070	---
(Prandtl Number)									
Calculated Parameter	Formula	70	100	200	300	400	500	600	°F
Reynold's Number, Re	ρVD/μ	1.0307E+06	1.5019E+06	3.2317E+06	4.8825E+06	6.3843E+06	7.7540E+06	8.1118E+06	---
Grashof Number, Gr	gβΔTL ³ /(μρ) ²	1.3522E+08	7.0314E+08	1.3383E+10	6.9351E+10	2.2021E+11	5.7264E+11	1.1964E+12	---
Rayleigh Number, Ra	GrPr	9.4382E+08	3.1712E+09	2.5562E+10	8.4608E+10	2.0920E+11	4.9189E+11	1.2802E+12	---
From [4]:									
Inside Surface Forced Convection Heat Transfer Coefficient:									
H _{forced} = 0.023Re ^{0.8} Pr ^{0.4} k/D		7,765.07	9,257.25	13,050.72	15,291.12	16,581.64	16,999.27	16,154.74	W/m ² -°C
		1,367.53	1,630.33	2,298.41	2,692.98	2,920.25	2,993.80	2,845.07	Btu/hr-ft ² -°F
		2.638E-03	3.145E-03	4.434E-03	5.195E-03	5.633E-03	5.775E-03	5.488E-03	Btu/sec-in ² -°F
From [4]:									
Inside Surface Natural Convection Heat Transfer Coefficient:									
Case:	Enclosed cylinder	C =	0.55	n =	0.25	(see page 256 of [4])			
H _{nee} = C(GrPr) ⁿ /L		231.44	329.18	597.32	811.84	984.52	1,113.82	1,187.70	W/m ² -°C
		40.76	57.97	105.20	142.98	173.39	196.16	209.17	Btu/hr-ft ² -°F
		7.863E-05	1.118E-04	2.029E-04	2.758E-04	3.345E-04	3.784E-04	4.035E-04	Btu/sec-in ² -°F

Table 4: First Partial Heat Transfer Coefficients for Region 3

Pipe Inside Diameter, D = 7.981 inches = 0.665 ft = 0.203 m		100% rated flow = 3,200 gpm @ T = 549 °F Density, ρ = 48.087 lbm/ft ³							
Flow, % of rated = 100%		Fluid Velocity, V = 20.522 ft/sec = 3,200.0 gpm = 1.234236214 Mb/hr							
Characteristic Length, L = D = 0.665 ft = 0.203 m		T _{fluid} - T _{surface} , ΔT = assumed to be 12% of fluid temperature = 8.40 12.00 24.00 36.00 48.00 60.00 72.00 °F							
= 4.67 6.67		= 20.00 26.67 33.33 40.00 °C							
Note: The above assumption is based on experience with past RPV heat transfer analyses.		Value at Fluid Temperature, T [7]							
Water Property	Conversion Factor [4]	70	100	200	300	400	500	600	Units
k (Thermal Conductivity)	1.7307	0.5997	0.6300	0.6784	0.6836	0.6611	0.6040	0.5071	W/m-°C
c _p (Specific Heat)	4.1869	4.185	4.179	4.229	4.313	4.522	4.982	6.322	Btu/hr-ft-°F
ρ (Density)	16.018	997.1	994.7	962.7	917.8	858.6	784.9	679.2	kJ/kg-°C
β (Volumetric Rate of Expansion)	1.8	62.3	62.1	60.1	57.3	53.6	49.0	42.4	Btu/lbm-°F
g (Gravitational Constant)	0.3048	1.89E-04	3.24E-04	6.66E-04	1.01E-03	1.40E-03	1.98E-03	3.15E-03	kg/m ³
μ (Dynamic Viscosity)	1.4881	1.05E-04	1.80E-04	3.70E-04	5.60E-04	7.80E-04	1.10E-03	1.75E-03	lbm/ft ³
Pr (Prandtl Number)		9.806	9.806	9.806	9.806	9.806	9.806	9.806	m ³ /m ³ -°C
Calculated Parameter	Formula	70	100	200	300	400	500	600	ft ³ /ft ³ -°F
Reynold's Number, Re	ρVD/μ	1.2700E+06	1.8507E+06	3.9821E+06	6.0161E+06	7.8665E+06	9.5543E+06	9.9952E+06	m/s ²
Grashof Number, Gr	gβΔTL ³ /(μρ) ²	7.2279E+07	3.7586E+08	7.1540E+09	3.7071E+10	1.1771E+11	3.0610E+11	6.3954E+11	ft/s ²
Rayleigh Number, Ra	GrPr	5.0451E+08	1.6951E+09	1.3664E+10	4.5226E+10	1.1183E+11	2.6294E+11	6.8430E+11	kg/m-s
From [4]:									lbm/ft-s
Inside Surface Forced Convection Heat Transfer Coefficient:									—
H _{forced} = 0.023Re ^{0.8} Pr ^{0.4} k/D		11,307.23	13,480.10	19,004.02	22,266.42	24,145.63	24,753.78	23,524.01	W/m ² -°C
		1,991.36	2,374.03	3,346.87	3,921.32	4,252.38	4,359.48	4,142.90	Btu/hr-ft ² -°F
		3.841E-03	4.580E-03	6.456E-03	7.564E-03	8.203E-03	8.409E-03	7.992E-03	Btu/sec-in ² -°F
From [4]:									
Inside Surface Natural Convection Heat Transfer Coefficient:									
Case: Enclosed cylinder	C =	0.55	n =	0.25 (see page 289 of [4])					
H _{free} = C(GrPr) ⁿ /kL		243.85	346.81	629.32	855.34	1,037.27	1,173.50	1,251.33	W/m ² -°C
		42.94	61.08	110.83	150.54	182.68	206.67	220.38	Btu/hr-ft ² -°F
		8.284E-05	1.178E-04	2.138E-04	2.906E-04	3.524E-04	3.987E-04	4.251E-04	Btu/sec-in ² -°F

Table 5: Second Partial Heat Transfer Coefficients for Region 3

Pipe Inside Diameter, D =	9.634	inches =	0.820	ft					
		=	0.250	m					
Outer Pipe, Inside radius, r _o =	4.917	inches =	0.410	ft					
		=	0.125	m					
Inner Pipe Outside Diameter, D =	8.625	inches =	0.719	ft					
		=	0.219	m					
Inner Pipe, Outside radius, r _i =	4.3125	inches =	0.359	ft					
		=	0.110	m					
Fluid Velocity, V =	13.517	ft/sec =	3200.000	gpm					
Characteristic Length, L = D =	0.820	ft =	0.250	m					
(Outside) T _{fluid} - T _{surface} , ΔT =	8.40	12.00	24.00	36.00	48.00	60.00	72.00	°F	
=	4.67	6.67	13.33	20.00	26.67	33.33	40.00	°C	

Water Property	Conversion Factor [4]	Value at Fluid Temperature, T [7]							Units
		70	100	200	300	400	500	600	°F
k	1.7307	0.5997	0.6300	0.6784	0.6836	0.6611	0.6040	0.5071	W/m-°C
(Thermal Conductivity)		0.3465	0.3640	0.3920	0.3950	0.3820	0.3490	0.2930	Btu/hr-ft-°F
c _p	4.1869	4.185	4.179	4.229	4.313	4.522	4.982	6.322	kJ/kg-°C
(Specific Heat)		1.000	0.998	1.010	1.030	1.080	1.190	1.510	Btu/lbm-°F
ρ	16.018	997.1	994.7	962.7	917.8	858.6	784.9	679.2	kg/m ³
(Density)		62.3	62.1	60.1	57.3	53.6	49.0	42.4	lbm/ft ³
β	1.8	1.89E-04	3.24E-04	6.66E-04	1.01E-03	1.40E-03	1.98E-03	3.15E-03	m ³ /m ³ -°C
(Volumetric Rate of Expansion)		1.05E-04	1.80E-04	3.70E-04	5.60E-04	7.80E-04	1.10E-03	1.75E-03	ft ³ /ft ³ -°F
g	0.3048	9.806	9.806	9.806	9.806	9.806	9.806	9.806	m/s ²
(Gravitational Constant)		32.17	32.17	32.17	32.17	32.17	32.17	32.17	ft/s ²
μ	1.4881	9.96E-04	6.82E-04	3.07E-04	1.93E-04	1.38E-04	1.04E-04	8.62E-05	kg/m-s
(Dynamic Viscosity)		6.69E-04	4.58E-04	2.06E-04	1.30E-04	9.30E-05	7.00E-05	5.79E-05	lbm/ft-s
Pr		6.980	4.510	1.910	1.220	0.950	0.859	1.070	---
(Prandtl Number)									
Calculated Parameter	Formula	70	100	200	300	400	500	600	°F
Reynold's Number, Re	ρVD/μ	1030724	1501950	3231741	4882481	6384268	7754027	8111787	---
Grashof Number, Gr	gβΔTL ³ /(μ/ρ) ²	135217684.2	703144247.6	13383382850	69350803914	2.2021E+11	5.72636E+11	1.19642E+12	---
Grashof Number, Gr _s	gβΔT(r _o -r _i) ³ /(μ/ρ) ³	3.14E+04	1.63E+05	3.11E+06	1.61E+07	5.11E+07	1.33E+08	2.78E+08	---
Rayleigh Number, Ra	GrPr	943819435.9	3171180557	25562261244	84607980776	2.092E+11	4.91894E+11	1.28017E+12	---
Rayleigh Number, Ra	Gr _s Pr	2.19E+05	7.37E+05	5.94E+06	1.97E+07	4.86E+07	1.14E+08	2.97E+08	---

From [4]:

Annulus Natural Convection Heat Transfer Coefficient:

Case:	Enclosed cylinder	C =	0.40	n =	0.20	(see page 291 of [5])			
H _{tee}	C(G _s Pr) ^{0.1} k/(r _o -r _i)	182.78	244.68	400.00	512.07	593.51	643.36	653.99	W/m ² -°C
		32.19	43.09	70.45	93.18	104.53	113.30	115.18	Btu/hr-ft ² -°F

Table 6: First Partial Heat Transfer Coefficients for Region 5

Pipe Inside Diameter, D = 7.981 inches = 0.665 ft = 0.203 m		100% rated flow = 3,200 gpm @ T = 549 °F Density, ρ = 48.087 lbm/ft ³							
Flow, % of rated = 100%		Fluid Velocity, V = 20.522 ft/sec = 3,200.0 gpm = 20.522 ft = 0.203 m							
Characteristic Length, L = D = 0.665 ft = 0.203 m		1.234236214 Mlb/hr							
T _{fluid} - T _{surface} , ΔT = assumed to be 12% of fluid temperature = 8.40 12.00 24.00 36.00 48.00 60.00 72.00 °F		= 4.67 6.67 13.33 20.00 26.67 33.33 40.00 °C							
Note: The above assumption is based on experience with past RPV heat transfer analyses.		Value at Fluid Temperature, T [7]							Units
Water Property	Conversion Factor [4]	70	100	200	300	400	500	600	°F
k (Thermal Conductivity)	1.7307	0.5997	0.6300	0.6784	0.6836	0.6611	0.6040	0.5071	W/m·°C
c _p (Specific Heat)	4.1869	4.185	4.179	4.229	4.313	4.522	4.982	6.322	kJ/kg·°C
ρ (Density)	16.018	997.1	994.7	962.7	917.8	858.6	784.9	679.2	Btu/lb·°F
β (Volumetric Rate of Expansion)	1.8	62.3	62.1	60.1	57.3	53.6	49.0	42.4	kg/m ³
g (Gravitational Constant)	0.3048	1.89E-04	3.24E-04	6.66E-04	1.01E-03	1.40E-03	1.98E-03	3.15E-03	lbm/ft ³
μ (Dynamic Viscosity)	1.4881	1.05E-04	1.80E-04	3.70E-04	5.60E-04	7.80E-04	1.10E-03	1.75E-03	m ³ /m ³ ·°C
Pr (Prandtl Number)		9.806	9.806	9.806	9.806	9.806	9.806	9.806	ft ³ /ft ³ ·°F
Calculated Parameter	Formula	70	100	200	300	400	500	600	m/s ²
Reynold's Number, Re	ρVD/μ	1.2700E+06	1.8507E+06	3.9821E+06	6.0161E+06	7.8665E+06	9.5543E+06	9.9952E+06	ft/s ²
Grashof Number, Gr	gβΔTL ³ /(μρ) ²	7.2279E+07	3.7586E+08	7.1540E+09	3.7071E+10	1.1771E+11	3.0610E+11	6.3954E+11	kg/m·s
Rayleigh Number, Ra	GrPr	5.0451E+08	1.6951E+09	1.3664E+10	4.5226E+10	1.1183E+11	2.6294E+11	6.8430E+11	lbm/ft·s
From [4]:									
Inside Surface Forced Convection Heat Transfer Coefficient:									
H _{forced} = 0.023Re ^{0.8} Pr ^{0.4} k/D		11,307.23	13,480.10	19,004.02	22,266.42	24,145.63	24,753.78	23,524.01	W/m ² ·°C
		1,991.36	2,374.03	3,346.87	3,921.42	4,252.38	4,359.48	4,142.90	Btu/hr-ft ² ·°F
		3.841E-03	4.580E-03	6.456E-03	7.564E-03	8.203E-03	8.409E-03	7.892E-03	Btu/sec-in ² ·°F
From [4]:									
Inside Surface Natural Convection Heat Transfer Coefficient:									
Case:	Enclosed cylinder	C =	0.55	n =	0.25	(see page 289 of [4])			
H _{free} = C(GrPr) ⁿ /k/L		243.85	346.81	629.32	855.34	1,037.27	1,173.50	1,251.33	W/m ² ·°C
		42.94	61.08	110.83	150.84	182.68	206.67	220.38	Btu/hr-ft ² ·°F
		8.284E-05	1.178E-04	2.138E-04	2.906E-04	3.524E-04	3.987E-04	4.251E-04	Btu/sec-in ² ·°F

Table 7: Second Partial Heat Transfer Coefficients for Region 5

Title = No Flow Region between nozzle and thermal sleeve.

Pipe Inside Diameter, D = 9.000 inches = 0.750 ft
= 0.229 m

Outer Pipe, Inside radius, r_o = 4.5 inches = 0.375 ft
= 0.114 m

Inner Pipe Outside Diameter, D = 8.625 inches = 0.719 ft
= 0.219 m

Inner Pipe, Outside radius, r_i = 4.3125 inches = 0.359 ft
= 0.110 m

Fluid Velocity, V = 16.138 ft/sec = 3200.000 gpm
Characteristic Length, L = D = 0.750 ft = 0.229 m

(Outside) T_{fluid} - T_{surface}, ΔT = 8.40 12.00 24.00 36.00 48.00 60.00 72.00 °F
= 4.67 6.67 13.33 20.00 26.67 33.33 40.00 °C

Water Property	Conversion Factor [4]	Value at Fluid Temperature, T [7]							Units
		70	100	200	300	400	500	600	°F
k	1.7307	21.11	37.78	93.33	148.89	204.44	260.00	315.56	°C
(Thermal Conductivity)		0.5997	0.6300	0.6784	0.6836	0.6611	0.6040	0.5071	W/m·°C
c _p	4.1869	0.3465	0.3640	0.3920	0.3950	0.3820	0.3490	0.2930	Btu/hr-ft·°F
(Specific Heat)		4.185	4.179	4.229	4.313	4.522	4.982	6.322	KJ/kg·°C
ρ	16.018	1.000	0.998	1.010	1.030	1.080	1.190	1.510	Btu/lbm·°F
(Density)		997.1	994.7	962.7	917.8	858.6	784.9	679.2	kg/m ³
β	1.8	62.3	62.1	60.1	57.3	53.6	49.0	42.4	lbm/ft ³
(Volumetric Rate of Expansion)		1.89E-04	3.24E-04	6.66E-04	1.01E-03	1.40E-03	1.98E-03	3.15E-03	m ³ /m ³ ·°C
g	0.3048	1.05E-04	1.80E-04	3.70E-04	5.60E-04	7.80E-04	1.10E-03	1.75E-03	ft ³ /ft ³ ·°F
(Gravitational Constant)		9.806	9.806	9.806	9.806	9.806	9.806	9.806	m/s ²
μ	1.4881	32.17	32.17	32.17	32.17	32.17	32.17	32.17	ft/s ²
(Dynamic Viscosity)		9.96E-04	6.82E-04	3.07E-04	1.93E-04	1.38E-04	1.04E-04	8.62E-05	kg/m·s
Pr		6.69E-04	4.58E-04	2.06E-04	1.30E-04	9.30E-05	7.00E-05	5.79E-05	lbm/ft·s
(Prandtl Number)		6.980	4.510	1.910	1.220	0.950	0.859	1.070	---
Calculated Parameter	Formula	70	100	200	300	400	500	600	°F
Reynold's Number, Re	ρVD/μ	1126238	1641130	3531215	5334924	6975877	8472567	8863480	---
Grashof Number, Gr	gβΔT ³ /(μρ) ²	103650263.4	538990790.5	10258947757	53160421562	1.68801E+11	4.3895E+11	9.17108E+11	---
Grashof Number, Gr _s	gβΔT(r _o -r _i) ³ /(μρ) ³	9.37E+02	4.87E+03	9.28E+04	4.81E+05	1.53E+06	3.97E+06	8.29E+06	---
Rayleigh Number, Ra	GrPr	723478838.9	2430848465	19594590215	64855714305	1.60361E+11	3.77058E+11	9.81306E+11	---
Rayleigh Number, Ra	Gr _s Pr	6.54E+03	2.20E+04	1.77E+05	5.86E+05	1.45E+06	3.41E+06	8.87E+06	---

From [4]:

Annulus Natural Convection Heat Transfer Coefficient:

Case: Enclosed cylinder C = 0.40 n = 0.20 (see page 291 of [5])

H_{free} = C(Gr_sPr)^{0.25}k/(r_o-r_i)

	70	100	200	300	400	500	600	Units
H _{free}	291.94	390.80	638.87	817.88	947.95	1,027.56	1,044.55	W/m ² ·°C
	51.41	68.83	112.51	144.04	166.95	180.97	183.96	Btu/hr-ft ² ·°F

Table 8: First Partial Heat Transfer Coefficients for Region 7

Pipe Inside Diameter, D = 7.981 inches = 0.665 ft = 0.203 m		100% rated flow = 3,200 gpm @ T = 549 °F Density, ρ = 48.087 lbm/ft ³								
Flow, % of rated = 100%		Fluid Velocity, V = 20.522 ft/sec = 3,200.0 gpm = 1.234236214 Mlb/hr								
Characteristic Length, L = D = 0.665 ft = 0.203 m		T _{fluid} - T _{surface} , ΔT = assumed to be 12% of fluid temperature = 8.40 12.00 24.00 36.00 48.00 60.00 72.00 °F								
Note: The above assumption is based on experience with past RPV heat transfer analyses.		= 4.67 6.67 13.33 20.00 26.67 33.33 40.00 °C								
Water Property	Conversion Factor [4]	Value at Fluid Temperature, T [7]								Units
		70	100	200	300	400	500	600	°F	
(Thermal Conductivity) k	1.7307	0.5997	0.6300	0.6784	0.6836	0.6611	0.6040	0.5071	W/m-°C	
(Specific Heat) c _p	4.1869	4.185	4.179	4.229	4.313	4.522	4.982	6.322	Btu/hr-ft-°F	
(Density) ρ	16.018	997.1	994.7	962.7	917.8	858.6	784.9	679.2	kJ/kg-°C	
(Volumetric Rate of Expansion) β	1.8	62.3	62.1	60.1	57.3	53.6	49.0	42.4	Btu/lbm-°F	
(Gravitational Constant) g	0.3048	1.89E-04	3.24E-04	6.66E-04	1.01E-03	1.40E-03	1.98E-03	3.15E-03	kg/m ³	
(Dynamic Viscosity) μ	1.4881	1.05E-04	1.80E-04	3.70E-04	5.60E-04	7.80E-04	1.10E-03	1.75E-03	lbm/ft ³	
(Prandtl Number) Pr		9.806	9.806	9.806	9.806	9.806	9.806	9.806	m/s ²	
Calculated Parameter	Formula	32.17	32.17	32.17	32.17	32.17	32.17	32.17	ft/s ²	
Reynold's Number, Re	ρVD/μ	9.96E-04	6.82E-04	3.07E-04	1.93E-04	1.38E-04	1.04E-04	8.62E-05	kg/m-s	
Grashof Number, Gr	gβΔTL ³ /(μρ) ²	6.69E-04	4.58E-04	2.06E-04	1.30E-04	9.30E-05	7.00E-05	5.79E-05	lbm/ft-s	
Rayleigh Number, Ra	GrPr	6.980	4.510	1.910	1.220	0.950	0.859	1.070	---	
From [4]:										
Inside Surface Forced Convection Heat Transfer Coefficient:										
H _{forced} = 0.023Re ^{0.8} Pr ^{0.4} k/D		11,307.23	13,480.10	19,004.02	22,266.42	24,145.63	24,753.78	23,524.01	W/m ² -°C	
		1,991.36	2,374.03	3,346.87	3,921.42	4,252.38	4,359.48	4,142.90	Btu/hr-ft ² -°F	
		3.841E-03	4.580E-03	6.456E-03	7.564E-03	8.203E-03	8.409E-03	7.992E-03	Btu/sec-in ² -°F	
From [4]:										
Inside Surface Natural Convection Heat Transfer Coefficient:										
Case: Enclosed cylinder		C = 0.55		n = 0.25 (see page 289 of [4])						
H _{free} = C(GrPr) ⁿ /kL		243.85	346.81	629.32	855.34	1,037.27	1,173.50	1,251.33	W/m ² -°C	
		42.94	61.08	110.83	150.62	182.68	206.67	220.38	Btu/hr-ft ² -°F	
		8.284E-05	1.478E-04	2.138E-04	2.906E-04	3.524E-04	3.987E-04	4.251E-04	Btu/sec-in ² -°F	

Table 9: Second Partial Heat Transfer Coefficients for Region 7

Pipe Inside Diameter, $D = 10.020$ inches = 0.835 ft
 = 0.255 m
 Outer Pipe, Inside radius, $r_o = 5.01$ inches = 0.418 ft
 = 0.127 m
 Inner Pipe Outside Diameter, $D = 8.625$ inches = 0.719 ft
 = 0.219 m
 Inner Pipe, Outside radius, $r_i = 4.3125$ inches = 0.359 ft
 = 0.110 m
 Fluid Velocity, $V = 13.020$ ft/sec = 3200.000 gpm
 Characteristic Length, $L = D = 0.835$ ft = 0.255 m
 (Outside) $T_{fluid} - T_{surface}, \Delta T =$
 = 8.40 12.00 24.00 36.00 48.00 60.00 72.00 °F
 = 4.67 6.67 13.33 20.00 26.67 33.33 40.00 °C

Water Property	Conversion Factor [4]	Value at Fluid Temperature, T [7]							Units
		70	100	200	300	400	500	600	
k (Thermal Conductivity)	1.7307	0.5997	0.6300	0.6784	0.6836	0.6611	0.6040	0.5071	W/m-°C
c_p (Specific Heat)	4.1869	4.185	4.179	4.229	4.313	4.522	4.982	6.322	Btu/hr-ft-°F
ρ (Density)	16.018	997.1	994.7	962.7	917.8	858.6	784.9	679.2	kJ/kg-°C
β (Volumetric Rate of Expansion)	1.8	62.3	62.1	60.1	57.3	53.6	49.0	42.4	Btu/lbm-°F
g (Gravitational Constant)	0.3048	1.89E-04	3.24E-04	6.66E-04	1.01E-03	1.40E-03	1.98E-03	3.15E-03	kg/m ³
μ (Dynamic Viscosity)	1.4881	1.05E-04	1.80E-04	3.70E-04	5.60E-04	7.80E-04	1.10E-03	1.75E-03	lbm/ft ³
Pr (Prandtl Number)		9.806	9.806	9.806	9.806	9.806	9.806	9.806	m/s ²
Calculated Parameter	Formula	32.17	32.17	32.17	32.17	32.17	32.17	32.17	ft/s ²
Reynold's Number, Re	$\rho V D / \mu$	9.96E-04	6.82E-04	3.07E-04	1.93E-04	1.38E-04	1.04E-04	8.62E-05	kg/m-s
Grashof Number, Gr	$g \beta \Delta T L^3 / (\mu / \rho)^2$	6.69E-04	4.58E-04	2.06E-04	1.30E-04	9.30E-05	7.00E-05	5.79E-05	lbm/ft-s
Grashof Number, Gr_s	$g \beta \Delta T (r_o - r_i)^3 / (\mu / \rho)^3$	6.980	4.510	1.910	1.220	0.950	0.859	1.070	---
Rayleigh Number, Ra	$Gr Pr$	3.37E+05	1.13E+06	9.12E+06	3.02E+07	7.46E+07	1.76E+08	4.57E+08	---
Rayleigh Number, Ra	$Gr_s Pr$								---

From [4]:

Annulus Natural Convection Heat Transfer Coefficient:

Case: Enclosed cylinder $C = 0.40$ $n = 0.20$ (see page 291 of [5])

$H_{nc} = C (Gr_s Pr)^n k / (r_o - r_i)$

	172.61	231.07	377.74	483.58	560.49	607.56	617.61	W/m ² -°C
	30.40	40.69	66.53	85.77	98.71	107.00	108.77	Btu/hr-ft ² -°F

Table 10: Third Partial Heat Transfer Coefficients for Region 7

Pipe Inside Diameter, D =	11.750	inches =	0.979	ft						
		=	0.298	m						
Outer Pipe, Inside radius, r _o =	5.875	inches =	0.490	ft						
		=	0.149	m						
Inner Pipe Outside Diameter, D =	10.750	inches =	0.896	ft						
		=	0.273	m						
Inner Pipe, Outside radius, r _i =	5.375	inches =	0.448	ft						
		=	0.137	m						
Fluid Velocity, V =	9.468	ft/sec =	3200.000	gpm						
Characteristic Length, L = D =	0.979	ft =	0.298	m						
(Outside) T _{fluid} - T _{surface} , ΔT =	8.40	12.00	24.00	36.00	48.00	60.00	72.00	°F		
=	4.67	6.67	13.33	20.00	26.67	33.33	40.00	°C		

Water Property	Conversion Factor [4]	Value at Fluid Temperature, T [7]								Units
		70	100	200	300	400	500	600	°F	
k	1.7307	21.11	37.78	93.33	148.89	204.44	260.00	315.56	°C	
(Thermal Conductivity)		0.5997	0.6300	0.6784	0.6836	0.6611	0.6040	0.5071	W/m ² ·°C	
		0.3465	0.3640	0.3920	0.3950	0.3820	0.3490	0.2930	Btu/hr-ft ² ·°F	
c _p	4.1869	4.185	4.179	4.229	4.313	4.522	4.982	6.322	kJ/kg·°C	
(Specific Heat)		1.000	0.998	1.010	1.030	1.080	1.190	1.510	Btu/lbm·°F	
ρ	16.018	997.1	994.7	962.7	917.8	858.6	784.9	679.2	kg/m ³	
(Density)		62.3	62.1	60.1	57.3	53.6	49.0	42.4	lbm/ft ³	
β	1.8	1.89E-04	3.24E-04	6.66E-04	1.01E-03	1.40E-03	1.98E-03	3.15E-03	m ³ /m ³ ·°C	
(Volumetric Rate of Expansion)		1.05E-04	1.80E-04	3.70E-04	5.60E-04	7.80E-04	1.10E-03	1.75E-03	ft ³ /ft ³ ·°F	
g	0.3048	9.806	9.806	9.806	9.806	9.806	9.806	9.806	m/s ²	
(Gravitational Constant)		32.17	32.17	32.17	32.17	32.17	32.17	32.17	ft/s ²	
μ	1.4881	9.96E-04	6.82E-04	3.07E-04	1.93E-04	1.38E-04	1.04E-04	8.62E-05	kg/m·s	
(Dynamic Viscosity)		6.69E-04	4.58E-04	2.06E-04	1.30E-04	9.30E-05	7.00E-05	5.79E-05	lbm/ft·s	
Pr		6.980	4.510	1.910	1.220	0.950	0.859	1.070	—	
(Prandtl Number)										
Calculated Parameter	Formula	70	100	200	300	400	500	600	°F	
Reynold's Number, Re	ρVD/μ	862650	1257036	2704761	4086325	5343225	6489626	6789048	—	
Grashof Number, Gr	gβΔTL ³ /(μ/ρ) ²	230651605.4	1199409312	22829105215	1.18297E+11	3.75631E+11	9.76791E+11	2.04083E+12	—	
Grashof Number, Gr _s	gβΔT(r _o -r _i) ³ /(μ/ρ) ²	1.78E+04	9.24E+04	1.76E+06	9.12E+06	2.89E+07	7.53E+07	1.57E+08	—	
Rayleigh Number, Ra	GrPr	1609948206	5409335995	43603590961	1.44323E+11	3.56849E+11	8.39063E+11	2.18369E+12	—	
Rayleigh Number, Ra	Gr _s Pr	1.24E+05	4.17E+05	3.36E+06	1.11E+07	2.75E+07	6.47E+07	1.68E+08	—	

From [4]:

Annulus Natural Convection Heat Transfer Coefficient:

Case:	Enclosed cylinder	C =	0.40	n =	0.20	(see page 291 of [5])		
H _{free}	C(Gr _s Pr) ^{0.25} /k(r _o -r _i)	197.20	263.98	431.55	552.46	640.32	694.10	
		34.73	46.49	76.00	97.30	112.77	122.24	
								W/m ² ·°C
								Btu/hr-ft ² ·°F

Table 11: First Partial Heat Transfer Coefficients for Region 9

Pipe Inside Diameter, D =	7.981	inches =	0.665	ft	100% rated flow =	3,200	gpm		
		=	0.203	m	@ T =	549	°F		
Flow, % of rated =	100%				Density, ρ =	48.087	lbm/ft ³		
Fluid Velocity, V =	20.522	ft/sec =	3,200.0	gpm =	1.234236214	Mlb/hr			
Characteristic Length, L = D =	0.665	ft =	0.203	m					
T _{fluid} - T _{surface} , ΔT = assumed to be 12% of fluid temperature =	8.40	12.00	24.00	36.00	48.00	60.00	72.00 °F		
Note: The above assumption is based on experience with past RPV heat transfer analyses.	4.67	6.67	13.33	20.00	26.67	33.33	40.00 °C		
		Value at Fluid Temperature, T [7]						Units	
Water Property	Conversion Factor [4]	70	100	200	300	400	500	600	°F
k	1.7307	0.5997	0.6300	0.6784	0.6836	0.6611	0.6040	0.5071	W/m ² ·°C
(Thermal Conductivity)		0.3465	0.3640	0.3920	0.3950	0.3820	0.3490	0.2930	Btu/hr-ft ² ·°F
c _p	4.1869	4.185	4.179	4.229	4.313	4.522	4.982	6.322	kJ/kg·°C
(Specific Heat)		1.000	0.998	1.010	1.030	1.080	1.190	1.510	Btu/lbm·°F
ρ	16.018	997.1	994.7	962.7	917.8	858.6	784.9	679.2	kg/m ³
(Density)		62.3	62.1	60.1	57.3	53.6	49.0	42.4	lbm/ft ³
β	1.8	1.89E-04	3.24E-04	6.66E-04	1.01E-03	1.40E-03	1.98E-03	3.15E-03	m ³ /m ³ ·°C
(Volumetric Rate of Expansion)		1.05E-04	1.80E-04	3.70E-04	5.60E-04	7.80E-04	1.10E-03	1.75E-03	ft ³ /ft ³ ·°F
g	0.3048	9.806	9.806	9.806	9.806	9.806	9.806	9.806	m/s ²
(Gravitational Constant)		32.17	32.17	32.17	32.17	32.17	32.17	32.17	ft/s ²
μ	1.4881	9.96E-04	6.82E-04	3.07E-04	1.93E-04	1.38E-04	1.04E-04	8.62E-05	kg/m-s
(Dynamic Viscosity)		6.69E-04	4.58E-04	2.06E-04	1.30E-04	9.30E-05	7.00E-05	5.79E-05	lbm/ft-s
Pr		6.980	4.510	1.910	1.220	0.950	0.859	1.070	---
(Prandtl Number)									
Calculated Parameter	Formula	70	100	200	300	400	500	600	°F
Reynold's Number, Re	ρVD/μ	1.2700E+06	1.8507E+06	3.9821E+06	6.0161E+06	7.9665E+06	9.5543E+06	9.9952E+06	---
Grashof Number, Gr	gβΔTL ³ /(μρ) ²	7.2279E+07	3.7586E+08	7.1540E+09	3.7071E+10	1.1771E+11	3.0610E+11	6.3954E+11	---
Rayleigh Number, Ra	GrPr	5.0451E+08	1.6951E+09	1.3664E+10	4.5226E+10	1.1183E+11	2.6294E+11	6.8430E+11	---
From [4]:									
Inside Surface Forced Convection Heat Transfer Coefficient:									
H _{forced} = 0.023Re ^{0.8} P ^{0.4} k/D		11,307.23	13,480.10	19,004.02	22,266.42	24,145.63	24,753.78	23,524.01	W/m ² ·°C
		1,991.36	2,374.03	3,346.87	3,921.42	4,252.38	4,359.48	4,142.90	Btu/hr-ft ² ·°F
		3.841E-03	4.580E-03	6.456E-03	7.564E-03	8.203E-03	8.409E-03	7.992E-03	Btu/sec-in ² ·°F
From [4]:									
Inside Surface Natural Convection Heat Transfer Coefficient:									
Case: Enclosed cylinder			C =	0.55	n =	0.25	(see page 289 of [4])		
H _{free} = C(GrPr) ⁿ k/L		243.85	346.81	629.32	855.34	1,037.27	1,173.50	1,251.33	W/m ² ·°C
		42.94	61.08	110.83	150.84	182.68	206.67	220.38	Btu/hr-ft ² ·°F
		8.284E-05	1.178E-04	2.138E-04	2.906E-04	3.524E-04	3.987E-04	4.251E-04	Btu/sec-in ² ·°F

Table 12: Second Partial Heat Transfer Coefficients for Region 9

Pipe Inside Diameter, D = 11.750 inches = 0.979 ft
 = 0.298 m
 Outer Pipe, Inside radius, r_o = 5.875 inches = 0.490 ft
 = 0.149 m
 Inner Pipe Outside Diameter, D = 8.625 inches = 0.719 ft
 = 0.219 m
 Inner Pipe, Outside radius, r_i = 4.3125 inches = 0.359 ft
 = 0.110 m
 Fluid Velocity, V = 9.468 ft/sec = 3200.000 gpm
 Characteristic Length, L = D = 0.979 ft = 0.298 m
 (Outside) T_{fluid} - T_{surface}, ΔT = 8.40 12.00 24.00 36.00 48.00 60.00 72.00 °F
 = 4.67 6.67 13.33 20.00 26.67 33.33 40.00 °C

Water Property	Conversion Factor [4]	Value at Fluid Temperature, T [7]							Units
		70	100	200	300	400	500	600	°F
k	1.7307	0.5997	0.6300	0.6784	0.6836	0.6611	0.6040	0.5071	W/m·°C
(Thermal Conductivity)		0.3465	0.3640	0.3920	0.3950	0.3820	0.3490	0.2930	Btu/hr-ft·°F
c _p	4.1869	4.185	4.179	4.229	4.313	4.522	4.982	6.322	kJ/kg·°C
(Specific Heat)		1.000	0.998	1.010	1.030	1.080	1.190	1.510	Btu/lbm·°F
ρ	16.018	997.1	994.7	962.7	917.8	858.6	784.9	679.2	kg/m ³
(Density)		62.3	62.1	60.1	57.3	53.6	49.0	42.4	lbm/ft ³
β	1.8	1.89E-04	3.24E-04	6.66E-04	1.01E-03	1.40E-03	1.98E-03	3.15E-03	m ³ /m ³ ·°C
(Volumetric Rate of Expansion)		1.05E-04	1.80E-04	3.70E-04	5.60E-04	7.80E-04	1.10E-03	1.75E-03	ft ³ /ft ³ ·°F
g	0.3048	9.806	9.806	9.806	9.806	9.806	9.806	9.806	m/s ²
(Gravitational Constant)		32.17	32.17	32.17	32.17	32.17	32.17	32.17	ft/s ²
μ	1.4881	9.96E-04	6.82E-04	3.07E-04	1.93E-04	1.38E-04	1.04E-04	8.62E-05	kg/m-s
(Dynamic Viscosity)		6.69E-04	4.58E-04	2.06E-04	1.30E-04	9.30E-05	7.00E-05	5.79E-05	lbm/ft-s
Pr		6.980	4.510	1.910	1.220	0.950	0.859	1.070	---
(Prandtl Number)									
Calculated Parameter	Formula	70	100	200	300	400	500	600	°F
Reynold's Number, Re	ρVD/μ	862650	1257036	2704761	4086325	5343225	6489626	6789048	---
Grashof Number, Gr	gβΔTL ³ /(μρ) ²	230651605.4	1199409312	22829105215	1.18297E+11	3.75631E+11	9.76791E+11	2.04083E+12	---
Grashof Number, Gr _s	gβΔT(r _o -r _i) ³ /(μρ) ³	5.42E+05	2.82E+06	5.37E+07	2.78E+08	8.83E+08	2.30E+09	4.80E+09	---
Rayleigh Number, Ra	GrPr	1609948206	5409335995	43603590961	1.44323E+11	3.56849E+11	8.39063E+11	2.18369E+12	---
Rayleigh Number, Ra	Gr _s Pr	3.79E+06	1.27E+07	1.03E+08	3.39E+08	8.39E+08	1.97E+09	5.13E+09	---

From [4]:

Annulus Natural Convection Heat Transfer Coefficient:

Case: Enclosed cylinder C = 0.40 n = 0.20 (see page 291 of (5))

H _{free}	C(Gr _s Pr) ^{0.25} /k/(r _o -r _i)	70	100	200	300	400	500	600	W/m ² ·°C
		22.02	29.47	48.18	61.62	71.49	77.50	78.78	Btu/hr-ft ² ·°F

Although the thermal sleeve was excluded from the analysis, its effect had to be included in the finite element model. For several thermal regions, the resultant HTC's had to be calculated from the partial heat transfer coefficients (HTC_i in Table 13). These are generated by "Heat Transfer Coefficients.xls".

$$\frac{1}{HTC_{Res}} = \left(\frac{1}{HTC_I} + \frac{T_I}{TC_I} \right) + \left(\frac{1}{HTC_{II}} + \frac{T_{II}}{TC_{II}} \right) + \left(\frac{1}{HTC_{III}} + \frac{T_{III}}{TC_{III}} \right)$$

Where:

- HTC_{Res} = Resultant HTC
- HTC_i = HTC of ith material
- T_i = Thickness of ith material
- TC_i = Thermal Conductivity of ith material

The reference for this equation is [4].

Table 13: Resultant Heat Transfer Coefficients for the Regions

100% Flow								
Regions	HTC I	Material		HTCII	Material		HTCIII	HTC resultant
		Thermal Conductivity, Btu/hr-ft-°F	Thickness [ft]		Thermal Conductivity, Btu/hr-ft-°F	Thickness [ft]		
R1	2,692.98							2693
R2								
R3	3,921.42	9.8	0.0268	90.18				71
R4								
R5	3,921.42	9.8	0.0268	144.04				101
R6								
R6B							97.30	97
R7A	3,921.42	9.8	0.0268	85.17				68
R7B	3,921.42	9.8	0.0268	85.17	9.8	0.0304	97.30	36
R8								
R9	3,921.42	9.8	0.0268	61.68				52
R10								
R11	500							500
R12	0.2							0.2

0% Flow								
Regions	HTC I	Material		HTCII	Material		HTCIII	HTC resultant
		Thermal Conductivity, Btu/hr-ft-°F	Thickness [ft]		Thermal Conductivity, Btu/hr-ft-°F	Thickness [ft]		
R1	142.98							143
R2								
R3	150.64	9.8	0.0268	90.18				49
R4								
R5	150.64	9.8	0.0268	144.04				61
R6								
R6B							97.30	97
R7A	150.64	9.8	0.0268	85.17				47
R7B	150.64	9.8	0.0268	85.17	9.8	0.0304	97.30	29
R8								
R9	150.64	9.8	0.0268	61.68				39
R10								
R11	500							500
R12	0.2							0.2

4.0 THERMAL AND PRESSURE LOAD RESULTS

The two flow dependent thermal load cases outlined in previous section were run on the core spray FEM. For ANSYS, the thermal transient input files "VY_16Q_T100.inp," "VY_16Q_T0.inp," for 100% and 0% flow, respectively. The stress input filenames are "VY_16Q_ST100.inp" and "VY_16Q_ST0.inp," respectively.

The limiting safe end location was chosen based on the highest thermal stress intensity at 100% flow. Node 3719 on the inside surface of the core spray nozzle was selected for the safe end analysis and shown in Figure 6.

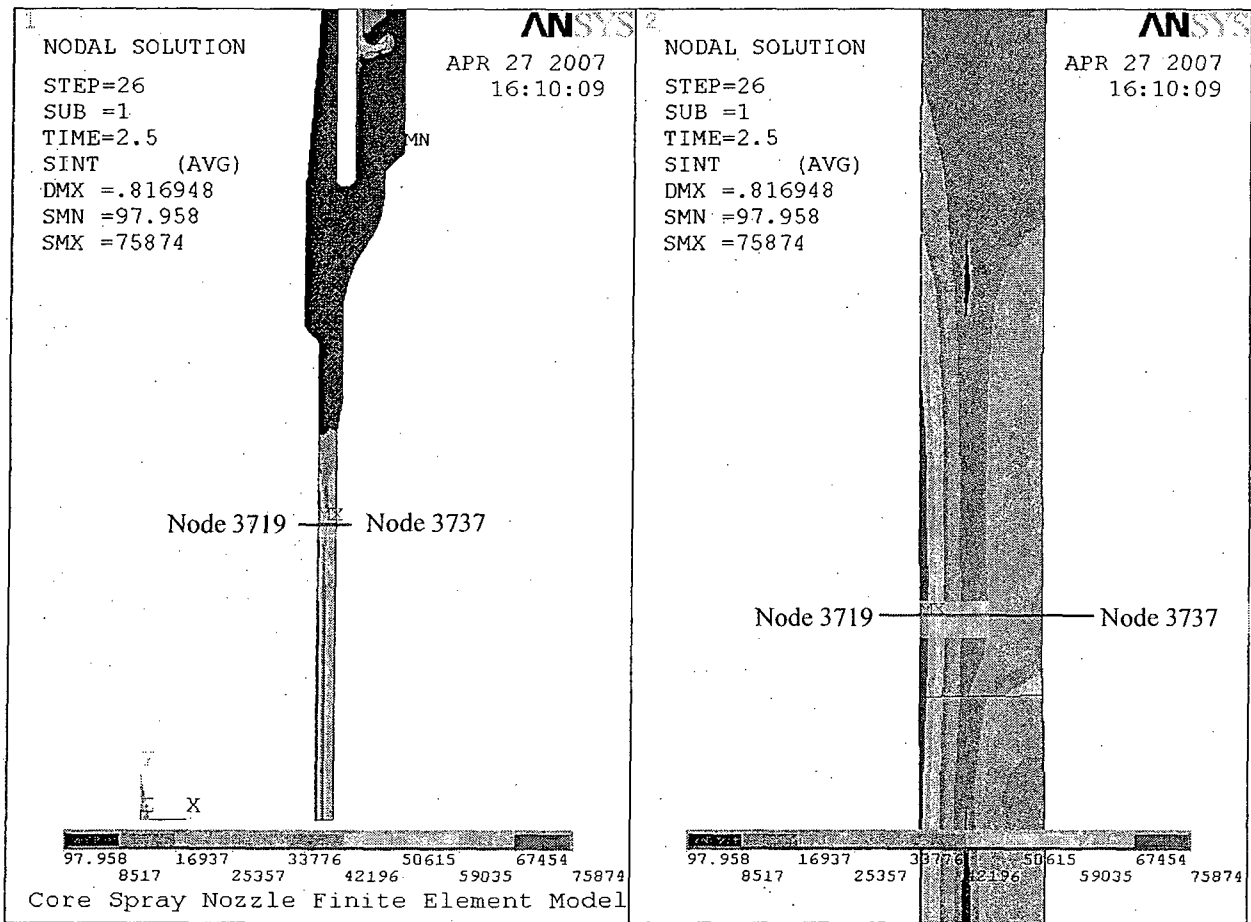


Figure 6: Safe End Critical Thermal Stress Location, Node 3719

The limiting blend radius location was chosen based upon the highest pressure stress intensity. Node 2166 on the inside surface of the blend radius was therefore selected for the nozzle forging analysis and shown in Figure 7. The highest thermal stress and pressure stress occur very close to the same location in the nozzle forging region. Therefore, this location is a reasonable choice for the limiting location.

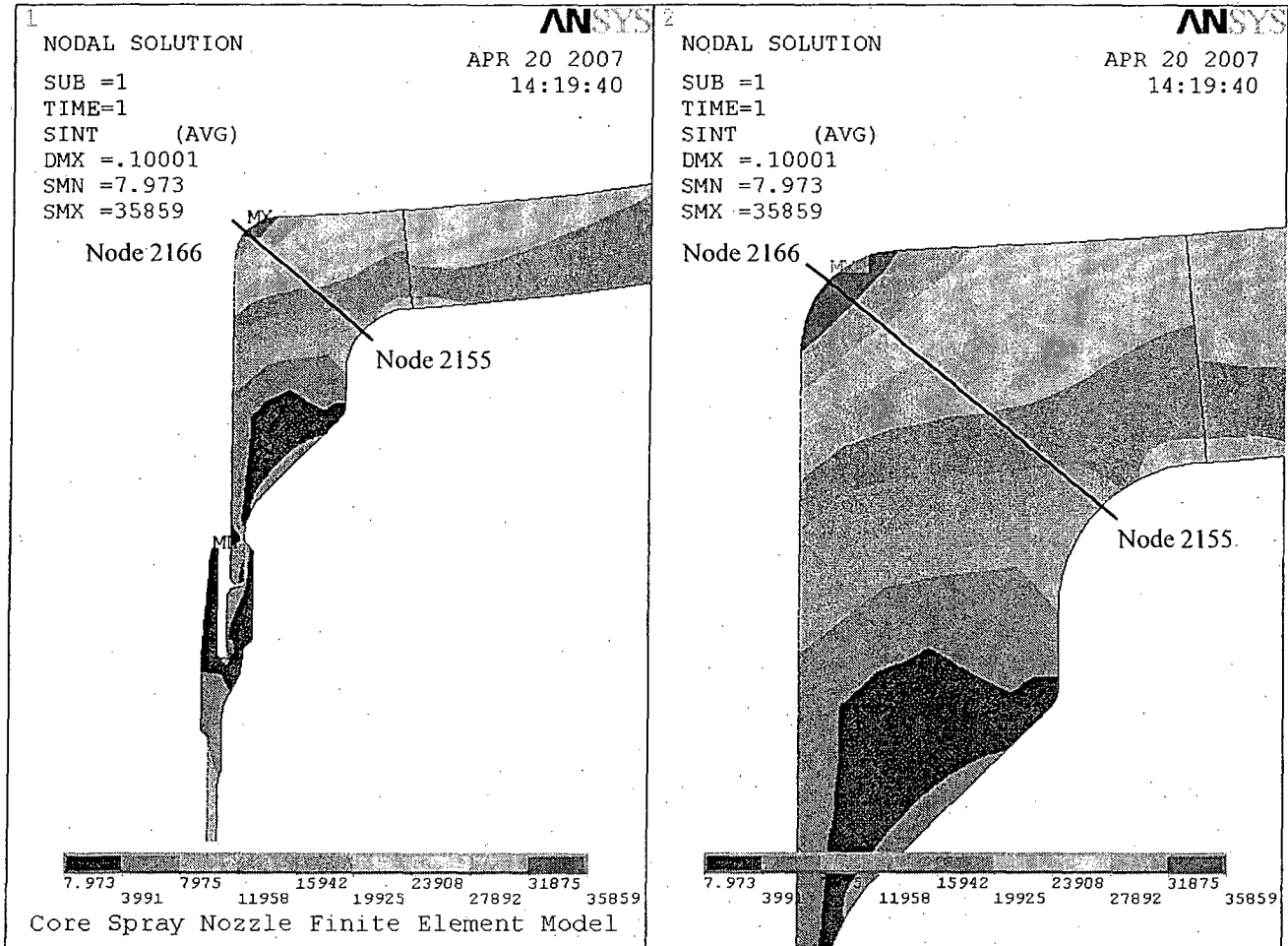


Figure 7: Blend Radius Limiting Pressure Stress Location, Node 2166



The stress intensity time history for the critical safe end and blend radius paths were extracted using the ANSYS post-processing file "extract100.inp" for 100% flow. This produced the two files, "SE_F100.out" and "BR_F100.out," which contain the thermal stress history. The membrane plus bending stresses and total stresses for the Green's Functions were extracted from these files to produce the four files "SE_F100.cln, BR_F100.cln" and "SE_F100_INSIDE.RED, BR_F100_INSIDE.RED."

The stress intensity time history for the critical safe end and blend radius paths were extracted using the ANSYS post-processing file "extract0.inp" for 0% flow. This produced the two files, "SE_F0.out" and "BR_F0.out," which contain the thermal stress history. The membrane plus bending stresses and total stresses for the Green's Functions were extracted from these files to produce the four files "SE_F0.cln, BR_F0.cln" and "SE_F0_INSIDE.RED, BR_F0_INSIDE.RED."

As the models were run with a 400°F step change in temperature, and the Green's Functions are for a 1°F step change in temperature, all data values were divided by 400. The governing Green's Functions for the core spray nozzle during 100% flow and 0% flow are shown in Figure 8 through Figure 11. The data for the Green's Functions is included in the files:

0% Flow Rate:

SE_Flow0_T_Green.xls
SE_Flow0_M+B-Green.xls
BLEND_Flow0_M+B_Green.xls
BLEND_Flow0_T_Green.xls

100 Flow Rate:

SE_Flow100_T_Green.xls
SE_Flow100_M+B-Green.xls
BLEND_Flow100_M+B_Green.xls
BLEND_Flow100_T_Green.xls

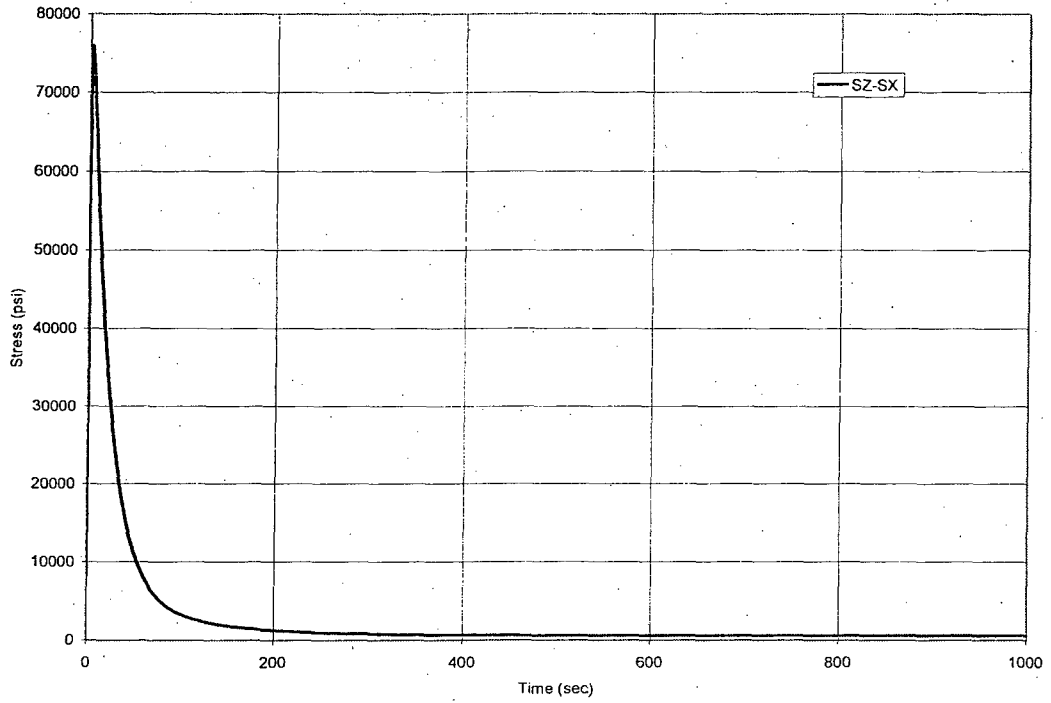


Figure 8: Safe End Total Stress History, 100% Flow

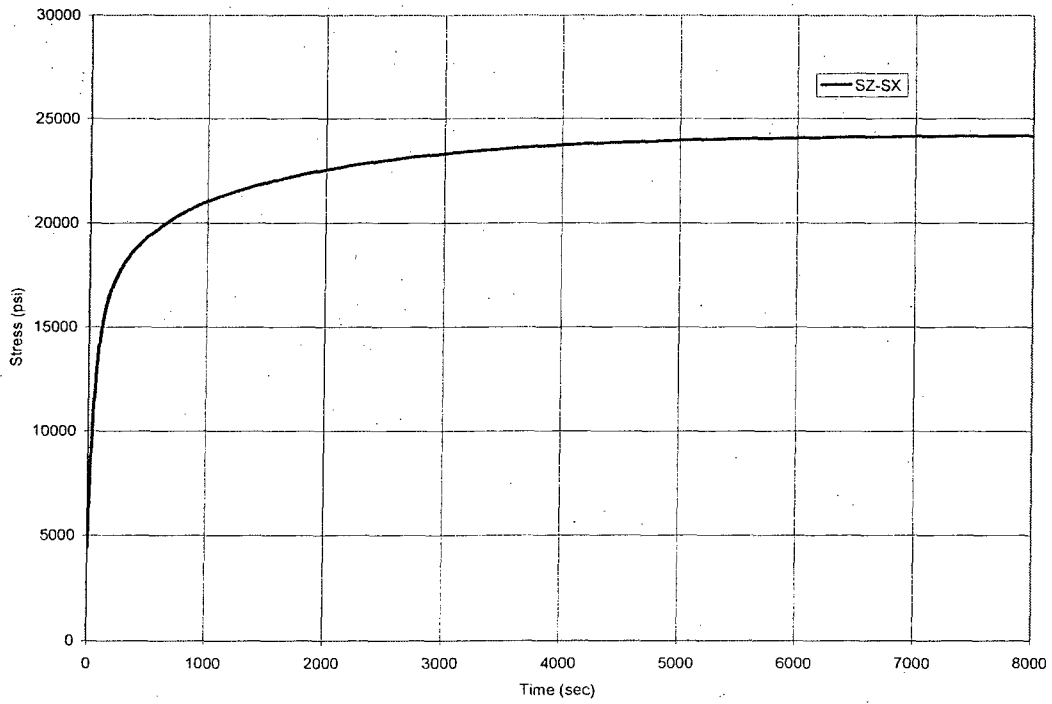


Figure 9: Blend Radius Total Stress History, 100% Flow

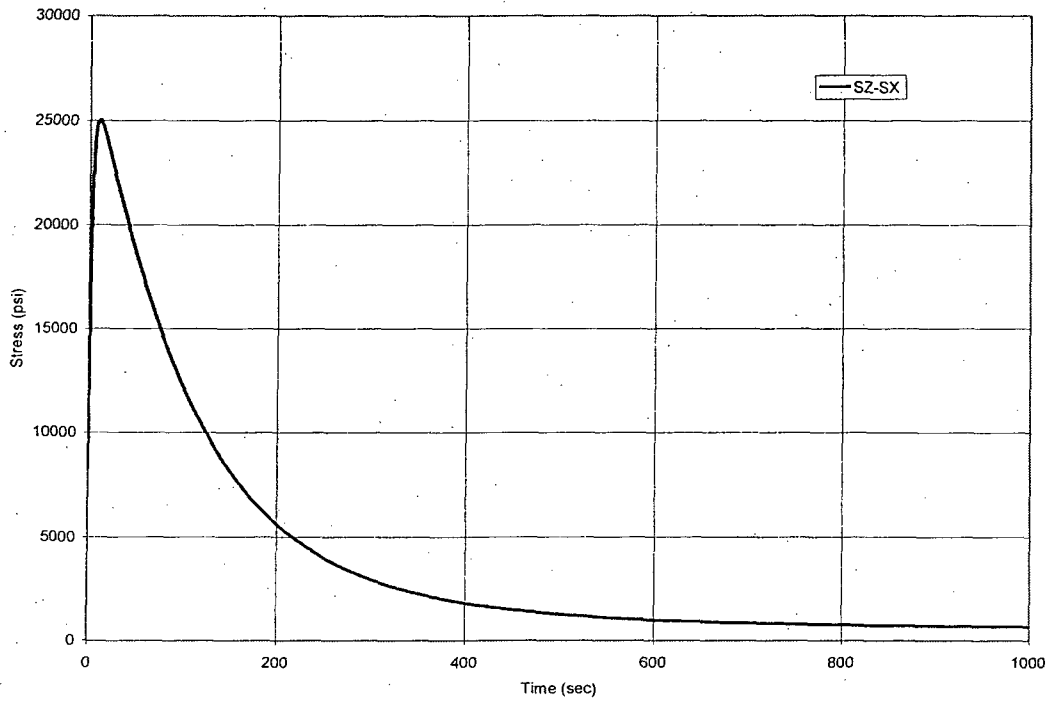


Figure 10: Safe End Total Stress History, 0% Flow

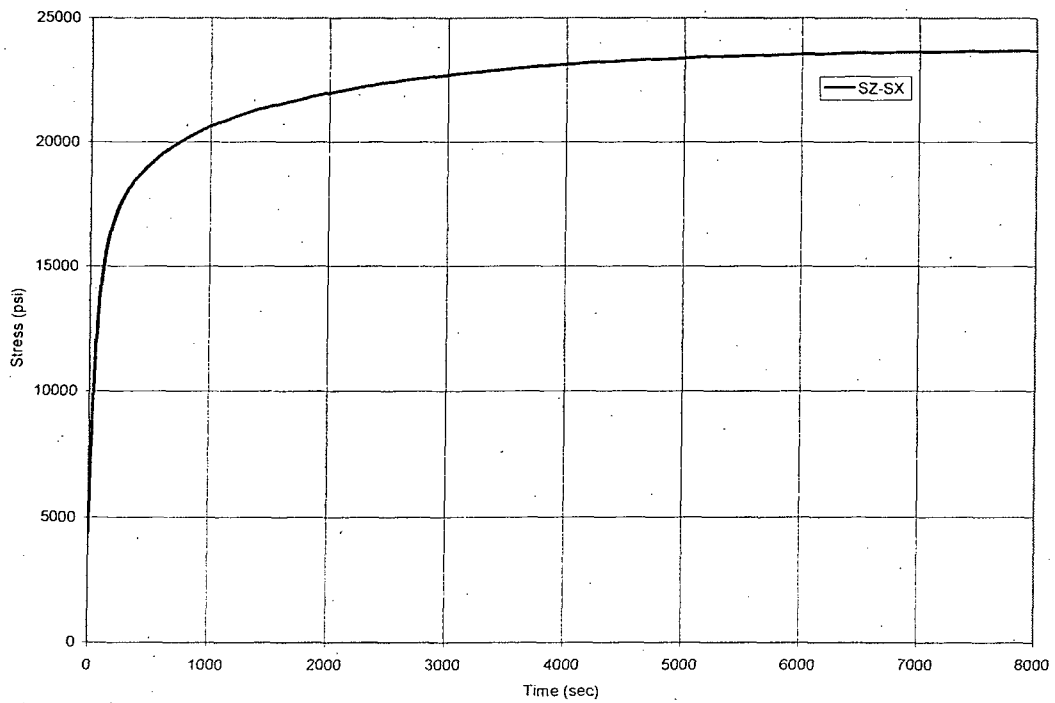


Figure 11: Blend Radius Total Stress History, 0% Flow



The pressure stress intensities for the safe end and blend radius paths were extracted using the ANSYS post-processing file "extractP.inp." This produced two files, SE_P.OUT for the safe end and BR_P.OUT for the blend radius.

Results of the internal pressure load case for Node 2166 (blend radius) is a total stress intensity of 35,860 psi and for Node 3719 (safe end), a total stress intensity of 12,030 psi. The membrane plus bending stress intensity at Node 2166 and Node 3719 are 34970 psi and 12,020 psi, respectively. Table 14 shows the final pressure results for the safe end and blend radius.

Table 14: Pressure Results (1,000 psi)

Location	Membrane plus Bending Stress Intensity (psi)	Total Stress Intensity (psi)
Safe End	12,020	12,030
Blend Radius	34,970	35,860

Results were also extracted from the vessel portion of the model to verify the accuracy of the results obtained from the ANSYS model, and to check the results due to the use of the 2.0 multiplier on the vessel radius. These results are contained in the file VESSEL_P.OUT. The radius of the finite element model (FEM) was multiplied by a factor of 2.0 [1] to account for the fact that the vessel portion of the 2D axisymmetric model is a sphere, but the true geometry is the intersection of two cylinders.

The equation for the membrane hoop stress in a sphere is:

$$\sigma = \left(\frac{(pressure) \times (radius)}{2 \times thickness} \right)$$

Considering a vessel base metal radius, R, of 105.906 inches increased by a factor of 2.0, a vessel base metal thickness, t, of 5.4375 inches, and an applied pressure, P, of 1,000 psi, the calculated stress for a sphere is $PR/(2t) = 19,477$ psi. This compares very well with the remote vessel wall membrane hoop stress from the ANSYS result file, VESSEL_P.OUT, of 18,530 psi. Thus, considering the peak total pressure stress of 35,860 psi reported above, the stress concentrating effect of the nozzle corner is $35,860/19,477 = 1.84$. In other words, the peak nozzle corner stress is 1.84 times higher than nominal vessel wall stress for the 2D axisymmetric model.

The equation for the membrane hoop stress in a cylinder is:

$$\sigma = \left(\frac{(\text{pressure}) \times (\text{radius})}{\text{thickness}} \right)$$

Based on the previous dimensions, the calculated stress for a cylinder without the 2.0 factor is 19,477 psi. Increasing this by a factor of 1.84 yields an expected peak nozzle corner stress of 35,838 psi, which would be expected from a cylindrical geometry that is representative of the nozzle configuration. Therefore, the result from the ANSYS file for the peak nozzle corner stress (35,860 psi) is close to the peak nozzle corner stress for a cylindrical geometry because of the use of the 2.0 multiplier. This is consistent with SI's experience where a factor of two increase in radius is typical for representing the three-dimensional (3D) effect in a 2D axisymmetric model.



5.0 REFERENCES

1. SI Calculation No. VY-16Q-308, Revision 0, "Core Spray Nozzle Finite Element Model".
2. ANSYS, Release 8.1A1 (w/Service Pack 1), ANSYS, Inc., June 2004.
3. American Society of Mechanical Engineers, Boiler and Pressure Vessel Code, Section II, Part D, 1998 Edition, 2000 Addenda.
4. J. P. Holman, "Heat Transfer," 4th Edition, McGraw-Hill, 1976.
5. J. P. Holman, "Heat Transfer," 5th Edition, 1981.
6. Entergy Design Input Record (DIR) EC No. 1773, Revision 0, "Environmental Fatigue Analysis for Vermont Yankee Nuclear Power Station," 7/3/07, SI File No. VY-16Q-209.
7. N. P. Cheremisinoff, "Heat Transfer Pocket Handbook," Gulf Publishing Co., 1984.
8. CB&I RPV Stress Report, Section T7, "Thermal Analysis of Core Spray Nozzle, Vermont Yankee Reactor Vessel, CB&I Contract 9-6201, SI File No. VY-16Q-206.



APPENDIX A
FINITE ELEMENT ANALYSIS FILES

ANSYS Input Files

File Name	Description
vy_csn_geom.inp	ANSYS input file includes the geometry and material properties
Heat Transfer Coefficients.xls	Excel file to calculate Heat Transfer coefficients
VY_16Q_P.inp	ANSYS input file for the pressure stress analysis
VY_16Q_T100.inp	ANSYS input file for the thermal analysis, 100% flow rate
VY_16Q_T0.inp	ANSYS input file for the thermal analysis, 0% flow rate
VY_16Q_ST100.inp	ANSYS input file for the thermal stress analysis, 100% flow rate
VY_16Q_ST0.inp	ANSYS input file for the thermal stress analysis, 0% flow rate
extract100.inp	ANSYS input file to extract the limiting paths, 100% flow rate
extract0.inp	ANSYS input file to extract the limiting paths, 100% flow rate
extractP.inp	ANSYS input file to extract the limiting paths
extractVessel.inp	ANSYS input file to extract the membrane stress in the vessel wall



ANSYS Output Files

File Name	Description
BR_F100.out	ANSYS output file, Results of running: extract100.inp, Blend Radius 100% Flow
SE_F100.out	ANSYS output file, Results of running: extract100.inp, Safe End 100% Flow
BR_F100.cln	Reduced ANSYS output file, contains the stress values in time, Blend Radius 100% Flow
SE_F100.cln	Reduced ANSYS output file, contains the stress values in time, Safe End 100% Flow
BR_F100_INSIDE.RED	Reduced ANSYS output file, contains detailed stress values in time, Blend Radius 100% Flow
SE_F100_INSIDE.RED	Reduced ANSYS output file, contains detailed stress values in time, Safe End 100% Flow
BR_F0.out	ANSYS output file, Results of running: extract0.inp, Blend Radius 0% Flow
SE_F0.out	ANSYS output file, Results of running: extract0.inp, Safe End 0% Flow
BR_F100.cln	Reduced ANSYS output file, contains the stress values in time, Blend Radius 0% Flow
SE_F100.cln	Reduced ANSYS output file, contains the stress values in time, Safe End 0% Flow
BR_F0_INSIDE.RED	Reduced ANSYS output file, contains detailed stress values in time, Blend Radius 0% Flow
SE_F0_INSIDE.RED	Reduced ANSYS output file, contains detailed stress values in time, Safe End 0% Flow



Structural Integrity Associates, Inc.

File No.: VY-16Q-310

NEC-JH_13

CALCULATION PACKAGE

Project No.: VY-16Q

PROJECT NAME:

Environmental Fatigue Analysis of VYNPS

CONTRACT NO.:

10150394

CLIENT:

Entergy Nuclear Operations, Inc.

PLANT:

Vermont Yankee Nuclear Power Station

CALCULATION TITLE:

Fatigue Analysis of Core Spray Nozzle

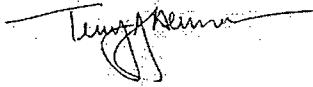
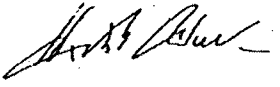

Document Revision	Affected Pages	Revision Description	Project Manager Approval Signature & Date	Preparer(s) & Checker(s) Signatures & Date
0	1-27 Appendix: A1-A2	Initial issue	Terry J. Herrmann  07/26/2007	Roland Horvath/ Minghao Qin  07/26/2007 Minghao Qin 07/26/2007 Carl Limpus  07/26/2007



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1.0 OBJECTIVE

The purpose of this calculation is to perform a revised fatigue analysis for the core spray nozzle. Two locations will be analyzed for fatigue acceptance: the blend radius (SA508 Class II) and the safe end (SB 166 N06600). Both locations are chosen based on the highest overall stress of the analysis performed in Reference [1]. A revised fatigue usage will be determined for both locations, the nozzle forging and safe end, respectively. In the end, the environmental fatigue usage factors will be determined for the limiting locations.

2.0 METHODOLOGY

In order to provide an overall approach and strategy for evaluating the core spray nozzle, the Green's Function methodology and associated ASME Code stress and fatigue analyses are described in this section.

Revised stress and fatigue analyses are being performed for the core spray nozzle using ASME Code, Section III methodology. These analyses are being performed to address license renewal requirements to evaluate environmental fatigue for this component in response to Generic Aging Lessons Learned (GALL), Report [12] requirements. The revised analysis is being performed to refine the fatigue usage so that an environmental fatigue factor can be determined for subsequent license renewal efforts.

Two sets of rules are available under ASME Code, Section III, Class 1 [8]. Subparagraph NB-3600 of Section III provides simplified rules for analysis of piping components, and NB-3200 allows for more detailed analysis of vessel components. The NB-3600 piping equations combine by absolute sum the stresses due to pressure, moments and through wall thermal gradient effects, regardless of where within the pipe cross-section the maximum value of the components of stress are located. By considering stress signs, affected surface (inside or outside) and azimuthal position, the stress ranges may be significantly reduced. In addition, NB-3600 assigns stress indices by which the stresses are multiplied to conservatively incorporate the effects of geometric discontinuities. In NB-3200, stress indices are not required, as the stresses are calculated by finite element analysis and consider applicable stress concentration factors. In addition, NB-3200 methodology accounts for the different locations within a component where stresses due to thermal, pressure or other mechanical loading are a maximum. This generally results in a net reduction of the stress ranges and consequently, in the calculated fatigue usage. Article 4 [14] methodology was originally used to evaluate the core spray nozzle. NB-3200 methodology, which is the modern day equivalent to Article 4, is used in this analysis to be consistent with the Section III design bases for this component, as well as to allow a more detailed analysis of this component. In addition, several of the conservatisms originally used in the original core spray nozzle evaluation (such as grouping of transients) are removed in the current evaluation so as to achieve a more refined CUF.

For the core spray nozzle evaluated as a part of this work, stress histories will be computed by a time integration of the product of a pre-determined Green's Function and the transient data. This Green's Function integration scheme is similar in concept to the well-known Duhamel theory used in



structural dynamics. A detailed derivation of this approach and examples of its application to specific plant locations is contained in Reference [11]. A general outline is provided in this section.

The steps involved in the evaluation are as follows:

- Develop finite element model
- Develop heat transfer coefficients and boundary conditions for the finite element model
- Develop Green's Functions
- Develop thermal transient definitions
- Perform stress analysis to determine stresses for all thermal transients
- Perform fatigue analysis

A Green's Function is derived by using finite-element methods to determine the transient stress response of the component to a step change in loading (usually a thermal shock). The critical location in the component is identified based on the maximum stress, and the thermal stress response over time is extracted for this location. This response to the input thermal step is the "Green's Function." Figure 8 shows a typical set of two Green's Functions, each for a different set of heat transfer coefficients (representing different flow rate conditions).

To compute the thermal stress response for an arbitrary transient, the loading parameter (usually local fluid temperature) is deconstructed into a series of step-loadings. By using the Green's Function, the response to each step can be quickly determined. By the principle of superposition, these can be added (algebraically) to determine the response to the original load history. The result is demonstrated in Figure 9. The input transient temperature history contains five step-changes of varying size, as shown in the upper plot in Figure 9. These five step changes produce the five successive stress responses in the lower plot shown in Figure 9. By adding all five response curves, the real-time stress response for the input thermal transient is computed.

The Green's Function methodology produces identical results compared to running the input transient through the finite element model. The advantage of using Green's Functions is that many individual transients can be run with a significant reduction of effort compared to running all transients through the finite element model. The trade-off in this process is that the Green's Functions are based on constant material properties and heat transfer coefficients. Therefore, these parameters are chosen to bound all transients that constitute the majority of fatigue usage, i.e., the heat transfer coefficients at 300°F bound the cold water injection transient. In addition, the instantaneous value for the coefficient of thermal expansion is used instead of the mean value for the coefficient of thermal expansion. This conservatism is more than offset by the benefit of not having to analyze every transient, which was done in the VY core spray nozzle evaluation.

Once the stress history is obtained for all transients using the Green's Function approach, the remainder of the fatigue analysis is carried out using traditional methodologies in accordance with ASME Code, Section III requirements.

Fatigue calculations are performed in accordance with ASME Code, Section III, Subsection NB-3200 methodology. Fatigue analysis is performed for the two limiting locations (one in the safe end



and one in the nozzle forging, representing the two materials of the nozzle assembly) using the Green's Functions developed for these two locations and 60-year projected cycle counts.

Three Structural Integrity utility programs will be used to perform the fatigue analysis. The first two calculate stresses in response to transients. The transients analyzed are those described in the thermal cycle diagrams [2] for the core spray nozzle. These transients are shown in Figure 1– Figure 6. The temperatures and pressures for these transients have been modified to account for power uprate [3]. The power uprate pressures and temperatures were used for this analysis. The last program calculates fatigue based on the stress output. The three programs are STRESS.EXE, P-V.EXE, and FATIGUE.EXE. The first program, STRESS.EXE, calculates a stress history in response to a thermal transient using a Green's Function. The second program, P-V.EXE, reduces the stress history to peaks and valleys, as required by ASME Code fatigue evaluation methods. The third program, FATIGUE.EXE, calculates fatigue from the reduced peak and valley history using ASME Code, Section III range-pair methodology. All three programs are explained in detail and have been independently verified for generic use in the Reference [4] calculation.

In order to perform the fatigue analysis, Green's Functions are developed using the finite element model. Then, input files with the necessary data are prepared and the three utility computer programs are run. The first program (STRESS.EXE) requires the following three input files:

- Input file "GREEN.DAT": This file contains the Green's Function for the location being evaluated. For each flow condition, two Green's Functions are determined: a membrane plus bending stress intensity Green's Function and a total stress intensity Green's Function. This allows computation of total stress, as well as membrane plus bending stress, which is necessary to compute K_e per ASME Code, Section III requirements.
- Input file "GREEN.CFG": This file is a configuration file containing parameters that define the Green's Function (i.e., number of points, temperature drop analyzed, etc.).
- Input file "TRANSNT.INP": This file contains the input transient definition for all thermal transients to be analyzed for the location being evaluated.

Pressure and piping stress intensities are also included for each transient case, based on pressure stress results from finite element analysis and attached piping load calculations.

The second program (P-V.EXE) simply extracts only the maxima and minima stress (i.e., the peaks and valleys) from the stress histories generated by program STRESS.EXE.

The third program (FATIGUE.EXE) performs the ASME Code peak event-pairing required to calculate a fatigue usage value. The input data consists of the output peak and valley history from program P-V.EXE, and a configuration input file that provides ASME Code configuration data relevant to the fatigue analysis (i.e., K_e parameters, S_m , Young's modulus, etc.). The output is the final fatigue calculation for the location being evaluated.

The Green's Function methodology described above uses standard industry stress and fatigue analysis practices, and is the same as the methodology used in typical stress reports. Special approval for the use of this methodology is therefore not required.



3.0 ANALYSIS

The transients analyzed for the core spray nozzle were developed based on the definitions in the original RPV Design Specification [10], as modified for EPU [3], as well as more recent definitions based on BWR operating experience [2] for BWR. The final transients evaluated in the stress and fatigue analyses are shown in Figure 1 thru Figure 6.

The fatigue analysis involves the preparing of input files for, and running of three programs [4]. The programs STRESS.EXE and P-V.EXE are run together through the use of a batch file. The program FATIGUE.EXE is run after processing the output from P_V.EXE.

The steps associated with this process are described in the following sub-sections.

3.1 Transient Definitions (for program STRESS.EXE)

The program STRESS.EXE requires the following three input files for analyzing an individual transient:

- Green.dat. There are 8 stress history functions obtained from References [1]. They represent the membrane plus bending and total stress intensities at the blend radius and safe end locations. Both of the blend radius and the safe end have two stress history functions for flow condition of 0% and 100%.
- Green.cfg is configured as described in Reference [4].
- Transnt.inp. These files are created to represent the selected transients obtained from the thermal cycle diagrams [2] and redefined by power uprate [3]. Table 1 and Table 2 contain the loading defined for each transient. Based upon the thermal cycle diagram for the RPV and the core spray nozzle, the transients are split into the following groups based upon flow rate:
 - Transients 02, 03, 11, 14, 21-23 and 24 are run at 0% flow.
 - Transient 30 runs at 100% flow rate per [3]. The transient of emergency shutdown is numbered as 30.

The remaining transients are not included in this analysis, as temperature changes from them are considered negligible to have impact on the results.

3.2 Peak and Valley Points of the Stress History (for program P-V.EXE)

The program P-V.exe is then run to extract the peaks and valleys from the STRESS.OUT file produced by the STRESS.EXE program. The only input required for this program is STRESS.OUT and it outputs all the peaks and valleys to P-V.OUT. Columns 2 through 5 of Table 5 (for the blend radius) and Table 6 (for the safe end) show the final peak and valley output. The pressure for column six is then filled in using the thermal cycle diagrams. Pressure and piping loads have to be added to the peak and valley points to calculate the final stress values used for fatigue analysis.



3.3 Pressure Load

The pressure stress associated with a 1000 psi internal pressure was determined in Reference [1]. These values are as follows:

Location	Membrane plus Bending Stress Intensity (psi)	Total Stress Intensity (psi)
Safe End	12,020	12,030
Blend Radius	34,970	35,860

These pressure stress values for each location were linearly scaled according to the pressure of the transient. The actual pressure for column 6 of Table 5 and Table 6 is obtained from Reference [2] and shown in Tables 1 and 2. The scaled pressure stress values are shown in columns 7 and 8 of Table 5 (for the blend radius) and Table 6 (for the safe end).

The pressure stress is combined with the peak and valley points to calculate the final stress values used for fatigue analysis.

3.4 Attached Piping Loads

Additionally, the piping stress intensity (stress caused by the attached piping) was determined. These piping forces and moments are determined as shown in Figure 7.

The following formulas are used to determine the maximum stress intensity in the nozzle at the two locations of interest. From engineering statics, the piping loads at the end of the model can be translated to the first cut (blend radius) and second cut (safe end) locations using the following equations:

$$\begin{aligned} \text{For Cut I: } & (M_x)_1 = M_x - F_y L_1 \\ & (M_y)_1 = M_y + F_x L_1 \end{aligned}$$

$$\begin{aligned} \text{For Cut II: } & (M_x)_2 = M_x - F_y L_2 \\ & (M_y)_2 = M_y + F_x L_2 \end{aligned}$$



The total bending moment and shear loads are obtained using the equations below:

$$\begin{aligned} \text{For Cut I: } M_{xy} &= \sqrt{(M_x)_1^2 + (M_y)_1^2} \\ F_{xy} &= \sqrt{(F_x)_1^2 + (F_y)_1^2} \end{aligned}$$

$$\begin{aligned} \text{For Cut II: } M_{xy} &= \sqrt{(M_x)_2^2 + (M_y)_2^2} \\ F_{xy} &= \sqrt{(F_x)_2^2 + (F_y)_2^2} \end{aligned}$$

The distributed loads for a thin-walled cylinder are obtained using the equations below:

$$\begin{aligned} N_z &= \frac{1}{\pi R_N} \left[\frac{1}{2} F_z + \frac{M_{xy}}{R_N} \right] \\ q_N &= \frac{1}{\pi R_N} \left[F_{xy} - \frac{M_z}{2R_N} \right] \end{aligned}$$

To determine the primary stresses, PM, due to internal pressure and piping loads, the following equations are used.

For Cut I, using thin-walled equations:

$$\begin{aligned} (P_M)_z &= \frac{Pa_N}{2t_N} + \frac{Nz}{t_N} \\ (P_M)_\theta &= \frac{Pa_N}{t_N} \\ (P_M)_R &= -P \\ \tau_M &= \frac{q_N}{t_N} \\ SI_{MAX} &= 2 \sqrt{\left(\frac{(P_M)_\theta - (P_M)_R}{2} \right)^2 + (\tau_M)_{z\theta}^2} \\ \text{or} \\ SI_{MAX} &= 2 \sqrt{\left(\frac{(P_M)_z - (P_M)_R}{2} \right)^2 + (\tau_M)_{z\theta}^2} \end{aligned}$$

Where:

L_1 = The length from the end of the nozzle where the piping loads are applied to the location of interest in the blend radius.



- L_2 = The length from the end of the nozzle where the piping loads are applied to the location of interest in the safe end.
- M_{xy} = The maximum bending moment in the xy plane.
- F_{yx} = The maximum shear force in the xy plane.
- N_z = The normal force per inch of circumference applied to the end of the nozzle in the z direction.
- q_N = The shear force per inch of circumference applied to the nozzle.
- R_N = The mid-wall nozzle radius.

Because pressure was not considered in this analysis, the equations used for Cut I are valid for Cut II. In addition, the equations can be simplified as follows:

$$(P_M)_z = \frac{N_z}{t_N}$$

$$(P_M)_\theta = 0$$

$$(P_M)_R = 0$$

$$\tau_M = \frac{q_N}{t_N}$$

$$SI_{MAX} = 2(\tau_M)_{z\theta}$$

or

$$SI_{MAX} = 2\sqrt{\left(\frac{N_z}{2t_N}\right)^2 + (\tau_M)_{z\theta}^2}$$

Per Reference [5], the core spray nozzle piping loads are as follows:

$F_x = 2,500$ lbs	$M_x = 22,000$ ft-lb = 264,000 in-lb
$F_y = 4,600$ lbs	$M_y = 7,100$ ft-lb = 85,200 in-lb
$F_z = 1,700$ lbs	$M_z = 8,800$ ft-lb = 150,600 in-lb

The location of the nozzle piping loads is assumed to be at the end of the connection of the safe end and the attached pipe. Therefore, the L1 is equal to 30.817 inches and the L2 is equal to 0.303 inches. The calculations for the blend radius and safe end are shown in Table 3 and Table 4. The first cut location is the middle of Green's Function cross section for the blend radius (Node 2181) per [1], and the second cut is from Node 3719 (inside) to Node 3737 (outside). The maximum stress intensities, due to piping loads are 322.52 psi at the blend radius and 6949.94 psi at the safe end, respectively. The piping load sign is set as the same as the thermal stress sign.

These piping stress values are scaled assuming no stress occurs at an ambient temperature of 70°F and the full values are reached at reactor design temperature, 575°F [2]. The scaled piping stress values are shown in columns 9 and 10 of Table 5 and Table 6. Columns 11 and 12 of Table 5 and Table 6 show the summation of all stresses for each thermal peak and valley stress point.

3.5 Fatigue Analysis (for program FATIGUE.EXE)

The number of cycles projected for the 60-year operating life is used for each transient, as obtained from Reference [2]. Column 13 in Table 5 and Table 6 shows the number of cycles associated with each transient.

The program FATIGUE.EXE performs the "ASME Code style" peak event pairing required to calculate a fatigue usage value. The input data for FATIGUE.CFG is as follows:

	Blend Radius (SA508 Class II)	Safe End (N06600)	Piping (Stainless Steel)
Parameters m and n for Computing K_c	2.0 & 0.2 (low alloy steel) [8]	1.7 & 0.3 [8]	1.7 & 0.3 [8]
Design Stress Intensity Values, S_m	26,700 psi [6] @ 600°F	23,300 psi [6] @ 600°F	17,000 psi [6] @ 600°F
Elastic Modulus from Applicable Fatigue Curve	30.0×10^6 psi [8]	28.3×10^6 psi [8]	28.3×10^6 psi [8]
Elastic Modulus Used in Finite Element Model (300°F)	26.7×10^6 psi [1]	29.8×10^6 psi [1]	27.0×10^6 psi [1]
The Geometric Stress Concentration Factor K_t	1.0	4.0 <small>See Note</small>	1.8 [14]

Note: Conservative bounding value per ASME Code, Section NB-3600 to cover thread and weld regions.

The results of the fatigue analyses are presented in Table 7 through Table 9 for the blend radius, safe end and stainless steel piping for 60 years, respectively.

The Core Spray piping adjacent to the safe end was also analyzed because of its proximity to the maximum safe end thermal stress location. For this fatigue analysis, the stress results of the safe end were used with stainless steel material properties and a value of 1.8 was selected for K_t at the weld location, based on the maximum value given in ASME Code, Section III, table NB-3681(a)-1 [8].

The results described are contained in EXCEL files BRresults.xls and SResults.xls, which are contained in the computer files.

4.0 FATIGUE USAGE RESULTS

The blend radius Cumulative Usage Factor (CUF) from system cycling is 0.0043 for 60 years. The safe end CUF is 0.0184 and the CUF of stainless steel piping is 0.0005 for 60 years.



5.0 ENVIRONMENTAL FATIGUE ANALYSIS

Per Reference [7], the dissolved Oxygen (DO) calculation shows the overall HWC availability is 47%. It means the pre-HWC is 53%.

The fatigue calculation will be re-performed for the nozzle base material, since cladding is structurally neglected in modern-day fatigue analyses, per ASME Code, Section III, NB-3122.3 [8]. This is also consistent with Sections 5.7.1 and 5.7.4 of NUREG/CR-6260 [9]. Therefore, the cladding will be neglected and EAF assessment of the nozzle base material is performed.

For the blend radius location, the environmental fatigue factors for pre-HWC and post-HWC are 11.14 and 8.82 from Table 4 of Reference [7]. It results in an EAF adjusted CUF of $(11.14 \times 53\% + 8.82 \times 47\%) \times 0.0043 = 0.0432$ for 60 years, which is acceptable (i.e., less than the allowable value of 1.0). The overall environmental multiplier is 10.05.

For the safe end location, the environmental fatigue factors for post-HWC and pre-HWC are all 1.49 from Reference [13]. It results in an EAF adjusted CUF of $1.49 \times 0.0184 = 0.0274$ for 60 years, which is acceptable (i.e., less than the allowable value of 1.0). The overall environmental multiplier is 1.49.

For the stainless steel piping, the environmental fatigue factors for post-HWC and pre-HWC are all 8.36 from Table 4 of Reference [7]. It results in an EAF adjusted CUF of $8.36 \times 0.0005 = 0.00418$ for 60 years, which is acceptable (i.e., less than the allowable value of 1.0). The overall environmental multiplier is 8.36.

A Fatigue Environmental Multiplier of 1.49 for Ni-Cr-Fe was applied to the safe end fatigue usage and 8.36 for stainless steel to the piping. This results in the safe end being the limiting location for fatigue.



6.0 REFERENCES

- 1 SI Calculation No. VY-16Q-309, Revision 0, "Core Spray Nozzle Green's Functions."
- 2 "Reactor Thermal Cycles for 60 Years of Operation," Attachment 1 of Entergy Design Input Record (DIR), Revision 1, EC No. 1773, Revision 0, "Environmental Fatigue Analysis for Vermont Yankee Nuclear Power Station," 7/26/07, SI File No. VY-16Q-209.
- 3 GE Certified Design Specification No. 26A6019, Revision 1, "Reactor Vessel - Extended Power Uprate," August 29, 2003, SI File No. VY-05Q-236.
- 4 Structural Integrity Associates Calculation (Generic) No. SW-SPVF-01Q-301, Revision 0, "STRESS.EXE, P-V.EXE, and FATIGUE.EXE Software Verification."
- 5 VY Drawing 5920-0024, Revision 11, Sht. No. 7, "Reactor Vessel," (GE Drawing No. 919D294), SI File No. VY-05Q-241.
- 6 American Society of Mechanical Engineers, Boiler and Pressure Vessel Code, Section II, Part D, 1998 Edition, 2000 Addenda.
- 7 SI Calculation No. VY-16Q-303, Revision 0, "Environmental Fatigue Evaluation of Reactor Recirculation Inlet Nozzle and Vessel Shell Bottom Head."
- 8 American Society of Mechanical Engineers, Boiler and Pressure Vessel Code, Section III Subsection NB, 1998 Edition, 2000 Addenda.
- 9 NUREG/CR-6260 (INEL-95/0045), "Application of NUREG/CR-5999 Interim Fatigue Curves to Selected Nuclear Power Plant Components," March 1995.
- 10 GE Design Specification No. 21A1115, Revision 4, "Vermont Yankee Reactor Pressure Vessel," October 21, 1969, SI File No. VY-05Q-210.
- 11 Kuo, A. Y., Tang, S. S., and Riccardella, P. C., "An On-Line Fatigue Monitoring System for Power Plants, Part I - Direct Calculation of Transient Peak Stress Through Transfer Matrices and Green's Functions," ASME PVP Conference, Chicago, 1986.
- 12 NUREG-1801, Revision 1, "Generic Aging Lessons Learned (GALL) Report," U. S. Nuclear Regulatory Commission, September 2005.
- 13 EPRI Report No. TR-105759, "An Environmental Factor Approach to Account for Reactor Water Effects in Light Water Reactor Pressure Vessel and Piping Fatigue Evaluations," December 1995.
- 14 American Society of Mechanical Engineers, Boiler and Pressure Vessel Code, Section III, Subsection A, Article 4, 1965 Edition with Winter 1966 Addenda.

Table 1: Blend Radius Transients^{1,2,3}

Transient Number	Time (s)	Temp (°F)	Time Step (s)	Pressure (psig)	Flow Rate (GPM)
2. Design HYD Test 120 Cycles	---	100	---	0	
				1100	
				50	
3. Startup 300 Cycles	0	100		0	0
	16164	549	16164	1010	(0%)
	24164	549	8000	1010	
11. Loss of Feedwater Pumps 10 Cycles	0	526		1010	0
	3	526	3	1190	(0%)
	13	526	10	1135	
	233	300	220	1135	
	2213	500	1980	1135	
	2393	300	180	885	
	6893	500	4500	1135	
	7313	300	420	675	
	7613	300	300	675	
	11213	400	3600	240	
	16577	549	5364	1010	
	16637	549	60	1010	
	16638	542	1	1010	
	16698	542	60	1010	
	16699	526	1	1010	
24699	526	8000	1010		
14. SRV Blowdown 1 Cycle	0	526		1010	0
	600	375	600	400	(0%)
	11580	70	10980	50	
	19580	70	8000	50	
21-23. Shutdown 300 Cycles	0	549		1010	0
	6264	375	6264	50	(0%)
	6864	330	600	50	
	16224	100	9360	50	
24224	100	8000	50		
24. Hydrostatic Test 1 Cycle	---	100	---	50	
				1563	
				50	
30. Emergency Shut Down 1 Cycle	0	549		1010	3200
	10	406	10	250	(100%)
	11	70	1	250	
	8011	70	8000	0	

Note:

1. Instant temperature change is 1 sec.
2. This is due to the length of the Green's Function. The transients are plotted using an 8000 second steady state increment.
3. The number of cycles for 60 years is from Reference [2].

Table 2: Safe End Transients^{1, 2, 3}

Transient Number	Time (s)	Temp (°F)	Time Step (s)	Pressure (psig)	Flow Rate (GPM)
2. Design HYD Test 120 Cycles	---	100	---	0	
				1100	
				50	
3. Startup 300 Cycles	0	100		0	0
	16164	549	16164	1010	(0%)
	17164	549	1000	1010	
11. Loss of Feedwater Pumps 10 Cycles	0	526		1010	0
	3	526	3	1190	(0%)
	13	526	10	1135	
	233	300	220	1135	
	2213	500	1980	1135	
	2393	300	180	885	
	6893	500	4500	1135	
	7313	300	420	675	
	7613	300	300	675	
	11213	400	3600	240	
	16577	549	5364	1010	
	16637	549	60	1010	
	16638	542	1	1010	
	16698	542	60	1010	
	16699	526	1	1010	
17699	526	1000	1010		
14. SRV Blowdown 1 Cycle	0	526		1010	0
	600	375	600	400	(0%)
	11580	70	10980	50	
	12580	70	1000	50	
21-23. Shutdown 300 Cycles	0	549		1010	0
	6264	375	6264	50	(0%)
	6864	330	600	50	
	16224	100	9360	50	
17224	100	1000	50		
12. Hydrostatic Test 1 cycle	---	100	---	50	
				1563	
				50	
30. Emergency Shut Down 1 Cycle	0	549		1010	3200
	10	406	10	250	(100%)
	11	70	1	250	
	1011	70	1000	0	

Note:

1. Instant temperature change is 1 sec.
2. The transients are plotted using a 1000 second steady state increment. The difference is due to the length of the Green's Function for the safe end.
3. The number of cycles for 60 years is from Reference [2].

Table 3: Maximum Piping Stress Intensity Calculations for Blend Radius

Blend Radius External Piping Loads		
Parameters		
$F_x =$	2.50	kips
$F_y =$	4.60	kips
$F_z =$	1.70	kips
$M_x =$	264.00	in-kips
$M_y =$	85.20	in-kips
$M_z =$	105.60	in-kips
OD=	18.87	in
ID=	11.750	in
$R_N =$	7.65	in
L =	30.82	in
$t_N =$	3.56	in
$(M_x)_2 =$	122.24	in-kips
$(M_y)_2 =$	162.24	in-kips
$M_{xy} =$	203.14	in-kips
$F_{xy} =$	5.24	kips
$N_z =$	1.14	kips/in
$q_N =$	-0.07	kips/in
Primary Membrane Stress Intensity		
$PM_z =$	0.32	ksi
$\tau =$	-0.02	ksi
$SI_{max} =$	0.32	ksi
$SI_{max} =$	322.52	psi

Note: The locations for Cut I and Cut II were defined in Reference [1] for safe end and blend radius paths, respectively.

Table 4: Maximum Piping Stress Intensity Calculations for Safe End

Safe End External Piping Loads		
Parameters		
$F_x =$	2.50	kips
$F_y =$	4.60	kips
$F_z =$	1.70	kips
$M_x =$	264.00	in-kips
$M_y =$	85.20	in-kips
$M_z =$	105.60	in-kips
OD=	10.82	in
ID=	9.834	in
$R_N =$	5.16	in
L =	0.30	in
$t_N =$	0.49	in
$(M_x)_1 =$	262.60	in-kips
$(M_y)_1 =$	85.96	in-kips
$M_{xy} =$	276.31	in-kips
$F_{xy} =$	5.24	kips
$N_z =$	3.35	kips/in
$q_N =$	-0.31	kips/in
Primary Membrane Stress Intensity		
$PM_z =$	6.84	ksi
$\tau =$	-0.63	ksi
$SI_{max} =$	6.95	ksi
$SI_{max} =$	6949.94	psi

Note: The locations for Cut I and Cut II were defined in Reference [1] for safe end and blend radius paths, respectively.



Table 5: Blend Radius Stress Summary

1	2	3	4	5	6	7	8	9	10	11	12	13
Transient Number	Time (s)	Total Stress (psi)	M+B Stress (psi)	Temperature F	Pressure (psig)	Total Pressure Stress (psi)	M+B Pressure Stress (psi)	Total Piping Stress (psi)	M+B Piping Stress (psi)	Total Total Stress (psi)	Total M+B Stress (psi)	Number of Cycles (60 years)
2				100	0	0	0	19	19	19	19	120
				100	1100	39446	38467	19	19	39465	38486	120
				100	50	1793	1749	19	19	1812	1768	120
3	0	23700	12600	100	0	0	0	19	19	23719	12619	300
	24164	2100	3180	549	1010	36219	35320	306	306	38625	38806	300
11	0	3209	3644	526	1010	36219	35320	291	291	39719	39255	10
	3	3209	3644	526	1190	42673	41614	291	291	46174	45550	10
	526	10458	5374	330	1135	40701	39691	166	166	51325	45231	10
	2222	5488	1664	490	1122	40235	39236	268	268	45991	41168	10
	2860	11776	7444	321	911	32668	31858	160	160	44605	39462	10
	6903	5435	3621	495	1124	40307	39306	272	272	46013	43199	10
	8012	12577	6791	390	627	22484	21926	204	204	35265	28921	10
14	16640	2772	4370	542	1010	36219	35320	301	301	39292	39991	10
	16991	3389	4115	526	1010	36219	35320	291	291	39899	39726	10
	24699	3209	3644	526	1010	36219	35320	291	291	39719	39255	10
	0	3209	3644	526	1010	36219	35320	291	291	39719	39255	1
21-23	19580	25122	13197	70	50	1793	1749	0	0	26915	14946	1
	0	2103	3161	549	1010	36219	35320	306	306	38628	38787	300
24	24224	23680	12568	100	50	1793	1749	19	19	25492	14336	300
				100	50	1793	1749	19	19	1812	1768	1
				100	1563	56049	54658	19	19	56068	54677	1
30				100	50	1793	1749	19	19	1812	1768	1
	0	2040	2950	549	1010	36219	35320	306	306	38565	38576	1
	8011	25700	14900	70	0	0	0	0	0	25700	14900	1

- NOTES: Column 1: Transient number identification.
 Column 2: Time during transient where a maxima or minima stress intensity occurs from P-V.OUT output file.
 Column 3: Maxima or minima total stress intensity from P-V.OUT output file.
 Column 4: Maxima or minima membrane plus bending stress intensity from P-V.OUT output file.
 Column 5: Temperature per total stress intensity.
 Column 6: Pressure per Table 1.
 Column 7: Total pressure stress intensity from the quantity (Column 6 x 35,860)/1000 [1].
 Column 8: Membrane plus bending pressure stress intensity from the quantity (Column 6 x 34,970)/1000 [1].
 Column 9: Total external stress from calculation in Table 3, 322.52 x (Column 5 -70°F)/(575°F -70°F).
 Column 10: Same as Column 9, but for M+B stress.
 Column 11: Sum of total stresses (Columns 3, 7, and 9).
 Column 12: Sum of membrane plus bending stresses (Columns 4, 8, and 10).
 Column 13: Number of cycles for the transient (60 years).

Table 6: Safe End Stress Summary

1	2	3	4	5	6	7	8	9	10	11	12	13
Transient Number	Time (s)	Total Stress (psi)	M+B Stress (psi)	Temperature F	Pressure (psig)	Total Pressure Stress (psi)	M+B Pressure Stress (psi)	Total Piping Stress (psi)	M+B Piping Stress (psi)	Total Total Stress (psi)	Total M+B Stress (psi)	Number of Cycles (60 years)
2				100	0	0	0	413	413	413	413	120
				100	1100	13233	13222	413	413	13646	13635	120
				100	50	602	601	413	413	1014	1014	120
3	0	661	759	100	0	0	0	413	413	1074	1172	300
	17164	9240	10700	549	1010	12150	12140	6592	6592	27982	29432	300
11	0	8802	10236	526	1010	12150	12140	6276	6276	27228	28652	10
	3	8802	10236	526	1190	14316	14304	6276	6276	29393	30815	10
	13	8802	10236	526	1135	13654	13643	6276	6276	28732	30154	10
	164	11645	11598	408	1135	13654	13643	4657	4657	29956	29898	10
	672	4808	5791	344	1135	13654	13643	3775	3775	22237	23209	10
	2374	11140	10841	361	912	10971	10962	4005	4005	26116	25808	10
	2955	4722	5577	325	916	11019	11010	3509	3509	19250	20096	10
	7054	9518	10162	441	959	11537	11527	5100	5100	26155	26789	10
	7930	4491	5276	309	637	7663	7657	3287	3287	15441	16219	10
	16709	9960	11116	526	1010	12150	12140	6276	6276	28386	29532	10
	17699	8802	10236	526	1010	12150	12140	6276	6276	27228	28652	10
14	0	8802	10236	526	1010	12150	12140	6276	6276	27228	28652	1
	152	9499	10570	497	855	10286	10277	5880	5880	25664	26727	1
	12580	91	95	70	50	602	601	0	0	693	696	1
21-23	0	9242	0	549	1010	12150	12140	6592	6592	27984	18732	300
	17224	664	0	100	50	602	601	413	413	1678	1014	300
24				100	50	602	601	413	413	1014	1014	1
				100	1563	18803	18787	413	413	19216	19200	1
				100	50	602	601	413	413	1014	1014	1
30	0	9280	10800	549	1010	12150	12140	6592	6592	28022	29532	1
	13	85600	44694	162	250	3002	3000	1260	1260	89862	48953	1
	1011	-12	-10	70	0	0	0	0	0	-12	-10	1

NOTES: Column 1: Transient number identification.

Column 2: Time during transient where a maxima or minima stress intensity occurs from P-V.OUT output file.

Column 3: Maxima or minima total stress intensity from P-V.OUT output file.

Column 4: Maxima or minima membrane plus bending stress intensity from P-V.OUT output file.

Column 5: Temperature per total stress intensity.

Column 6: Pressure per Table 2.

Column 7: Total pressure stress intensity from the quantity (Column 6 x 12,030)/1000.

Column 8: Membrane plus bending pressure stress intensity from the quantity (Column 6 x 12,020)/1000.

Column 9: Total external stress from calculation in Table 4, -6949.94 x (Column 5-70°F)/(575°F -70°F).

Column 10: Same as Column 9, but for M+B stress.

Column 11: Sum of total stresses (Columns 3, 7, and 9).

Column 12: Sum of membrane plus bending stresses (Columns 4, 8, and 10).

Column 13: Number of cycles for the transient (60 years).



Table 7: Fatigue Results for Blend Radius (60 Years)

LOCATION = LOCATION NO. 2 -- BLEND RADIUS
 FATIGUE CURVE = 1 (1 = CARBON/LOW ALLOY, 2 = STAINLESS STEEL)
 m = 2.0
 n = .2
 Sm = 26700. psi
 Ecurve = 3.000E+07 psi
 Eanalysis = 2.670E+07 psi
 Kt = 1.00

MAX	MIN	RANGE	MEM+BEND	Ke	Salt	Napplied	Nallowed	U
56068.	19.	56049.	54658.	1.000	31488.	1.000E+00	1.896E+04	.0001
51325.	19.	51306.	45212.	1.000	28824.	1.000E+01	2.501E+04	.0004
46174.	19.	46155.	45531.	1.000	25930.	1.000E+01	3.460E+04	.0003
46013.	19.	45994.	43180.	1.000	25839.	1.000E+01	3.498E+04	.0003
45991.	19.	45972.	41149.	1.000	25827.	1.000E+01	3.503E+04	.0003
44605.	19.	44586.	39443.	1.000	25048.	1.000E+01	3.848E+04	.0003
39899.	19.	39880.	39707.	1.000	22404.	1.000E+01	5.695E+04	.0002
39719.	19.	39700.	39236.	1.000	22303.	1.000E+01	5.824E+04	.0002
39719.	19.	39700.	39236.	1.000	22303.	1.000E+01	5.824E+04	.0002
39719.	19.	39700.	39236.	1.000	22303.	1.000E+00	5.824E+04	.0000
39465.	19.	39446.	38467.	1.000	22161.	3.800E+01	6.012E+04	.0006
39465.	1812.	37653.	36718.	1.000	21153.	8.200E+01	7.572E+04	.0011
39292.	1812.	37480.	38223.	1.000	21056.	1.000E+01	7.747E+04	.0001
38628.	1812.	36816.	37019.	1.000	20683.	2.800E+01	8.466E+04	.0003
38628.	1812.	36816.	37019.	1.000	20683.	1.000E+00	8.466E+04	.0000
38628.	1812.	36816.	37019.	1.000	20683.	1.000E+00	8.466E+04	.0000
38628.	23719.	14909.	26168.	1.000	8376.	2.700E+02	5.366E+07	.0000
38625.	23719.	14906.	26187.	1.000	8374.	3.000E+01	5.375E+07	.0000
38625.	25492.	13133.	24470.	1.000	7378.	2.700E+02	3.042E+08	.0000
38565.	25492.	13073.	24240.	1.000	7344.	1.000E+00	3.374E+08	.0000
35265.	25492.	9773.	14585.	1.000	5490.	1.000E+01	1.000E+20	.0000
26915.	25492.	1423.	610.	1.000	799.	1.000E+00	1.000E+20	.0000
25700.	25492.	208.	564.	1.000	117.	1.000E+00	1.000E+20	.0000

TOTAL USAGE FACTOR = .0043



Table 8: Fatigue Results for Safe End (60 Years)

LOCATION = LOCATION NO. 1 -- SAFE END
FATIGUE CURVE = 2 (1 = CARBON/LOW ALLOY, 2 = STAINLESS STEEL)
m = 1.7
n = .3
Sm = 23300. psi
Ecurve = 2.830E+07 psi
Eanalysis = 2.980E+07 psi
Kt = 4.00

Table with 9 columns: MAX, MIN, RANGE, MEM+BEND, Ke, Salt, Napplied, Nallowed, U. It contains multiple rows of numerical data representing fatigue results for various locations and conditions.

TOTAL USAGE FACTOR = .0184

Table 9: Fatigue Results for Stainless Steel Piping (60 Years)

LOCATION = LOCATION NO. 1 -- SS Piping
 FATIGUE CURVE = 2 (1 = CARBON/LOW ALLOY, 2 = STAINLESS STEEL)
 m = 1.7
 n = .3
 Sm = 17000. psi
 Ecurve = 2.830E+07 psi
 Eanalysis = 2.700E+07 psi
 Kt = 1.80

MAX	MIN	RANGE	MEM+BEND	Ke	Salt	Napplied	Nallowed	U
89862.	-12.	89874.	48963.	1.000	67629.	1.000E+00	8.006E+03	.0001
29956.	413.	29543.	29485.	1.000	27845.	1.000E+01	1.042E+06	.0000
29393.	413.	28980.	30402.	1.000	27934.	1.000E+01	1.031E+06	.0000
28732.	413.	28319.	29741.	1.000	27310.	1.000E+01	1.110E+06	.0000
28386.	413.	27973.	29119.	1.000	26868.	1.000E+01	1.171E+06	.0000
28022.	413.	27609.	29119.	1.000	26678.	1.000E+00	1.198E+06	.0000
27984.	413.	27571.	18319.	1.000	22130.	7.900E+01	2.272E+06	.0000
27984.	693.	27291.	18036.	1.000	21864.	1.000E+00	2.392E+06	.0000
27984.	1014.	26970.	17718.	1.000	21563.	1.200E+02	2.539E+06	.0000
27984.	1014.	26970.	17718.	1.000	21563.	1.000E+00	2.539E+06	.0000
27984.	1014.	26970.	17718.	1.000	21563.	1.000E+00	2.539E+06	.0000
27984.	1074.	26910.	17560.	1.000	21465.	9.800E+01	2.588E+06	.0000
27982.	1074.	26908.	28260.	1.000	25950.	2.020E+02	1.311E+06	.0002
27982.	1678.	26304.	28418.	1.000	25700.	9.800E+01	1.354E+06	.0001
27228.	1678.	25550.	27638.	1.000	24978.	1.000E+01	1.485E+06	.0000
27228.	1678.	25550.	27638.	1.000	24978.	1.000E+01	1.485E+06	.0000
27228.	1678.	25550.	27638.	1.000	24978.	1.000E+00	1.485E+06	.0000
26155.	1678.	24477.	25775.	1.000	23634.	1.000E+01	1.779E+06	.0000
26116.	1678.	24438.	24794.	1.000	23202.	1.000E+01	1.889E+06	.0000
25664.	1678.	23986.	25713.	1.000	23351.	1.000E+00	1.850E+06	.0000
22237.	1678.	20559.	22195.	1.000	20080.	1.000E+01	3.442E+06	.0000
19250.	1678.	17572.	19082.	1.000	17209.	1.000E+01	7.481E+06	.0000
19216.	1678.	17538.	18186.	1.000	16816.	1.000E+00	8.600E+06	.0000
15441.	1678.	13763.	15205.	1.000	13588.	1.000E+01	1.000E+20	.0000
13646.	1678.	11968.	12621.	1.000	11564.	1.200E+02	1.000E+20	.0000
TOTAL USAGE FACTOR =								.0005

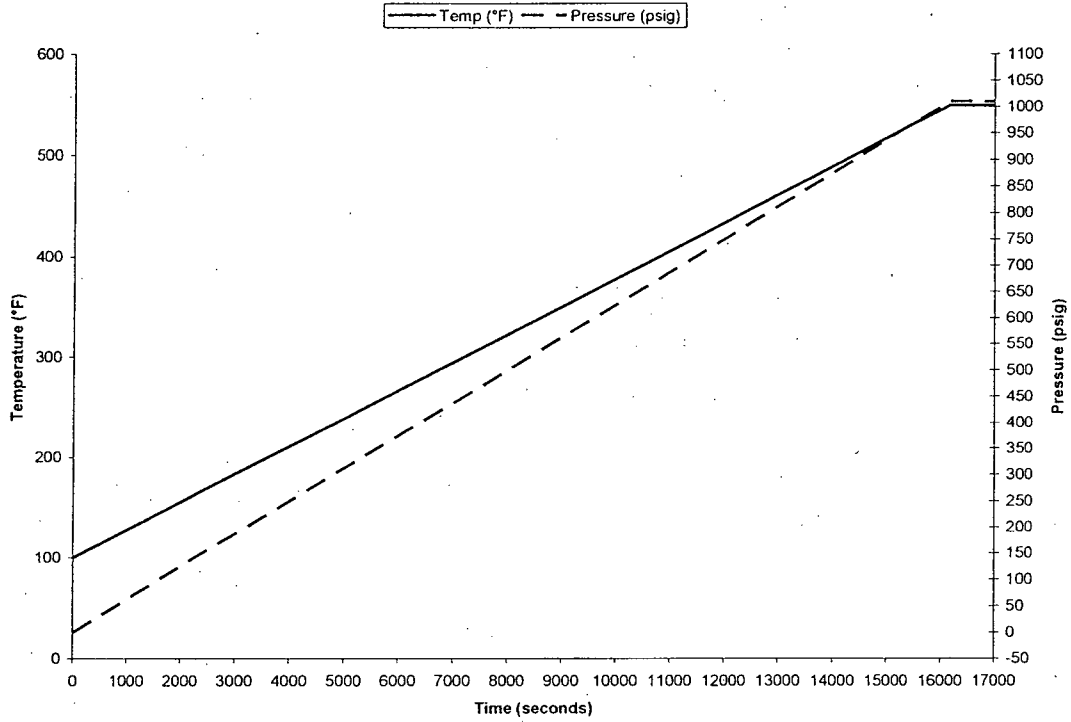


Figure 1: Transient 03: Start Up

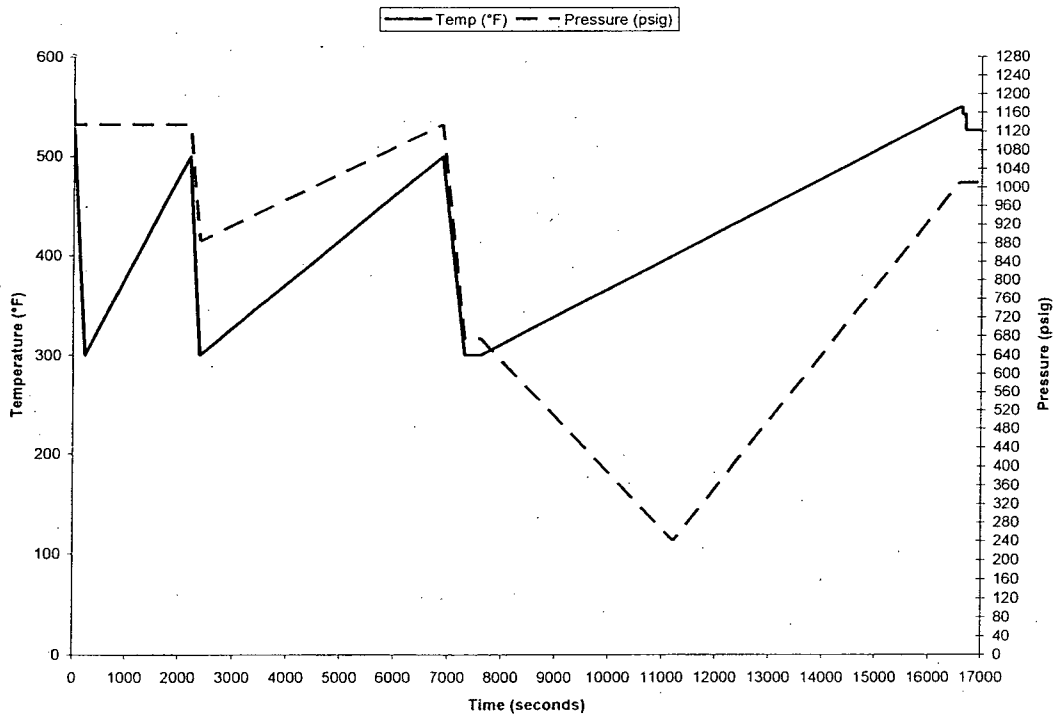


Figure 2: Transient 11: Loss of Feedwater Pumps, Isolation Valves Close

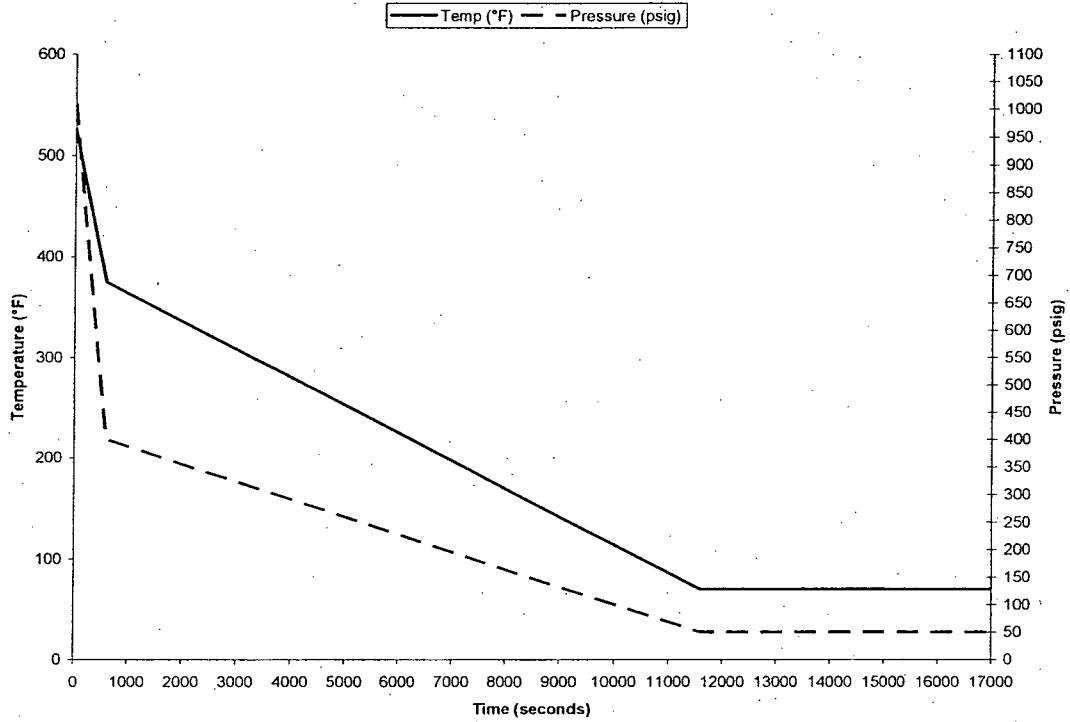


Figure 3: Transient 14: Single Relief of Safety Valve Blow Down

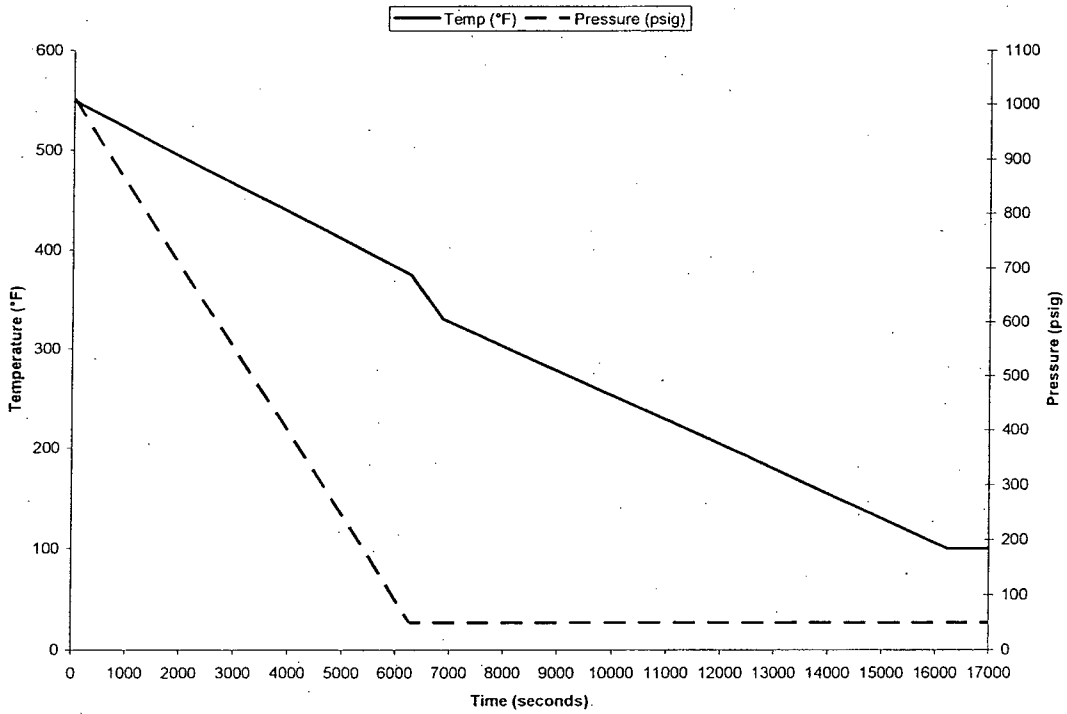


Figure 4: Transient 21-23: Shut Down Vessel Flooding

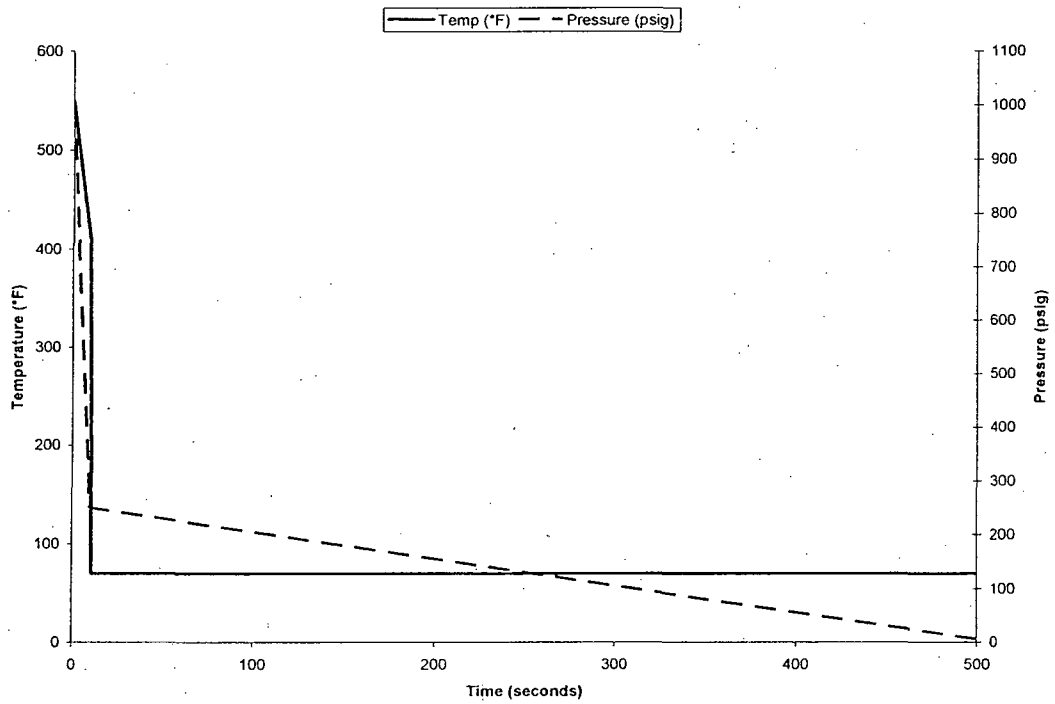


Figure 5: Transient 30: Emergency Shut Down 100% Flow (Safe End)

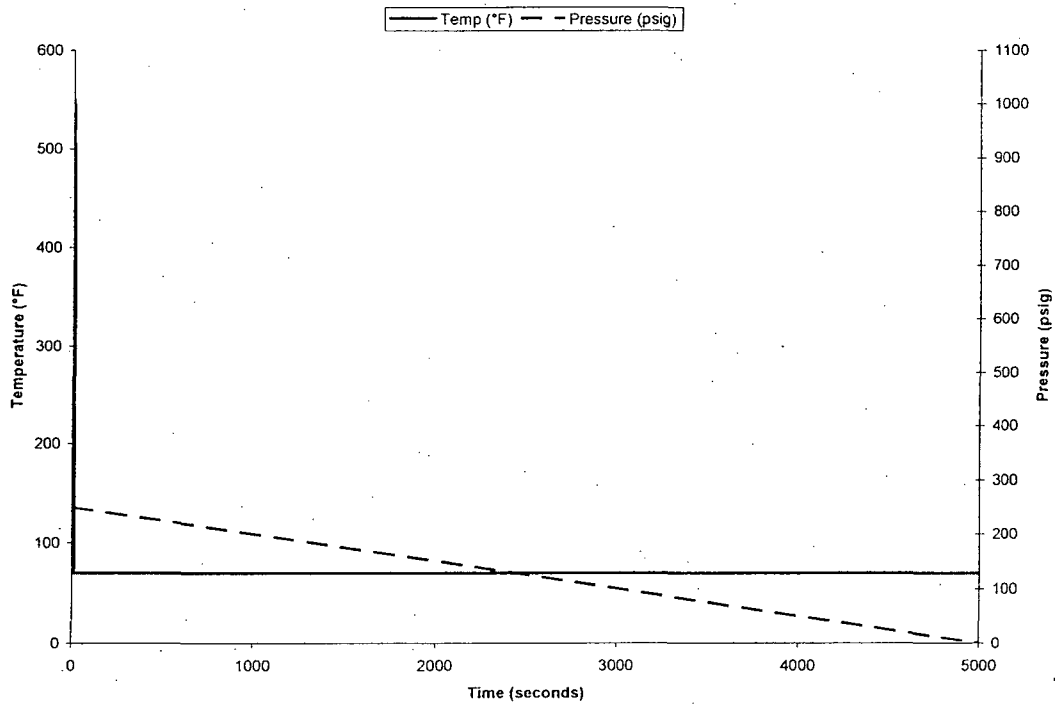


Figure 6: Transient 30: Emergency Shut Down 100% Flow (Blend Radius)

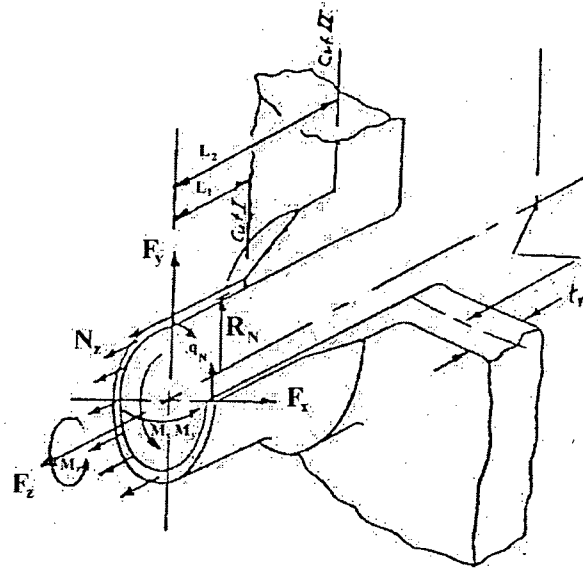


Figure 7: External Forces and Moments on the Core Spray Nozzle

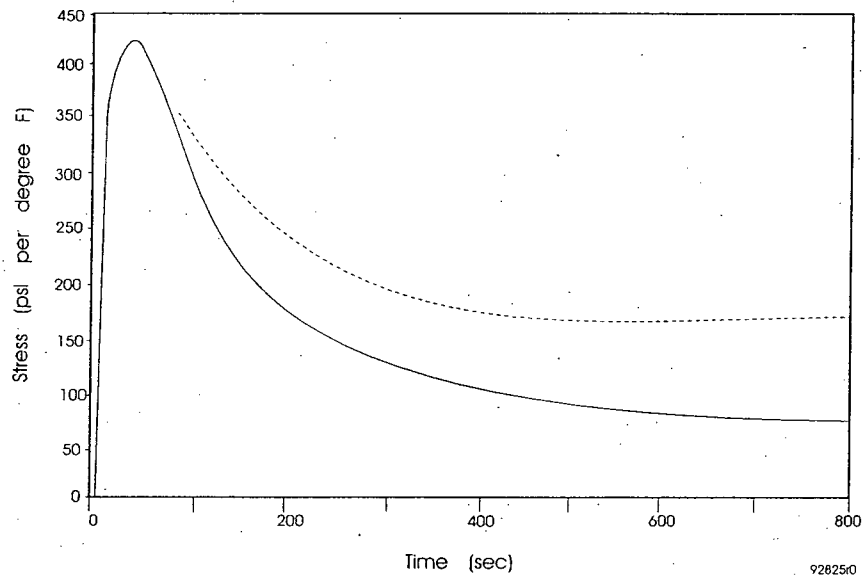


Figure 8: Typical Green's Functions for Thermal Transient Stress

Note: A typical set of two Green's Functions is shown, each for a different set of heat transfer coefficients (representing different flow rate conditions).

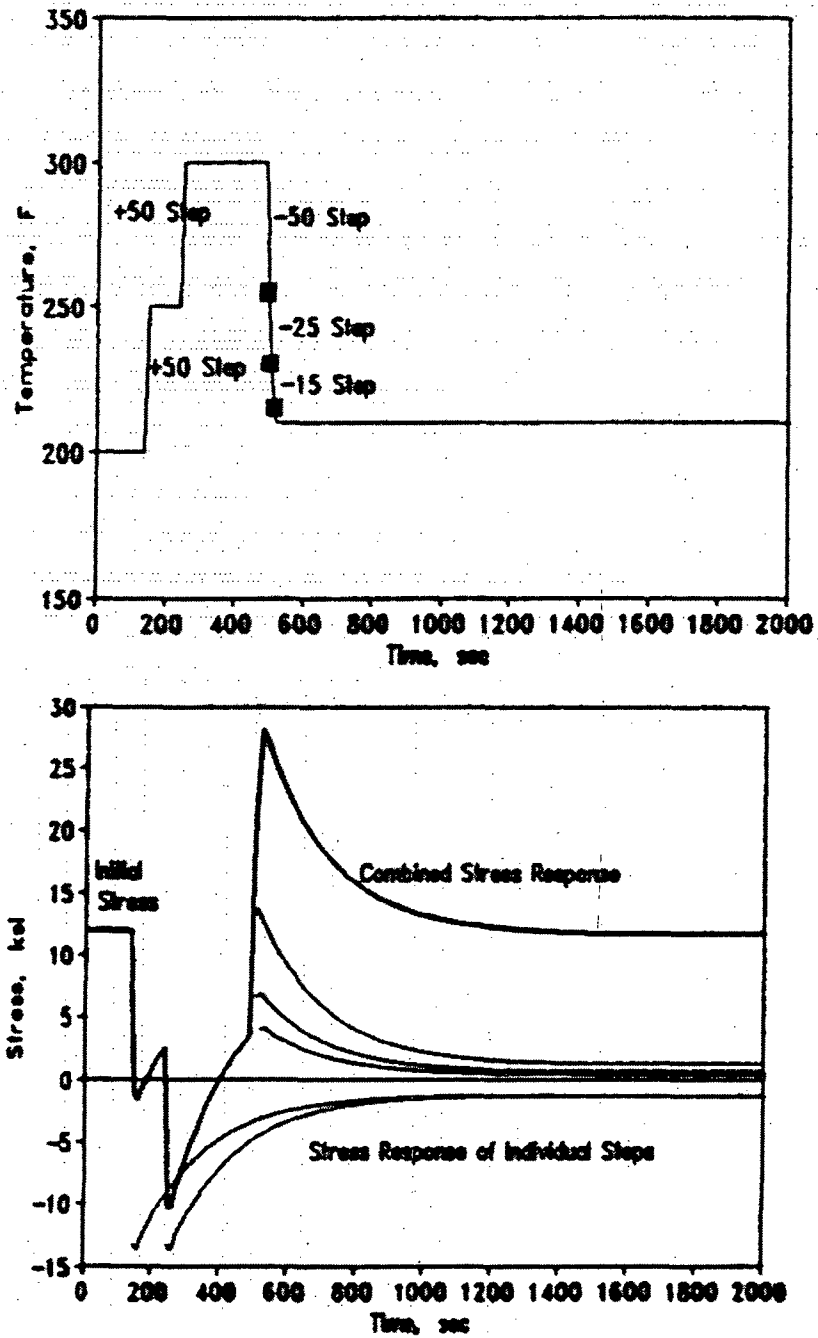


Figure 9: Typical Stress Response Using Green's Functions



APPENDIX A
INPUT AND OUTPUT FILES



Input Files

File Name	Description
TRANSNT_03.INP	Text file describing transient 03 for STRESS.EXE
TRANSNT_11.INP	Text file describing transient 11 for STRESS.EXE
TRANSNT_14.INP	Text file describing transient 14 for STRESS.EXE
TRANSNT_21_22_23.INP	Text file describing transients 21-23 for STRESS.EXE
TRANSNT_30.INP	Text file describing transient 30 for STRESS.EXE

Output Files

File Name	Description
P-V_03.OUT	Output text file of STRESS.EXE and P-V.EXE, Stress peaks and valleys of transient 03
P-V_11.OUT	Output text file of STRESS.EXE and P-V.EXE, Stress peaks and valleys of transient 11
P-V_14.OUT	Output text file of STRESS.EXE and P-V.EXE, Stress peaks and valleys of transient 14
P-V_21_22_23.OUT	Output text file of STRESS.EXE and P-V.EXE, Stress peaks and valleys of transients 21-23
P-V_30.OUT	Output text file of STRESS.EXE and P-V.EXE, Stress peaks and valleys of transient 30



Structural Integrity Associates, Inc.

File No.: VY-16Q-311

NEC-JH_14

CALCULATION PACKAGE

Project No.: VY-16Q

PROJECT NAME:

Environmental Fatigue Analysis of VYNPS

CONTRACT NO.:

10150394

CLIENT:

Entergy Nuclear Operations, Inc.

PLANT:

Vermont Yankee Nuclear Power Station

CALCULATION TITLE:

Feedwater Class 1 Piping Fatigue Analysis

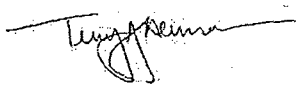
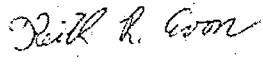
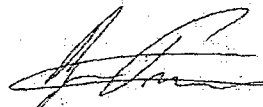
Document Revision	Affected Pages	Revision Description	Project Manager Approval Signature & Date	Preparer(s) & Checker(s) Signatures & Date
0	1-17, A1 - A38, B1 - B45 In Computer Files	Initial Issue	Terry J. Herrmann 7/20/2007 	Keith R. Evon 7/16/2007  Ryan V. Perry 7/16/2007 

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1.0 OBJECTIVE

The purpose of this calculation is to perform an ASME Section III, NB-3600 fatigue calculation (including environmental fatigue) of the Vermont Yankee (VY) Class 1 feedwater piping located inside the drywell (originally analyzed to B31.1 requirements). This section of piping was originally identified in the Recommendation Report [6] for installing a fatigue monitoring system at VY.

The fatigue calculation performed herein is not a certified ASME Code NB-3600 stress and fatigue analysis. Rather, it is an evaluation for the purposes of establishing fatigue usage to accommodate fatigue monitoring of the subject B31.1 piping. Although the PIPESTRESS program implements all ASME Code NB-3600 equations, only the fatigue usage results are utilized. All stress limit checks, although calculated by the program, are ignored since satisfactory stress limit checks were performed as a part of the already existing governing B31.1 stress analyses for all piping systems.

2.0 METHODOLOGY

The Class 1 Loop A feedwater piping system line extending from anchor HD-36 to reactor pressure vessel (RPV) nozzles N-4A and N-4B was evaluated. This includes a portion of the HPCI line to support HPCI-HD35A [7], so that the appropriate stiffness affects of this line on the feedwater piping are included. This evaluation is also considered valid for the Loop B line extending from anchor HD-39 to RPV nozzles N-4C and N-4D for the following reasons:

1. The Class 1 sections of Loop A and Loop B are mirror images of each other. This evaluation includes piping beyond the Class 1 boundary check valve so that its influence on the Class 1 piping is taken into account. The final fatigue analysis will only consider points on the Class 1 portion of the piping.
2. A 14" HPCI line tees into Loop A and a 4" RCIC line tees into Loop B. The HPCI line is more than three times the size of the RCIC line and will therefore have a greater influence on the feedwater piping.
3. The transients defined in this calculation are the bounding set for the two loops.

The operating conditions for the Class 1 portion of the feedwater line were defined based on References [11 and 12]. The resulting piping transient definitions are specified in Table 1. For each thermal cycle, the operating temperatures for Regions I through V define the conditions to be applied to the model.

Region boundaries are defined at branches, transitions, or locations where temperature and flow conditions change. These boundary locations are also shown in Figure 1. A listing of the PIPESTRESS input file "FWHPCI.FRE" is given in Appendix A and is also included in the project computer files.

Table 1: Thermal Cycle Definitions for Feedwater Line

Transient Cycle	Description (1)	Piping Region (3)	Thermal Conditions (2)									Pressure Conditions		No. of Cycles (1)
			Oper. Temp. (°F)	T _{init} (°F)	T _{final} (°F)	Time (sec.)	Rate (°F/hr)	T _{avg} (°F)	Flow (%)	Ratio	(gpm)(4)	P _{init} (psig)	P _{final} (psig)	
1	Design Hydrotest (Leak Test) (+)	I	100	70	100	1800	60	85	0	1	200.0	0.0	1100	120
		IIa	100	70	100	1800	60	85	0	1	150.0	0.0	1100	
		IIb	100	70	100	1800	60	85	0	1	150.0	0.0	1100	
		III	100	70	100	1800	60	85	0	1	150.0	0.0	50	
		III	100	70	100	1800	60	85	0	1	200.0	0.0	1100	
		IV	100	70	100	1800	60	85	0	1/2	100.0	0.0	1100	
		IVa	100	70	100	1800	60	85	0	1/2	100.0	0.0	1100	
		IVb	100	70	100	1800	60	85	0	1/2	100.0	0.0	1100	
V	100	70	100	1800	60	85	0	1/2	100.0	0.0	1100			
2	Design Hydrotest (Leak Test) (-)	I	100	100	100	0	0	100	0	1	200.0	1100.0	50	120
		IIa	100	100	100	0	0	100	0	1	150.0	1100.0	50	
		IIb	100	100	100	0	0	100	0	1	150.0	1100.0	50	
		III	100	100	100	0	0	100	0	1	150.0	50.0	50	
		III	100	100	100	0	0	100	0	1	200.0	1100.0	50	
		IV	100	100	100	0	0	100	0	1/2	100.0	1100.0	50	
		IVa	100	100	100	0	0	100	0	1/2	100.0	1100.0	50	
		IVb	100	100	100	0	0	100	0	1/2	100.0	1100.0	50	
V	100	100	100	0	0	100	0	1/2	100.0	1100.0	50			
3	Startup (+)	I	150	100	150	16164	11.1	125	0	1	200.0	50.0	1010	300
		IIa	150	100	150	16164	11.1	125	0	1	150.0	50.0	1010	
		IIb	125	100	125	16164	5.6	113	0	1	150.0	50.0	1010	
		II	100	100	100	16164	0.0	100	0	1	150.0	50.0	50	
		III	150	100	150	16164	11.1	125	0	1	200.0	50.0	1010	
		IV	150	100	150	16164	11.1	125	0	1/2	100.0	50.0	1010	
		IVa	283	100	283	16164	40.8	192	0	1/2	100.0	50.0	1010	
		IVb	416	100	416	16164	70.4	258	0	1/2	100.0	50.0	1010	
V	549	100	549	16164	100	325	0	1/2	100.0	50.0	1010			
4	Turbine Roll & Increase to Rated Power 1 (-) (Includes 10 SCRAM, Loss of Feedwater Pumps and 300 Hot Standby - Feedwater Cycling)	I	100	150	100	0	STEP	125	15	1	1377.0	1010.0	1010	610
		IIa	100	150	100	0	STEP	125	0	1	150.0	1010.0	1010	
		IIb	100	125	100	0	STEP	113	0	1	150.0	1010.0	1010	
		II	100	100	100	0	STEP	100	0	1	150.0	50.0	50	
		III	100	150	100	0	STEP	125	15	1	1377.0	1010.0	1010	
		IV	100	150	100	0	STEP	125	15	1/2	688.5	1010.0	1010	
		IVa	100	283	100	0	STEP	192	15	1/2	688.5	1010.0	1010	
		IVb	100	416	100	0	STEP	258	15	1/2	688.5	1010.0	1010	
V	100	549	100	0	STEP	325	15	1/2	688.5	1010.0	1010			
5	Turbine Roll & Increase to Rated Power 2 (+) (Includes 10 SCRAM, Loss of Feedwater Pumps, 1 Reactor Overpressure, 228 Other SCRAMS and 60 Turbine Generator Trip)	I	260	100	260	0	STEP	180	15	1	1377.0	1010.0	1010	599
		IIa	260	100	260	0	STEP	180	0	1	150.0	1010.0	1010	
		IIb	180	100	180	0	STEP	140	0	1	150.0	1010.0	1010	
		II	100	100	100	0	STEP	100	0	1	150.0	50.0	50	
		III	260	100	260	0	STEP	180	15	1	1377.0	1010.0	1010	
		IV	260	100	260	0	STEP	180	15	1/2	688.5	1010.0	1010	
		IVa	260	100	260	0	STEP	180	15	1/2	688.5	1010.0	1010	
		IVb	260	100	260	0	STEP	180	15	1/2	688.5	1010.0	1010	
V	260	100	260	0	STEP	180	15	1/2	688.5	1010.0	1010			
6	Turbine Roll & Increase to Rated Power 3 (+) (Includes 10 SCRAM, Loss of Feedwater Pumps, 1 Reactor Overpressure, 228 Other SCRAMS and 60 Turbine Generator Trip)	I	392	260	392	1800	264	326	100	1	9180.0	1010.0	1010	599
		IIa	392	260	392	1800	264	326	0	1	150.0	1010.0	1010	
		IIb	246	180	246	1800	132	213	0	1	150.0	1010.0	1010	
		II	100	100	100	1800	0	100	0	1	150.0	50.0	50	
		III	392	260	392	1800	264	326	100	1	9180.0	1010.0	1010	
		IV	392	260	392	1800	264	326	100	1/2	4590.0	1010.0	1010	
		IVa	392	260	392	1800	264	326	100	1/2	4590.0	1010.0	1010	
		IVb	392	260	392	1800	264	326	100	1/2	4590.0	1010.0	1010	
V	392	260	392	1800	264	326	100	1/2	4590.0	1010.0	1010			
7	Daily Reduction to 75% Power (-)	I	310	392	310	900	-328	351	75	1	6885.0	1010.0	1010	10000
		IIa	310	392	310	900	-328	351	0	1	150.0	1010.0	1010	
		IIb	205	246	205	900	-164	226	0	1	150.0	1010.0	1010	
		II	100	100	100	900	0	100	0	1	150.0	50.0	50	
		III	310	392	310	900	-328	351	75	1	6885.0	1010.0	1010	
		IV	310	392	310	900	-328	351	75	1/2	3442.5	1010.0	1010	
		IVa	310	392	310	900	-328	351	75	1/2	3442.5	1010.0	1010	
		IVb	310	392	310	900	-328	351	75	1/2	3442.5	1010.0	1010	
V	310	392	310	900	-328	351	75	1/2	3442.5	1010.0	1010			
8	Daily Reduction to 75% Power (+)	I	392	310	392	900	328	351	75	1	6885.0	1010.0	1010	10000
		IIa	392	310	392	900	328	351	0	1	150.0	1010.0	1010	
		IIb	246	205	246	900	164	226	0	1	150.0	1010.0	1010	
		II	100	100	100	900	0	100	0	1	150.0	50.0	50	
		III	392	310	392	900	328	351	75	1	6885.0	1010.0	1010	
		IV	392	310	392	900	328	351	75	1/2	3442.5	1010.0	1010	
		IVa	392	310	392	900	328	351	75	1/2	3442.5	1010.0	1010	
		IVb	392	310	392	900	328	351	75	1/2	3442.5	1010.0	1010	
V	392	310	392	900	328	351	75	1/2	3442.5	1010.0	1010			
9	Weekly Reduction to 50% Power (-)	I	280	392	280	1800	-224	336	50	1	4590.0	1010.0	1010	2000
		IIa	280	392	280	1800	-224	336	0	1	150.0	1010.0	1010	
		IIb	190	246	190	1800	-112	218	0	1	150.0	1010.0	1010	
		II	100	100	100	1800	0	100	0	1	150.0	50.0	50	
		III	280	392	280	1800	-224	336	50	1	4590.0	1010.0	1010	
		IV	280	392	280	1800	-224	336	50	1/2	2295.0	1010.0	1010	
		IVa	280	392	280	1800	-224	336	50	1/2	2295.0	1010.0	1010	
		IVb	280	392	280	1800	-224	336	50	1/2	2295.0	1010.0	1010	
V	280	392	280	1800	-224	336	50	1/2	2295.0	1010.0	1010			

For notes, see last page of table.

Table 1: Thermal Cycle Definitions for Feedwater Line (continued)

Transient Cycle	Description (1)	Piping Region (3)	Thermal Conditions (2)								Pressure Conditions		No. of Cycles (1)	
			Oper. Temp. (°F)	T _{init} (°F)	T _{final} (°F)	Time (sec.)	Rate (°F/hr)	T _{avg} (°F)	Flow (%)	Ratio	(gpm)(4)	Pinit (psig)		Pfinal (psig)
10	Weekly Reduction to 50% Power (+)	I	392	280	392	1800	224	336	50	1	4590.0	1010.0	1010	2000
		IIa	392	280	392	1800	224	336	0	1	150.0	1010.0	1010	
		IIb	246	190	246	1800	112	218	0	1	150.0	1010.0	1010	
		II	100	100	100	1800	0	100	0	1	150.0	50.0	50	
		III	392	280	392	1800	224	336	50	1	4590.0	1010.0	1010	
		IV	392	280	392	1800	224	336	50	1/2	2295.0	1010.0	1010	
		IVa	392	280	392	1800	224	336	50	1/2	2295.0	1010.0	1010	
11	Loss of Feedwater Heater, Turbine Trip 1 (-) (Includes 10 Loss of Feedwater Heater Turbine Trip, and 300 Reduction to 0% Power)	I	265	392	265	1800	-254	329	50	1	4590.0	1010.0	1010	310
		IIa	265	392	265	1800	-254	329	0	1	150.0	1010.0	1010	
		IIb	182.5	246	182.5	1800	-127	214	0	1	150.0	1010.0	1010	
		II	100	100	100	1800	0	100	0	1	150.0	50.0	50	
		III	265	392	265	1800	-254	329	50	1	4590.0	1010.0	1010	
		IV	265	392	265	1800	-254	329	50	1/2	2295.0	1010.0	1010	
		IVa	265	392	265	1800	-254	329	50	1/2	2295.0	1010.0	1010	
12	Loss of Feedwater Heater, Turbine Trip 2 (-)	I	90	265	90	360	-1750	178	15	1	1377.0	1010.0	1010	10
		IIa	90	265	90	360	-1750	178	0	1	150.0	1010.0	1010	
		IIb	95	182.5	95	360	-875	139	0	1	150.0	1010.0	1010	
		II	100	100	100	360	0	100	0	1	150.0	50.0	50	
		III	90	265	90	360	-1750	178	15	1	1377.0	1010.0	1010	
		IV	90	265	90	360	-1750	178	15	1/2	688.5	1010.0	1010	
		IVa	90	265	90	360	-1750	178	15	1/2	688.5	1010.0	1010	
13	Loss of Feedwater Heater, Turbine Trip 3 (+)	I	265	90	265	900	700	178	15	1	1377.0	1010.0	1010	10
		IIa	265	90	265	900	700	178	0	1	150.0	1010.0	1010	
		IIb	182.5	95	182.5	900	350	139	0	1	150.0	1010.0	1010	
		II	100	100	100	900	0	100	0	1	150.0	50.0	50	
		III	265	90	265	900	700	178	15	1	1377.0	1010.0	1010	
		IV	265	90	265	900	700	178	15	1/2	688.5	1010.0	1010	
		IVa	265	90	265	900	700	178	15	1/2	688.5	1010.0	1010	
14	Loss of Feedwater Heater, Turbine Trip 4 (+)	I	392	265	392	1800	254	329	50	1	4590.0	1010.0	1010	10
		IIa	392	265	392	1800	254	329	0	1	150.0	1010.0	1010	
		IIb	246	182.5	246	1800	127	214	0	1	150.0	1010.0	1010	
		II	100	100	100	1800	0	100	0	1	150.0	50.0	50	
		III	392	265	392	1800	254	329	50	1	4590.0	1010.0	1010	
		IV	392	265	392	1800	254	329	50	1/2	2295.0	1010.0	1010	
		IVa	392	265	392	1800	254	329	50	1/2	2295.0	1010.0	1010	
15	Loss of Feedwater Heater, FW Heater Bypass (-)	I	265	392	265	90	-5080	329	100	1	9180.0	1010.0	1010	70
		IIa	265	392	265	90	-5080	329	0	1	150.0	1010.0	1010	
		IIb	182.5	246	182.5	90	-2540	214	0	1	150.0	1010.0	1010	
		II	100	100	100	90	0	100	0	1	150.0	50.0	50	
		III	265	392	265	90	-5080	329	100	1	9180.0	1010.0	1010	
		IV	265	392	265	90	-5080	329	100	1/2	4590.0	1010.0	1010	
		IVa	265	392	265	90	-5080	329	100	1/2	4590.0	1010.0	1010	
16	Loss of Feedwater Heater, FW Heater Bypass (+)	I	392	265	392	180	2540	329	100	1	9180.0	1010.0	1010	70
		IIa	392	265	392	180	2540	329	0	1	150.0	1010.0	1010	
		IIb	246	182.5	246	180	1270	214	0	1	150.0	1010.0	1010	
		II	100	100	100	180	0	100	0	1	150.0	50.0	50	
		III	392	265	392	180	2540	329	100	1	9180.0	1010.0	1010	
		IV	392	265	392	180	2540	329	100	1/2	4590.0	1010.0	1010	
		IVa	392	265	392	180	2540	329	100	1/2	4590.0	1010.0	1010	
17	SCRAM, T.G. Trip, Reactor Overpressure, and All Other Scrams 1 (-) (Includes 1 Reactor Overpressure, 228 Other SCRAMS and 60 Turbine Generator Trip)	I	275	392	275	60	-7020	334	110	1	10098.0	1010.0	1010	289
		IIa	275	392	275	60	-7020	334	0	1	150.0	1010.0	1010	
		IIb	187.5	246	187.5	60	-3510	217	0	1	150.0	1010.0	1010	
		II	100	100	100	60	0	100	0	1	150.0	50.0	50	
		III	275	392	275	60	-7020	334	110	1	10098.0	1010.0	1010	
		IV	275	392	275	60	-7020	334	110	1/2	5049.0	1010.0	1010	
		IVa	275	392	275	60	-7020	334	110	1/2	5049.0	1010.0	1010	
18	SCRAM, T.G. Trip, Reactor Overpressure, and All Other Scrams 2 (-) (Includes 1 Reactor Overpressure, 228 Other SCRAMS and 60 Turbine Generator Trip)	I	100	275	100	900	-700	188	3	1	275.4	1010.0	1010	289
		IIa	100	275	100	900	-700	188	0	1	150.0	1010.0	1010	
		IIb	100	187.5	100	900	-350	144	0	1	150.0	1010.0	1010	
		II	100	100	100	900	0	100	0	1	150.0	50.0	50	
		III	100	275	100	900	-700	188	3	1	275.4	1010.0	1010	
		IV	100	275	100	900	-700	188	3	1/2	137.7	1010.0	1010	
		IVa	100	275	100	900	-700	188	3	1/2	137.7	1010.0	1010	

For notes, see last page of table.

Table 1: Thermal Cycle Definitions for Feedwater Line (continued)

Transient Cycle	Description (1)	Piping Region (3)	Thermal Conditions (2)							Pressure Conditions			No. of Cycles (1)	
			Oper. Temp. (°F)	T _{init} (°F)	T _{final} (°F)	Time (sec.)	Rate (°F/hr)	T _{avg} (°F)	Flow (%)	Ratio	(gpm)(4)	P _{init} (psig)		P _{final} (psig)
19	Hot Standby 1 (+)	I	265	265	265	0	STEP	265	0	1	200.0	1010.0	1010	300
		IIa	265	265	265	0	STEP	265	0	1	150.0	1010.0	1010	
		IIb	182.5	182.5	182.5	0	STEP	183	0	1	150.0	1010.0	1010	
		II	100	100	100	0	STEP	100	0	1	150.0	50.0	50	
		III	265	265	265	0	STEP	265	0	1	200.0	1010.0	1010	
		IV	265	265	265	0	STEP	265	0	1/2	100.0	1010.0	1010	
		IVa	323	265	323	0	STEP	294	0	1/2	100.0	1010.0	1010	
		IVb	382	265	382	0	STEP	324	0	1/2	100.0	1010.0	1010	
		V	440	265	440	0	STEP	353	0	1/2	100.0	1010.0	1010	
20	Hot Standby 2 (+)	I	265	265	265	0	0	265	0	1	200.0	1010.0	1010	300
		IIa	265	265	265	0	0	265	0	1	150.0	1010.0	1010	
		IIb	182.5	182.5	182.5	0	0	183	0	1	150.0	1010.0	1010	
		II	100	100	100	0	0	100	0	1	150.0	50.0	50	
		III	265	265	265	0	0	265	0	1	200.0	1010.0	1010	
		IV	265	265	265	0	0	265	0	1/2	100.0	1010.0	1010	
		IVa	360	323	360	3924	34	342	0	1/2	100.0	1010.0	1010	
		IVb	454	382	454	3924	66	418	0	1/2	100.0	1010.0	1010	
		V	549	440	549	3924	100	495	0	1/2	100.0	1010.0	1010	
21	Hot Standby 3 (-)	I	150	265	150	4140	-100	208	0	1	200.0	1010.0	1010	300
		IIa	150	265	150	4140	-100	208	0	1	150.0	1010.0	1010	
		IIb	125	182.5	125	4140	-50	154	0	1	150.0	1010.0	1010	
		II	100	100	100	0	0	100	0	1	150.0	50.0	50	
		III	150	265	150	4140	-100	208	0	1	200.0	1010.0	1010	
		IV	150	265	150	4140	-100	208	0	1/2	100.0	1010.0	1010	
		IVa	283	360	283	4140	-67	322	0	1/2	100.0	1010.0	1010	
		IVb	416	454	416	4140	-33	435	0	1/2	100.0	1010.0	1010	
		V	549	549	549	0	0	549	0	1/2	100.0	1010.0	1010	
22	Shutdown 1 (-)	I	150	150	150	0	0	150	0	1	200.0	1010.0	170	300
		IIa	150	150	150	0	0	150	0	1	150.0	1010.0	170	
		IIb	125	125	125	0	0	125	0	1	150.0	1010.0	170	
		II	100	100	100	0	0	100	0	1	150.0	50.0	50	
		III	150	150	150	0	0	150	0	1	200.0	1010.0	170	
		IV	150	150	150	0	0	150	0	1/2	100.0	1010.0	170	
		IVa	225	283	225	6264	-33	254	0	1/2	100.0	1010.0	170	
		IVb	300	416	300	6264	-67	358	0	1/2	100.0	1010.0	170	
		V	375	549	375	6264	-100	462	0	1/2	100.0	1010.0	170	
23	Shutdown 2 (-)	I	150	150	150	0	0	150	0	1	200.0	170.0	88	300
		IIa	150	150	150	0	0	150	0	1	150.0	170.0	88	
		IIb	125	125	125	0	0	125	0	1	150.0	170.0	88	
		II	100	100	100	0	0	100	0	1	150.0	50.0	50	
		III	150	150	150	0	0	150	0	1	200.0	170.0	88	
		IV	150	150	150	0	0	150	0	1/2	100.0	170.0	88	
		IVa	210	225	210	600	-90	218	0	1/2	100.0	170.0	88	
		IVb	270	300	270	600	-180	285	0	1/2	100.0	170.0	88	
		V	330	375	330	600	-270	353	0	1/2	100.0	170.0	88	
24	Shutdown 3 (-)	I	100	150	100	8280	-22	125	0	1	200.0	88.0	50	300
		IIa	100	150	100	8280	-22	125	0	1	150.0	88.0	50	
		IIb	100	125	100	8280	-11	113	0	1	150.0	88.0	50	
		II	100	100	100	8280	0	100	0	1	150.0	50.0	50	
		III	100	150	100	8280	-22	125	0	1	200.0	88.0	50	
		IV	100	150	100	8280	-22	125	0	1/2	100.0	88.0	50	
		IVa	100	210	100	8280	-48	155	0	1/2	100.0	88.0	50	
		IVb	100	270	100	8280	-74	185	0	1/2	100.0	88.0	50	
		V	100	330	100	8280	-100	215	0	1/2	100.0	88.0	50	
25	SCRAM, Loss of Feedwater Pumps 1 (+)	I	392	392	392	12	0	392	0	1	200.0	1010.0	1190	10
		IIa	392	392	392	12	0	392	0	1	150.0	1010.0	1190	
		IIb	246	246	246	12	0	246	0	1	150.0	1010.0	1190	
		II	100	100	100	12	0	100	0	1	150.0	50.0	50	
		III	392	392	392	12	0	392	0	1	200.0	1010.0	1190	
		IV	392	392	392	12	0	392	0	1/2	100.0	1010.0	1190	
		IVa	450	392	450	12	17400	421	0	1/2	100.0	1010.0	1190	
		IVb	507	392	507	12	34500	450	0	1/2	100.0	1010.0	1190	
		V	565	392	565	12	51900	479	0	1/2	100.0	1010.0	1190	
26	SCRAM, Loss of Feedwater Pumps 2 (-) (First HPCI)	I	50	392	50	0	STEP	221	40	1	3672.0	1190.0	1135	10
		IIa	50	392	50	0	STEP	221	40	1	3672.0	1190.0	1135	
		IIb	50	246	50	0	STEP	148	40	1	3672.0	1190.0	1135	
		II	50	100	50	0	STEP	75	40	1	3672.0	50.0	1135	
		III	50	392	50	0	STEP	221	40	1	3672.0	1190.0	1135	
		IV	50	392	50	0	STEP	221	40	1/2	1836.0	1190.0	1135	
		IVa	50	450	50	0	STEP	250	40	1/2	1836.0	1190.0	1135	
		IVb	50	507	50	0	STEP	279	40	1/2	1836.0	1190.0	1135	
		V	50	565	50	0	STEP	308	40	1/2	1836.0	1190.0	1135	
27	SCRAM, Loss of Feedwater Pumps 3 (+)	I	150	50	150	1380	261	100	0	1	200.0	1135.0	1135	10
		IIa	150	50	150	1380	261	100	0	1	150.0	1135.0	1135	
		IIb	125	50	125	1380	196	88	0	1	150.0	1135.0	1135	
		II	100	50	100	1380	130	75	0	1	150.0	1135.0	50	
		III	150	50	150	1380	261	100	0	1	200.0	1135.0	1135	
		IV	150	50	150	1380	261	100	0	1/2	100.0	1135.0	1135	
		IVa	247	50	247	1380	514	149	0	1/2	100.0	1135.0	1135	
		IVb	343	50	343	1380	764	197	0	1/2	100.0	1135.0	1135	
		V	440	50	440	1380	1017	245	0	1/2	100.0	1135.0	1135	

For notes, see last page of table.

Table 1: Thermal Cycle Definitions for Feedwater Line (continued)

Transient Cycle	Description (1)	Piping Region (3)	Thermal Conditions (2)								Pressure Conditions		No. of Cycles (1)
			Oper. Temp. (°F)	T _{init} (°F)	T _{final} (°F)	Time (sec.)	Rate (°F/hr)	T _{avg} (°F)	Flow (%)	Ratio	(gpm)(4)	P _{init} (psig)	
28	SCRAM, Loss of Feedwater Pumps 4 (+)	I	150	150	150	0	STEP	150	0	1	200.0	1135.0	1135
		IIa	150	150	150	0	STEP	150	0	1	150.0	1135.0	1135
		IIb	125	125	125	0	STEP	125	0	1	150.0	1135.0	1135
		II	100	100	100	0	STEP	100	0	1	150.0	50.0	50
		III	150	150	150	0	STEP	150	0	1	200.0	1135.0	1135
		IV	150	150	150	0	STEP	150	0	1/2	100.0	1135.0	1135
		IVa	288	247	288	0	STEP	268	0	1/2	100.0	1135.0	1135
		IVb	427	343	427	0	STEP	385	0	1/2	100.0	1135.0	1135
V	565	440	565	0	STEP	503	0	1/2	100.0	1135.0	1135		
29	SCRAM, Loss of Feedwater Pumps 5 (-) (Second HPCI)	I	50	150	50	0	STEP	100	30	1	2754.0	1135.0	885
		IIa	50	150	50	0	STEP	100	30	1	2754.0	1135.0	885
		IIb	50	125	50	0	STEP	88	30	1	2754.0	1135.0	885
		II	50	100	50	0	STEP	75	30	1	2754.0	1135.0	885
		III	50	150	50	0	STEP	100	30	1	2754.0	1135.0	885
		IV	50	150	50	0	STEP	100	30	1/2	1377.0	1135.0	885
		IVa	50	288	50	0	STEP	169	30	1/2	1377.0	1135.0	885
		IVb	50	427	50	0	STEP	239	30	1/2	1377.0	1135.0	885
V	50	565	50	0	STEP	308	30	1/2	1377.0	1135.0	885		
30	SCRAM, Loss of Feedwater Pumps 6 (+)	I	150	50	150	3060	118	100	0	1	200.0	885.0	1060
		IIa	150	50	150	3060	118	100	0	1	150.0	885.0	1060
		IIb	125	50	125	3060	88	88	0	1	150.0	885.0	1060
		II	100	50	100	3060	59	75	0	1	150.0	885.0	50
		III	150	50	150	3060	118	100	0	1	200.0	885.0	1060
		IV	150	50	150	3060	118	100	0	1/2	100.0	885.0	1060
		IVa	247	50	247	3060	232	149	0	1/2	100.0	885.0	1060
		IVb	343	50	343	3060	345	197	0	1/2	100.0	885.0	1060
V	440	50	440	3060	459	245	0	1/2	100.0	885.0	1060		
31	SCRAM, Loss of Feedwater Pumps 7 (+)	I	150	150	150	0	STEP	150	0	1	200.0	1060.0	1135
		IIa	150	150	150	0	STEP	150	0	1	150.0	1060.0	1135
		IIb	125	125	125	0	STEP	125	0	1	150.0	1060.0	1135
		II	100	100	100	0	STEP	100	0	1	150.0	50.0	50
		III	150	150	150	0	STEP	150	0	1	200.0	1060.0	1135
		IV	150	150	150	0	STEP	150	0	1/2	100.0	1060.0	1135
		IVa	283	247	283	0	STEP	265	0	1/2	100.0	1060.0	1135
		IVb	416	343	416	0	STEP	380	0	1/2	100.0	1060.0	1135
V	549	440	549	0	STEP	495	0	1/2	100.0	1060.0	1135		
32	SCRAM, Loss of Feedwater Pumps 8 (-) (Third HPCI)	I	50	150	50	0	STEP	100	17	1	1560.6	1135.0	675
		IIa	50	150	50	0	STEP	100	17	1	1560.6	1135.0	675
		IIb	50	125	50	0	STEP	88	17	1	1560.6	1135.0	675
		II	50	100	50	0	STEP	75	17	1	1560.6	50.0	675
		III	50	150	50	0	STEP	100	17	1	1560.6	1135.0	675
		IV	50	150	50	0	STEP	100	17	1/2	780.3	1135.0	675
		IVa	50	283	50	0	STEP	167	17	1/2	780.3	1135.0	675
		IVb	50	416	50	0	STEP	233	17	1/2	780.3	1135.0	675
V	50	549	50	0	STEP	300	17	1/2	780.3	1135.0	675		
33	SCRAM, Loss of Feedwater Pumps 9 (+)	I	150	50	150	300	1200	100	0	1	200.0	675.0	675
		IIa	150	50	150	300	1200	100	0	1	150.0	675.0	675
		IIb	125	50	125	300	900	88	0	1	150.0	675.0	675
		II	100	50	100	300	600	75	0	1	150.0	675.0	50
		III	150	50	150	300	1200	100	0	1	200.0	675.0	675
		IV	150	50	150	300	1200	100	0	1/2	100.0	675.0	675
		IVa	200	50	200	300	1800	125	0	1/2	100.0	675.0	675
		IVb	250	50	250	300	2400	150	0	1/2	100.0	675.0	675
V	300	50	300	300	3000	175	0	1/2	100.0	675.0	675		
34	SCRAM, Loss of Feedwater Pumps 10 (+)	I	150	150	150	8964	0	150	0	1	200.0	240.0	1010
		IIa	150	150	150	8964	0	150	0	1	150.0	240.0	1010
		IIb	125	125	125	8964	0	125	0	1	150.0	240.0	1010
		II	100	100	100	8964	0	100	0	1	150.0	50.0	50
		III	150	150	150	8964	0	150	0	1	200.0	240.0	1010
		IV	150	150	150	8964	0	150	0	1/2	100.0	240.0	1010
		IVa	283	200	283	8964	33	242	0	1/2	100.0	240.0	1010
		IVb	416	250	416	8964	67	333	0	1/2	100.0	240.0	1010
V	549	300	549	8964	100	425	0	1/2	100.0	240.0	1010		
35	SCRAM, SRV Blowdown 1 (-)	I	275	392	275	60	-7020	334	110	1	10098.0	1010.0	885
		IIa	275	392	275	60	-7020	334	0	1	150.0	1010.0	885
		IIb	187.5	246	187.5	60	-3510	217	0	1	150.0	1010.0	885
		II	100	100	100	60	0	100	0	1	150.0	50.0	50
		III	275	392	275	60	-7020	334	110	1	10098.0	1010.0	885
		IV	275	392	275	60	-7020	334	110	1/2	5049.0	1010.0	885
		IVa	275	392	275	60	-7020	334	110	1/2	5049.0	1010.0	885
		IVb	275	392	275	60	-7020	334	110	1/2	5049.0	1010.0	885
V	275	392	275	60	-7020	334	110	1/2	5049.0	1010.0	885		
36	SCRAM, SRV Blowdown 2 (-)	I	100	275	100	900	-700	188	3	1	275.4	885.0	50
		IIa	100	275	100	900	-700	188	0	1	150.0	885.0	50
		IIb	100	187.5	100	900	-350	144	0	1	150.0	885.0	50
		II	100	100	100	900	0	100	0	1	150.0	50.0	50
		III	100	275	100	900	-700	188	3	1	275.4	885.0	50
		IV	100	275	100	900	-700	188	3	1/2	137.7	885.0	50
		IVa	100	275	100	900	-700	188	3	1/2	137.7	885.0	50
		IVb	100	275	100	900	-700	188	3	1/2	137.7	885.0	50
V	100	275	100	900	-700	188	3	1/2	137.7	885.0	50		

For notes, see last page of table.

Table 1: Thermal Cycle Definitions for Feedwater Line (continued)

Transient Cycle	Description (1)	Piping Region (3)	Thermal Conditions (2)								Pressure Conditions		No. of Cycles (1)	
			Oper. Temp. (°F)	T _{init} (°F)	T _{final} (°F)	Time (sec.)	Rate (°F/hr)	T _{avg} (°F)	Flow (%)	Ratio	(gpm)(4)	P _{init} (psig)		P _{final} (psig)
37	Hydrostatic Test (+)	I	100	100	100	0	0	100	0	1	200.0	50.0	1563	1
		IIa	100	100	100	0	0	100	0	1	150.0	50.0	1563	
		IIb	100	100	100	0	0	100	0	1	150.0	50.0	1563	
		II	100	100	100	0	0	100	0	1	150.0	50.0	50	
		III	100	100	100	0	0	100	0	1	200.0	50.0	1563	
		IV	100	100	100	0	0	100	0	1/2	100.0	50.0	1563	
		IVa	100	100	100	0	0.0	100	0	1/2	100.0	50.0	1563	
IVb	100	100	100	0	0.0	100	0	1/2	100.0	50.0	1563			
V	100	100	100	0	0	100	0	1/2	100.0	50.0	1563			
38	Hydrostatic Test (-)	I	100	100	100	0	0	100	0	1	200.0	1563.0	50	1
		IIa	100	100	100	0	0	100	0	1	150.0	1563.0	50	
		IIb	100	100	100	0	0	100	0	1	150.0	1563.0	50	
		II	100	100	100	0	0	100	0	1	150.0	50.0	50	
		III	100	100	100	0	0	100	0	1	200.0	1563.0	50	
		IV	100	100	100	0	0	100	0	1/2	100.0	1563.0	50	
		IVa	100	100	100	0	0.0	100	0	1/2	100.0	1563.0	50	
IVb	100	100	100	0	0.0	100	0	1/2	100.0	1563.0	50			
V	100	100	100	0	0	100	0	1/2	100.0	1563.0	50			
39	SCRAM, T.G. Trip, Reactor Overpressure, and All Other Scrams 1 (-) (Includes 1 Reactor Overpressure, 228 Other SCRAMS and 60 Turbine Generator Trip)	I	392	392	392	60	0	392	110	1	10098.0	1010.0	1375	289
		IIa	392	392	392	60	0	392	0	1	150.0	1010.0	1375	
		IIb	246	246	246	60	0	246	0	1	150.0	1010.0	1375	
		II	100	100	100	60	0	100	0	1	150.0	50.0	50	
		III	392	392	392	60	0	392	110	1	10098.0	1010.0	1375	
		IV	392	392	392	60	0	392	110	1/2	5049.0	1010.0	1375	
		IVa	392	392	392	60	0	392	110	1/2	5049.0	1010.0	1375	
IVb	392	392	392	60	0	392	110	1/2	5049.0	1010.0	1375			
V	392	392	392	60	0	392	110	1/2	5049.0	1010.0	1375			
40	SCRAM, T.G. Trip, Reactor Overpressure, and All Other Scrams 2 (-) (Includes 1 Reactor Overpressure, 228 Other SCRAMS and 60 Turbine Generator Trip)	I	392	392	392	900	0	392	3	1	275.4	1375.0	940	289
		IIa	392	392	392	900	0	392	0	1	150.0	1375.0	940	
		IIb	246	246	246	900	0	246	0	1	150.0	1375.0	940	
		II	100	100	100	900	0	100	0	1	150.0	50.0	50	
		III	392	392	392	900	0	392	3	1	275.4	1375.0	940	
		IV	392	392	392	900	0	392	3	1/2	137.7	1375.0	940	
		IVa	392	392	392	900	0	392	3	1/2	137.7	1375.0	940	
IVb	392	392	392	900	0	392	3	1/2	137.7	1375.0	940			
V	392	392	392	900	0	392	3	1/2	137.7	1375.0	940			
41	SCRAM, T.G. Trip, Reactor Overpressure, and All Other Scrams 3 (-) (Includes 1 Reactor Overpressure, 228 Other SCRAMS and 60 Turbine Generator Trip)	I	392	392	392	900	0	392	3	1	275.4	940.0	1010	289
		IIa	392	392	392	900	0	392	0	1	150.0	940.0	1010	
		IIb	246	246	246	900	0	246	0	1	150.0	940.0	1010	
		II	100	100	100	900	0	100	0	1	150.0	50.0	50	
		III	392	392	392	900	0	392	3	1	275.4	940.0	1010	
		IV	392	392	392	900	0	392	3	1/2	137.7	940.0	1010	
		IVa	392	392	392	900	0	392	3	1/2	137.7	940.0	1010	
IVb	392	392	392	900	0	392	3	1/2	137.7	940.0	1010			
V	392	392	392	900	0	392	3	1/2	137.7	940.0	1010			
42	Hot Standby, Feedwater Cycling 1 (+)	I	125	100	125	60	1500	113	0	1	200.0	1010.0	1010	300
		IIa	125	100	125	60	1500	113	0	1	150.0	1010.0	1010	
		IIb	112.5	100	112.5	60	750	106	0	1	150.0	1010.0	1010	
		II	100	100	100	60	0	100	0	1	150.0	50.0	50	
		III	125	100	125	60	1500	113	0	1	200.0	1010.0	1010	
		IV	125	100	125	60	1500	113	0	1/2	100.0	1010.0	1010	
		IVa	180	100	180	60	4800	140	0	1/2	100.0	1010.0	1010	
IVb	235	100	235	60	8100	168	0	1/2	100.0	1010.0	1010			
V	290	100	290	60	11400	195	0	1/2	100.0	1010.0	1010			
43	Hot Standby, Feedwater Cycling 2 (+)	I	150	125	150	210	429	138	0	1	200.0	1010.0	1010	300
		IIa	150	125	150	210	429	138	0	1	150.0	1010.0	1010	
		IIb	125	112.5	125	210	214	119	0	1	150.0	1010.0	1010	
		II	100	100	100	210	0	100	0	1	150.0	50.0	50	
		III	150	125	150	210	429	138	0	1	200.0	1010.0	1010	
		IV	150	125	150	210	429	138	0	1/2	100.0	1010.0	1010	
		IVa	283	180	283	210	1766	232	0	1/2	100.0	1010.0	1010	
IVb	416	235	416	210	3103	326	0	1/2	100.0	1010.0	1010			
V	549	290	549	210	4440	420	0	1/2	100.0	1010.0	1010			

For notes, see next page.



Table 1: Thermal Cycle Definitions for Feedwater Line (concluded)

Notes:

1. From Reference [13].
2. Normal operating conditions are 1,010 psig, 549°F (steam dome), 392°F (feedwater), and 4590 gpm (feedwater nozzle) [14].
3. See Figure 1.
4. For the transients where flow is stopped, the natural convection heat transfer coefficient was used. The same approximate value was used within each region. These values are:
 - 200 gpm for Regions I and III.
 - 150 gpm for Regions II, IIa, and IIb.
 - 100 gpm for Regions IV and V.

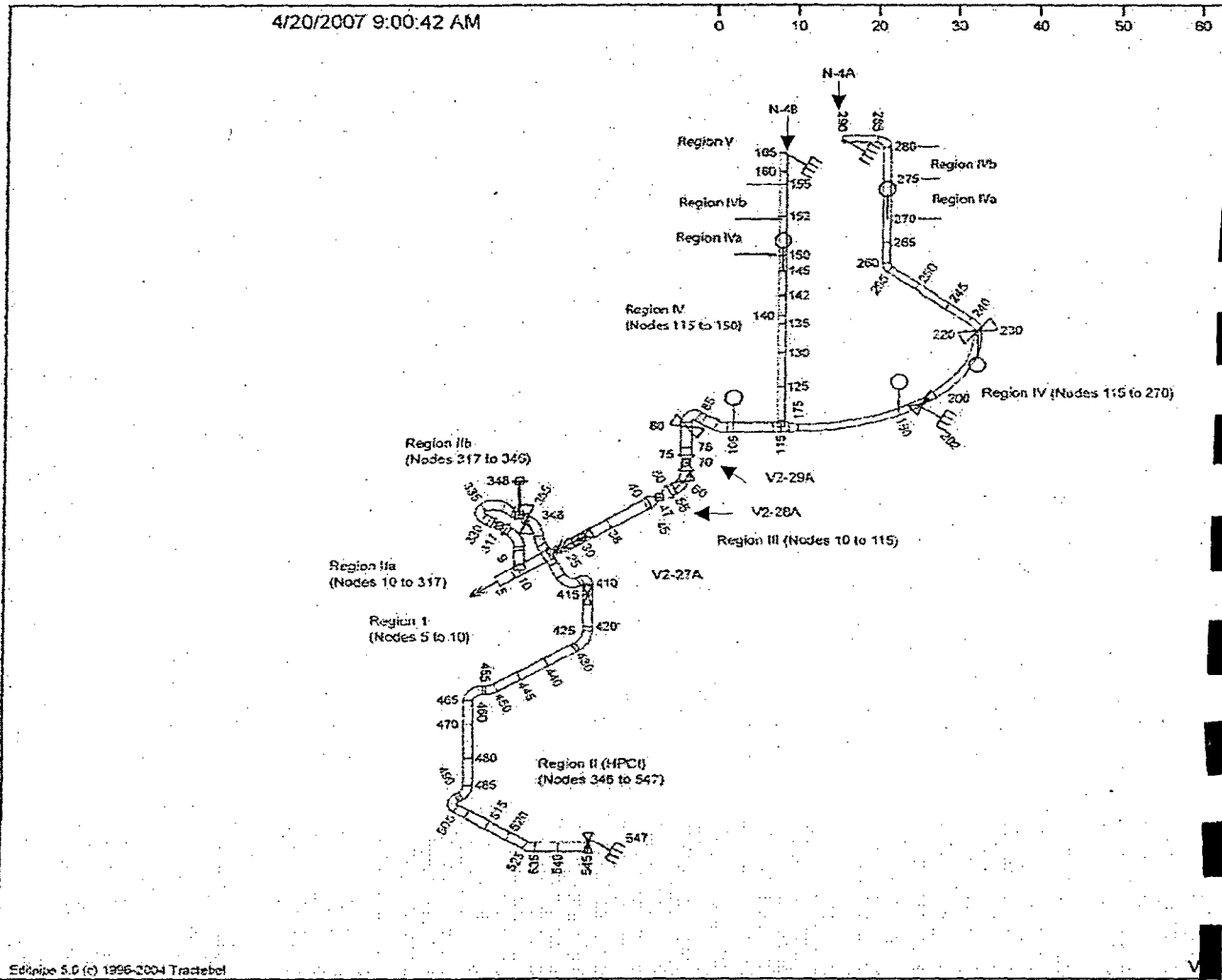


Figure 1: Feedwater/HPCI Piping from Anchor HD-36 to RPV Nozzles N-4A and

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3.0 ASSUMPTIONS/DESIGN INPUT

In order to take advantage of improvements in the ASME Code that result in a lower calculated fatigue usage, this evaluation is done to the ASME Boiler and Pressure Vessel Code, Section III, 1998 Edition with 2000 Addenda [9]. The 1998 Edition of Section III (with 2000 Addenda) has been accepted by the US NRC for use in design analyses. Although there are a few restrictions on the application of this Edition, they involve the use of optional increased allowables that are not being used in this calculation.

A piping model was created using PIPESTRESS [1]. The calculation [2] that had previously analyzed the subject Class 1 feedwater piping contains the ADLPIPE input file used to create the PIPESTRESS input file for this evaluation. Valve dimensions and properties were also obtained from the ADLPIPE input file. The piping model is composed of one carbon steel grade (maximum carbon content of 0.30 %) [2]. Temperature dependent material properties were used with values obtained from Reference [5]. Table 2 summarizes these values. The resulting PIPESTRESS model (including boundary conditions) is shown in Figure 1. The drawings for both feedwater loops [3, 4] and the HPCI line [7] were also consulted to aid in building the PIPESTRESS model.

Assumptions:

- 1) The weight of insulation is included in the analysis and PIPESTRESS calculates the heat transfer effects of insulation.
- 2) Node 545 is the end of the as-modeled HPCI piping system. This is appropriate because of the distance from the HPCI/Feedwater tee, six pipe supports in the segment and multiple pipe direction changes.

The feedwater and HPCI line sizes are specified in the previous calculation [2] and are shown in Table 3.

Table 2: Material Properties for Feedwater System Class 1 Piping [2 App. E, 5]

SA 106 B and SA-234 WPB (Carbon Silicon Steel, C-Si)							
Temperature (°F)	Young's Modulus ($\times 10^6$ psi)	Coefficient of Linear Thermal Expansion (in/100 ft)	Mean Coefficient of Thermal Expansion ⁽¹⁾ (10^{-6} /in/in/°F)	Thermal Conductivity ⁽¹⁾ (btu/hr/ft/°F)	Thermal Diffusivity ⁽¹⁾ (ft ² /hr)	Yield Stress S _y (ksi)	Design Stress Intensity S _m (ksi)
50	29.6	0 ⁽²⁾				35.0	20.0
70	29.5	0	6.4	27.5	0.529	35.0	20.0
100	29.3	0.2		27.6	0.512	35.0	20.0
150				27.6	0.496		
200	28.8	1.0		27.6	0.486	32.1	20.0
250				27.4	0.467		
300	28.3	1.9		27.2	0.453	31.0	20.0
350				27.0	0.440		
400	27.7	2.8		26.7	0.428	29.9	20.0
450				26.3	0.413		
500	27.3	3.7		25.9	0.398	28.5	18.9
550				25.5	0.387		
600	26.7	4.7		25.0	0.374	26.8	17.3

Notes:

1. These properties are used for the transient analysis only.
2. Assumed equivalent to the value at 70°F.

The material properties applied in the analyses are taken from ASME Section II Part D 1998 Edition with 2000 Addenda. This is consistent with information provided in the Design Input Record (page 13 of VY EC No. 1773, SI File No. VY-16Q-209). The use of a later code edition than that used for the original design code is acceptable since later editions typically reflect more accurate material properties than was published in prior Code editions.



Table 3: Feedwater/HPCI Piping Size Information [2]

	16" FW Downstream of V2-29A	16" FW Upstream of V2-29A	10" FW	14" HPCI
Pipe Schedule	80	120	120	120
Fittings Schedule	120	---	120	---
Piping O.D. (in.)	16.0	16.0	10.75	14.0
Piping Nom. Wall (in.)	0.843	1.218	0.843	1.093
Fitting Nom. Wall (in.)	1.218	---	0.843	---
Pipe Weight ¹ (lb/ft)	136.46	192.3	89.20	150.7
Insulation Weight (lb/ft)	14.64	11.98	8.92	10.65 ²

Note:

1. Weight of contents automatically added by the PIPESTRESS Program.
2. Insulation weight assumed to be consistent with thickness (2 inches) and composition of insulation on the 16" FW upstream of V2-29A.



4.0 ANALYSIS

Through-wall thermal gradient terms were calculated by the PIPESTRESS program for all of the transients. Table 1 defines each thermal cycle definition (i.e., transient load case) and the region of the modeled piping those conditions are applicable.

The forces and moments due to differential thermal expansion need to be included in the fatigue evaluation. The differential thermal expansion cases as analyzed by the piping program, PIPESTRESS, correspond to the end temperature and pressure of the transient. Table 4 lists the thermal expansion cases.

The material properties were obtained from the ASME Code Section II, 1998 Edition, Part D, with 2000 Addenda [5]. E and α are taken at 70°F, and k, ρ , and c_p are taken at the average temperature over the range of the individual transients.

The internal heat transfer coefficient h for the transients with flow occurring in the pipe is calculated based on the following relation for forced convection [8]:

$$h = 0.023 \text{ Re}^{0.8} \text{ Pr}^{0.4} k/D$$

Where Re = Reynolds number
Pr = Prandtl number

The heat transfer coefficients were calculated by PIPESTRESS using the above relation. The flow rates described for each transient in Table 1 were used. For the transients where flow is stopped, the natural convection heat transfer coefficient was used. The formula for h is [8]:

$$h = 0.55 (\text{Gr Pr})^{0.25} k/L$$

Where Gr = Grashof Number
L = pipe diameter

PIPESTRESS only has the forced convection heat transfer formula built in, so an equivalent flow rate was determined that would give the same heat transfer coefficient as the free convection coefficient.

As discussed in the next section, the PIPESTRESS input file "FWHPCI.FRE" will be run and analyzed to Section III, Subsection NB-3600 of ASME 1998 Edition [9] in order to evaluate acceptable fatigue usage values for the Class 1 feedwater loop A system. The code option available in PIPESTRESS is the 1998 edition without addenda. This is acceptable as the 1999 and 2000 addenda to the 1998 code did not change the fatigue analysis method which PIPESTRESS uses.

A Listing of the PIPESTRESS input is included as Appendix A.



Table 4: Thermal Cycle Load Cases

Load Set	Transients Represented	Region I Temp. (°F)	Region IIa Temp. (°F)	Region IIb Temp. (°F)	Region II Temp. (°F)	Region III Temp. (°F)	Region IV Temp. (°F)	Region IVa Temp. (°F)	Region IVb Temp. (°F)	Region V Temp. (°F)	Vessel Temp. (°F)	Region II Pressure (psig)	All other Regions Pressure (psig)
1	1	100	100	100	100	100	100	100	100	100	100	50	1100
2	2, 24, 36, 38	100	100	100	100	100	100	100	100	100	100	50	50
3	3, 21, 34, 43	150	150	125	100	150	150	283	416	549	549	50	1010
4	5	260	260	180	100	260	260	260	260	260	549	50	1010
5	6, 8, 10, 14, 16	392	392	246	100	392	392	392	392	392	549	50	1010
6	7	310	310	205	100	310	310	310	310	310	549	50	1010
7	9	280	280	190	100	280	280	280	280	280	549	50	1010
8	11, 13, 15	265	265	182.5	100	265	265	265	265	265	549	50	1010
9	12	90	90	95	100	90	90	90	90	90	549	50	1010
10	20	265	265	182.5	100	265	265	360	454	549	549	50	1010
11	22	150	150	125	100	150	150	225	300	375	375	50	170
12	23	150	150	125	100	150	150	210	270	330	330	50	88
13	25	392	392	246	100	392	392	450	507	565	565	50	1190
14	26	50	50	50	50	50	50	50	50	50	565	1135	1135
15	27	150	150	125	100	150	150	247	343	440	565	50	1135
16	28	150	150	125	100	150	150	288	427	565	565	50	1135
17	30	150	150	125	100	150	150	247	343	440	555	50	1060
18	31	150	150	125	100	150	150	283	416	549	565	50	1135
19	32	50	50	50	50	50	50	50	50	50	502	675	675
20	33	150	150	125	100	150	150	200	250	300	502	50	675
21	35	275	275	187.5	100	275	275	275	275	275	549	50	885
22	37	100	100	100	100	100	100	100	100	100	100	50	1563
23	39	392	392	246	100	392	392	392	392	392	600	50	1375
24	40	392	392	246	100	392	392	392	392	392	539	50	940
25	41	392	392	246	100	392	392	392	392	392	549	50	1010
26	17	275	275	187.5	100	275	275	275	275	275	539	50	1010
27	19	265	265	182.5	100	265	265	323	382	440	549	50	1010
28	4	100	100	100	100	100	100	100	100	100	549	50	1010
29	18	100	100	100	100	100	100	100	100	100	539	50	1010
30	42	125	125	112.5	100	125	125	180	235	290	549	50	1010
31	29	50	50	50	50	50	50	50	50	50	532	885	885

5.0 RESULTS OF ANALYSIS

Since the piping at VY was designed in accordance with USAS B31.1 methodology, fatigue analysis does not exist for the piping. Therefore, fatigue calculations are being developed for selected locations in the Class 1 piping systems at VY. This will result in detailed, Class 1 fatigue calculations for each selected location. Piping models and transient definitions have been developed for the Class 1 portion of the feedwater system, as documented in the previous sections of this calculation.

The limiting total fatigue usage for the analyzed feedwater/HPCI piping system occurs at Node 155 on the riser to the feedwater nozzle. The total usage at this location is $U = 0.1661$ (per the PIPESTRESS report FWHPCL.PRF) which passes Class 1 fatigue evaluation. The second highest total fatigue usage for the analyzed feedwater/HPCI piping system occurs at Node 175, the 16" to 10" reducer on the feedwater piping. The total usage at this location is $U = 0.1114$ (per the PIPESTRESS report FWHPCL.PRF) which passes Class 1 fatigue evaluation. The environmental fatigue multiplier to use from Reference [10] is 1.74. The total usage including environmental effects is therefore 0.289.

Appendix B contains the fatigue usage summary for node 155.



6.0 REFERENCES

1. PIPESTRESS, Version 3.5.1+0.26, DST Computer Services S.A., QA-1670-301, June, 2004.
2. HPCI/FW Piping Stress Information. ADLPIPE listing for FDW & HPCI piping from Calculation No. VYC-551, Rev. 2, Appendix A, SI File No. VY-05Q-229.
3. Vermont Yankee Nuclear Power Corp. Drawing No. VYI-FDW-Part 5, Rev. 1, "Piping Isometric Feedwater: Drywell-Main Steam Tunnel (FDW) Part 5," SI File No. VY-05Q-221.
4. Vermont Yankee Nuclear Power Corp. Drawing No. VYI-FDW-Part 5A, Rev. 1, "Piping Isometric Feedwater: Main Steam Tunnel and Drywell FDW-Part 5A," SI File No. VY-05Q-221.
5. American Society of Mechanical Engineers Boiler & Pressure Vessel Code, Section II, Materials, Part D, "Properties (Customary)," 1998 Edition including the 2000 Addenda.
6. Structural Integrity Associates Report No. SIR-01-130, Revision 0, "System Review and Recommendations for a Transient and Fatigue Monitoring System at the Vermont Yankee Nuclear Power Station," February 2002, SI File No. VY-05Q-401.
7. Vermont Yankee Nuclear Power Corp. Drawing No. VYI-HPCI-Part 5, Rev. 0, "Piping Isometric Drawing High Pressure Coolant Injection Main Steam Tunnel-Torus Area (HPCI) Part 5," SI File No. VY-05Q-223.
8. Holman, J.P., *Heat Transfer*, Fifth Edition, McGraw-Hill, 1981.
9. American Society of Mechanical Engineers Boiler & Pressure Vessel Code, Section III, Rules for Construction of Nuclear Facility Components, 1998 Edition including the 2000 Addenda.
10. Structural Integrity Associates Calculation No. VY-16Q-303, Revision 0, "Environmental Fatigue Evaluation of Reactor Recirculation Inlet Nozzle and Vessel Shell/Bottom Head."
11. "Reactor Thermal Cycles," Attachment 1, page 2, of Entergy Design Input Record (DIR) EC No. 1773, Revision 0, "Environmental Fatigue Analysis for Vermont Yankee Nuclear Power Station," 7/3/07, SI File No. VY-16Q-209.
12. "Nozzle Thermal Cycles (Feedwater)," Attachment 1, page 3, of Entergy Design Input Record (DIR) EC No. 1773, Revision 0, "Environmental Fatigue Analysis for Vermont Yankee Nuclear Power Station," 7/3/07, SI File No. VY-16Q-209.
13. "Reactor Thermal Cycles for 60 Years of Operation," Attachment 1 of Entergy Design Input Record (DIR) EC No. 1773, Revision 0, "Environmental Fatigue Analysis for Vermont Yankee Nuclear Power Station," 7/3/07, SI File No. VY-16Q-209.
14. GE Certified Design Specification No. 26A6019, Revision 1, "REACTOR VESSEL – EXTENDED POWER UPRATE," August 29, 2003, SI File No. VY-05Q-236.



APPENDIX A

PIPESTRESS INPUT FILE ("FWHPCIFRE")

(Pages A1 – A38)

```

IDEN JB=2      *Job number (1 to 9999)
CD=1          *1=ASME Section III
VA=0         *0=Calculate          2=Verify
GR=-Y        *Direction of gravity
IU=1         *Input units          0=SIU          1=USA
OU=1         *Output units         0=SIU          1=USA
CH=$         *Delimiter character
AB=T         *FREE errors =abort
PL=$Vermont Yankee$
EN=$KRES$

TITL BL=3     *Modeling option:
              * 3 =uniform mass for static analysis
              *      lumped mass for dynamic analysis
              *      rotational inertia ignored
GL=1         *Report forces/moment  0=Global      1=Local      2=G et L
SU=1         *Support summary       0=No          1=Yes
CV=15        *Code version - See Manual
HS=1         *Highest 20 stress ratios for each case
MD=1         *Hot modulus
TI=$Vermont Yankee Feedwater Piping$
              $SI Fatigue Analysis$
FREQ RF=1 RP=8 FR=33 MP=20 MX=70 TI=$SEISMIC$

```

```

*****
**** THERMAL CYCLE LOAD CASES****
*****

```

```

LCAS RF=0 CA=1 TY=0 TI=$LC-1$      *TC-1
LCAS RF=0 CA=2 TY=0 TI=$LC-2$      *TC-2,24,36,38
LCAS RF=0 CA=3 TY=0 TI=$LC-3$      *TC-3,21,34,43
LCAS RF=0 CA=4 TY=0 TI=$LC-4$      *TC-5
LCAS RF=0 CA=5 TY=0 TI=$LC-5$      *TC-6,8,10,14,16
LCAS RF=0 CA=6 TY=0 TI=$LC-6$      *TC-7
LCAS RF=0 CA=7 TY=0 TI=$LC-7$      *TC-9
LCAS RF=0 CA=8 TY=0 TI=$LC-8$      *TC-11,13,15
LCAS RF=0 CA=9 TY=0 TI=$LC-9$      *TC-12
LCAS RF=0 CA=10 TY=0 TI=$LC-10$     *TC-20
LCAS RF=0 CA=11 TY=0 TI=$LC-11$     *TC-22
LCAS RF=0 CA=12 TY=0 TI=$LC-12$     *TC-23
LCAS RF=0 CA=13 TY=0 TI=$LC-13$     *TC-25
LCAS RF=0 CA=14 TY=0 TI=$LC-14$     *TC-26, 29
LCAS RF=0 CA=15 TY=0 TI=$LC-15$     *TC-27
LCAS RF=0 CA=16 TY=0 TI=$LC-16$     *TC-28
LCAS RF=0 CA=17 TY=0 TI=$LC-17$     *TC-30
LCAS RF=0 CA=18 TY=0 TI=$LC-18$     *TC-31
LCAS RF=0 CA=19 TY=0 TI=$LC-19$     *TC-32
LCAS RF=0 CA=20 TY=0 TI=$LC-20$     *TC-33
LCAS RF=0 CA=21 TY=0 TI=$LC-21$     *TC-35
LCAS RF=0 CA=22 TY=0 TI=$LC-22$     *TC-37
LCAS RF=0 CA=23 TY=0 TI=$LC-23$     *TC-39
LCAS RF=0 CA=24 TY=0 TI=$LC-24$     *TC-40
LCAS RF=0 CA=25 TY=0 TI=$LC-25$     *TC-41
LCAS RF=0 CA=26 TY=0 TI=$LC-26$     *TC-17
LCAS RF=0 CA=27 TY=0 TI=$LC-27$     *TC-19
LCAS RF=0 CA=28 TY=0 TI=$LC-28$     *TC-4
LCAS RF=0 CA=29 TY=0 TI=$LC-29$     *TC-18
LCAS RF=0 CA=30 TY=0 TI=$LC-30$     *TC-42

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LCAS RF=0 CA=31 TY=0 TI=\$LIC-31 *TC-29

LCAS RF=6 CA=32 TY=6 TI=\$SAM\$

**** WEIGHT CASES****

LCAS CA=101 RF=1 TY=3 TI=\$OPERATING WEIGHT\$

LCAS CA=102 RF=2 TY=4 TI=\$HYDROTEST WEIGHT\$

*

**** THERMAL TRANSIENT CASES****

TCAS CA=201 RP=1 TI=\$Design Hydrotest +\$

TCAS CA=202 RP=1 TI=\$Design Hydrotest -\$

TCAS CA=203 RP=1 TI=\$Startup +\$

TCAS CA=204 RP=1 TI=\$TRoll & Inc. PWR1 -\$

TCAS CA=205 RP=1 TI=\$TRoll & Inc. PWR2 +\$

TCAS CA=206 RP=1 TI=\$TRoll & Inc. PWR3 +\$

TCAS CA=207 RP=1 TI=\$DlyReduction to 75% -\$

TCAS CA=208 RP=1 TI=\$DlyReduction to 75% +\$

TCAS CA=209 RP=1 TI=\$WklyReduct to 50% -\$

TCAS CA=210 RP=1 TI=\$WklyReduct to 50% +\$

TCAS CA=211 RP=1 TI=\$LOFWH+TT 1 -\$

TCAS CA=212 RP=1 TI=\$LOFWH+TT 2 -\$

TCAS CA=213 RP=1 TI=\$LOFWH+TT 3 +\$

TCAS CA=214 RP=1 TI=\$LOFWH+TT 4 +\$

TCAS CA=215 RP=1 TI=\$LOFWH+PFWHTR Byp -\$

TCAS CA=216 RP=1 TI=\$LOFWH+PFWHTR Byp +\$

TCAS CA=217 RP=1 TI=\$SCRAM+TT+AllOtrScm -\$

TCAS CA=218 RP=1 TI=\$SCRAM+TT+AllOtrScm -\$

TCAS CA=219 RP=1 TI=\$HotStandby 1 +\$

TCAS CA=220 RP=1 TI=\$HotStandby 2 +\$

TCAS CA=221 RP=1 TI=\$HotStandby 3 -\$

TCAS CA=222 RP=1 TI=\$Shutdown 1 -\$

TCAS CA=223 RP=1 TI=\$Shutdown 2 -\$

TCAS CA=224 RP=1 TI=\$Shutdown 3 -\$

TCAS CA=225 RP=1 TI=\$SCRAM+LOFWP1 +\$

TCAS CA=226 RP=1 TI=\$SCRAM+LOFWP2 -\$

TCAS CA=227 RP=1 TI=\$SCRAM+LOFWP3 +\$

TCAS CA=228 RP=1 TI=\$SCRAM+LOFWP4 +\$

TCAS CA=229 RP=1 TI=\$SCRAM+LOFWP5 -\$

TCAS CA=230 RP=1 TI=\$SCRAM+LOFWP6 +\$

TCAS CA=231 RP=1 TI=\$SCRAM+LOFWP7 +\$

TCAS CA=232 RP=1 TI=\$SCRAM+LOFWP8 -\$

TCAS CA=233 RP=1 TI=\$SCRAM+LOFWP9 +\$

TCAS CA=234 RP=1 TI=\$SCRAM+LOFWP10+\$

TCAS CA=235 RP=1 TI=\$SCRAM+SRVBLDN1-\$

TCAS CA=236 RP=1 TI=\$SCRAM+SRVBLDN2-\$

TCAS CA=237 RP=1 TI=\$Hydro Test +\$

TCAS CA=238 RP=1 TI=\$Hydro Test -\$

TCAS CA=239 RP=1 TI=\$SCRAM+TG+OPres1 -\$

TCAS CA=240 RP=1 TI=\$SCRAM+TG+OPres2 -\$

TCAS CA=241 RP=1 TI=\$SCRAM+TG+OPres3 -\$

TCAS CA=242 RP=1 TI=\$HotSby_FWcyc +\$

TCAS CA=243 RP=1 TI=\$HotSby_FWcyc +\$

**** SEISMIC CASES****

RCAS CA=103 EQ=3 EV=1 TY=1 SU=1 LO=1 FX=1 FY=1 FZ=1 TI=\$OBE INERTIAS

*

** *****

**** LOAD COMBINATION CASES *

** *****

CCAS RF=1 CA=104 ME=1 FL=1 C1=103 CY=10 TI=\$OBE\$
 CCAS RF=1 CA=401 SS=1 ME=1 EQ=3 C1=102 C2=103 TI=\$EQUATION 9 LEVEL B\$
 CCAS RF=1 CA=402 SS=1 ME=3 F1=1 C1=103 C2=6 C3=32 TI=\$NORMAL+OBE\$
 CCAS RF=1 CA=403 SS=1 ME=3 F1=-1 C1=103 C2=6 C3=32 TI=\$NORMAL-OBE\$

*

**** LOAD SETS****

*RF field is the highest temperature and pressure of the transient
 *PR and MO fields are the final temperature and pressure of the transient

LSET RF=1	RP=1	CY=120	PR=1	MO=1	TR=+201	TI=\$Design Hydrotest +	LS-1\$
LSET RF=2	RP=1	CY=120	PR=2	MO=2	TR=-202	TI=\$Design Hydrotest -	LS-2\$
LSET RF=3	RP=1	CY=300	PR=3	MO=3	TR=+203	TI=\$Startup +	LS-3\$
LSET RF=3	RP=1	CY=610	PR=28	MO=28	TR=-204	TI=\$TRoll & Inc. PWR1 -	LS-4\$
LSET RF=4	RP=1	CY=599	PR=4	MO=4	TR=+205	TI=\$TRoll & Inc. PWR2 +	LS-5\$
LSET RF=5	RP=1	CY=599	PR=5	MO=5	TR=+206	TI=\$TRoll & Inc. PWR3 +	LS-6\$
LSET RF=5	RP=1	CY=10000	PR=6	MO=6	TR=-207	TI=\$DlyReduction to 75% -	LS-7\$
LSET RF=5	RP=1	CY=10000	PR=5	MO=5	TR=+208	TI=\$DlyReduction to 75% +	LS-8\$
LSET RF=5	RP=1	CY=2000	PR=7	MO=7	TR=-209	TI=\$WklyReduct to 50% -	LS-9\$
LSET RF=5	RP=1	CY=2000	PR=5	MO=5	TR=+210	TI=\$WklyReduct to 50% +	LS-10\$
LSET RF=5	RP=1	CY=310	PR=8	MO=8	TR=-211	TI=\$LOFWH+TT 1 -	LS-11\$
LSET RF=8	RP=1	CY=10	PR=9	MO=9	TR=-212	TI=\$LOFWH+TT 2 -	LS-12\$
LSET RF=8	RP=1	CY=10	PR=8	MO=8	TR=+213	TI=\$LOFWH+TT 3 +	LS-13\$
LSET RF=5	RP=1	CY=10	PR=5	MO=5	TR=+214	TI=\$LOFWH+TT 4 +	LS-14\$
LSET RF=5	RP=1	CY=70	PR=8	MO=8	TR=-215	TI=\$LOFWH+PFWHTR Byp -	LS-15\$
LSET RF=5	RP=1	CY=70	PR=5	MO=5	TR=+216	TI=\$LOFWH+PFWHTR Byp +	LS-16\$
LSET RF=5	RP=1	CY=289	PR=26	MO=26	TR=-217	TI=\$SCRAM+TT+AllOtrScm -	LS-17\$
LSET RF=26	RP=1	CY=289	PR=29	MO=29	TR=-218	TI=\$SCRAM+TT+AllOtrScm -	LS-18\$
LSET RF=27	RP=1	CY=300	PR=27	MO=27	TR=+219	TI=\$HotStandby 1 +	LS-19\$
LSET RF=10	RP=1	CY=300	PR=10	MO=10	TR=+220	TI=\$HotStandby 2 +	LS-20\$
LSET RF=10	RP=1	CY=300	PR=3	MO=3	TR=-221	TI=\$HotStandby 3 -	LS-21\$
LSET RF=3	RP=1	CY=300	PR=11	MO=11	TR=-222	TI=\$Shutdown 1 -	LS-22\$
LSET RF=11	RP=1	CY=300	PR=12	MO=12	TR=-223	TI=\$Shutdown 2 -	LS-23\$
LSET RF=12	RP=1	CY=300	PR=2	MO=2	TR=-224	TI=\$Shutdown 3 -	LS-24\$
LSET RF=13	RP=1	CY=10	PR=13	MO=13	TR=+225	TI=\$SCRAM+LOFWP1 +	LS-25\$
LSET RF=13	RP=1	CY=10	PR=14	MO=14	TR=-226	TI=\$SCRAM+LOFWP2 -	LS-26\$
LSET RF=15	RP=1	CY=10	PR=15	MO=15	TR=+227	TI=\$SCRAM+LOFWP3 +	LS-27\$
LSET RF=16	RP=1	CY=10	PR=16	MO=16	TR=+228	TI=\$SCRAM+LOFWP4 +	LS-28\$
LSET RF=16	RP=1	CY=10	PR=31	MO=31	TR=-229	TI=\$SCRAM+LOFWP5 -	LS-29\$
LSET RF=17	RP=1	CY=10	PR=17	MO=17	TR=+230	TI=\$SCRAM+LOFWP6 +	LS-30\$
LSET RF=18	RP=1	CY=10	PR=18	MO=18	TR=+231	TI=\$SCRAM+LOFWP7 +	LS-31\$
LSET RF=18	RP=1	CY=10	PR=19	MO=19	TR=-232	TI=\$SCRAM+LOFWP8 -	LS-32\$
LSET RF=20	RP=1	CY=10	PR=20	MO=20	TR=+233	TI=\$SCRAM+LOFWP9 +	LS-33\$
LSET RF=3	RP=1	CY=10	PR=3	MO=3	TR=+234	TI=\$SCRAM+LOFWP10+	LS-34\$
LSET RF=5	RP=1	CY=1	PR=21	MO=21	TR=-235	TI=\$SCRAM+SRVBLDN1-	LS-35\$
LSET RF=21	RP=1	CY=1	PR=2	MO=2	TR=-236	TI=\$SCRAM+SRVBLDN2-	LS-36\$
LSET RF=22	RP=1	CY=1	PR=22	MO=22	TR=+237	TI=\$Hydro Test +	LS-37\$
LSET RF=2	RP=1	CY=1	PR=2	MO=2	TR=-238	TI=\$Hydro Test -	LS-38\$
LSET RF=23	RP=1	CY=289	PR=23	MO=23	TR=-239	TI=\$SCRAM+TG+OPres1 -	LS-39\$
LSET RF=24	RP=1	CY=289	PR=24	MO=24	TR=-240	TI=\$SCRAM+TG+OPres2 -	LS-40\$

LSET RF=25 RP=1 CY=289 PR=25 MO=25 TR=-241 TI=\$SCRAM+TG+OPres3 - LS-41\$
 LSET RF=30 RP=1 CY=300 PR=30 MO=30 TR=+242 TI=\$HotSbyFWcyc + LS-42\$
 LSET RF=3 RP=1 CY=300 PR=3 MO=3 TR=+243 TI=\$HotSbyFWcyc + LS-43\$

*
 LSET RF=6 CY=5 FL=1 PR=6 MO=402 TI=\$NORMAL+OBE LS-132\$
 LSET RF=6 CY=5 FL=1 PR=6 MO=403 TI=\$NORMAL-OBE LS-133\$

 **** RESPONSE SPECTRA****

*SSE response spectra conservatively used

SPEC FS=OBE EV=1 ME=3 FP=1 TI=\$RESPONSE\$

LV=1 DX=1 DY=1 DZ=1

DI=X

0.30/0.125	0.80/0.300	2.00/0.650	3.00/0.725	3.50/1.000	4.40/1.200
5.00/1.900	5.75/2.850	6.00/3.375	8.25/3.375	9.00/3.000	10.00/2.400
14.00/1.325	19.00/1.600	21.00/1.000	22.00/0.800	30.00/0.700	36.00/0.650

DI=Y

0.30/0.075	1.25/0.250	1.75/0.325	2.40/0.450	2.75/0.475	3.80/0.500
4.40/0.500	4.80/0.600	7.25/0.600	7.50/0.700	8.50/0.700	10.00/0.925
12.00/1.450	16.00/1.900	18.00/1.700	20.00/0.750	25.00/0.450	30.00/0.350
36.00/0.325	36.10/0.325	36.20/0.325	36.30/0.325	36.40/0.325	36.50/0.325

DI=Z

0.30/0.150	1.00/0.350	2.00/0.625	4.00/1.000	4.50/1.400	5.00/2.000
5.75/2.950	6.00/3.450	6.25/3.800	8.75/3.800	10.00/2.625	12.0/2.150
15.00/1.300	17.50/1.450	20.00/0.875	30.00/0.650	36.00/0.650	36.10/0.650

 **** MATERIAL PROPERTIES ****

* SA-106 Grade B and SA-234 WPB

MATH CD=106 EX=0 TY=1 *C-Si

*MATD TE=-100 EH=30.2 EX=0 SM=20 SY=35

MATD TE=50 EH=29.6 EX=0 SM=20 SY=35

MATD TE=70 EH=29.5 EX=0 SM=20 SY=35

MATD TE=100 EH=29.3 EX=0.2 SM=20.0 SY=35

MATD TE=200 EH=28.8 EX=1.0 SM=20.0 SY=32.1

MATD TE=300 EH=28.3 EX=1.9 SM=20.0 SY=31

MATD TE=400 EH=27.7 EX=2.8 SM=20.0 SY=29.9

MATD TE=500 EH=27.3 EX=3.7 SM=18.9 SY=28.5

MATD TE=600 EH=26.7 EX=4.7 SM=17.3 SY=26.8

*** Cross Sectional Properties

*REGION I- LINE 16 INCH FDW-16 SCH. 120 Run from 5 to 10

*Anchor HD36 to HPCI brnch

CROS CD=1 OD=16.0 WT=1.218 MA=204.28

SO=1 ST=1 IN=0

*FEEDWATER Valves - V2-27A, V2-28A, V2-29A

CROS CD=2 OD=24.0 WT=2.436 MA=0.12

SO=1 ST=1 IN=0 KL=1

*REGION III- LINE 16 INCH FDW-16 SCH. 80

*Piping Downstream of Valve V2-29A TO FW TEE

CROS CD=3 OD=16.0 WT=0.843 MA=151.1

SO=1 ST=1 IN=0

*REGION III- LINE 16 INCH FDW-16 SCH. 120

*Fittings Downstream of Valve V2-29A TO FW TEE

CROS CD=4 OD=16.0 WT=1.218 MA=204.28

SO=1 ST=1 IN=0

*REGION IV & V- LINES 10 INCH INCH FDW-21 AND 10 INCH FDW-19 SCH. 120

*Piping Downstream of FW TEE TO NOZZLES

CROS CD=5 OD=10.75 WT=0.843 MA=98.12

SO=1 ST=1 IN=0

*REGION II- LINE 14 INCH HPCI-15A SCH. 120 FROM NODE 10 TO 547

CROS CD=6 OD=14.0 WT=1.093 MA=161.35

SO=1 ST=1 IN=1

*REGION II- HPCI Valves

CROS CD=7 OD=21.0 WT=2.186 MA=0.12

SO=1 ST=1 IN=1 KL=1

*

* STRUCTURE AND LOADS

*

DESN TE=400.0 PR=1900.0 *FEEDWATER AND HPCI PIPING

*BEGIN REGION I

*Same for all regions except II

OPER CA=1 TE=100 PR=1100

OPER CA=22 TE=100 PR=1563

OPER CA=28 TE=100 PR=1010

OPER CA=29 TE=100 PR=1010

*Same for all regions

OPER CA=2 TE=100 PR=50

OPER CA=19 TE=50 PR=675

OPER CA=31 TE=50 PR=885

*Unique

OPER CA=3 TE=150 PR=1010

OPER CA=4 TE=260 PR=1010

OPER CA=5 TE=392 PR=1010

OPER CA=6 TE=310 PR=1010

OPER CA=7 TE=280 PR=1010

OPER CA=8 TE=265 PR=1010

OPER CA=9 TE=90 PR=1010

OPER CA=10 TE=265 PR=1010

OPER CA=11 TE=150 PR=170

OPER CA=12 TE=150 PR=88

OPER CA=13 TE=392 PR=1190

OPER CA=14 TE=50 PR=1135

OPER CA=15 TE=150 PR=1135

OPER CA=16 TE=150 PR=1135

OPER CA=17 TE=150 PR=1060

OPER CA=18 TE=150 PR=1135

OPER CA=20 TE=150 PR=675

OPER CA=21 TE=275 PR=885

OPER CA=23 TE=392 PR=1375

OPER CA=24 TE=392 PR=940

OPER CA=25 TE=392 PR=1010

OPER CA=26 TE=275 PR=1010

OPER CA=27 TE=265 PR=1010



OPER CA=30 TE=125 PR=1010

*

TRAN CA=201 IS=1 FS=1 IT=70 FT=100 TT=1800 FL=200 IP=15 FP=1115 TP=1800
 TRAN CA=202 IS=1 FS=1 IT=100 FT=100 TT=0 FL=200 IP=1115 FP=65 TP=0
 TRAN CA=203 IS=1 FS=1 IT=100 FT=150 TT=16164 FL=200 IP=65 FP=1025 TP=16164
 TRAN CA=204 IS=1 FS=1 IT=150 FT=100 TT=0 FL=1377 IP=1025 FP=1025 TP=0
 TRAN CA=205 IS=1 FS=1 IT=100 FT=260 TT=0 FL=1377 IP=1025 FP=1025 TP=0
 TRAN CA=206 IS=1 FS=1 IT=260 FT=392 TT=1800 FL=9180 IP=1025 FP=1025 TP=1800
 TRAN CA=207 IS=1 FS=1 IT=392 FT=310 TT=900 FL=6885 IP=1025 FP=1025 TP=900
 TRAN CA=208 IS=1 FS=1 IT=310 FT=392 TT=900 FL=6885 IP=1025 FP=1025 TP=900
 TRAN CA=209 IS=1 FS=1 IT=392 FT=280 TT=1800 FL=4590 IP=1025 FP=1025 TP=1800
 TRAN CA=210 IS=1 FS=1 IT=280 FT=392 TT=1800 FL=4590 IP=1025 FP=1025 TP=1800
 TRAN CA=211 IS=1 FS=1 IT=392 FT=265 TT=1800 FL=4590 IP=1025 FP=1025 TP=1800
 TRAN CA=212 IS=1 FS=1 IT=265 FT=90 TT=360 FL=1377 IP=1025 FP=1025 TP=360
 TRAN CA=213 IS=1 FS=1 IT=90 FT=265 TT=900 FL=1377 IP=1025 FP=1025 TP=900
 TRAN CA=214 IS=1 FS=1 IT=265 FT=392 TT=1800 FL=4590 IP=1025 FP=1025 TP=1800
 TRAN CA=215 IS=1 FS=1 IT=392 FT=265 TT=90 FL=9180 IP=1025 FP=1025 TP=90
 TRAN CA=216 IS=1 FS=1 IT=265 FT=392 TT=180 FL=9180 IP=1025 FP=1025 TP=180
 TRAN CA=217 IS=1 FS=1 IT=392 FT=275 TT=60 FL=10098 IP=1025 FP=1025 TP=60
 TRAN CA=218 IS=1 FS=1 IT=275 FT=100 TT=900 FL=275.4 IP=1025 FP=1025 TP=900
 TRAN CA=219 *IS=1 FS=1 IT=265 FT=265 TT=0 FL=200 IP=1025 FP=1025 TP=0
 TRAN CA=220 *IS=1 FS=1 IT=265 FT=265 TT=0 FL=200 IP=1025 FP=1025 TP=0
 TRAN CA=221 IS=1 FS=1 IT=265 FT=150 TT=4140 FL=200 IP=1025 FP=1025 TP=4140
 TRAN CA=222 IS=1 FS=1 IT=150 FT=150 TT=0 FL=200 IP=1025 FP=185 TP=0
 TRAN CA=223 IS=1 FS=1 IT=150 FT=150 TT=0 FL=200 IP=185 FP=103 TP=0
 TRAN CA=224 IS=1 FS=1 IT=150 FT=100 TT=8280 FL=200 IP=103 FP=65 TP=8280
 TRAN CA=225 IS=1 FS=1 IT=392 FT=392 TT=12 FL=200 IP=1025 FP=1205 TP=12
 TRAN CA=226 IS=1 FS=1 IT=392 FT=50 TT=0 FL=3672 IP=1205 FP=1150 TP=0
 TRAN CA=227 IS=1 FS=1 IT=50 FT=150 TT=1380 FL=200 IP=1150 FP=1150 TP=1380
 TRAN CA=228 IS=1 FS=1 IT=150 FT=150 TT=0 FL=200 IP=1150 FP=1150 TP=0
 TRAN CA=229 IS=1 FS=1 IT=150 FT=50 TT=0 FL=2754 IP=1150 FP=900 TP=0
 TRAN CA=230 IS=1 FS=1 IT=50 FT=150 TT=3060 FL=200 IP=900 FP=1075 TP=3060
 TRAN CA=231 IS=1 FS=1 IT=150 FT=150 TT=0 FL=200 IP=1075 FP=1150 TP=0
 TRAN CA=232 IS=1 FS=1 IT=150 FT=50 TT=0 FL=1560.6 IP=1150 FP=690 TP=0
 TRAN CA=233 IS=1 FS=1 IT=50 FT=150 TT=300 FL=200 IP=690 FP=690 TP=300
 TRAN CA=234 IS=1 FS=1 IT=150 FT=150 TT=8964 FL=200 IP=255 FP=1025 TP=8964
 TRAN CA=235 IS=1 FS=1 IT=392 FT=275 TT=60 FL=10098 IP=1025 FP=900 TP=60
 TRAN CA=236 IS=1 FS=1 IT=275 FT=100 TT=900 FL=275.4 IP=900 FP=65 TP=900
 TRAN CA=237 IS=1 FS=1 IT=100 FT=100 TT=0 FL=200 IP=65 FP=1578 TP=0
 TRAN CA=238 IS=1 FS=1 IT=100 FT=100 TT=0 FL=200 IP=1578 FP=65 TP=0
 TRAN CA=239 IS=1 FS=1 IT=392 FT=392 TT=60 FL=10098 IP=1025 FP=1390 TP=60
 TRAN CA=240 IS=1 FS=1 IT=392 FT=392 TT=900 FL=275.4 IP=1390 FP=955 TP=900
 TRAN CA=241 IS=1 FS=1 IT=392 FT=392 TT=900 FL=275.4 IP=955 FP=1025 TP=900
 TRAN CA=242 IS=1 FS=1 IT=100 FT=125 TT=60 FL=200 IP=1025 FP=1025 TP=60
 TRAN CA=243 IS=1 FS=1 IT=125 FT=150 TT=210 FL=200 IP=1025 FP=1025 TP=210

PAIR CA=201 CO=27.6 DI=0.521 EX=6.4 * Tavg=85
 PAIR CA=202 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
 PAIR CA=203 CO=27.6 DI=0.504 EX=6.4 * Tavg=125
 PAIR CA=204 CO=27.6 DI=0.504 EX=6.4 * Tavg=125
 PAIR CA=205 CO=27.6 DI=0.490 EX=6.4 * Tavg=180
 PAIR CA=206 CO=27.1 DI=0.446 EX=6.4 * Tavg=326
 PAIR CA=207 CO=27.0 DI=0.440 EX=6.4 * Tavg=351
 PAIR CA=208 CO=27.0 DI=0.440 EX=6.4 * Tavg=351
 PAIR CA=209 CO=27.1 DI=0.444 EX=6.4 * Tavg=336



PAIR CA=210 CO=27.1 DI=0.444 EX=6.4 * Tavg=336
PAIR CA=211 CO=27.1 DI=0.445 EX=6.4 * Tavg=329
PAIR CA=212 CO=27.6 DI=0.490 EX=6.4 * Tavg=178
PAIR CA=213 CO=27.6 DI=0.490 EX=6.4 * Tavg=178
PAIR CA=214 CO=27.1 DI=0.445 EX=6.4 * Tavg=329
PAIR CA=215 CO=27.1 DI=0.445 EX=6.4 * Tavg=329
PAIR CA=216 CO=27.1 DI=0.445 EX=6.4 * Tavg=329
PAIR CA=217 CO=27.1 DI=0.444 EX=6.4 * Tavg=334
PAIR CA=218 CO=27.6 DI=0.488 EX=6.4 * Tavg=188
PAIR CA=219 CO=27.3 DI=0.463 EX=6.4 * Tavg=265
PAIR CA=220 CO=27.3 DI=0.463 EX=6.4 * Tavg=265
PAIR CA=221 CO=27.6 DI=0.483 EX=6.4 * Tavg=208
PAIR CA=222 CO=27.6 DI=0.496 EX=6.4 * Tavg=150
PAIR CA=223 CO=27.6 DI=0.496 EX=6.4 * Tavg=150
PAIR CA=224 CO=27.6 DI=0.504 EX=6.4 * Tavg=125
PAIR CA=225 CO=26.7 DI=0.430 EX=6.4 * Tavg=392
PAIR CA=226 CO=27.5 DI=0.478 EX=6.4 * Tavg=221
PAIR CA=227 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
PAIR CA=228 CO=27.6 DI=0.496 EX=6.4 * Tavg=150
PAIR CA=229 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
PAIR CA=230 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
PAIR CA=231 CO=27.6 DI=0.496 EX=6.4 * Tavg=150
PAIR CA=232 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
PAIR CA=233 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
PAIR CA=234 CO=27.6 DI=0.496 EX=6.4 * Tavg=150
PAIR CA=235 CO=27.1 DI=0.444 EX=6.4 * Tavg=334
PAIR CA=236 CO=27.6 DI=0.488 EX=6.4 * Tavg=188
PAIR CA=237 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
PAIR CA=238 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
PAIR CA=239 CO=26.7 DI=0.430 EX=6.4 * Tavg=392
PAIR CA=240 CO=26.7 DI=0.430 EX=6.4 * Tavg=392
PAIR CA=241 CO=26.7 DI=0.430 EX=6.4 * Tavg=392
PAIR CA=242 CO=27.6 DI=0.508 EX=6.4 * Tavg=113
PAIR CA=243 CO=27.6 DI=0.500 EX=6.4 * Tavg=138

*REGION I GEOMETRY

* RUN 1 FROM ANCHOR HD36 TO HPCI brnCH- FDW-16 LINE A

MATL CD=106

CROS CD=1

COOR PT=5 AX=0 AY=0 AZ=0 *ANCHOR HD36

JUNC PT=5

TANG PT=9 DZ=-2.75 EW=1

TANG PT=10 DZ=-1 *WELDING TEE PER ANSI B16.9

*-----
*END REGION I
*-----

*BEGIN REGION 3
*-----

*OPER cards same as those for region I
*

TRAN CA=201 IS=1 FS=1 IT=70 FT=100 TT=1800 FL=200 IP=15 FP=1115 TP=1800
TRAN CA=202 IS=1 FS=1 IT=100 FT=100 TT=0 FL=200 IP=1115 FP=65 TP=0
TRAN CA=203 IS=1 FS=1 IT=100 FT=150 TT=16164 FL=200 IP=65 FP=1025 TP=16164
TRAN CA=204 IS=1 FS=1 IT=150 FT=100 TT=0 FL=1377 IP=1025 FP=1025 TP=0
TRAN CA=205 IS=1 FS=1 IT=100 FT=260 TT=0 FL=1377 IP=1025 FP=1025 TP=0
TRAN CA=206 IS=1 FS=1 IT=260 FT=392 TT=1800 FL=9180 IP=1025 FP=1025 TP=1800



TRAN CA=207 IS=1 FS=1 IT=392 FT=310 TT=900 FL=6885 IP=1025 FP=1025 TP=900
 TRAN CA=208 IS=1 FS=1 IT=310 FT=392 TT=900 FL=6885 IP=1025 FP=1025 TP=900
 TRAN CA=209 IS=1 FS=1 IT=392 FT=280 TT=1800 FL=4590 IP=1025 FP=1025 TP=1800
 TRAN CA=210 IS=1 FS=1 IT=280 FT=392 TT=1800 FL=4590 IP=1025 FP=1025 TP=1800
 TRAN CA=211 IS=1 FS=1 IT=392 FT=265 TT=1800 FL=4590 IP=1025 FP=1025 TP=1800
 TRAN CA=212 IS=1 FS=1 IT=265 FT=90 TT=360 FL=1377 IP=1025 FP=1025 TP=360
 TRAN CA=213 IS=1 FS=1 IT=90 FT=265 TT=900 FL=1377 IP=1025 FP=1025 TP=900
 TRAN CA=214 IS=1 FS=1 IT=265 FT=392 TT=1800 FL=4590 IP=1025 FP=1025 TP=1800
 TRAN CA=215 IS=1 FS=1 IT=392 FT=265 TT=90 FL=9180 IP=1025 FP=1025 TP=90
 TRAN CA=216 IS=1 FS=1 IT=265 FT=392 TT=180 FL=9180 IP=1025 FP=1025 TP=180
 TRAN CA=217 IS=1 FS=1 IT=392 FT=275 TT=60 FL=10098 IP=1025 FP=1025 TP=60
 TRAN CA=218 IS=1 FS=1 IT=275 FT=100 TT=900 FL=275.4 IP=1025 FP=1025 TP=900
 TRAN CA=219 *IS=1 FS=1 IT=265 FT=265 TT=0 FL=200 IP=1025 FP=1025 TP=0
 TRAN CA=220 *IS=1 FS=1 IT=265 FT=265 TT=0 FL=200 IP=1025 FP=1025 TP=0
 TRAN CA=221 IS=1 FS=1 IT=265 FT=150 TT=4140 FL=200 IP=1025 FP=1025 TP=4140
 TRAN CA=222 IS=1 FS=1 IT=150 FT=150 TT=0 FL=200 IP=1025 FP=185 TP=0
 TRAN CA=223 IS=1 FS=1 IT=150 FT=150 TT=0 FL=200 IP=185 FP=103 TP=0
 TRAN CA=224 IS=1 FS=1 IT=150 FT=100 TT=8280 FL=200 IP=103 FP=65 TP=8280
 TRAN CA=225 IS=1 FS=1 IT=392 FT=392 TT=12 FL=200 IP=1025 FP=1205 TP=12
 TRAN CA=226 IS=1 FS=1 IT=392 FT=50 TT=0 FL=3672 IP=1205 FP=1150 TP=0
 TRAN CA=227 IS=1 FS=1 IT=50 FT=150 TT=1380 FL=200 IP=1150 FP=1150 TP=1380
 TRAN CA=228 *IS=1 FS=1 IT=150 FT=150 TT=0 FL=200 IP=1150 FP=1150 TP=0
 TRAN CA=229 IS=1 FS=1 IT=150 FT=50 TT=0 FL=2754 IP=1150 FP=900 TP=0
 TRAN CA=230 IS=1 FS=1 IT=50 FT=150 TT=3060 FL=200 IP=900 FP=1075 TP=3060
 TRAN CA=231 IS=1 FS=1 IT=150 FT=150 TT=0 FL=200 IP=1075 FP=1150 TP=0
 TRAN CA=232 IS=1 FS=1 IT=150 FT=50 TT=0 FL=1560.6 IP=1150 FP=690 TP=0
 TRAN CA=233 IS=1 FS=1 IT=50 FT=150 TT=300 FL=200 IP=690 FP=690 TP=300
 TRAN CA=234 IS=1 FS=1 IT=150 FT=150 TT=8964 FL=200 IP=255 FP=1025 TP=8964
 TRAN CA=235 IS=1 FS=1 IT=392 FT=275 TT=60 FL=10098 IP=1025 FP=900 TP=60
 TRAN CA=236 IS=1 FS=1 IT=275 FT=100 TT=900 FL=275.4 IP=900 FP=65 TP=900
 TRAN CA=237 IS=1 FS=1 IT=100 FT=100 TT=0 FL=200 IP=65 FP=1578 TP=0
 TRAN CA=238 IS=1 FS=1 IT=100 FT=100 TT=0 FL=200 IP=1578 FP=65 TP=0
 TRAN CA=239 IS=1 FS=1 IT=392 FT=392 TT=60 FL=10098 IP=1025 FP=1390 TP=60
 TRAN CA=240 IS=1 FS=1 IT=392 FT=392 TT=900 FL=275.4 IP=1390 FP=955 TP=900
 TRAN CA=241 IS=1 FS=1 IT=392 FT=392 TT=900 FL=275.4 IP=955 FP=1025 TP=900
 TRAN CA=242 IS=1 FS=1 IT=100 FT=125 TT=60 FL=200 IP=1025 FP=1025 TP=60
 TRAN CA=243 IS=1 FS=1 IT=125 FT=150 TT=210 FL=200 IP=1025 FP=1025 TP=210

*
 PAIR CA=201 CO=27.6 DI=0.521 EX=6.4 * Tavq=85
 PAIR CA=202 CO=27.6 DI=0.512 EX=6.4 * Tavq=100
 PAIR CA=203 CO=27.6 DI=0.504 EX=6.4 * Tavq=125
 PAIR CA=204 CO=27.6 DI=0.504 EX=6.4 * Tavq=125
 PAIR CA=205 CO=27.6 DI=0.490 EX=6.4 * Tavq=180
 PAIR CA=206 CO=27.1 DI=0.446 EX=6.4 * Tavq=326
 PAIR CA=207 CO=27.0 DI=0.440 EX=6.4 * Tavq=351
 PAIR CA=208 CO=27.0 DI=0.440 EX=6.4 * Tavq=351
 PAIR CA=209 CO=27.1 DI=0.444 EX=6.4 * Tavq=336
 PAIR CA=210 CO=27.1 DI=0.444 EX=6.4 * Tavq=336
 PAIR CA=211 CO=27.1 DI=0.445 EX=6.4 * Tavq=329
 PAIR CA=212 CO=27.6 DI=0.490 EX=6.4 * Tavq=178
 PAIR CA=213 CO=27.6 DI=0.490 EX=6.4 * Tavq=178
 PAIR CA=214 CO=27.1 DI=0.445 EX=6.4 * Tavq=329
 PAIR CA=215 CO=27.1 DI=0.445 EX=6.4 * Tavq=329
 PAIR CA=216 CO=27.1 DI=0.445 EX=6.4 * Tavq=329
 PAIR CA=217 CO=27.1 DI=0.444 EX=6.4 * Tavq=334



PAIR CA=218 CO=27.6 DI=0.488 EX=6.4 * Tavg=188
PAIR CA=219 CO=27.3 DI=0.463 EX=6.4 * Tavg=265
PAIR CA=220 CO=27.3 DI=0.463 EX=6.4 * Tavg=265
PAIR CA=221 CO=27.6 DI=0.483 EX=6.4 * Tavg=208
PAIR CA=222 CO=27.6 DI=0.496 EX=6.4 * Tavg=150
PAIR CA=223 CO=27.6 DI=0.496 EX=6.4 * Tavg=150
PAIR CA=224 CO=27.6 DI=0.504 EX=6.4 * Tavg=125
PAIR CA=225 CO=26.7 DI=0.430 EX=6.4 * Tavg=392
PAIR CA=226 CO=27.5 DI=0.478 EX=6.4 * Tavg=221
PAIR CA=227 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
PAIR CA=228 CO=27.6 DI=0.496 EX=6.4 * Tavg=150
PAIR CA=229 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
PAIR CA=230 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
PAIR CA=231 CO=27.6 DI=0.496 EX=6.4 * Tavg=150
PAIR CA=232 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
PAIR CA=233 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
PAIR CA=234 CO=27.6 DI=0.496 EX=6.4 * Tavg=150
PAIR CA=235 CO=27.1 DI=0.444 EX=6.4 * Tavg=334
PAIR CA=236 CO=27.6 DI=0.488 EX=6.4 * Tavg=188
PAIR CA=237 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
PAIR CA=238 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
PAIR CA=239 CO=26.7 DI=0.430 EX=6.4 * Tavg=392
PAIR CA=240 CO=26.7 DI=0.430 EX=6.4 * Tavg=392
PAIR CA=241 CO=26.7 DI=0.430 EX=6.4 * Tavg=392
PAIR CA=242 CO=27.6 DI=0.508 EX=6.4 * Tavg=113
PAIR CA=243 CO=27.6 DI=0.500 EX=6.4 * Tavg=138

*

*REGION III GEOMETRY

CROS CD=1

*JUNC PT=10

TANG PT=11 DZ=-1 EW=1

TANG PT=15 DZ=-4.17

TANG PT=20 DZ=-0.333 EW=1 *TA=1

CROS CD=2

VALV PT=22 DZ=-1.333 PL=1 MA=2.7 *VALVE V2-27A

VALV PT=25 DZ=-1.333 PL=2 EW=1 *TA=1

CROS CD=1

TANG PT=30 DZ=-2.792

LUMP PT=30 MA=1.285

TANG PT=38 DZ=-4.6

TANG PT=40 DZ=-6.317

TANG PT=45 DZ=-0.625 EW=1 *TA=1

CROS CD=2

VALV PT=47 DZ=-1.792 PL=1 MA=2.7 *VALVE V2-28A

VALV PT=50 DZ=-1.792 PL=2 EW=1 *TA=1

CROS CD=1

*TANG PT=55 DZ=-2.791 EW=1

TANG PT=55 DZ=-.791 EW=1

*BRAD PT=65 RA=2 SD=2 EW=1 Used this to determine midpoint viw .prd output

BEND PT=60 X1=0 Y1=0 Z1=-.828 X2=0 Y2=.586 Z2=-.586

BEND PT=65 X1=0 Y1=.586 Z1=-.586 X2=0 Y2=.828 Z2=0

*TANG PT=67 DY=2.084 EW=1 *TA=1

TANG PT=67 DY=.084

CROS CD=2

VALV PT=70 DY=1.333 PL=1 MA=3.25 *VALVE V2-29A



VALV PT=75 DY=1.333 PL=2 EW=1 *TA=1
 CROS CD=3
 TANG PT=78 DY=1.25
 TANG PT=80 DY=3.5
 TANG PT=82 DY=2.667 EW=1
 CROS CD=4
 BRAD PT=85 RA=2 EW=1
 CROS CD=3
 TANG PT=90 DX=2.875
 TANG PT=95 DX=2.875 EW=1
 CROS CD=4
 BRAD PT=100 RA=2 EW=1
 CROS CD=3
 TANG PT=105 DX=1.12 DZ=-1.12
 TANG PT=110 DX=3.477 DZ=-3.477 EW=1
 CROS CD=4
 TANG PT=115 DX=0.7071 DZ=-0.7071 EW=1

*

*END REGION III

*BEGIN REGION IV

*OPER cards same as those for regions I and III

TRAN CA=201 IS=1 FS=1 IT=70 FT=100 TT=1800 FL=100 IP=15 FP=1115 TP=1800
 TRAN CA=202 IS=1 FS=1 IT=100 FT=100 TT=0 FL=100 IP=1115 FP=65 TP=0
 TRAN CA=203 IS=1 FS=1 IT=100 FT=150 TT=16164 FL=100 IP=65 FP=1025 TP=16164
 TRAN CA=204 IS=1 FS=1 IT=150 FT=100 TT=0 FL=688.5 IP=1025 FP=1025 TP=0
 TRAN CA=205 IS=1 FS=1 IT=100 FT=260 TT=0 FL=688.5 IP=1025 FP=1025 TP=0
 TRAN CA=206 IS=1 FS=1 IT=260 FT=392 TT=1800 FL=4590 IP=1025 FP=1025 TP=1800
 TRAN CA=207 IS=1 FS=1 IT=392 FT=310 TT=900 FL=3442.5 IP=1025 FP=1025 TP=900
 TRAN CA=208 IS=1 FS=1 IT=310 FT=392 TT=900 FL=3442.5 IP=1025 FP=1025 TP=900
 TRAN CA=209 IS=1 FS=1 IT=392 FT=280 TT=1800 FL=2295 IP=1025 FP=1025 TP=1800
 TRAN CA=210 IS=1 FS=1 IT=280 FT=392 TT=1800 FL=2295 IP=1025 FP=1025 TP=1800
 TRAN CA=211 IS=1 FS=1 IT=392 FT=265 TT=1800 FL=2295 IP=1025 FP=1025 TP=1800
 TRAN CA=212 IS=1 FS=1 IT=265 FT=90 TT=360 FL=688.5 IP=1025 FP=1025 TP=360
 TRAN CA=213 IS=1 FS=1 IT=90 FT=265 TT=900 FL=688.5 IP=1025 FP=1025 TP=900
 TRAN CA=214 IS=1 FS=1 IT=265 FT=392 TT=1800 FL=2295 IP=1025 FP=1025 TP=1800
 TRAN CA=215 IS=1 FS=1 IT=392 FT=265 TT=90 FL=4590 IP=1025 FP=1025 TP=90
 TRAN CA=216 IS=1 FS=1 IT=265 FT=392 TT=180 FL=4590 IP=1025 FP=1025 TP=180
 TRAN CA=217 IS=1 FS=1 IT=392 FT=275 TT=60 FL=5049 IP=1025 FP=1025 TP=60
 TRAN CA=218 IS=1 FS=1 IT=275 FT=100 TT=900 FL=137.7 IP=1025 FP=1025 TP=900
 TRAN CA=219 *IS=1 FS=1 IT=265 FT=265 TT=0 FL=100 IP=1025 FP=1025 TP=0
 TRAN CA=220 *IS=1 FS=1 IT=265 FT=265 TT=0 FL=100 IP=1025 FP=1025 TP=0
 TRAN CA=221 IS=1 FS=1 IT=265 FT=150 TT=4140 FL=100 IP=1025 FP=1025 TP=4140
 TRAN CA=222 IS=1 FS=1 IT=150 FT=150 TT=0 FL=100 IP=1025 FP=185 TP=0
 TRAN CA=223 IS=1 FS=1 IT=150 FT=150 TT=0 FL=100 IP=185 FP=103 TP=0
 TRAN CA=224 IS=1 FS=1 IT=150 FT=100 TT=8280 FL=100 IP=103 FP=65 TP=8280
 TRAN CA=225 IS=1 FS=1 IT=392 FT=392 TT=12 FL=100 IP=1025 FP=1205 TP=12
 TRAN CA=226 IS=1 FS=1 IT=392 FT=50 TT=0 FL=1836 IP=1205 FP=1150 TP=0
 TRAN CA=227 IS=1 FS=1 IT=50 FT=150 TT=1380 FL=100 IP=1150 FP=1150 TP=1380
 TRAN CA=228 *IS=1 FS=1 IT=150 FT=150 TT=0 FL=100 IP=1150 FP=1150 TP=0
 TRAN CA=229 IS=1 FS=1 IT=150 FT=50 TT=0 FL=1377 IP=1150 FP=900 TP=0
 TRAN CA=230 IS=1 FS=1 IT=50 FT=150 TT=3060 FL=100 IP=900 FP=1075 TP=3060
 TRAN CA=231 IS=1 FS=1 IT=150 FT=150 TT=0 FL=100 IP=1075 FP=1150 TP=0



TRAN CA=232 IS=1 FS=1 IT=150 FT=50 TT=0 FL=780.3 IP=1150 FP=690 TP=0
 TRAN CA=233 IS=1 FS=1 IT=50 FT=150 TT=300 FL=100 IP=690 FP=690 TP=300
 TRAN CA=234 IS=1 FS=1 IT=150 FT=150 TT=8964 FL=100 IP=255 FP=1025 TP=8964
 TRAN CA=235 IS=1 FS=1 IT=392 FT=275 TT=60 FL=5049 IP=1025 FP=900 TP=60
 TRAN CA=236 IS=1 FS=1 IT=275 FT=100 TT=900 FL=137.7 IP=900 FP=65 TP=900
 TRAN CA=237 IS=1 FS=1 IT=100 FT=100 TT=0 FL=100 IP=65 FP=1578 TP=0
 TRAN CA=238 IS=1 FS=1 IT=100 FT=100 TT=0 FL=100 IP=1578 FP=65 TP=0
 TRAN CA=239 IS=1 FS=1 IT=392 FT=392 TT=60 FL=5049 IP=1025 FP=1390 TP=60
 TRAN CA=240 IS=1 FS=1 IT=392 FT=392 TT=900 FL=137.7 IP=1390 FP=955 TP=900
 TRAN CA=241 IS=1 FS=1 IT=392 FT=392 TT=900 FL=137.7 IP=955 FP=1025 TP=900
 TRAN CA=242 IS=1 FS=1 IT=100 FT=125 TT=60 FL=100 IP=1025 FP=1025 TP=60
 TRAN CA=243 IS=1 FS=1 IT=125 FT=150 TT=210 FL=100 IP=1025 FP=1025 TP=210

*
 PAIR CA=201 CO=27.6 DI=0.521 EX=6.4 * Tavg=85
 PAIR CA=202 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
 PAIR CA=203 CO=27.6 DI=0.504 EX=6.4 * Tavg=125
 PAIR CA=204 CO=27.6 DI=0.504 EX=6.4 * Tavg=125
 PAIR CA=205 CO=27.6 DI=0.490 EX=6.4 * Tavg=180
 PAIR CA=206 CO=27.1 DI=0.446 EX=6.4 * Tavg=326
 PAIR CA=207 CO=27.0 DI=0.440 EX=6.4 * Tavg=351
 PAIR CA=208 CO=27.0 DI=0.440 EX=6.4 * Tavg=351
 PAIR CA=209 CO=27.1 DI=0.444 EX=6.4 * Tavg=336
 PAIR CA=210 CO=27.1 DI=0.444 EX=6.4 * Tavg=336
 PAIR CA=211 CO=27.1 DI=0.445 EX=6.4 * Tavg=329
 PAIR CA=212 CO=27.6 DI=0.490 EX=6.4 * Tavg=178
 PAIR CA=213 CO=27.6 DI=0.490 EX=6.4 * Tavg=178
 PAIR CA=214 CO=27.1 DI=0.445 EX=6.4 * Tavg=329
 PAIR CA=215 CO=27.1 DI=0.445 EX=6.4 * Tavg=329
 PAIR CA=216 CO=27.1 DI=0.445 EX=6.4 * Tavg=329
 PAIR CA=217 CO=27.1 DI=0.444 EX=6.4 * Tavg=334
 PAIR CA=218 CO=27.6 DI=0.488 EX=6.4 * Tavg=188
 PAIR CA=219 CO=27.3 DI=0.463 EX=6.4 * Tavg=265
 PAIR CA=220 CO=27.3 DI=0.463 EX=6.4 * Tavg=265
 PAIR CA=221 CO=27.6 DI=0.483 EX=6.4 * Tavg=208
 PAIR CA=222 CO=27.6 DI=0.496 EX=6.4 * Tavg=150
 PAIR CA=223 CO=27.6 DI=0.496 EX=6.4 * Tavg=150
 PAIR CA=224 CO=27.6 DI=0.504 EX=6.4 * Tavg=125
 PAIR CA=225 CO=26.7 DI=0.430 EX=6.4 * Tavg=392
 PAIR CA=226 CO=27.5 DI=0.478 EX=6.4 * Tavg=221
 PAIR CA=227 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
 PAIR CA=228 CO=27.6 DI=0.496 EX=6.4 * Tavg=150
 PAIR CA=229 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
 PAIR CA=230 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
 PAIR CA=231 CO=27.6 DI=0.496 EX=6.4 * Tavg=150
 PAIR CA=232 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
 PAIR CA=233 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
 PAIR CA=234 CO=27.6 DI=0.496 EX=6.4 * Tavg=150
 PAIR CA=235 CO=27.1 DI=0.444 EX=6.4 * Tavg=334
 PAIR CA=236 CO=27.6 DI=0.488 EX=6.4 * Tavg=188
 PAIR CA=237 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
 PAIR CA=238 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
 PAIR CA=239 CO=26.7 DI=0.430 EX=6.4 * Tavg=392
 PAIR CA=240 CO=26.7 DI=0.430 EX=6.4 * Tavg=392
 PAIR CA=241 CO=26.7 DI=0.430 EX=6.4 * Tavg=392
 PAIR CA=242 CO=27.6 DI=0.508 EX=6.4 * Tavg=113

PAIR CA=243 CO=27.6 DI=0.500 EX=6.4 * Tavg=138

*
 *REGION IV GEOMETRY DOWNSTREAM OF FW brnCH TEE/REDUCER - 10 INCH PIPING
 *RUN FROM FW TEE TO ELBOW BEFORE NOZZLE NBA, NODE 275
 CROS CD=4
 TANG PT=170 DX=0.7071 DZ=-0.7071 EW=1
 ERED PT=175 DX=0.825 DZ=-0.825 AN=30
 CROS CD=5
 *RUN FROM FW TEE TO ELBOW BEFORE NOZZLE N4B, NODE 152
 BEND PT=190 X1=4.813 Y1=0 Z1=-4.813 X2=1.283 Y2=0 Z2=-6.685
 BEND PT=200 X1=0.449 Y1=0 Z1=-2.342 X2=-0.059 Y2=0 Z2=-2.384
 STRU PT=201 DX=.198 DZ=.9802
 STRU PT=202 DX=.198 DZ=.9802
 ANCH PT=202
 JUNC PT=200
 BEND PT=220 X1=-0.2196 Y1=0 Z1=-8.859 X2=-6.266 Y2=0 Z2=-6.266
 TANG PT=225 DX=-0.3388 DZ=-0.3388
 TANG PT=230 DX=-0.3388 DZ=-0.3388
 TANG PT=235 DX=-1.002 DZ=-1.002
 BRAD PT=240 RA=1.25
 TANG PT=245 DX=-2.693 DY=3.196 DZ=2.693
 TANG PT=250 DX=-2.693 DY=3.196 DZ=2.693
 TANG PT=255 DX=-2.693 DY=3.196 DZ=2.693
 BRAD PT=260 RA=1.25
 TANG PT=265 DY=3.958
 TANG PT=270 DY=3.959

*-----
 *END REGION IV
 *-----

*BEGIN REGION IVa
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OPER CA=3 TE=283 PR=1010
 OPER CA=4 TE=260 PR=1010
 OPER CA=5 TE=392 PR=1010
 OPER CA=6 TE=310 PR=1010
 OPER CA=7 TE=280 PR=1010
 OPER CA=8 TE=265 PR=1010
 OPER CA=9 TE=90 PR=1010
 OPER CA=10 TE=360 PR=1010
 OPER CA=11 TE=225 PR=170
 OPER CA=12 TE=210 PR=88
 OPER CA=13 TE=450 PR=1190
 OPER CA=14 TE=50 PR=1135
 OPER CA=15 TE=247 PR=1135
 OPER CA=16 TE=288 PR=1135
 OPER CA=17 TE=247 PR=1060
 OPER CA=18 TE=283 PR=1135

 OPER CA=20 TE=200 PR=675
 OPER CA=21 TE=275 PR=885

 OPER CA=23 TE=392 PR=1375
 OPER CA=24 TE=392 PR=940
 OPER CA=25 TE=392 PR=1010
 OPER CA=26 TE=275 PR=1010



OPER CA=27 TE=323 PR=1010
OPER CA=30 TE=180 PR=1010

*

TRAN CA=201 IS=1 FS=1 IT=70 FT=100 TT=1800 FL=100 IP=15 FP=1115 TP=1800
TRAN CA=202 IS=1 FS=1 IT=100 FT=100 TT=0 FL=100 IP=1115 FP=65 TP=0
TRAN CA=203 IS=1 FS=1 IT=100 FT=283 TT=16164 FL=100 IP=65 FP=1025 TP=16164
TRAN CA=204 IS=1 FS=1 IT=283 FT=100 TT=0 FL=688.5 IP=1025 FP=1025 TP=0
TRAN CA=205 IS=1 FS=1 IT=100 FT=260 TT=0 FL=688.5 IP=1025 FP=1025 TP=0
TRAN CA=206 IS=1 FS=1 IT=260 FT=392 TT=1800 FL=4590 IP=1025 FP=1025 TP=1800
TRAN CA=207 IS=1 FS=1 IT=392 FT=310 TT=900 FL=3442.5 IP=1025 FP=1025 TP=900
TRAN CA=208 IS=1 FS=1 IT=310 FT=392 TT=900 FL=3442.5 IP=1025 FP=1025 TP=900
TRAN CA=209 IS=1 FS=1 IT=392 FT=280 TT=1800 FL=2295 IP=1025 FP=1025 TP=1800
TRAN CA=210 IS=1 FS=1 IT=280 FT=392 TT=1800 FL=2295 IP=1025 FP=1025 TP=1800
TRAN CA=211 IS=1 FS=1 IT=392 FT=265 TT=1800 FL=2295 IP=1025 FP=1025 TP=1800
TRAN CA=212 IS=1 FS=1 IT=265 FT=90 TT=360 FL=688.5 IP=1025 FP=1025 TP=360
TRAN CA=213 IS=1 FS=1 IT=90 FT=265 TT=900 FL=688.5 IP=1025 FP=1025 TP=900
TRAN CA=214 IS=1 FS=1 IT=265 FT=392 TT=1800 FL=2295 IP=1025 FP=1025 TP=1800
TRAN CA=215 IS=1 FS=1 IT=392 FT=265 TT=90 FL=4590 IP=1025 FP=1025 TP=90
TRAN CA=216 IS=1 FS=1 IT=265 FT=392 TT=180 FL=4590 IP=1025 FP=1025 TP=180
TRAN CA=217 IS=1 FS=1 IT=392 FT=275 TT=60 FL=5049 IP=1025 FP=1025 TP=60
TRAN CA=218 IS=1 FS=1 IT=275 FT=100 TT=900 FL=137.7 IP=1025 FP=1025 TP=900
TRAN CA=219 IS=1 FS=1 IT=265 FT=323 TT=0 FL=100 IP=1025 FP=1025 TP=0
TRAN CA=220 IS=1 FS=1 IT=323 FT=360 TT=3924 FL=100 IP=1025 FP=1025 TP=3924
TRAN CA=221 IS=1 FS=1 IT=360 FT=283 TT=4140 FL=100 IP=1025 FP=1025 TP=4140
TRAN CA=222 IS=1 FS=1 IT=283 FT=225 TT=6264 FL=100 IP=1025 FP=185 TP=6264
TRAN CA=223 IS=1 FS=1 IT=225 FT=210 TT=600 FL=100 IP=185 FP=103 TP=600
TRAN CA=224 IS=1 FS=1 IT=210 FT=100 TT=8280 FL=100 IP=103 FP=65 TP=8280
TRAN CA=225 IS=1 FS=1 IT=392 FT=450 TT=12 FL=100 IP=1025 FP=1205 TP=12
TRAN CA=226 IS=1 FS=1 IT=450 FT=50 TT=0 FL=1836 IP=1205 FP=1150 TP=0
TRAN CA=227 IS=1 FS=1 IT=50 FT=247 TT=1380 FL=100 IP=1150 FP=1150 TP=1380
TRAN CA=228 IS=1 FS=1 IT=247 FT=288 TT=0 FL=100 IP=1150 FP=1150 TP=0
TRAN CA=229 IS=1 FS=1 IT=288 FT=50 TT=0 FL=1377 IP=1150 FP=900 TP=0
TRAN CA=230 IS=1 FS=1 IT=50 FT=247 TT=3060 FL=100 IP=900 FP=1075 TP=3060
TRAN CA=231 IS=1 FS=1 IT=247 FT=283 TT=0 FL=100 IP=1075 FP=1150 TP=0
TRAN CA=232 IS=1 FS=1 IT=283 FT=50 TT=0 FL=780.3 IP=1150 FP=690 TP=0
TRAN CA=233 IS=1 FS=1 IT=50 FT=200 TT=300 FL=100 IP=690 FP=690 TP=300
TRAN CA=234 IS=1 FS=1 IT=200 FT=283 TT=8964 FL=100 IP=255 FP=1025 TP=8964
TRAN CA=235 IS=1 FS=1 IT=392 FT=275 TT=60 FL=5049 IP=1025 FP=900 TP=60
TRAN CA=236 IS=1 FS=1 IT=275 FT=100 TT=900 FL=137.7 IP=900 FP=65 TP=900
TRAN CA=237 IS=1 FS=1 IT=100 FT=100 TT=0 FL=100 IP=65 FP=1578 TP=0
TRAN CA=238 IS=1 FS=1 IT=100 FT=100 TT=0 FL=100 IP=1578 FP=65 TP=0
TRAN CA=239 IS=1 FS=1 IT=392 FT=392 TT=60 FL=5049 IP=1025 FP=1390 TP=60
TRAN CA=240 IS=1 FS=1 IT=392 FT=392 TT=900 FL=137.7 IP=1390 FP=955 TP=900
TRAN CA=241 IS=1 FS=1 IT=392 FT=392 TT=900 FL=137.7 IP=955 FP=1025 TP=900
TRAN CA=242 IS=1 FS=1 IT=100 FT=180 TT=60 FL=100 IP=1025 FP=1025 TP=60
TRAN CA=243 IS=1 FS=1 IT=180 FT=283 TT=210 FL=100 IP=1025 FP=1025 TP=210

PAIR CA=201 CO=27.6 DI=0.521 EX=6.4 * Tavg=85
PAIR CA=202 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
PAIR CA=203 CO=27.6 DI=0.488 EX=6.4 * Tavg=192
PAIR CA=204 CO=27.6 DI=0.488 EX=6.4 * Tavg=192
PAIR CA=205 CO=27.6 DI=0.490 EX=6.4 * Tavg=180
PAIR CA=206 CO=27.1 DI=0.446 EX=6.4 * Tavg=326
PAIR CA=207 CO=27.0 DI=0.440 EX=6.4 * Tavg=351
PAIR CA=208 CO=27.0 DI=0.440 EX=6.4 * Tavg=351



PAIR CA=209 CO=27.1 DI=0.444 EX=6.4 * Tavg=336
PAIR CA=210 CO=27.1 DI=0.444 EX=6.4 * Tavg=336
PAIR CA=211 CO=27.1 DI=0.445 EX=6.4 * Tavg=329
PAIR CA=212 CO=27.6 DI=0.490 EX=6.4 * Tavg=178
PAIR CA=213 CO=27.6 DI=0.490 EX=6.4 * Tavg=178
PAIR CA=214 CO=27.1 DI=0.445 EX=6.4 * Tavg=329
PAIR CA=215 CO=27.1 DI=0.445 EX=6.4 * Tavg=329
PAIR CA=216 CO=27.1 DI=0.445 EX=6.4 * Tavg=329
PAIR CA=217 CO=27.1 DI=0.444 EX=6.4 * Tavg=334
PAIR CA=218 CO=27.6 DI=0.488 EX=6.4 * Tavg=188
PAIR CA=219 CO=27.2 DI=0.455 EX=6.4 * Tavg=294
PAIR CA=220 CO=27.0 DI=0.442 EX=6.4 * Tavg=342
PAIR CA=221 CO=27.1 DI=0.447 EX=6.4 * Tavg=322
PAIR CA=222 CO=27.4 DI=0.466 EX=6.4 * Tavg=254
PAIR CA=223 CO=27.5 DI=0.479 EX=6.4 * Tavg=218
PAIR CA=224 CO=27.6 DI=0.495 EX=6.4 * Tavg=155
PAIR CA=225 CO=26.5 DI=0.422 EX=6.4 * Tavg=421
PAIR CA=226 CO=27.4 DI=0.467 EX=6.4 * Tavg=250
PAIR CA=227 CO=27.6 DI=0.496 EX=6.4 * Tavg=149
PAIR CA=228 CO=27.3 DI=0.462 EX=6.4 * Tavg=268
PAIR CA=229 CO=27.6 DI=0.492 EX=6.4 * Tavg=169
PAIR CA=230 CO=27.6 DI=0.496 EX=6.4 * Tavg=149
PAIR CA=231 CO=27.3 DI=0.463 EX=6.4 * Tavg=265
PAIR CA=232 CO=27.6 DI=0.493 EX=6.4 * Tavg=167
PAIR CA=233 CO=27.6 DI=0.504 EX=6.4 * Tavg=125
PAIR CA=234 CO=27.4 DI=0.470 EX=6.4 * Tavg=242
PAIR CA=235 CO=27.1 DI=0.444 EX=6.4 * Tavg=334
PAIR CA=236 CO=27.6 DI=0.488 EX=6.4 * Tavg=188
PAIR CA=237 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
PAIR CA=238 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
PAIR CA=239 CO=26.7 DI=0.430 EX=6.4 * Tavg=392
PAIR CA=240 CO=26.7 DI=0.430 EX=6.4 * Tavg=392
PAIR CA=241 CO=26.7 DI=0.430 EX=6.4 * Tavg=392
PAIR CA=242 CO=27.6 DI=0.499 EX=6.4 * Tavg=140
PAIR CA=243 CO=27.5 DI=0.474 EX=6.4 * Tavg=232

TANG PT=275 DY=6.583 EW=0

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*END REGION IVa
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*BEGIN REGION IVb
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OPER CA=3 TE=416 PR=1010
OPER CA=4 TE=260 PR=1010
OPER CA=5 TE=392 PR=1010
OPER CA=6 TE=310 PR=1010
OPER CA=7 TE=280 PR=1010
OPER CA=8 TE=265 PR=1010
OPER CA=9 TE=90 PR=1010
OPER CA=10 TE=454 PR=1010
OPER CA=11 TE=300 PR=170
OPER CA=12 TE=270 PR=88
OPER CA=13 TE=507 PR=1190
OPER CA=14 TE=50 PR=1135



OPER CA=15 TE=343 PR=1135
OPER CA=16 TE=427 PR=1135
OPER CA=17 TE=343 PR=1060
OPER CA=18 TE=416 PR=1135

OPER CA=20 TE=250 PR=675
OPER CA=21 TE=275 PR=885

OPER CA=23 TE=392 PR=1375
OPER CA=24 TE=392 PR=940
OPER CA=25 TE=392 PR=1010
OPER CA=26 TE=275 PR=1010
OPER CA=27 TE=382 PR=1010
OPER CA=30 TE=235 PR=1010

TRAN CA=201 IS=1 FS=1 IT=70 FT=100 TT=1800 FL=100 IP=15 FP=1115 TP=1800
TRAN CA=202 IS=1 FS=1 IT=100 FT=100 TT=0 FL=100 IP=1115 FP=65 TP=0
TRAN CA=203 IS=1 FS=1 IT=100 FT=416 TT=16164 FL=100 IP=65 FP=1025 TP=16164
TRAN CA=204 IS=1 FS=1 IT=416 FT=100 TT=0 FL=688.5 IP=1025 FP=1025 TP=0
TRAN CA=205 IS=1 FS=1 IT=100 FT=260 TT=0 FL=688.5 IP=1025 FP=1025 TP=0
TRAN CA=206 IS=1 FS=1 IT=260 FT=392 TT=1800 FL=4590 IP=1025 FP=1025 TP=1800
TRAN CA=207 IS=1 FS=1 IT=392 FT=310 TT=900 FL=3442.5 IP=1025 FP=1025 TP=900
TRAN CA=208 IS=1 FS=1 IT=310 FT=392 TT=900 FL=3442.5 IP=1025 FP=1025 TP=900
TRAN CA=209 IS=1 FS=1 IT=392 FT=280 TT=1800 FL=2295 IP=1025 FP=1025 TP=1800
TRAN CA=210 IS=1 FS=1 IT=280 FT=392 TT=1800 FL=2295 IP=1025 FP=1025 TP=1800
TRAN CA=211 IS=1 FS=1 IT=392 FT=265 TT=1800 FL=2295 IP=1025 FP=1025 TP=1800
TRAN CA=212 IS=1 FS=1 IT=265 FT=90 TT=360 FL=688.5 IP=1025 FP=1025 TP=360
TRAN CA=213 IS=1 FS=1 IT=90 FT=265 TT=900 FL=688.5 IP=1025 FP=1025 TP=900
TRAN CA=214 IS=1 FS=1 IT=265 FT=392 TT=1800 FL=2295 IP=1025 FP=1025 TP=1800
TRAN CA=215 IS=1 FS=1 IT=392 FT=265 TT=90 FL=4590 IP=1025 FP=1025 TP=90
TRAN CA=216 IS=1 FS=1 IT=265 FT=392 TT=180 FL=4590 IP=1025 FP=1025 TP=180
TRAN CA=217 IS=1 FS=1 IT=392 FT=275 TT=60 FL=5049 IP=1025 FP=1025 TP=60
TRAN CA=218 IS=1 FS=1 IT=275 FT=100 TT=900 FL=137.7 IP=1025 FP=1025 TP=900
TRAN CA=219 IS=1 FS=1 IT=265 FT=382 TT=0 FL=100 IP=1025 FP=1025 TP=0
TRAN CA=220 IS=1 FS=1 IT=382 FT=454 TT=3924 FL=100 IP=1025 FP=1025 TP=3924
TRAN CA=221 IS=1 FS=1 IT=454 FT=416 TT=4140 FL=100 IP=1025 FP=1025 TP=4140
TRAN CA=222 IS=1 FS=1 IT=416 FT=300 TT=6264 FL=100 IP=1025 FP=185 TP=6264
TRAN CA=223 IS=1 FS=1 IT=300 FT=270 TT=600 FL=100 IP=185 FP=103 TP=600
TRAN CA=224 IS=1 FS=1 IT=270 FT=100 TT=8280 FL=100 IP=103 FP=65 TP=8280
TRAN CA=225 IS=1 FS=1 IT=392 FT=507 TT=12 FL=100 IP=1025 FP=1205 TP=12
TRAN CA=226 IS=1 FS=1 IT=507 FT=50 TT=0 FL=1836 IP=1205 FP=1150 TP=0
TRAN CA=227 IS=1 FS=1 IT=50 FT=343 TT=1380 FL=100 IP=1150 FP=1150 TP=1380
TRAN CA=228 IS=1 FS=1 IT=343 FT=427 TT=0 FL=100 IP=1150 FP=1150 TP=0
TRAN CA=229 IS=1 FS=1 IT=427 FT=50 TT=0 FL=1377 IP=1150 FP=900 TP=0
TRAN CA=230 IS=1 FS=1 IT=50 FT=343 TT=3060 FL=100 IP=900 FP=1075 TP=3060
TRAN CA=231 IS=1 FS=1 IT=343 FT=416 TT=0 FL=100 IP=1075 FP=1150 TP=0
TRAN CA=232 IS=1 FS=1 IT=416 FT=50 TT=0 FL=780.3 IP=1150 FP=690 TP=0
TRAN CA=233 IS=1 FS=1 IT=50 FT=250 TT=300 FL=100 IP=690 FP=690 TP=300
TRAN CA=234 IS=1 FS=1 IT=250 FT=416 TT=8964 FL=100 IP=255 FP=1025 TP=8964
TRAN CA=235 IS=1 FS=1 IT=392 FT=275 TT=60 FL=5049 IP=1025 FP=900 TP=60
TRAN CA=236 IS=1 FS=1 IT=275 FT=100 TT=900 FL=137.7 IP=900 FP=65 TP=900
TRAN CA=237 IS=1 FS=1 IT=100 FT=100 TT=0 FL=100 IP=65 FP=1578 TP=0
TRAN CA=238 IS=1 FS=1 IT=100 FT=100 TT=0 FL=100 IP=1578 FP=65 TP=0
TRAN CA=239 IS=1 FS=1 IT=392 FT=392 TT=60 FL=5049 IP=1025 FP=1390 TP=60
TRAN CA=240 IS=1 FS=1 IT=392 FT=392 TT=900 FL=137.7 IP=1390 FP=955 TP=900
TRAN CA=241 IS=1 FS=1 IT=392 FT=392 TT=900 FL=137.7 IP=955 FP=1025 TP=900
TRAN CA=242 IS=1 FS=1 IT=100 FT=235 TT=60 FL=100 IP=1025 FP=1025 TP=60

TRAN CA=243 IS=1 FS=1 IT=235 FT=416 TT=210 FL=100 IP=1025 FP=1025 TP=210

PAIR CA=201 CO=27.6 DI=0.521 EX=6.4 * Tavg=85
PAIR CA=202 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
PAIR CA=203 CO=27.4 DI=0.465 EX=6.4 * Tavg=258
PAIR CA=204 CO=27.4 DI=0.465 EX=6.4 * Tavg=258
PAIR CA=205 CO=27.6 DI=0.490 EX=6.4 * Tavg=180
PAIR CA=206 CO=27.1 DI=0.446 EX=6.4 * Tavg=326
PAIR CA=207 CO=27.0 DI=0.440 EX=6.4 * Tavg=351
PAIR CA=208 CO=27.0 DI=0.440 EX=6.4 * Tavg=351
PAIR CA=209 CO=27.1 DI=0.444 EX=6.4 * Tavg=336
PAIR CA=210 CO=27.1 DI=0.444 EX=6.4 * Tavg=336
PAIR CA=211 CO=27.1 DI=0.445 EX=6.4 * Tavg=329
PAIR CA=212 CO=27.6 DI=0.490 EX=6.4 * Tavg=178
PAIR CA=213 CO=27.6 DI=0.490 EX=6.4 * Tavg=178
PAIR CA=214 CO=27.1 DI=0.445 EX=6.4 * Tavg=329
PAIR CA=215 CO=27.1 DI=0.445 EX=6.4 * Tavg=329
PAIR CA=216 CO=27.1 DI=0.445 EX=6.4 * Tavg=329
PAIR CA=217 CO=27.1 DI=0.444 EX=6.4 * Tavg=334
PAIR CA=218 CO=27.6 DI=0.488 EX=6.4 * Tavg=188
PAIR CA=219 CO=27.1 DI=0.447 EX=6.4 * Tavg=324
PAIR CA=220 CO=26.6 DI=0.423 EX=6.4 * Tavg=418
PAIR CA=221 CO=26.4 DI=0.418 EX=6.4 * Tavg=435
PAIR CA=222 CO=27.0 DI=0.438 EX=6.4 * Tavg=358
PAIR CA=223 CO=27.3 DI=0.457 EX=6.4 * Tavg=285
PAIR CA=224 CO=27.6 DI=0.489 EX=6.4 * Tavg=185
PAIR CA=225 CO=26.3 DI=0.413 EX=6.4 * Tavg=450
PAIR CA=226 CO=27.3 DI=0.459 EX=6.4 * Tavg=279
PAIR CA=227 CO=27.6 DI=0.487 EX=6.4 * Tavg=197
PAIR CA=228 CO=26.8 DI=0.432 EX=6.4 * Tavg=385
PAIR CA=229 CO=27.4 DI=0.471 EX=6.4 * Tavg=239
PAIR CA=230 CO=27.6 DI=0.487 EX=6.4 * Tavg=197
PAIR CA=231 CO=26.8 DI=0.433 EX=6.4 * Tavg=380
PAIR CA=232 CO=27.5 DI=0.473 EX=6.4 * Tavg=233
PAIR CA=233 CO=27.6 DI=0.496 EX=6.4 * Tavg=150
PAIR CA=234 CO=27.1 DI=0.444 EX=6.4 * Tavg=333
PAIR CA=235 CO=27.1 DI=0.444 EX=6.4 * Tavg=334
PAIR CA=236 CO=27.6 DI=0.488 EX=6.4 * Tavg=188
PAIR CA=237 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
PAIR CA=238 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
PAIR CA=239 CO=26.7 DI=0.430 EX=6.4 * Tavg=392
PAIR CA=240 CO=26.7 DI=0.430 EX=6.4 * Tavg=392
PAIR CA=241 CO=26.7 DI=0.430 EX=6.4 * Tavg=392
PAIR CA=242 CO=27.6 DI=0.492 EX=6.4 * Tavg=168
PAIR CA=243 CO=27.1 DI=0.446 EX=6.4 * Tavg=326

*
TANG PT=280 DY=6.583

*-----
*END REGION IVb
*-----

*BEGIN REGION V TO NOZZLE N4A, NODE 290
*-----

OPER CA=3 TE=549 PR=1010
OPER CA=4 TE=260 PR=1010

OPER CA=5 TE=392 PR=1010
OPER CA=6 TE=310 PR=1010
OPER CA=7 TE=280 PR=1010
OPER CA=8 TE=265 PR=1010
OPER CA=9 TE=90 PR=1010
OPER CA=10 TE=549 PR=1010
OPER CA=11 TE=375 PR=170
OPER CA=12 TE=330 PR=88
OPER CA=13 TE=565 PR=1190
OPER CA=14 TE=50 PR=1135
OPER CA=15 TE=440 PR=1135
OPER CA=16 TE=565 PR=1135
OPER CA=17 TE=440 PR=1060
OPER CA=18 TE=549 PR=1135

OPER CA=20 TE=300 PR=675
OPER CA=21 TE=275 PR=885

OPER CA=23 TE=392 PR=1375
OPER CA=24 TE=392 PR=940
OPER CA=25 TE=392 PR=1010
OPER CA=26 TE=275 PR=1010
OPER CA=27 TE=440 PR=1010
OPER CA=30 TE=290 PR=1010
TRAN CA=201 IS=1 FS=1 IT=70 FT=100 TT=1800 FL=100 IP=15 FP=1115 TP=1800
TRAN CA=202 IS=1 FS=1 IT=100 FT=100 TT=0 FL=100 IP=1115 FP=65 TP=0
TRAN CA=203 IS=1 FS=1 IT=100 FT=549 TT=16164 FL=100 IP=65 FP=1025 TP=16164
TRAN CA=204 IS=1 FS=1 IT=549 FT=100 TT=0 FL=688.5 IP=1025 FP=1025 TP=0
TRAN CA=205 IS=1 FS=1 IT=100 FT=260 TT=0 FL=688.5 IP=1025 FP=1025 TP=0
TRAN CA=206 IS=1 FS=1 IT=260 FT=392 TT=1800 FL=4590 IP=1025 FP=1025 TP=1800
TRAN CA=207 IS=1 FS=1 IT=392 FT=310 TT=900 FL=3442.5 IP=1025 FP=1025 TP=900
TRAN CA=208 IS=1 FS=1 IT=310 FT=392 TT=900 FL=3442.5 IP=1025 FP=1025 TP=900
TRAN CA=209 IS=1 FS=1 IT=392 FT=280 TT=1800 FL=2295 IP=1025 FP=1025 TP=1800
TRAN CA=210 IS=1 FS=1 IT=280 FT=392 TT=1800 FL=2295 IP=1025 FP=1025 TP=1800
TRAN CA=211 IS=1 FS=1 IT=392 FT=265 TT=1800 FL=2295 IP=1025 FP=1025 TP=1800
TRAN CA=212 IS=1 FS=1 IT=265 FT=90 TT=360 FL=688.5 IP=1025 FP=1025 TP=360
TRAN CA=213 IS=1 FS=1 IT=90 FT=265 TT=900 FL=688.5 IP=1025 FP=1025 TP=900
TRAN CA=214 IS=1 FS=1 IT=265 FT=392 TT=1800 FL=2295 IP=1025 FP=1025 TP=1800
TRAN CA=215 IS=1 FS=1 IT=392 FT=265 TT=90 FL=4590 IP=1025 FP=1025 TP=90
TRAN CA=216 IS=1 FS=1 IT=265 FT=392 TT=180 FL=4590 IP=1025 FP=1025 TP=180
TRAN CA=217 IS=1 FS=1 IT=392 FT=275 TT=60 FL=5049 IP=1025 FP=1025 TP=60
TRAN CA=218 IS=1 FS=1 IT=275 FT=100 TT=900 FL=137.7 IP=1025 FP=1025 TP=900
TRAN CA=219 IS=1 FS=1 IT=265 FT=440 TT=0 FL=100 IP=1025 FP=1025 TP=0
TRAN CA=220 IS=1 FS=1 IT=440 FT=549 TT=3924 FL=100 IP=1025 FP=1025 TP=3924
TRAN CA=221 *IS=1 FS=1 IT=549 FT=549 TT=0 FL=100 IP=1025 FP=1025 TP=0
TRAN CA=222 IS=1 FS=1 IT=549 FT=375 TT=6264 FL=100 IP=1025 FP=185 TP=6264
TRAN CA=223 IS=1 FS=1 IT=375 FT=330 TT=600 FL=100 IP=185 FP=103 TP=600
TRAN CA=224 IS=1 FS=1 IT=330 FT=100 TT=8280 FL=100 IP=103 FP=65 TP=8280
TRAN CA=225 IS=1 FS=1 IT=392 FT=565 TT=12 FL=100 IP=1025 FP=1205 TP=12
TRAN CA=226 IS=1 FS=1 IT=565 FT=50 TT=0 FL=1836 IP=1205 FP=1150 TP=0
TRAN CA=227 IS=1 FS=1 IT=50 FT=440 TT=1380 FL=100 IP=1150 FP=1150 TP=1380
TRAN CA=228 IS=1 FS=1 IT=440 FT=565 TT=0 FL=100 IP=1150 FP=1150 TP=0
TRAN CA=229 IS=1 FS=1 IT=565 FT=50 TT=0 FL=1377 IP=1150 FP=900 TP=0
TRAN CA=230 IS=1 FS=1 IT=50 FT=440 TT=3060 FL=100 IP=900 FP=1075 TP=3060
TRAN CA=231 IS=1 FS=1 IT=440 FT=549 TT=0 FL=100 IP=1075 FP=1150 TP=0
TRAN CA=232 IS=1 FS=1 IT=549 FT=50 TT=0 FL=780.3 IP=1150 FP=690 TP=0



TRAN CA=233 IS=1 FS=1 IT=50 FT=300 TT=300 FL=100 IP=690 FP=690 TP=300
TRAN CA=234 IS=1 FS=1 IT=300 FT=549 TT=8964 FL=100 IP=255 FP=1025 TP=8964
TRAN CA=235 IS=1 FS=1 IT=392 FT=275 TT=60 FL=5049 IP=1025 FP=900 TP=60
TRAN CA=236 IS=1 FS=1 IT=275 FT=100 TT=900 FL=137.7 IP=900 FP=65 TP=900
TRAN CA=237 IS=1 FS=1 IT=100 FT=100 TT=0 FL=100 IP=65 FP=1578 TP=0
TRAN CA=238 IS=1 FS=1 IT=100 FT=100 TT=0 FL=100 IP=1578 FP=65 TP=0
TRAN CA=239 IS=1 FS=1 IT=392 FT=392 TT=60 FL=5049 IP=1025 FP=1390 TP=60
TRAN CA=240 IS=1 FS=1 IT=392 FT=392 TT=900 FL=137.7 IP=1390 FP=955 TP=900
TRAN CA=241 IS=1 FS=1 IT=392 FT=392 TT=900 FL=137.7 IP=955 FP=1025 TP=900
TRAN CA=242 IS=1 FS=1 IT=100 FT=290 TT=60 FL=100 IP=1025 FP=1025 TP=60
TRAN CA=243 IS=1 FS=1 IT=290 FT=549 TT=210 FL=100 IP=1025 FP=1025 TP=210

PAIR CA=201 CO=27.6 DI=0.521 EX=6.4 * Tavg=85
PAIR CA=202 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
PAIR CA=203 CO=27.1 DI=0.447 EX=6.4 * Tavg=325
PAIR CA=204 CO=27.1 DI=0.447 EX=6.4 * Tavg=325
PAIR CA=205 CO=27.6 DI=0.490 EX=6.4 * Tavg=180
PAIR CA=206 CO=27.1 DI=0.446 EX=6.4 * Tavg=326
PAIR CA=207 CO=27.0 DI=0.440 EX=6.4 * Tavg=351
PAIR CA=208 CO=27.0 DI=0.440 EX=6.4 * Tavg=351
PAIR CA=209 CO=27.1 DI=0.444 EX=6.4 * Tavg=336
PAIR CA=210 CO=27.1 DI=0.444 EX=6.4 * Tavg=336
PAIR CA=211 CO=27.1 DI=0.445 EX=6.4 * Tavg=329
PAIR CA=212 CO=27.6 DI=0.490 EX=6.4 * Tavg=178
PAIR CA=213 CO=27.6 DI=0.490 EX=6.4 * Tavg=178
PAIR CA=214 CO=27.1 DI=0.445 EX=6.4 * Tavg=329
PAIR CA=215 CO=27.1 DI=0.445 EX=6.4 * Tavg=329
PAIR CA=216 CO=27.1 DI=0.445 EX=6.4 * Tavg=329
PAIR CA=217 CO=27.1 DI=0.444 EX=6.4 * Tavg=334
PAIR CA=218 CO=27.6 DI=0.488 EX=6.4 * Tavg=188
PAIR CA=219 CO=27.0 DI=0.439 EX=6.4 * Tavg=353
PAIR CA=220 CO=25.9 DI=0.400 EX=6.4 * Tavg=495
PAIR CA=221 CO=25.5 DI=0.387 EX=6.4 * Tavg=549
PAIR CA=222 CO=26.2 DI=0.409 EX=6.4 * Tavg=462
PAIR CA=223 CO=27.0 DI=0.439 EX=6.4 * Tavg=353
PAIR CA=224 CO=27.5 DI=0.480 EX=6.4 * Tavg=215
PAIR CA=225 CO=26.1 DI=0.404 EX=6.4 * Tavg=479
PAIR CA=226 CO=27.2 DI=0.451 EX=6.4 * Tavg=308
PAIR CA=227 CO=27.4 DI=0.469 EX=6.4 * Tavg=245
PAIR CA=228 CO=25.9 DI=0.397 EX=6.4 * Tavg=503
PAIR CA=229 CO=27.2 DI=0.451 EX=6.4 * Tavg=308
PAIR CA=230 CO=27.4 DI=0.469 EX=6.4 * Tavg=245
PAIR CA=231 CO=25.9 DI=0.400 EX=6.4 * Tavg=495
PAIR CA=232 CO=27.2 DI=0.453 EX=6.4 * Tavg=300
PAIR CA=233 CO=27.6 DI=0.491 EX=6.4 * Tavg=175
PAIR CA=234 CO=26.5 DI=0.421 EX=6.4 * Tavg=425
PAIR CA=235 CO=27.1 DI=0.444 EX=6.4 * Tavg=334
PAIR CA=236 CO=27.6 DI=0.488 EX=6.4 * Tavg=188
PAIR CA=237 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
PAIR CA=238 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
PAIR CA=239 CO=26.7 DI=0.430 EX=6.4 * Tavg=392
PAIR CA=240 CO=26.7 DI=0.430 EX=6.4 * Tavg=392
PAIR CA=241 CO=26.7 DI=0.430 EX=6.4 * Tavg=392
PAIR CA=242 CO=27.6 DI=0.487 EX=6.4 * Tavg=195
PAIR CA=243 CO=26.5 DI=0.422 EX=6.4 * Tavg=420



BRAD PT=285 RA=1.25
TANG PT=290 DX=-4.007 DZ=4.007 EW=1
NOZZ PT=290 *NOZZLE N4A
AMVT CA=1 PT=290 DX=0.0196 DY=0.1069 DZ=-0.0196
AMVT CA=2 PT=290 DX=0.0196 DY=0.1069 DZ=-0.0196
AMVT CA=3 PT=290 DX=0.3130 DY=1.7067 DZ=-0.3130
AMVT CA=4 PT=290 DX=0.3130 DY=1.7067 DZ=-0.3130
AMVT CA=5 PT=290 DX=0.3130 DY=1.7067 DZ=-0.3130
AMVT CA=6 PT=290 DX=0.3130 DY=1.7067 DZ=-0.3130
AMVT CA=7 PT=290 DX=0.3130 DY=1.7067 DZ=-0.3130
AMVT CA=8 PT=290 DX=0.3130 DY=1.7067 DZ=-0.3130
AMVT CA=9 PT=290 DX=0.3130 DY=1.7067 DZ=-0.3130
AMVT CA=10 PT=290 DX=0.3130 DY=1.7067 DZ=-0.3130
AMVT CA=11 PT=290 DX=0.1993 DY=1.0867 DZ=-0.1993
AMVT CA=12 PT=290 DX=0.1699 DY=0.9264 DZ=-0.1699
AMVT CA=13 PT=290 DX=0.3234 DY=1.7637 DZ=-0.3234
AMVT CA=14 PT=290 DX=0.3234 DY=1.7637 DZ=-0.3234
AMVT CA=15 PT=290 DX=0.3234 DY=1.7637 DZ=-0.3234
AMVT CA=16 PT=290 DX=0.3234 DY=1.7637 DZ=-0.3234
AMVT CA=17 PT=290 DX=0.3169 DY=1.7281 DZ=-0.3169
AMVT CA=18 PT=290 DX=0.3234 DY=1.7637 DZ=-0.3234
AMVT CA=19 PT=290 DX=0.2823 DY=1.5392 DZ=-0.2823
AMVT CA=20 PT=290 DX=0.2823 DY=1.5392 DZ=-0.2823
AMVT CA=21 PT=290 DX=0.3130 DY=1.7067 DZ=-0.3130
AMVT CA=22 PT=290 DX=0.0196 DY=0.1069 DZ=-0.0196
AMVT CA=23 PT=290 DX=0.3463 DY=1.8884 DZ=-0.3463
AMVT CA=24 PT=290 DX=0.3064 DY=1.6711 DZ=-0.3064
AMVT CA=25 PT=290 DX=0.3130 DY=1.7067 DZ=-0.3130
AMVT CA=26 PT=290 DX=0.3064 DY=1.6711 DZ=-0.3064
AMVT CA=27 PT=290 DX=0.3130 DY=1.7067 DZ=-0.3130
AMVT CA=28 PT=290 DX=0.3130 DY=1.7067 DZ=-0.3130
AMVT CA=29 PT=290 DX=0.3064 DY=1.6711 DZ=-0.3064
AMVT CA=30 PT=290 DX=0.3130 DY=1.7067 DZ=-0.3130
AMVT CA=31 PT=290 DX=0.3019 DY=1.6461 DZ=-0.3019

AMVT CA=32 PT=290 DX=-.09 DY=.015 DZ=-.093

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*END REGION V
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*BEGIN REGION IV
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OPER CA=3 TE=150 PR=1010
OPER CA=4 TE=260 PR=1010
OPER CA=5 TE=392 PR=1010
OPER CA=6 TE=310 PR=1010
OPER CA=7 TE=280 PR=1010
OPER CA=8 TE=265 PR=1010
OPER CA=9 TE=90 PR=1010
OPER CA=10 TE=265 PR=1010
OPER CA=11 TE=150 PR=170
OPER CA=12 TE=150 PR=88
OPER CA=13 TE=392 PR=1190
OPER CA=14 TE=50 PR=1135
OPER CA=15 TE=150 PR=1135
OPER CA=16 TE=150 PR=1135
OPER CA=17 TE=150 PR=1060



Structural Integrity Associates, Inc.

OPER CA=18 TE=150 PR=1135

OPER CA=20 TE=150 PR=675

OPER CA=21 TE=275 PR=885

OPER CA=23 TE=392 PR=1375

OPER CA=24 TE=392 PR=940

OPER CA=25 TE=392 PR=1010

OPER CA=26 TE=275 PR=1010

OPER CA=27 TE=265 PR=1010

OPER CA=30 TE=125 PR=1010

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TRAN CA=201 IS=1 FS=1 IT=70 FT=100 TT=1800 FL=100 IP=15 FP=1115 TP=1800
TRAN CA=202 IS=1 FS=1 IT=100 FT=100 TT=0 FL=100 IP=1115 FP=65 TP=0
TRAN CA=203 IS=1 FS=1 IT=100 FT=150 TT=16164 FL=100 IP=65 FP=1025 TP=16164
TRAN CA=204 IS=1 FS=1 IT=150 FT=100 TT=0 FL=688.5 IP=1025 FP=1025 TP=0
TRAN CA=205 IS=1 FS=1 IT=100 FT=260 TT=0 FL=688.5 IP=1025 FP=1025 TP=0
TRAN CA=206 IS=1 FS=1 IT=260 FT=392 TT=1800 FL=4590 IP=1025 FP=1025 TP=1800
TRAN CA=207 IS=1 FS=1 IT=392 FT=310 TT=900 FL=3442.5 IP=1025 FP=1025 TP=900
TRAN CA=208 IS=1 FS=1 IT=310 FT=392 TT=900 FL=3442.5 IP=1025 FP=1025 TP=900
TRAN CA=209 IS=1 FS=1 IT=392 FT=280 TT=1800 FL=2295 IP=1025 FP=1025 TP=1800
TRAN CA=210 IS=1 FS=1 IT=280 FT=392 TT=1800 FL=2295 IP=1025 FP=1025 TP=1800
TRAN CA=211 IS=1 FS=1 IT=392 FT=265 TT=1800 FL=2295 IP=1025 FP=1025 TP=1800
TRAN CA=212 IS=1 FS=1 IT=265 FT=90 TT=360 FL=688.5 IP=1025 FP=1025 TP=360
TRAN CA=213 IS=1 FS=1 IT=90 FT=265 TT=900 FL=688.5 IP=1025 FP=1025 TP=900
TRAN CA=214 IS=1 FS=1 IT=265 FT=392 TT=1800 FL=2295 IP=1025 FP=1025 TP=1800
TRAN CA=215 IS=1 FS=1 IT=392 FT=265 TT=90 FL=4590 IP=1025 FP=1025 TP=90
TRAN CA=216 IS=1 FS=1 IT=265 FT=392 TT=180 FL=4590 IP=1025 FP=1025 TP=180
TRAN CA=217 IS=1 FS=1 IT=392 FT=275 TT=60 FL=5049 IP=1025 FP=1025 TP=60
TRAN CA=218 IS=1 FS=1 IT=275 FT=100 TT=900 FL=137.7 IP=1025 FP=1025 TP=900
TRAN CA=219 *IS=1 FS=1 IT=265 FT=265 TT=0 FL=100 IP=1025 FP=1025 TP=0
TRAN CA=220 *IS=1 FS=1 IT=265 FT=265 TT=0 FL=100 IP=1025 FP=1025 TP=0
TRAN CA=221 IS=1 FS=1 IT=265 FT=150 TT=4140 FL=100 IP=1025 FP=1025 TP=4140
TRAN CA=222 IS=1 FS=1 IT=150 FT=150 TT=0 FL=100 IP=1025 FP=185 TP=0
TRAN CA=223 IS=1 FS=1 IT=150 FT=150 TT=0 FL=100 IP=185 FP=103 TP=0
TRAN CA=224 IS=1 FS=1 IT=150 FT=100 TT=8280 FL=100 IP=103 FP=65 TP=8280
TRAN CA=225 IS=1 FS=1 IT=392 FT=392 TT=12 FL=100 IP=1025 FP=1205 TP=12
TRAN CA=226 IS=1 FS=1 IT=392 FT=50 TT=0 FL=1836 IP=1205 FP=1150 TP=0
TRAN CA=227 IS=1 FS=1 IT=50 FT=150 TT=1380 FL=100 IP=1150 FP=1150 TP=1380
TRAN CA=228 IS=1 FS=1 IT=150 FT=150 TT=0 FL=100 IP=1150 FP=1150 TP=0
TRAN CA=229 IS=1 FS=1 IT=150 FT=50 TT=0 FL=1377 IP=1150 FP=900 TP=0
TRAN CA=230 IS=1 FS=1 IT=50 FT=150 TT=3060 FL=100 IP=900 FP=1075 TP=3060
TRAN CA=231 IS=1 FS=1 IT=150 FT=150 TT=0 FL=100 IP=1075 FP=1150 TP=0
TRAN CA=232 IS=1 FS=1 IT=150 FT=50 TT=0 FL=780.3 IP=1150 FP=690 TP=0
TRAN CA=233 IS=1 FS=1 IT=50 FT=150 TT=300 FL=100 IP=690 FP=690 TP=300
TRAN CA=234 IS=1 FS=1 IT=150 FT=150 TT=8964 FL=100 IP=255 FP=1025 TP=8964
TRAN CA=235 IS=1 FS=1 IT=392 FT=275 TT=60 FL=5049 IP=1025 FP=900 TP=60
TRAN CA=236 IS=1 FS=1 IT=275 FT=100 TT=900 FL=137.7 IP=900 FP=65 TP=900
TRAN CA=237 IS=1 FS=1 IT=100 FT=100 TT=0 FL=100 IP=65 FP=1578 TP=0
TRAN CA=238 IS=1 FS=1 IT=100 FT=100 TT=0 FL=100 IP=1578 FP=65 TP=0
TRAN CA=239 IS=1 FS=1 IT=392 FT=392 TT=60 FL=5049 IP=1025 FP=1390 TP=60
TRAN CA=240 IS=1 FS=1 IT=392 FT=392 TT=900 FL=137.7 IP=1390 FP=955 TP=900
TRAN CA=241 IS=1 FS=1 IT=392 FT=392 TT=900 FL=137.7 IP=955 FP=1025 TP=900
TRAN CA=242 IS=1 FS=1 IT=100 FT=125 TT=60 FL=100 IP=1025 FP=1025 TP=60
TRAN CA=243 IS=1 FS=1 IT=125 FT=150 TT=210 FL=100 IP=1025 FP=1025 TP=210



*
PAIR CA=201 CO=27.6 DI=0.521 EX=6.4 * Tavg=85
PAIR CA=202 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
PAIR CA=203 CO=27.6 DI=0.504 EX=6.4 * Tavg=125
PAIR CA=204 CO=27.6 DI=0.504 EX=6.4 * Tavg=125
PAIR CA=205 CO=27.6 DI=0.490 EX=6.4 * Tavg=180
PAIR CA=206 CO=27.1 DI=0.446 EX=6.4 * Tavg=326
PAIR CA=207 CO=27.0 DI=0.440 EX=6.4 * Tavg=351
PAIR CA=208 CO=27.0 DI=0.440 EX=6.4 * Tavg=351
PAIR CA=209 CO=27.1 DI=0.444 EX=6.4 * Tavg=336
PAIR CA=210 CO=27.1 DI=0.444 EX=6.4 * Tavg=336
PAIR CA=211 CO=27.1 DI=0.445 EX=6.4 * Tavg=329
PAIR CA=212 CO=27.6 DI=0.490 EX=6.4 * Tavg=178
PAIR CA=213 CO=27.6 DI=0.490 EX=6.4 * Tavg=178
PAIR CA=214 CO=27.1 DI=0.445 EX=6.4 * Tavg=329
PAIR CA=215 CO=27.1 DI=0.445 EX=6.4 * Tavg=329
PAIR CA=216 CO=27.1 DI=0.445 EX=6.4 * Tavg=329
PAIR CA=217 CO=27.1 DI=0.444 EX=6.4 * Tavg=334
PAIR CA=218 CO=27.6 DI=0.488 EX=6.4 * Tavg=188
PAIR CA=219 CO=27.3 DI=0.463 EX=6.4 * Tavg=265
PAIR CA=220 CO=27.3 DI=0.463 EX=6.4 * Tavg=265
PAIR CA=221 CO=27.6 DI=0.483 EX=6.4 * Tavg=208
PAIR CA=222 CO=27.6 DI=0.496 EX=6.4 * Tavg=150
PAIR CA=223 CO=27.6 DI=0.496 EX=6.4 * Tavg=150
PAIR CA=224 CO=27.6 DI=0.504 EX=6.4 * Tavg=125
PAIR CA=225 CO=26.7 DI=0.430 EX=6.4 * Tavg=392
PAIR CA=226 CO=27.5 DI=0.478 EX=6.4 * Tavg=221
PAIR CA=227 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
PAIR CA=228 CO=27.6 DI=0.496 EX=6.4 * Tavg=150
PAIR CA=229 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
PAIR CA=230 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
PAIR CA=231 CO=27.6 DI=0.496 EX=6.4 * Tavg=150
PAIR CA=232 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
PAIR CA=233 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
PAIR CA=234 CO=27.6 DI=0.496 EX=6.4 * Tavg=150
PAIR CA=235 CO=27.1 DI=0.444 EX=6.4 * Tavg=334
PAIR CA=236 CO=27.6 DI=0.488 EX=6.4 * Tavg=188
PAIR CA=237 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
PAIR CA=238 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
PAIR CA=239 CO=26.7 DI=0.430 EX=6.4 * Tavg=392
PAIR CA=240 CO=26.7 DI=0.430 EX=6.4 * Tavg=392
PAIR CA=241 CO=26.7 DI=0.430 EX=6.4 * Tavg=392
PAIR CA=242 CO=27.6 DI=0.508 EX=6.4 * Tavg=113
PAIR CA=243 CO=27.6 DI=0.500 EX=6.4 * Tavg=138

*
*REGION IV GEOMETRY DOWNSTREAM OF FW brnCH TEE/REDUCER - 10 INCH PIPING
*RUN FROM FW TEE TO ELBOW BEFORE NOZZLE N4A, NODE 155
JUNC PT=115
CROS CD=5
BRAN PT=120 DX=-0.5022 DY=0.596 DZ=-0.5022 TE=1 EW=1
TANG PT=125 DX=-2.594 DY=3.078 DZ=-2.594
TANG PT=130 DX=-2.594 DY=3.078 DZ=-2.594
TANG PT=135 DX=-2.594 DY=3.078 DZ=-2.594 EW=0



BRAD PT=140 RA=1.25 EW=0
TANG PT=142 DY=4
TANG PT=145 DY=4
TANG PT=150 DY=2.53

*-----
*END REGION IV
*-----

*BEGIN REGION IVa
*-----

OPER CA=3 TE=283 PR=1010
OPER CA=4 TE=260 PR=1010
OPER CA=5 TE=392 PR=1010
OPER CA=6 TE=310 PR=1010
OPER CA=7 TE=280 PR=1010
OPER CA=8 TE=265 PR=1010
OPER CA=9 TE=90 PR=1010
OPER CA=10 TE=360 PR=1010
OPER CA=11 TE=225 PR=170
OPER CA=12 TE=210 PR=88
OPER CA=13 TE=450 PR=1190
OPER CA=14 TE=50 PR=1135
OPER CA=15 TE=247 PR=1135
OPER CA=16 TE=288 PR=1135
OPER CA=17 TE=247 PR=1060
OPER CA=18 TE=283 PR=1135

OPER CA=20 TE=200 PR=675
OPER CA=21 TE=275 PR=885

OPER CA=23 TE=392 PR=1375
OPER CA=24 TE=392 PR=940
OPER CA=25 TE=392 PR=1010
OPER CA=26 TE=275 PR=1010
OPER CA=27 TE=323 PR=1010
OPER CA=30 TE=180 PR=1010

*
TRAN CA=201 IS=1 FS=1 IT=70 FT=100 TT=1800 FL=100 IP=15 FP=1115 TP=1800
TRAN CA=202 IS=1 FS=1 IT=100 FT=100 TT=0 FL=100 IP=1115 FP=65 TP=0
TRAN CA=203 IS=1 FS=1 IT=100 FT=283 TT=16164 FL=100 IP=65 FP=1025 TP=16164
TRAN CA=204 IS=1 FS=1 IT=283 FT=100 TT=0 FL=688.5 IP=1025 FP=1025 TP=0
TRAN CA=205 IS=1 FS=1 IT=100 FT=260 TT=0 FL=688.5 IP=1025 FP=1025 TP=0
TRAN CA=206 IS=1 FS=1 IT=260 FT=392 TT=1800 FL=4590 IP=1025 FP=1025 TP=1800
TRAN CA=207 IS=1 FS=1 IT=392 FT=310 TT=900 FL=3442.5 IP=1025 FP=1025 TP=900
TRAN CA=208 IS=1 FS=1 IT=310 FT=392 TT=900 FL=3442.5 IP=1025 FP=1025 TP=900
TRAN CA=209 IS=1 FS=1 IT=392 FT=280 TT=1800 FL=2295 IP=1025 FP=1025 TP=1800
TRAN CA=210 IS=1 FS=1 IT=280 FT=392 TT=1800 FL=2295 IP=1025 FP=1025 TP=1800
TRAN CA=211 IS=1 FS=1 IT=392 FT=265 TT=1800 FL=2295 IP=1025 FP=1025 TP=1800
TRAN CA=212 IS=1 FS=1 IT=265 FT=90 TT=360 FL=688.5 IP=1025 FP=1025 TP=360
TRAN CA=213 IS=1 FS=1 IT=90 FT=265 TT=900 FL=688.5 IP=1025 FP=1025 TP=900
TRAN CA=214 IS=1 FS=1 IT=265 FT=392 TT=1800 FL=2295 IP=1025 FP=1025 TP=1800
TRAN CA=215 IS=1 FS=1 IT=392 FT=265 TT=90 FL=4590 IP=1025 FP=1025 TP=90
TRAN CA=216 IS=1 FS=1 IT=265 FT=392 TT=180 FL=4590 IP=1025 FP=1025 TP=180
TRAN CA=217 IS=1 FS=1 IT=392 FT=275 TT=60 FL=5049 IP=1025 FP=1025 TP=60
TRAN CA=218 IS=1 FS=1 IT=275 FT=100 TT=900 FL=137.7 IP=1025 FP=1025 TP=900
TRAN CA=219 IS=1 FS=1 IT=265 FT=323 TT=0 FL=100 IP=1025 FP=1025 TP=0
TRAN CA=220 IS=1 FS=1 IT=323 FT=360 TT=3924 FL=100 IP=1025 FP=1025 TP=3924

TRAN CA=221 IS=1 FS=1 IT=360 FT=283 TT=4140 FL=100 IP=1025 FP=1025 TP=4140
 TRAN CA=222 IS=1 FS=1 IT=283 FT=225 TT=6264 FL=100 IP=1025 FP=185 TP=6264
 TRAN CA=223 IS=1 FS=1 IT=225 FT=210 TT=600 FL=100 IP=185 FP=103 TP=600
 TRAN CA=224 IS=1 FS=1 IT=210 FT=100 TT=8280 FL=100 IP=103 FP=65 TP=8280
 TRAN CA=225 IS=1 FS=1 IT=392 FT=450 TT=12 FL=100 IP=1025 FP=1205 TP=12
 TRAN CA=226 IS=1 FS=1 IT=450 FT=50 TT=0 FL=1836 IP=1205 FP=1150 TP=0
 TRAN CA=227 IS=1 FS=1 IT=50 FT=247 TT=1380 FL=100 IP=1150 FP=1150 TP=1380
 TRAN CA=228 IS=1 FS=1 IT=247 FT=288 TT=0 FL=100 IP=1150 FP=1150 TP=0
 TRAN CA=229 IS=1 FS=1 IT=288 FT=50 TT=0 FL=1377 IP=1150 FP=900 TP=0
 TRAN CA=230 IS=1 FS=1 IT=50 FT=247 TT=3060 FL=100 IP=900 FP=1075 TP=3060
 TRAN CA=231 IS=1 FS=1 IT=247 FT=283 TT=0 FL=100 IP=1075 FP=1150 TP=0
 TRAN CA=232 IS=1 FS=1 IT=283 FT=50 TT=0 FL=780.3 IP=1150 FP=690 TP=0
 TRAN CA=233 IS=1 FS=1 IT=50 FT=200 TT=300 FL=100 IP=690 FP=690 TP=300
 TRAN CA=234 IS=1 FS=1 IT=200 FT=283 TT=8964 FL=100 IP=255 FP=1025 TP=8964
 TRAN CA=235 IS=1 FS=1 IT=392 FT=275 TT=60 FL=5049 IP=1025 FP=900 TP=60
 TRAN CA=236 IS=1 FS=1 IT=275 FT=100 TT=900 FL=137.7 IP=900 FP=65 TP=900
 TRAN CA=237 IS=1 FS=1 IT=100 FT=100 TT=0 FL=100 IP=65 FP=1578 TP=0
 TRAN CA=238 IS=1 FS=1 IT=100 FT=100 TT=0 FL=100 IP=1578 FP=65 TP=0
 TRAN CA=239 IS=1 FS=1 IT=392 FT=392 TT=60 FL=5049 IP=1025 FP=1390 TP=60
 TRAN CA=240 IS=1 FS=1 IT=392 FT=392 TT=900 FL=137.7 IP=1390 FP=955 TP=900
 TRAN CA=241 IS=1 FS=1 IT=392 FT=392 TT=900 FL=137.7 IP=955 FP=1025 TP=900
 TRAN CA=242 IS=1 FS=1 IT=100 FT=180 TT=60 FL=100 IP=1025 FP=1025 TP=60
 TRAN CA=243 IS=1 FS=1 IT=180 FT=283 TT=210 FL=100 IP=1025 FP=1025 TP=210

PAIR CA=201 CO=27.6 DI=0.521 EX=6.4 * Tavg=85
 PAIR CA=202 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
 PAIR CA=203 CO=27.6 DI=0.488 EX=6.4 * Tavg=192
 PAIR CA=204 CO=27.6 DI=0.488 EX=6.4 * Tavg=192
 PAIR CA=205 CO=27.6 DI=0.490 EX=6.4 * Tavg=180
 PAIR CA=206 CO=27.1 DI=0.446 EX=6.4 * Tavg=326
 PAIR CA=207 CO=27.0 DI=0.440 EX=6.4 * Tavg=351
 PAIR CA=208 CO=27.0 DI=0.440 EX=6.4 * Tavg=351
 PAIR CA=209 CO=27.1 DI=0.444 EX=6.4 * Tavg=336
 PAIR CA=210 CO=27.1 DI=0.444 EX=6.4 * Tavg=336
 PAIR CA=211 CO=27.1 DI=0.445 EX=6.4 * Tavg=329
 PAIR CA=212 CO=27.6 DI=0.490 EX=6.4 * Tavg=178
 PAIR CA=213 CO=27.6 DI=0.490 EX=6.4 * Tavg=178
 PAIR CA=214 CO=27.1 DI=0.445 EX=6.4 * Tavg=329
 PAIR CA=215 CO=27.1 DI=0.445 EX=6.4 * Tavg=329
 PAIR CA=216 CO=27.1 DI=0.445 EX=6.4 * Tavg=329
 PAIR CA=217 CO=27.1 DI=0.444 EX=6.4 * Tavg=334
 PAIR CA=218 CO=27.6 DI=0.488 EX=6.4 * Tavg=188
 PAIR CA=219 CO=27.2 DI=0.455 EX=6.4 * Tavg=294
 PAIR CA=220 CO=27.0 DI=0.442 EX=6.4 * Tavg=342
 PAIR CA=221 CO=27.1 DI=0.447 EX=6.4 * Tavg=322
 PAIR CA=222 CO=27.4 DI=0.466 EX=6.4 * Tavg=254
 PAIR CA=223 CO=27.5 DI=0.479 EX=6.4 * Tavg=218
 PAIR CA=224 CO=27.6 DI=0.495 EX=6.4 * Tavg=155
 PAIR CA=225 CO=26.5 DI=0.422 EX=6.4 * Tavg=421
 PAIR CA=226 CO=27.4 DI=0.467 EX=6.4 * Tavg=250
 PAIR CA=227 CO=27.6 DI=0.496 EX=6.4 * Tavg=149
 PAIR CA=228 CO=27.3 DI=0.462 EX=6.4 * Tavg=268
 PAIR CA=229 CO=27.6 DI=0.492 EX=6.4 * Tavg=169
 PAIR CA=230 CO=27.6 DI=0.496 EX=6.4 * Tavg=149
 PAIR CA=231 CO=27.3 DI=0.463 EX=6.4 * Tavg=265



PAIR CA=232 CO=27.6 DI=0.493 EX=6.4 * Tavg=167
PAIR CA=233 CO=27.6 DI=0.504 EX=6.4 * Tavg=125
PAIR CA=234 CO=27.4 DI=0.470 EX=6.4 * Tavg=242
PAIR CA=235 CO=27.1 DI=0.444 EX=6.4 * Tavg=334
PAIR CA=236 CO=27.6 DI=0.488 EX=6.4 * Tavg=188
PAIR CA=237 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
PAIR CA=238 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
PAIR CA=239 CO=26.7 DI=0.430 EX=6.4 * Tavg=392
PAIR CA=240 CO=26.7 DI=0.430 EX=6.4 * Tavg=392
PAIR CA=241 CO=26.7 DI=0.430 EX=6.4 * Tavg=392
PAIR CA=242 CO=27.6 DI=0.499 EX=6.4 * Tavg=140
PAIR CA=243 CO=27.5 DI=0.474 EX=6.4 * Tavg=232

*
TANG PT=152 DY=6.53 EW=0
*

*END REGION IVa

*BEGIN REGION IVb

OPER CA=3 TE=416 PR=1010
OPER CA=4 TE=260 PR=1010
OPER CA=5 TE=392 PR=1010
OPER CA=6 TE=310 PR=1010
OPER CA=7 TE=280 PR=1010
OPER CA=8 TE=265 PR=1010
OPER CA=9 TE=90 PR=1010
OPER CA=10 TE=454 PR=1010
OPER CA=11 TE=300 PR=170
OPER CA=12 TE=270 PR=88
OPER CA=13 TE=507 PR=1190
OPER CA=14 TE=50 PR=1135
OPER CA=15 TE=343 PR=1135
OPER CA=16 TE=427 PR=1135
OPER CA=17 TE=343 PR=1060
OPER CA=18 TE=416 PR=1135

OPER CA=20 TE=250 PR=675
OPER CA=21 TE=275 PR=885

OPER CA=23 TE=392 PR=1375
OPER CA=24 TE=392 PR=940
OPER CA=25 TE=392 PR=1010
OPER CA=26 TE=275 PR=1010
OPER CA=27 TE=382 PR=1010
OPER CA=30 TE=235 PR=1010

TRAN CA=201 IS=1 FS=1 IT=70 FT=100 TT=1800 FL=100 IP=15 FP=1115 TP=1800
TRAN CA=202 IS=1 FS=1 IT=100 FT=100 TT=0 FL=100 IP=1115 FP=65 TP=0
TRAN CA=203 IS=1 FS=1 IT=100 FT=416 TT=16164 FL=100 IP=65 FP=1025 TP=16164
TRAN CA=204 IS=1 FS=1 IT=416 FT=100 TT=0 FL=688.5 IP=1025 FP=1025 TP=0
TRAN CA=205 IS=1 FS=1 IT=100 FT=260 TT=0 FL=688.5 IP=1025 FP=1025 TP=0
TRAN CA=206 IS=1 FS=1 IT=260 FT=392 TT=1800 FL=4590 IP=1025 FP=1025 TP=1800
TRAN CA=207 IS=1 FS=1 IT=392 FT=310 TT=900 FL=3442.5 IP=1025 FP=1025 TP=900
TRAN CA=208 IS=1 FS=1 IT=310 FT=392 TT=900 FL=3442.5 IP=1025 FP=1025 TP=900
TRAN CA=209 IS=1 FS=1 IT=392 FT=280 TT=1800 FL=2295 IP=1025 FP=1025 TP=1800



TRAN CA=210 IS=1 FS=1 IT=280 FT=392 TT=1800 FL=2295 IP=1025 FP=1025 TP=1800
 TRAN CA=211 IS=1 FS=1 IT=392 FT=265 TT=1800 FL=2295 IP=1025 FP=1025 TP=1800
 TRAN CA=212 IS=1 FS=1 IT=265 FT=90 TT=360 FL=688.5 IP=1025 FP=1025 TP=360
 TRAN CA=213 IS=1 FS=1 IT=90 FT=265 TT=900 FL=688.5 IP=1025 FP=1025 TP=900
 TRAN CA=214 IS=1 FS=1 IT=265 FT=392 TT=1800 FL=2295 IP=1025 FP=1025 TP=1800
 TRAN CA=215 IS=1 FS=1 IT=392 FT=265 TT=90 FL=4590 IP=1025 FP=1025 TP=90
 TRAN CA=216 IS=1 FS=1 IT=265 FT=392 TT=180 FL=4590 IP=1025 FP=1025 TP=180
 TRAN CA=217 IS=1 FS=1 IT=392 FT=275 TT=60 FL=5049 IP=1025 FP=1025 TP=60
 TRAN CA=218 IS=1 FS=1 IT=275 FT=100 TT=900 FL=137.7 IP=1025 FP=1025 TP=900
 TRAN CA=219 IS=1 FS=1 IT=265 FT=382 TT=0 FL=100 IP=1025 FP=1025 TP=0
 TRAN CA=220 IS=1 FS=1 IT=382 FT=454 TT=3924 FL=100 IP=1025 FP=1025 TP=3924
 TRAN CA=221 IS=1 FS=1 IT=454 FT=416 TT=4140 FL=100 IP=1025 FP=1025 TP=4140
 TRAN CA=222 IS=1 FS=1 IT=416 FT=300 TT=6264 FL=100 IP=1025 FP=185 TP=6264
 TRAN CA=223 IS=1 FS=1 IT=300 FT=270 TT=600 FL=100 IP=185 FP=103 TP=600
 TRAN CA=224 IS=1 FS=1 IT=270 FT=100 TT=8280 FL=100 IP=103 FP=65 TP=8280
 TRAN CA=225 IS=1 FS=1 IT=392 FT=507 TT=12 FL=100 IP=1025 FP=1205 TP=12
 TRAN CA=226 IS=1 FS=1 IT=507 FT=50 TT=0 FL=1836 IP=1205 FP=1150 TP=0
 TRAN CA=227 IS=1 FS=1 IT=50 FT=343 TT=1380 FL=100 IP=1150 FP=1150 TP=1380
 TRAN CA=228 IS=1 FS=1 IT=343 FT=427 TT=0 FL=100 IP=1150 FP=1150 TP=0
 TRAN CA=229 IS=1 FS=1 IT=427 FT=50 TT=0 FL=1377 IP=1150 FP=900 TP=0
 TRAN CA=230 IS=1 FS=1 IT=50 FT=343 TT=3060 FL=100 IP=900 FP=1075 TP=3060
 TRAN CA=231 IS=1 FS=1 IT=343 FT=416 TT=0 FL=100 IP=1075 FP=1150 TP=0
 TRAN CA=232 IS=1 FS=1 IT=416 FT=50 TT=0 FL=780.3 IP=1150 FP=690 TP=0
 TRAN CA=233 IS=1 FS=1 IT=50 FT=250 TT=300 FL=100 IP=690 FP=690 TP=300
 TRAN CA=234 IS=1 FS=1 IT=250 FT=416 TT=8964 FL=100 IP=255 FP=1025 TP=8964
 TRAN CA=235 IS=1 FS=1 IT=392 FT=275 TT=60 FL=5049 IP=1025 FP=900 TP=60
 TRAN CA=236 IS=1 FS=1 IT=275 FT=100 TT=900 FL=137.7 IP=900 FP=65 TP=900
 TRAN CA=237 IS=1 FS=1 IT=100 FT=100 TT=0 FL=100 IP=65 FP=1578 TP=0
 TRAN CA=238 IS=1 FS=1 IT=100 FT=100 TT=0 FL=100 IP=1578 FP=65 TP=0
 TRAN CA=239 IS=1 FS=1 IT=392 FT=392 TT=60 FL=5049 IP=1025 FP=1390 TP=60
 TRAN CA=240 IS=1 FS=1 IT=392 FT=392 TT=900 FL=137.7 IP=1390 FP=955 TP=900
 TRAN CA=241 IS=1 FS=1 IT=392 FT=392 TT=900 FL=137.7 IP=955 FP=1025 TP=900
 TRAN CA=242 IS=1 FS=1 IT=100 FT=235 TT=60 FL=100 IP=1025 FP=1025 TP=60
 TRAN CA=243 IS=1 FS=1 IT=235 FT=416 TT=210 FL=100 IP=1025 FP=1025 TP=210

PAIR CA=201 CO=27.6 DI=0.521 EX=6.4 * Tavg=85
 PAIR CA=202 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
 PAIR CA=203 CO=27.4 DI=0.465 EX=6.4 * Tavg=258
 PAIR CA=204 CO=27.4 DI=0.465 EX=6.4 * Tavg=258
 PAIR CA=205 CO=27.6 DI=0.490 EX=6.4 * Tavg=180
 PAIR CA=206 CO=27.1 DI=0.446 EX=6.4 * Tavg=326
 PAIR CA=207 CO=27.0 DI=0.440 EX=6.4 * Tavg=351
 PAIR CA=208 CO=27.0 DI=0.440 EX=6.4 * Tavg=351
 PAIR CA=209 CO=27.1 DI=0.444 EX=6.4 * Tavg=336
 PAIR CA=210 CO=27.1 DI=0.444 EX=6.4 * Tavg=336
 PAIR CA=211 CO=27.1 DI=0.445 EX=6.4 * Tavg=329
 PAIR CA=212 CO=27.6 DI=0.490 EX=6.4 * Tavg=178
 PAIR CA=213 CO=27.6 DI=0.490 EX=6.4 * Tavg=178
 PAIR CA=214 CO=27.1 DI=0.445 EX=6.4 * Tavg=329
 PAIR CA=215 CO=27.1 DI=0.445 EX=6.4 * Tavg=329
 PAIR CA=216 CO=27.1 DI=0.445 EX=6.4 * Tavg=329
 PAIR CA=217 CO=27.1 DI=0.444 EX=6.4 * Tavg=334
 PAIR CA=218 CO=27.6 DI=0.488 EX=6.4 * Tavg=188
 PAIR CA=219 CO=27.1 DI=0.447 EX=6.4 * Tavg=324
 PAIR CA=220 CO=26.6 DI=0.423 EX=6.4 * Tavg=418
 PAIR CA=221 CO=26.4 DI=0.418 EX=6.4 * Tavg=435

PAIR CA=222 CO=27.0 DI=0.438 EX=6.4 * Tavg=358
PAIR CA=223 CO=27.3 DI=0.457 EX=6.4 * Tavg=285
PAIR CA=224 CO=27.6 DI=0.489 EX=6.4 * Tavg=185
PAIR CA=225 CO=26.3 DI=0.413 EX=6.4 * Tavg=450
PAIR CA=226 CO=27.3 DI=0.459 EX=6.4 * Tavg=279
PAIR CA=227 CO=27.6 DI=0.487 EX=6.4 * Tavg=197
PAIR CA=228 CO=26.8 DI=0.432 EX=6.4 * Tavg=385
PAIR CA=229 CO=27.4 DI=0.471 EX=6.4 * Tavg=239
PAIR CA=230 CO=27.6 DI=0.487 EX=6.4 * Tavg=197
PAIR CA=231 CO=26.8 DI=0.433 EX=6.4 * Tavg=380
PAIR CA=232 CO=27.5 DI=0.473 EX=6.4 * Tavg=233
PAIR CA=233 CO=27.6 DI=0.496 EX=6.4 * Tavg=150
PAIR CA=234 CO=27.1 DI=0.444 EX=6.4 * Tavg=333
PAIR CA=235 CO=27.1 DI=0.444 EX=6.4 * Tavg=334
PAIR CA=236 CO=27.6 DI=0.488 EX=6.4 * Tavg=188
PAIR CA=237 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
PAIR CA=238 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
PAIR CA=239 CO=26.7 DI=0.430 EX=6.4 * Tavg=392
PAIR CA=240 CO=26.7 DI=0.430 EX=6.4 * Tavg=392
PAIR CA=241 CO=26.7 DI=0.430 EX=6.4 * Tavg=392
PAIR CA=242 CO=27.6 DI=0.492 EX=6.4 * Tavg=168
PAIR CA=243 CO=27.1 DI=0.446 EX=6.4 * Tavg=326

*
TANG PT=155 DY=6.523

*-----
*END REGION IVb

*-----
*BEGIN REGION V GEOMETRY TO NOZZLE N4B, NODE 165

*-----
OPER CA=3 TE=549 PR=1010
OPER CA=4 TE=260 PR=1010
OPER CA=5 TE=392 PR=1010
OPER CA=6 TE=310 PR=1010
OPER CA=7 TE=280 PR=1010
OPER CA=8 TE=265 PR=1010
OPER CA=9 TE=90 PR=1010
OPER CA=10 TE=549 PR=1010
OPER CA=11 TE=375 PR=170
OPER CA=12 TE=330 PR=88
OPER CA=13 TE=565 PR=1190
OPER CA=14 TE=50 PR=1135
OPER CA=15 TE=440 PR=1135
OPER CA=16 TE=565 PR=1135
OPER CA=17 TE=440 PR=1060
OPER CA=18 TE=549 PR=1135

OPER CA=20 TE=300 PR=675
OPER CA=21 TE=275 PR=885

OPER CA=23 TE=392 PR=1375
OPER CA=24 TE=392 PR=940
OPER CA=25 TE=392 PR=1010
OPER CA=26 TE=275 PR=1010
OPER CA=27 TE=440 PR=1010
OPER CA=30 TE=290 PR=1010



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TRAN CA=201 IS=1 FS=1 IT=70 FT=100 TT=1800 FL=100 IP=15 FP=1115 TP=1800
 TRAN CA=202 IS=1 FS=1 IT=100 FT=100 TT=0 FL=100 IP=1115 FP=65 TP=0
 TRAN CA=203 IS=1 FS=1 IT=100 FT=549 TT=16164 FL=100 IP=65 FP=1025 TP=16164
 TRAN CA=204 IS=1 FS=1 IT=549 FT=100 TT=0 FL=688.5 IP=1025 FP=1025 TP=0
 TRAN CA=205 IS=1 FS=1 IT=100 FT=260 TT=0 FL=688.5 IP=1025 FP=1025 TP=0
 TRAN CA=206 IS=1 FS=1 IT=260 FT=392 TT=1800 FL=4590 IP=1025 FP=1025 TP=1800
 TRAN CA=207 IS=1 FS=1 IT=392 FT=310 TT=900 FL=3442.5 IP=1025 FP=1025 TP=900
 TRAN CA=208 IS=1 FS=1 IT=310 FT=392 TT=900 FL=3442.5 IP=1025 FP=1025 TP=900
 TRAN CA=209 IS=1 FS=1 IT=392 FT=280 TT=1800 FL=2295 IP=1025 FP=1025 TP=1800
 TRAN CA=210 IS=1 FS=1 IT=280 FT=392 TT=1800 FL=2295 IP=1025 FP=1025 TP=1800
 TRAN CA=211 IS=1 FS=1 IT=392 FT=265 TT=1800 FL=2295 IP=1025 FP=1025 TP=1800
 TRAN CA=212 IS=1 FS=1 IT=265 FT=90 TT=360 FL=688.5 IP=1025 FP=1025 TP=360
 TRAN CA=213 IS=1 FS=1 IT=90 FT=265 TT=900 FL=688.5 IP=1025 FP=1025 TP=900
 TRAN CA=214 IS=1 FS=1 IT=265 FT=392 TT=1800 FL=2295 IP=1025 FP=1025 TP=1800
 TRAN CA=215 IS=1 FS=1 IT=392 FT=265 TT=90 FL=4590 IP=1025 FP=1025 TP=90
 TRAN CA=216 IS=1 FS=1 IT=265 FT=392 TT=180 FL=4590 IP=1025 FP=1025 TP=180
 TRAN CA=217 IS=1 FS=1 IT=392 FT=275 TT=60 FL=5049 IP=1025 FP=1025 TP=60
 TRAN CA=218 IS=1 FS=1 IT=275 FT=100 TT=900 FL=137.7 IP=1025 FP=1025 TP=900
 TRAN CA=219 IS=1 FS=1 IT=265 FT=440 TT=0 FL=100 IP=1025 FP=1025 TP=0
 TRAN CA=220 IS=1 FS=1 IT=440 FT=549 TT=3924 FL=100 IP=1025 FP=1025 TP=3924
 TRAN CA=221 *IS=1 FS=1 IT=549 FT=549 TT=0 FL=100 IP=1025 FP=1025 TP=0
 TRAN CA=222 IS=1 FS=1 IT=549 FT=375 TT=6264 FL=100 IP=1025 FP=185 TP=6264
 TRAN CA=223 IS=1 FS=1 IT=375 FT=330 TT=600 FL=100 IP=185 FP=103 TP=600
 TRAN CA=224 IS=1 FS=1 IT=330 FT=100 TT=8280 FL=100 IP=103 FP=65 TP=8280
 TRAN CA=225 IS=1 FS=1 IT=392 FT=565 TT=12 FL=100 IP=1025 FP=1205 TP=12
 TRAN CA=226 IS=1 FS=1 IT=565 FT=50 TT=0 FL=1836 IP=1205 FP=1150 TP=0
 TRAN CA=227 IS=1 FS=1 IT=50 FT=440 TT=1380 FL=100 IP=1150 FP=1150 TP=1380
 TRAN CA=228 IS=1 FS=1 IT=440 FT=565 TT=0 FL=100 IP=1150 FP=1150 TP=0
 TRAN CA=229 IS=1 FS=1 IT=565 FT=50 TT=0 FL=1377 IP=1150 FP=900 TP=0
 TRAN CA=230 IS=1 FS=1 IT=50 FT=440 TT=3060 FL=100 IP=900 FP=1075 TP=3060
 TRAN CA=231 IS=1 FS=1 IT=440 FT=549 TT=0 FL=100 IP=1075 FP=1150 TP=0
 TRAN CA=232 IS=1 FS=1 IT=549 FT=50 TT=0 FL=780.3 IP=1150 FP=690 TP=0
 TRAN CA=233 IS=1 FS=1 IT=50 FT=300 TT=300 FL=100 IP=690 FP=690 TP=300
 TRAN CA=234 IS=1 FS=1 IT=300 FT=549 TT=8964 FL=100 IP=255 FP=1025 TP=8964
 TRAN CA=235 IS=1 FS=1 IT=392 FT=275 TT=60 FL=5049 IP=1025 FP=900 TP=60
 TRAN CA=236 IS=1 FS=1 IT=275 FT=100 TT=900 FL=137.7 IP=900 FP=65 TP=900
 TRAN CA=237 IS=1 FS=1 IT=100 FT=100 TT=0 FL=100 IP=65 FP=1578 TP=0
 TRAN CA=238 IS=1 FS=1 IT=100 FT=100 TT=0 FL=100 IP=1578 FP=65 TP=0
 TRAN CA=239 IS=1 FS=1 IT=392 FT=392 TT=60 FL=5049 IP=1025 FP=1390 TP=60
 TRAN CA=240 IS=1 FS=1 IT=392 FT=392 TT=900 FL=137.7 IP=1390 FP=955 TP=900
 TRAN CA=241 IS=1 FS=1 IT=392 FT=392 TT=900 FL=137.7 IP=955 FP=1025 TP=900
 TRAN CA=242 IS=1 FS=1 IT=100 FT=290 TT=60 FL=100 IP=1025 FP=1025 TP=60
 TRAN CA=243 IS=1 FS=1 IT=290 FT=549 TT=210 FL=100 IP=1025 FP=1025 TP=210

PAIR CA=201 CO=27.6 DI=0.521 EX=6.4 * Tavg=85
 PAIR CA=202 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
 PAIR CA=203 CO=27.1 DI=0.447 EX=6.4 * Tavg=325
 PAIR CA=204 CO=27.1 DI=0.447 EX=6.4 * Tavg=325
 PAIR CA=205 CO=27.6 DI=0.490 EX=6.4 * Tavg=180
 PAIR CA=206 CO=27.1 DI=0.446 EX=6.4 * Tavg=326
 PAIR CA=207 CO=27.0 DI=0.440 EX=6.4 * Tavg=351
 PAIR CA=208 CO=27.0 DI=0.440 EX=6.4 * Tavg=351
 PAIR CA=209 CO=27.1 DI=0.444 EX=6.4 * Tavg=336
 PAIR CA=210 CO=27.1 DI=0.444 EX=6.4 * Tavg=336
 PAIR CA=211 CO=27.1 DI=0.445 EX=6.4 * Tavg=329
 PAIR CA=212 CO=27.6 DI=0.490 EX=6.4 * Tavg=178



PAIR CA=213 CO=27.6 DI=0.490 EX=6.4 * Tavg=178
PAIR CA=214 CO=27.1 DI=0.445 EX=6.4 * Tavg=329
PAIR CA=215 CO=27.1 DI=0.445 EX=6.4 * Tavg=329
PAIR CA=216 CO=27.1 DI=0.445 EX=6.4 * Tavg=329
PAIR CA=217 CO=27.1 DI=0.444 EX=6.4 * Tavg=334
PAIR CA=218 CO=27.6 DI=0.488 EX=6.4 * Tavg=188
PAIR CA=219 CO=27.0 DI=0.439 EX=6.4 * Tavg=353
PAIR CA=220 CO=25.9 DI=0.400 EX=6.4 * Tavg=495
PAIR CA=221 CO=25.5 DI=0.387 EX=6.4 * Tavg=549
PAIR CA=222 CO=26.2 DI=0.409 EX=6.4 * Tavg=462
PAIR CA=223 CO=27.0 DI=0.439 EX=6.4 * Tavg=353
PAIR CA=224 CO=27.5 DI=0.480 EX=6.4 * Tavg=215
PAIR CA=225 CO=26.1 DI=0.404 EX=6.4 * Tavg=479
PAIR CA=226 CO=27.2 DI=0.451 EX=6.4 * Tavg=308
PAIR CA=227 CO=27.4 DI=0.469 EX=6.4 * Tavg=245
PAIR CA=228 CO=25.9 DI=0.397 EX=6.4 * Tavg=503
PAIR CA=229 CO=27.2 DI=0.451 EX=6.4 * Tavg=308
PAIR CA=230 CO=27.4 DI=0.469 EX=6.4 * Tavg=245
PAIR CA=231 CO=25.9 DI=0.400 EX=6.4 * Tavg=495
PAIR CA=232 CO=27.2 DI=0.453 EX=6.4 * Tavg=300
PAIR CA=233 CO=27.6 DI=0.491 EX=6.4 * Tavg=175
PAIR CA=234 CO=26.5 DI=0.421 EX=6.4 * Tavg=425
PAIR CA=235 CO=27.1 DI=0.444 EX=6.4 * Tavg=334
PAIR CA=236 CO=27.6 DI=0.488 EX=6.4 * Tavg=188
PAIR CA=237 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
PAIR CA=238 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
PAIR CA=239 CO=26.7 DI=0.430 EX=6.4 * Tavg=392
PAIR CA=240 CO=26.7 DI=0.430 EX=6.4 * Tavg=392
PAIR CA=241 CO=26.7 DI=0.430 EX=6.4 * Tavg=392
PAIR CA=242 CO=27.6 DI=0.487 EX=6.4 * Tavg=195
PAIR CA=243 CO=26.5 DI=0.422 EX=6.4 * Tavg=420
BRAD PT=160 RA=1.25
TANG PT=165 DX=-4.007 DZ=-4.007 EW=1
NOZZ PT=165 *NOZZLE N4B
AMVT CA=1 PT=165 DX=0.0196 DY=0.1069 DZ=0.0196
AMVT CA=2 PT=165 DX=0.0196 DY=0.1069 DZ=0.0196
AMVT CA=3 PT=165 DX=0.3130 DY=1.7067 DZ=0.3130
AMVT CA=4 PT=165 DX=0.3130 DY=1.7067 DZ=0.3130
AMVT CA=5 PT=165 DX=0.3130 DY=1.7067 DZ=0.3130
AMVT CA=6 PT=165 DX=0.3130 DY=1.7067 DZ=0.3130
AMVT CA=7 PT=165 DX=0.3130 DY=1.7067 DZ=0.3130
AMVT CA=8 PT=165 DX=0.3130 DY=1.7067 DZ=0.3130
AMVT CA=9 PT=165 DX=0.3130 DY=1.7067 DZ=0.3130
AMVT CA=10 PT=165 DX=0.3130 DY=1.7067 DZ=0.3130
AMVT CA=11 PT=165 DX=0.1993 DY=1.0867 DZ=0.1993
AMVT CA=12 PT=165 DX=0.1699 DY=0.9264 DZ=0.1699
AMVT CA=13 PT=165 DX=0.3234 DY=1.7637 DZ=0.3234
AMVT CA=14 PT=165 DX=0.3234 DY=1.7637 DZ=0.3234
AMVT CA=15 PT=165 DX=0.3234 DY=1.7637 DZ=0.3234
AMVT CA=16 PT=165 DX=0.3234 DY=1.7637 DZ=0.3234
AMVT CA=17 PT=165 DX=0.3169 DY=1.7281 DZ=0.3169
AMVT CA=18 PT=165 DX=0.3234 DY=1.7637 DZ=0.3234
AMVT CA=19 PT=165 DX=0.2823 DY=1.5392 DZ=0.2823
AMVT CA=20 PT=165 DX=0.2823 DY=1.5392 DZ=0.2823
AMVT CA=21 PT=165 DX=0.3130 DY=1.7067 DZ=0.3130
AMVT CA=22 PT=165 DX=0.0196 DY=0.1069 DZ=0.0196



AMVT CA=23 PT=165 DX=0.3463 DY=1.8884 DZ=0.3463
AMVT CA=24 PT=165 DX=0.3064 DY=1.6711 DZ=0.3064
AMVT CA=25 PT=165 DX=0.3130 DY=1.7067 DZ=0.3130
AMVT CA=26 PT=165 DX=0.3064 DY=1.6711 DZ=0.3064
AMVT CA=27 PT=165 DX=0.3130 DY=1.7067 DZ=0.3130
AMVT CA=28 PT=165 DX=0.3130 DY=1.7067 DZ=0.3130
AMVT CA=29 PT=165 DX=0.3064 DY=1.6711 DZ=0.3064
AMVT CA=30 PT=165 DX=0.3130 DY=1.7067 DZ=0.3130
AMVT CA=31 PT=165 DX=0.3019 DY=1.6461 DZ=0.3019
AMVT CA=32 PT=165 DX=-.09 DY=.015 DZ=-.093

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*END REGION V

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*REGION II GEOMETRY - HPCI Line brnch

CROS CD=6

JUNC PT=10

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*BEGIN REGION IIa

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OPER CA=3 TE=150 PR=1010
OPER CA=4 TE=260 PR=1010
OPER CA=5 TE=392 PR=1010
OPER CA=6 TE=310 PR=1010
OPER CA=7 TE=280 PR=1010
OPER CA=8 TE=265 PR=1010
OPER CA=9 TE=90 PR=1010
OPER CA=10 TE=265 PR=1010
OPER CA=11 TE=150 PR=170
OPER CA=12 TE=150 PR=88
OPER CA=13 TE=392 PR=1190
OPER CA=14 TE=50 PR=1135
OPER CA=15 TE=150 PR=1135
OPER CA=16 TE=150 PR=1135
OPER CA=17 TE=150 PR=1060
OPER CA=18 TE=150 PR=1135

OPER CA=20 TE=150 PR=675
OPER CA=21 TE=275 PR=885

OPER CA=23 TE=392 PR=1375
OPER CA=24 TE=392 PR=940
OPER CA=25 TE=392 PR=1010
OPER CA=26 TE=275 PR=1010
OPER CA=27 TE=265 PR=1010

OPER CA=30 TE=125 PR=1010

TRAN CA=201 IS=1 FS=1 IT=70 FT=100 TT=1800 FL=150 IP=15 FP=1115 TP=1800
TRAN CA=202 IS=1 FS=1 IT=100 FT=100 TT=0 FL=150 IP=1115 FP=65 TP=0
TRAN CA=203 IS=1 FS=1 IT=100 FT=150 TT=16164 FL=150 IP=65 FP=1025 TP=16164
TRAN CA=204 IS=1 FS=1 IT=150 FT=100 TT=0 FL=150 IP=1025 FP=1025 TP=0
TRAN CA=205 IS=1 FS=1 IT=100 FT=260 TT=0 FL=150 IP=1025 FP=1025 TP=0
TRAN CA=206 IS=1 FS=1 IT=260 FT=392 TT=1800 FL=150 IP=1025 FP=1025 TP=1800
TRAN CA=207 IS=1 FS=1 IT=392 FT=310 TT=900 FL=150 IP=1025 FP=1025 TP=900
TRAN CA=208 IS=1 FS=1 IT=310 FT=392 TT=900 FL=150 IP=1025 FP=1025 TP=900
TRAN CA=209 IS=1 FS=1 IT=392 FT=280 TT=1800 FL=150 IP=1025 FP=1025 TP=1800

TRAN CA=210 IS=1 FS=1 IT=280 FT=392 TT=1800 FL=150 IP=1025 FP=1025 TP=1800
 TRAN CA=211 IS=1 FS=1 IT=392 FT=265 TT=1800 FL=150 IP=1025 FP=1025 TP=1800
 TRAN CA=212 IS=1 FS=1 IT=265 FT=90 TT=360 FL=150 IP=1025 FP=1025 TP=360
 TRAN CA=213 IS=1 FS=1 IT=90 FT=265 TT=900 FL=150 IP=1025 FP=1025 TP=900
 TRAN CA=214 IS=1 FS=1 IT=265 FT=392 TT=1800 FL=150 IP=1025 FP=1025 TP=1800
 TRAN CA=215 IS=1 FS=1 IT=392 FT=265 TT=90 FL=150 IP=1025 FP=1025 TP=90
 TRAN CA=216 IS=1 FS=1 IT=265 FT=392 TT=180 FL=150 IP=1025 FP=1025 TP=180
 TRAN CA=217 IS=1 FS=1 IT=392 FT=275 TT=60 FL=150 IP=1025 FP=1025 TP=60
 TRAN CA=218 IS=1 FS=1 IT=275 FT=100 TT=900 FL=150 IP=1025 FP=1025 TP=900
 TRAN CA=219 *IS=1 FS=1 IT=265 FT=265 TT=0 FL=150 IP=1025 FP=1025 TP=0
 TRAN CA=220 *IS=1 FS=1 IT=265 FT=265 TT=0 FL=150 IP=1025 FP=1025 TP=0
 TRAN CA=221 IS=1 FS=1 IT=265 FT=150 TT=4140 FL=150 IP=1025 FP=1025 TP=4140
 TRAN CA=222 IS=1 FS=1 IT=150 FT=150 TT=0 FL=150 IP=1025 FP=185 TP=0
 TRAN CA=223 IS=1 FS=1 IT=150 FT=150 TT=0 FL=150 IP=185 FP=103 TP=0
 TRAN CA=224 IS=1 FS=1 IT=150 FT=100 TT=8280 FL=150 IP=103 FP=65 TP=8280
 TRAN CA=225 IS=1 FS=1 IT=392 FT=392 TT=12 FL=150 IP=1025 FP=1205 TP=12
 TRAN CA=226 IS=1 FS=1 IT=392 FT=50 TT=0 FL=3672 IP=1205 FP=1150 TP=0
 TRAN CA=227 IS=1 FS=1 IT=50 FT=150 TT=1380 FL=150 IP=1150 FP=1150 TP=1380
 TRAN CA=228 IS=1 FS=1 IT=150 FT=150 TT=0 FL=150 IP=1150 FP=1150 TP=0
 TRAN CA=229 IS=1 FS=1 IT=150 FT=50 TT=0 FL=2754 IP=1150 FP=900 TP=0
 TRAN CA=230 IS=1 FS=1 IT=50 FT=150 TT=3060 FL=150 IP=900 FP=1075 TP=3060
 TRAN CA=231 IS=1 FS=1 IT=150 FT=150 TT=0 FL=150 IP=1075 FP=1150 TP=0
 TRAN CA=232 IS=1 FS=1 IT=150 FT=50 TT=0 FL=1560.6 IP=1150 FP=690 TP=0
 TRAN CA=233 IS=1 FS=1 IT=50 FT=150 TT=300 FL=150 IP=690 FP=690 TP=300
 TRAN CA=234 IS=1 FS=1 IT=150 FT=150 TT=8964 FL=150 IP=255 FP=1025 TP=8964
 TRAN CA=235 IS=1 FS=1 IT=392 FT=275 TT=60 FL=150 IP=1025 FP=900 TP=60
 TRAN CA=236 IS=1 FS=1 IT=275 FT=100 TT=900 FL=150 IP=900 FP=65 TP=900
 TRAN CA=237 IS=1 FS=1 IT=100 FT=100 TT=0 FL=150 IP=65 FP=1578 TP=0
 TRAN CA=238 IS=1 FS=1 IT=100 FT=100 TT=0 FL=150 IP=1578 FP=65 TP=0
 TRAN CA=239 IS=1 FS=1 IT=392 FT=392 TT=60 FL=150 IP=1025 FP=1390 TP=60
 TRAN CA=240 IS=1 FS=1 IT=392 FT=392 TT=900 FL=150 IP=1390 FP=955 TP=900
 TRAN CA=241 IS=1 FS=1 IT=392 FT=392 TT=900 FL=150 IP=955 FP=1025 TP=900
 TRAN CA=242 IS=1 FS=1 IT=100 FT=125 TT=60 FL=150 IP=1025 FP=1025 TP=60
 TRAN CA=243 IS=1 FS=1 IT=125 FT=150 TT=210 FL=150 IP=1025 FP=1025 TP=210

 PAIR CA=201 CO=27.6 DI=0.521 EX=6.4 * Tavg=85
 PAIR CA=202 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
 PAIR CA=203 CO=27.6 DI=0.504 EX=6.4 * Tavg=125
 PAIR CA=204 CO=27.6 DI=0.504 EX=6.4 * Tavg=125
 PAIR CA=205 CO=27.6 DI=0.490 EX=6.4 * Tavg=180
 PAIR CA=206 CO=27.1 DI=0.446 EX=6.4 * Tavg=326
 PAIR CA=207 CO=27.0 DI=0.440 EX=6.4 * Tavg=351
 PAIR CA=208 CO=27.0 DI=0.440 EX=6.4 * Tavg=351
 PAIR CA=209 CO=27.1 DI=0.444 EX=6.4 * Tavg=336
 PAIR CA=210 CO=27.1 DI=0.444 EX=6.4 * Tavg=336
 PAIR CA=211 CO=27.1 DI=0.445 EX=6.4 * Tavg=329
 PAIR CA=212 CO=27.6 DI=0.490 EX=6.4 * Tavg=178
 PAIR CA=213 CO=27.6 DI=0.490 EX=6.4 * Tavg=178
 PAIR CA=214 CO=27.1 DI=0.445 EX=6.4 * Tavg=329
 PAIR CA=215 CO=27.1 DI=0.445 EX=6.4 * Tavg=329
 PAIR CA=216 CO=27.1 DI=0.445 EX=6.4 * Tavg=329
 PAIR CA=217 CO=27.1 DI=0.444 EX=6.4 * Tavg=334
 PAIR CA=218 CO=27.6 DI=0.488 EX=6.4 * Tavg=188
 PAIR CA=219 CO=27.3 DI=0.463 EX=6.4 * Tavg=265
 PAIR CA=220 CO=27.3 DI=0.463 EX=6.4 * Tavg=265
 PAIR CA=221 CO=27.6 DI=0.483 EX=6.4 * Tavg=208



PAIR CA=222 CO=27.6 DI=0.496 EX=6.4 * Tavg=150
PAIR CA=223 CO=27.6 DI=0.496 EX=6.4 * Tavg=150
PAIR CA=224 CO=27.6 DI=0.504 EX=6.4 * Tavg=125
PAIR CA=225 CO=26.7 DI=0.430 EX=6.4 * Tavg=392
PAIR CA=226 CO=27.5 DI=0.478 EX=6.4 * Tavg=221
PAIR CA=227 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
PAIR CA=228 CO=27.6 DI=0.496 EX=6.4 * Tavg=150
PAIR CA=229 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
PAIR CA=230 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
PAIR CA=231 CO=27.6 DI=0.496 EX=6.4 * Tavg=150
PAIR CA=232 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
PAIR CA=233 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
PAIR CA=234 CO=27.6 DI=0.496 EX=6.4 * Tavg=150
PAIR CA=235 CO=27.1 DI=0.444 EX=6.4 * Tavg=334
PAIR CA=236 CO=27.6 DI=0.488 EX=6.4 * Tavg=188
PAIR CA=237 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
PAIR CA=238 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
PAIR CA=239 CO=26.7 DI=0.430 EX=6.4 * Tavg=392
PAIR CA=240 CO=26.7 DI=0.430 EX=6.4 * Tavg=392
PAIR CA=241 CO=26.7 DI=0.430 EX=6.4 * Tavg=392
PAIR CA=242 CO=27.6 DI=0.508 EX=6.4 * Tavg=113
PAIR CA=243 CO=27.6 DI=0.500 EX=6.4 * Tavg=138

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BRAN PT=301 DY=1 TE=1 EW=1
TANG PT=302 DY=2.333
TANG PT=305 DY=2.333 EW=1
BRAD PT=310 RA=1.75 EW=1
TANG PT=315 DX=-2.333 EW=1
CROS CD=7
VALV PT=317 DX=-1.167 PL=1 MA=2.05

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*BEGIN REGION Iib
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OPER CA=3 TE=125 PR=1010
OPER CA=4 TE=180 PR=1010
OPER CA=5 TE=246 PR=1010
OPER CA=6 TE=205 PR=1010
OPER CA=7 TE=190 PR=1010
OPER CA=8 TE=182.5 PR=1010
OPER CA=9 TE=95 PR=1010
OPER CA=10 TE=182.5 PR=1010
OPER CA=11 TE=125 PR=170
OPER CA=12 TE=125 PR=88
OPER CA=13 TE=246 PR=1190
OPER CA=14 TE=50 PR=1135
OPER CA=15 TE=125 PR=1135
OPER CA=16 TE=125 PR=1135
OPER CA=17 TE=125 PR=1060
OPER CA=18 TE=125 PR=1135

OPER CA=20 TE=125 PR=675
OPER CA=21 TE=187.5 PR=885

OPER CA=23 TE=246 PR=1375

OPER CA=24 TE=246 PR=940
 OPER CA=25 TE=246 PR=1010
 OPER CA=26 TE=187.5 PR=1010
 OPER CA=27 TE=182.5 PR=1010

OPER CA=30 TE=112.5 PR=1010

TRAN CA=201 IS=1 FS=1 IT=70 FT=100 TT=1800 FL=150 IP=15 FP=1115 TP=1800
 TRAN CA=202 IS=1 FS=1 IT=100 FT=100 TT=0 FL=150 IP=1115 FP=65 TP=0
 TRAN CA=203 IS=1 FS=1 IT=100 FT=125 TT=16164 FL=150 IP=65 FP=1025 TP=16164
 TRAN CA=204 IS=1 FS=1 IT=125 FT=100 TT=0 FL=150 IP=1025 FP=1025 TP=0
 TRAN CA=205 IS=1 FS=1 IT=100 FT=180 TT=0 FL=150 IP=1025 FP=1025 TP=0
 TRAN CA=206 IS=1 FS=1 IT=180 FT=246 TT=1800 FL=150 IP=1025 FP=1025 TP=1800
 TRAN CA=207 IS=1 FS=1 IT=246 FT=205 TT=900 FL=150 IP=1025 FP=1025 TP=900
 TRAN CA=208 IS=1 FS=1 IT=205 FT=246 TT=900 FL=150 IP=1025 FP=1025 TP=900
 TRAN CA=209 IS=1 FS=1 IT=246 FT=190 TT=1800 FL=150 IP=1025 FP=1025 TP=1800
 TRAN CA=210 IS=1 FS=1 IT=190 FT=246 TT=1800 FL=150 IP=1025 FP=1025 TP=1800
 TRAN CA=211 IS=1 FS=1 IT=246 FT=182.5 TT=1800 FL=150 IP=1025 FP=1025 TP=1800
 TRAN CA=212 IS=1 FS=1 IT=182.5 FT=95 TT=360 FL=150 IP=1025 FP=1025 TP=360
 TRAN CA=213 IS=1 FS=1 IT=95 FT=182.5 TT=900 FL=150 IP=1025 FP=1025 TP=900
 TRAN CA=214 IS=1 FS=1 IT=182.5 FT=246 TT=1800 FL=150 IP=1025 FP=1025 TP=1800
 TRAN CA=215 IS=1 FS=1 IT=246 FT=182.5 TT=90 FL=150 IP=1025 FP=1025 TP=90
 TRAN CA=216 IS=1 FS=1 IT=182.5 FT=180 TT=180 FL=150 IP=1025 FP=1025 TP=180
 TRAN CA=217 IS=1 FS=1 IT=246 FT=187.5 TT=60 FL=150 IP=1025 FP=1025 TP=60
 TRAN CA=218 IS=1 FS=1 IT=187.5 FT=100 TT=900 FL=150 IP=1025 FP=1025 TP=900
 TRAN CA=219 *IS=1 FS=1 IT=182.5 FT=182.5 TT=0 FL=150 IP=1025 FP=1025 TP=0
 TRAN CA=220 *IS=1 FS=1 IT=182.5 FT=182.5 TT=0 FL=150 IP=1025 FP=1025 TP=0
 TRAN CA=221 IS=1 FS=1 IT=182.5 FT=125 TT=4140 FL=150 IP=1025 FP=1025 TP=4140
 TRAN CA=222 IS=1 FS=1 IT=125 FT=125 TT=0 FL=150 IP=1025 FP=185 TP=0
 TRAN CA=223 IS=1 FS=1 IT=125 FT=125 TT=0 FL=150 IP=185 FP=103 TP=0
 TRAN CA=224 IS=1 FS=1 IT=125 FT=100 TT=8280 FL=150 IP=103 FP=65 TP=8280
 TRAN CA=225 *IS=1 FS=1 IT=246 FT=246 TT=12 FL=150 IP=1025 FP=1205 TP=12
 TRAN CA=226 IS=1 FS=1 IT=246 FT=50 TT=0 FL=3672 IP=1205 FP=1150 TP=0
 TRAN CA=227 IS=1 FS=1 IT=50 FT=125 TT=1380 FL=150 IP=1150 FP=1150 TP=1380
 TRAN CA=228 *IS=1 FS=1 IT=125 FT=125 TT=0 FL=150 IP=1150 FP=1150 TP=0
 TRAN CA=229 IS=1 FS=1 IT=125 FT=50 TT=0 FL=2754 IP=1150 FP=900 TP=0
 TRAN CA=230 IS=1 FS=1 IT=50 FT=125 TT=3060 FL=150 IP=900 FP=1075 TP=3060
 TRAN CA=231 IS=1 FS=1 IT=125 FT=125 TT=0 FL=150 IP=1075 FP=1150 TP=0
 TRAN CA=232 IS=1 FS=1 IT=125 FT=50 TT=0 FL=1560.6 IP=1150 FP=690 TP=0
 TRAN CA=233 IS=1 FS=1 IT=50 FT=125 TT=300 FL=150 IP=690 FP=690 TP=300
 TRAN CA=234 IS=1 FS=1 IT=125 FT=125 TT=8964 FL=150 IP=255 FP=1025 TP=8964
 TRAN CA=235 IS=1 FS=1 IT=246 FT=187.5 TT=60 FL=150 IP=1025 FP=900 TP=60
 TRAN CA=236 IS=1 FS=1 IT=187.5 FT=100 TT=900 FL=150 IP=900 FP=65 TP=900
 TRAN CA=237 IS=1 FS=1 IT=100 FT=100 TT=0 FL=150 IP=65 FP=1578 TP=0
 TRAN CA=238 IS=1 FS=1 IT=100 FT=100 TT=0 FL=150 IP=1578 FP=65 TP=0
 TRAN CA=239 IS=1 FS=1 IT=246 FT=246 TT=60 FL=150 IP=1025 FP=1390 TP=60
 TRAN CA=240 IS=1 FS=1 IT=246 FT=246 TT=900 FL=150 IP=1390 FP=955 TP=900
 TRAN CA=241 IS=1 FS=1 IT=246 FT=246 TT=900 FL=150 IP=955 FP=1025 TP=900
 TRAN CA=242 IS=1 FS=1 IT=100 FT=112.5 TT=60 FL=150 IP=1025 FP=1025 TP=60
 TRAN CA=243 IS=1 FS=1 IT=112.5 FT=125 TT=210 FL=150 IP=1025 FP=1025 TP=210

PAIR CA=201 CO=27.6 DI=0.521 EX=6.4 * Tavg=85
 PAIR CA=202 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
 PAIR CA=203 CO=27.6 DI=0.508 EX=6.4 * Tavg=113
 PAIR CA=204 CO=27.6 DI=0.508 EX=6.4 * Tavg=113
 PAIR CA=205 CO=27.6 DI=0.499 EX=6.4 * Tavg=140



PAIR CA=206 CO=27.5 DI=0.481 EX=6.4 * Tavg=213
PAIR CA=207 CO=27.5 DI=0.476 EX=6.4 * Tavg=226
PAIR CA=208 CO=27.5 DI=0.476 EX=6.4 * Tavg=226
PAIR CA=209 CO=27.5 DI=0.479 EX=6.4 * Tavg=218
PAIR CA=210 CO=27.5 DI=0.479 EX=6.4 * Tavg=218
PAIR CA=211 CO=27.5 DI=0.481 EX=6.4 * Tavg=214
PAIR CA=212 CO=27.6 DI=0.500 EX=6.4 * Tavg=139
PAIR CA=213 CO=27.6 DI=0.500 EX=6.4 * Tavg=139
PAIR CA=214 CO=27.5 DI=0.481 EX=6.4 * Tavg=214
PAIR CA=215 CO=27.5 DI=0.481 EX=6.4 * Tavg=214
PAIR CA=216 CO=27.5 DI=0.481 EX=6.4 * Tavg=214
PAIR CA=217 CO=27.5 DI=0.480 EX=6.4 * Tavg=217
PAIR CA=218 CO=27.6 DI=0.498 EX=6.4 * Tavg=144
PAIR CA=219 CO=27.6 DI=0.489 EX=6.4 * Tavg=183
PAIR CA=220 CO=27.6 DI=0.489 EX=6.4 * Tavg=183
PAIR CA=221 CO=27.6 DI=0.495 EX=6.4 * Tavg=154
PAIR CA=222 CO=27.6 DI=0.504 EX=6.4 * Tavg=125
PAIR CA=223 CO=27.6 DI=0.504 EX=6.4 * Tavg=125
PAIR CA=224 CO=27.6 DI=0.508 EX=6.4 * Tavg=113
PAIR CA=225 CO=27.4 DI=0.469 EX=6.4 * Tavg=246
PAIR CA=226 CO=27.6 DI=0.497 EX=6.4 * Tavg=148
PAIR CA=227 CO=27.6 DI=0.519 EX=6.4 * Tavg=88
PAIR CA=228 CO=27.6 DI=0.504 EX=6.4 * Tavg=125
PAIR CA=229 CO=27.6 DI=0.519 EX=6.4 * Tavg=88
PAIR CA=230 CO=27.6 DI=0.519 EX=6.4 * Tavg=88
PAIR CA=231 CO=27.6 DI=0.504 EX=6.4 * Tavg=125
PAIR CA=232 CO=27.6 DI=0.519 EX=6.4 * Tavg=88
PAIR CA=233 CO=27.6 DI=0.519 EX=6.4 * Tavg=88
PAIR CA=234 CO=27.6 DI=0.504 EX=6.4 * Tavg=125
PAIR CA=235 CO=27.5 DI=0.480 EX=6.4 * Tavg=217
PAIR CA=236 CO=27.6 DI=0.498 EX=6.4 * Tavg=144
PAIR CA=237 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
PAIR CA=238 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
PAIR CA=239 CO=27.4 DI=0.469 EX=6.4 * Tavg=246
PAIR CA=240 CO=27.4 DI=0.469 EX=6.4 * Tavg=246
PAIR CA=241 CO=27.4 DI=0.469 EX=6.4 * Tavg=246
PAIR CA=242 CO=27.6 DI=0.510 EX=6.4 * Tavg=106
PAIR CA=243 CO=27.6 DI=0.506 EX=6.4 * Tavg=119
VALV PT=320 DX=-1.167 PL=2 EW=1
CROS CD=6
TANG PT=325 DX=-0.666
TANG PT=330 DX=-2.667 EW=1
BRAD PT=335 RA=1.75 EW=1
TANP DZ=-3.5
BRAD PT=340 RA=1.75 EW=1
TANG PT=345 DX=3.333 EW=1
CROS CD=7
VALV PT=346 DX=1.167 PL=1 MA=1.725

*-----
*END REGION IIB
*-----

*BEGIN REGION II
*-----

OPER CA=1 TE=100 PR=50

OPER CA=3 TE=100 PR=50



OPER CA=4 TE=100 PR=50
OPER CA=5 TE=100 PR=50
OPER CA=6 TE=100 PR=50
OPER CA=7 TE=100 PR=50
OPER CA=8 TE=100 PR=50
OPER CA=9 TE=100 PR=50
OPER CA=10 TE=100 PR=50
OPER CA=11 TE=100 PR=50
OPER CA=12 TE=100 PR=50
OPER CA=13 TE=100 PR=50
OPER CA=14 TE=50 PR=1135
OPER CA=15 TE=100 PR=50
OPER CA=16 TE=100 PR=50
OPER CA=17 TE=100 PR=50
OPER CA=18 TE=100 PR=50

OPER CA=20 TE=100 PR=50
OPER CA=21 TE=100 PR=50
OPER CA=22 TE=100 PR=50
OPER CA=23 TE=100 PR=50
OPER CA=24 TE=100 PR=50
OPER CA=25 TE=100 PR=50
OPER CA=26 TE=100 PR=50
OPER CA=27 TE=100 PR=50
OPER CA=28 TE=100 PR=50
OPER CA=29 TE=100 PR=50
OPER CA=30 TE=100 PR=50

TRAN CA=201 IS=1 FS=1 IT=70 FT=100 TT=1800 FL=150 IP=15 FP=65 TP=1800
TRAN CA=202 *IS=1 FS=1 IT=100 FT=100 TT=0 FL=150 IP=65 FP=65 TP=0
TRAN CA=203 *IS=1 FS=1 IT=100 FT=100 TT=16164 FL=150 IP=65 FP=65 TP=16164
TRAN CA=204 *IS=1 FS=1 IT=100 FT=100 TT=0 FL=150 IP=65 FP=65 TP=0
TRAN CA=205 *IS=1 FS=1 IT=100 FT=100 TT=0 FL=150 IP=65 FP=65 TP=0
TRAN CA=206 *IS=1 FS=1 IT=100 FT=100 TT=1800 FL=150 IP=65 FP=65 TP=1800
TRAN CA=207 *IS=1 FS=1 IT=100 FT=100 TT=900 FL=150 IP=65 FP=65 TP=900
TRAN CA=208 *IS=1 FS=1 IT=100 FT=100 TT=900 FL=150 IP=65 FP=65 TP=900
TRAN CA=209 *IS=1 FS=1 IT=100 FT=100 TT=1800 FL=150 IP=65 FP=65 TP=1800
TRAN CA=210 *IS=1 FS=1 IT=100 FT=100 TT=1800 FL=150 IP=65 FP=65 TP=1800
TRAN CA=211 *IS=1 FS=1 IT=100 FT=100 TT=1800 FL=150 IP=65 FP=65 TP=1800
TRAN CA=212 *IS=1 FS=1 IT=100 FT=100 TT=360 FL=150 IP=65 FP=65 TP=360
TRAN CA=213 *IS=1 FS=1 IT=100 FT=100 TT=900 FL=150 IP=65 FP=65 TP=900
TRAN CA=214 *IS=1 FS=1 IT=100 FT=100 TT=1800 FL=150 IP=65 FP=65 TP=1800
TRAN CA=215 *IS=1 FS=1 IT=100 FT=100 TT=90 FL=150 IP=65 FP=65 TP=90
TRAN CA=216 *IS=1 FS=1 IT=100 FT=100 TT=180 FL=150 IP=65 FP=65 TP=180
TRAN CA=217 *IS=1 FS=1 IT=100 FT=100 TT=60 FL=150 IP=65 FP=65 TP=60
TRAN CA=218 *IS=1 FS=1 IT=100 FT=100 TT=900 FL=150 IP=65 FP=65 TP=900
TRAN CA=219 *IS=1 FS=1 IT=100 FT=100 TT=0 FL=150 IP=65 FP=65 TP=0
TRAN CA=220 *IS=1 FS=1 IT=100 FT=100 TT=0 FL=150 IP=65 FP=65 TP=0
TRAN CA=221 *IS=1 FS=1 IT=100 FT=100 TT=0 FL=150 IP=65 FP=65 TP=0
TRAN CA=222 *IS=1 FS=1 IT=100 FT=100 TT=0 FL=150 IP=65 FP=65 TP=0
TRAN CA=223 *IS=1 FS=1 IT=100 FT=100 TT=0 FL=150 IP=65 FP=65 TP=0
TRAN CA=224 *IS=1 FS=1 IT=100 FT=100 TT=8280 FL=150 IP=65 FP=65 TP=8280
TRAN CA=225 *IS=1 FS=1 IT=100 FT=100 TT=12 FL=150 IP=65 FP=65 TP=12
TRAN CA=226 IS=1 FS=1 IT=100 FT=50 TT=0 FL=3672 IP=65 FP=1150 TP=0
TRAN CA=227 IS=1 FS=1 IT=50 FT=100 TT=1380 FL=150 IP=1150 FP=65 TP=1380
TRAN CA=228 *IS=1 FS=1 IT=100 FT=100 TT=0 FL=150 IP=65 FP=65 TP=0



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TRAN CA=229 IS=1 FS=1 IT=100 FT=50 TT=0 FL=2754 IP=1150 FP=900 TP=0
TRAN CA=230 IS=1 FS=1 IT=50 FT=100 TT=3060 FL=150 IP=900 FP=65 TP=3060
TRAN CA=231 *IS=1 FS=1 IT=100 FT=100 TT=0 FL=150 IP=65 FP=65 TP=0
TRAN CA=232 IS=1 FS=1 IT=100 FT=50 TT=0 FL=1560.6 IP=65 FP=690 TP=0
TRAN CA=233 IS=1 FS=1 IT=50 FT=100 TT=300 FL=150 IP=690 FP=65 TP=300
TRAN CA=234 *IS=1 FS=1 IT=100 FT=100 TT=8964 FL=150 IP=65 FP=65 TP=8964
TRAN CA=235 *IS=1 FS=1 IT=100 FT=100 TT=60 FL=150 IP=65 FP=65 TP=60
TRAN CA=236 *IS=1 FS=1 IT=100 FT=100 TT=900 FL=150 IP=65 FP=65 TP=900
TRAN CA=237 *IS=1 FS=1 IT=100 FT=100 TT=0 FL=150 IP=65 FP=65 TP=0
TRAN CA=238 *IS=1 FS=1 IT=100 FT=100 TT=0 FL=150 IP=65 FP=65 TP=0
TRAN CA=239 *IS=1 FS=1 IT=100 FT=100 TT=60 FL=150 IP=65 FP=65 TP=60
TRAN CA=240 *IS=1 FS=1 IT=100 FT=100 TT=900 FL=150 IP=65 FP=65 TP=900
TRAN CA=241 *IS=1 FS=1 IT=100 FT=100 TT=900 FL=150 IP=65 FP=65 TP=900
TRAN CA=242 *IS=1 FS=1 IT=100 FT=100 TT=60 FL=150 IP=65 FP=65 TP=60
TRAN CA=243 *IS=1 FS=1 IT=100 FT=100 TT=210 FL=150 IP=65 FP=65 TP=210

PAIR CA=201 CO=27.6 DI=0.521 EX=6.4 * Tavg=85
PAIR CA=202 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
PAIR CA=203 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
PAIR CA=204 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
PAIR CA=205 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
PAIR CA=206 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
PAIR CA=207 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
PAIR CA=208 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
PAIR CA=209 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
PAIR CA=210 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
PAIR CA=211 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
PAIR CA=212 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
PAIR CA=213 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
PAIR CA=214 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
PAIR CA=215 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
PAIR CA=216 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
PAIR CA=217 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
PAIR CA=218 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
PAIR CA=219 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
PAIR CA=220 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
PAIR CA=221 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
PAIR CA=222 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
PAIR CA=223 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
PAIR CA=224 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
PAIR CA=225 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
PAIR CA=226 CO=27.5 DI=0.526 EX=6.4 * Tavg=75
PAIR CA=227 CO=27.5 DI=0.526 EX=6.4 * Tavg=75
PAIR CA=228 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
PAIR CA=229 CO=27.5 DI=0.526 EX=6.4 * Tavg=75
PAIR CA=230 CO=27.5 DI=0.526 EX=6.4 * Tavg=75
PAIR CA=231 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
PAIR CA=232 CO=27.5 DI=0.526 EX=6.4 * Tavg=75
PAIR CA=233 CO=27.5 DI=0.526 EX=6.4 * Tavg=75
PAIR CA=234 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
PAIR CA=235 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
PAIR CA=236 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
PAIR CA=237 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
PAIR CA=238 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
PAIR CA=239 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
PAIR CA=240 CO=27.6 DI=0.512 EX=6.4 * Tavg=100



PAIR CA=241 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
PAIR CA=242 CO=27.6 DI=0.512 EX=6.4 * Tavg=100
PAIR CA=243 CO=27.6 DI=0.512 EX=6.4 * Tavg=100

VALV PT=350 DX=1.167 PL=2 EW=1
CROS CD=6
TANG PT=355 DX=0.167
TANG PT=360 DX=2.083 EW=1
BRAD PT=380 RA=1.75 EW=1
TANP DY=-2.479
BRAD PT=390 RA=1.75 EW=1
TANG PT=392 DX=2.585 DY=-2.585
TANG PT=395 DX=2.585 DY=-2.585 EW=1
BRAD PT=400 RA=1.75 EW=1
TANG PT=405 DZ=-3.417
BRAD PT=410 RA=1.17 EW=1
TANG PT=415 DY=-3
TANG PT=420 DY=-5.25
TANG PT=425 DY=-2.417
BRAD PT=430 RA=1.75 EW=1
TANG PT=435 DZ=2.333
TANG PT=440 DZ=4.757
TANG PT=445 DZ=4.757
TANG PT=450 DZ=4.757
BRAD PT=455 RA=1.75 EW=1
TANG PT=460 DX=-1.989 DZ=1.989
BRAD PT=465 RA=1.75 EW=1
TANG PT=470 DY=-5.722
TANG PT=480 DY=-5.722
TANG PT=485 DY=-5.722
BRAD PT=490 RA=1.17 EW=1
TANG PT=495 DZ=1.667
TANG PT=500 DZ=2.0833
BRAD PT=505 RA=1.75 EW=1
TANG PT=510 DX=3.682
TANG PT=515 DX=3.682
TANG PT=520 DX=3.682
TANG PT=525 DX=3.682
BRAD PT=535 RA=1.75
TANG PT=540 DX=2.556 DZ=-2.556
TANG PT=545 DX=2.555 DZ=-2.555
STRU PT=546 DX=-.7071 DZ=-.7071
STRU PT=547 DX=-.7071 DZ=-.7071
ANCH PT=547

*-----
*END REGION II
*-----
*

*** VALVE OPERATOR ***
CROS CD=7
JUNC PT=346
VALV PT=348 DY=5.567 PL=3 MA=2.52
*** SUPPORTS AND ANCHORS ***
CSUP PT=105 DY=1 KP=5000 PI=0 *FW-9
CSUP PT=190 DY=1 KP=1000 PI=0 *FW-6
CSUP PT=220 DY=-1 KP=1000 PI=0 *FW-4



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CSUP PT=270 DY=1 KP=1000 PI=0 *FW-2
CSUP PT=145 DY=1 KP=1000 PI=0 *FW-7
*
RSTN PT=230 DX=-0.6123 DY=-0.5 DZ=0.6123 SP=370 *FW-3
*
RSTN PT=80 DX=1.0 SP=200 *FDW-H10

RSTN PT=201 DX=0.198 DZ=0.9802 SP=1000 *FDW-H23
RSTN PT=546 DX=-0.7071 DZ=-0.7071 SP=1000 *BELLOWS
RSTN PT=355 DY=1.0 *HPCI-H31
RSTN PT=415 DX=1.0 DZ=1.0 *HPCI-H32
RSTN PT=30 DX=1.0 DY=1.0 *FDW-HD37 FLUED HEAD
ROTR PT=30 RZ=1 *FDW_HD37 FLUED HEAD
RSTN PT=60 DX=1.0 DY=1.0 SP=200 *FDW-H24
RSTN PT=5 DX=1 DY=1 DZ=1
ROTR PT=5 RZ=1
ENDP



APPENDIX B

PIPESTRESS OUTPUT FILE ("FWHPCI.PRF")



Structural Integrity Associates, Inc.

DST COMPUTER SERVICES S. A.

F-4

PAGE NO. 97

++ DST/PIPESTRESS ++

Vermont Yankee

Version 3.5.1+026 PC-EXE

Release: Jun

CALCULATION NUMBER 2 CODE SECTION III CLASS 1 ASME-1998
Vermont Yankee Feedwater PipingSI Fatigue Analysis

KRE

2007/07/11 11:35:07 [1

DELTA T1 IN DEGREES
PRESSURES IN PSI
STRESSES IN PSI
GLOBAL MOMENTS IN FT-LB

SUMMARY OF LOAD SETS AT POINT 155 LR ELBOW 155 TO 160

LOAD SET NO.	LOAD SET DESCRIPTION	CYCLES	DYNAM. CYCLES	PRESSURE	MOMENT			TRANSIENT STRESSES			DELTA T1
					X	Y	Z	EQ. 10	EQ. 11	EQ. 13	
1	Design Hydrotest +	120		1100.	55.	22.	54.	0.	50.	0.	0.
2	Design Hydrotest -	120		50.	56.	21.	54.	0.	0.	0.	0.
3	Startup +	300		1010.	14505.	-695.	-12403.	25110.	25114.	12555.	0.
4	TRoll & Inc. PWR1 -	610		1010.	24012.	-983.	-21319.	-25109.	-54064.	-12555.	-169.
5	TRoll & Inc. PWR2 +	599		1010.	13760.	-442.	-11409.	0.	15049.	0.	78.
6	TRoll & Inc. PWR3 +	599		1010.	4661.	45.	-2630.	0.	252.	0.	1.
7	DlyReduction to 75% -	10000		1010.	10313.	-259.	-8082.	0.	-317.	0.	-1.
8	DlyReduction to 75% +	10000		1010.	4661.	45.	-2630.	0.	317.	0.	1.
9	WklyReduct to 50% -	2000		1010.	12381.	-369.	-10078.	0.	-215.	0.	-1.
10	WklyReduct to 50% +	2000		1010.	4661.	45.	-2630.	0.	215.	0.	1.
11	LOFWH+TT 1 -	310		1010.	13415.	-424.	-11077.	0.	-243.	0.	-1.
12	LOFWH+TT 2 -	10		1010.	24520.	-1009.	-21810.	0.	-1480.	0.	-7.
13	LOFWH+TT 3 +	10		1010.	13415.	-424.	-11077.	0.	619.	0.	3.
14	LOFWH+TT 4 +	10		1010.	4661.	45.	-2630.	0.	243.	0.	1.
15	LOFWH+PEWHTR Byp -	70		1010.	13415.	-424.	-11077.	0.	-4784.	0.	-25.
16	LOFWH+PEWHTR Byp +	70		1010.	4661.	45.	-2630.	0.	2433.	0.	13.
17	SCRAM+TT+AllOtrScm -	289		1010.	12192.	-365.	-9935.	0.	-6434.	0.	-34.
18	SCRAM+TT+AllOtrScm -	289		1010.	23480.	-960.	-20844.	0.	-593.	0.	-3.
19	HotStandby 1 +	300		1010.	10450.	-359.	-8332.	10951.	10969.	5476.	39.
20	HotStandby 2 +	300		1010.	8500.	-316.	-6524.	17948.	17967.	8974.	0.
21	HotStandby 3 -	300		1010.	14505.	-695.	-12403.	0.	0.	0.	0.
22	Shutdown 1 -	300		170.	7904.	-355.	-6594.	-25111.	-25113.	-12555.	-0.
23	Shutdown 2 -	300		88.	6239.	-269.	-5132.	-14160.	-14241.	-7080.	-1.
24	Shutdown 3 -	300		50.	56.	21.	54.	-11328.	-11330.	-5664.	-0.
25	SCRAM+LOFWP1 +	10		1190.	2347.	86.	-451.	10950.	10965.	5475.	42.
26	SCRAM+LOFWP2 -	10		1135.	26504.	-1110.	-23646.	-10950.	-57147.	-5475.	-251.
27	SCRAM+LOFWP3 +	10		1135.	17301.	-771.	-14962.	18318.	18515.	9159.	5.
28	SCRAM+LOFWP4 +	10		1135.	15136.	-727.	-12959.	26076.	26097.	13038.	31.
29	SCRAM+LOFWP5 -	10		885.	24686.	-1036.	-22025.	-26053.	-64906.	-13027.	-223.
30	SCRAM+LOFWP6 +	10		1060.	16738.	-747.	-14459.	18317.	18413.	9159.	2.
31	SCRAM+LOFWP7 +	10		1135.	15415.	-733.	-13217.	25130.	25158.	12565.	27.

File No.: VY-16Q-311

Revision: 0



++ DST/PIPESTRESS ++

Vermont Yankee

Version 3.5.1+026 PC-EXE

Release: Jun

CALCULATION NUMBER 2 CODE SECTION III CLASS 1 ASME-1998
Vermont Yankee Feedwater PipingSI Fatigue Analysis

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SUMMARY OF LOAD SETS AT POINT 155 LR ELBOW 155 TO 160 GLOBAL MOMENTS IN FT-LB

Table with columns: LOAD SET NO., LOAD SET DESCRIPTION, CYCLES, DYNAM. CYCLES, PRESSURE, MOMENT X, MOMENT Y, MOMENT Z, TRANSIENT STRESSES EQ. 10, EQ. 11, EQ. 13, DELTA T1. Rows include SCRAM+LOFWP8, SCRAM+LOFWP9, SCRAM+LOFWP10, SCRAM+SRVBLDN1, SCRAM+SRVBLDN2, Hydro Test, SCRAM+TG+OPres1, SCRAM+TG+OPres2, SCRAM+TG+OPres3, HotSbyFWcyc, NORMAL+OBE, NORMAL-OBE, WEIGHT, DYNAMIC FLAG= 1.

Table with columns: B1, C1, K1, B2, C2, K2, C3, K3, C3PRIM, C4, Z, DIAM/TH, MATERIAL, E. Values: 0.106, 1.247, 1.000, 2.022, 3.034, 1.000, 1.000, 1.000, 0.500, 1.100, 0.60321E+02, 12.752, CARBON STEEL, 0.295.

THIS ANALYSIS IS FOR THE BODY OF THE FITTING.

File No.: VY-16Q-311
Revision: 0



++ DST/PIPESTRESS ++

Vermont Yankee

Version 3.5.1+026 PC-EXE

Release: Jun

CALCULATION NUMBER 2 CODE SECTION III CLASS 1 ASME-1998
Vermont Yankee Feedwater PipingSI Fatigue Analysis

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FATIGUE ANALYSIS AT POINT 155, LR ELBOW 155 TO 160

INDIVIDUAL STRESS RANGE CHECK

DELTA T1 IN DEGREES
STRESSES IN PSI

LOAD SET PAIR	SN EQN.10	SE EQN.12	DELTA T1 RANGE	EQN.13	SP EQN.11	KE	SALT EQN.14	ALLOW CYCLES	3*SM	ALLOWABLE FOR DELTA T1 RANGE
28 29	62066.b	7949.	255.1	35508.	100939.	1.317	66456.	1808.	53580.	759.3
29 31	60890.b	7720.	251.0	35035.	99771.	1.273	63498.	2050.	53580.	763.1
29 43	60634.b	8457.	247.4	34041.	99802.	1.263	63040.	2098.	53580.	763.1
29 34	60622.b	8457.	224.3	34035.	99482.	1.263	62816.	2122.	53580.	763.1
3 29	60614.b	8457.	224.3	34031.	99470.	1.263	62793.	2125.	53580.	763.1
28 32	61463.b	6621.	198.3	36705.	90035.	1.294	58265.	2697.	53580.	763.1
20 29	58527.b	13533.	224.4	30450.	97400.	1.185	57693.	2783.	53580.	763.1
25 29	58182.b	18755.	266.0	28382.	97049.	1.172	56861.	2915.	53580.	759.3
31 32	60288.b	6392.	194.2	36232.	88867.	1.219	54146.	3406.	54348.	766.8
4 28	59539.b	7360.	201.2	34042.	88515.	1.222	54103.	3415.	53580.	763.1
32 43	60031.b	7129.	190.6	35238.	88898.	1.209	53745.	3488.	54348.	766.8
32 34	60020.b	7129.	167.6	35233.	88578.	1.209	53533.	3532.	54348.	766.8
3 32	60011.b	7129.	167.6	35228.	88566.	1.208	53512.	3536.	54348.	766.8
4 31	58364.b	7131.	197.0	33569.	87347.	1.148	50128.	4354.	54348.	766.8
4 43	58108.b	7868.	193.5	32576.	87378.	1.138	49733.	4465.	54348.	766.8
4 34	58096.b	7868.	170.4	32570.	87058.	1.138	49533.	4523.	54348.	766.8
3 4	58088.b	7868.	170.4	32566.	87046.	1.138	49512.	4529.	54348.	766.8
25 32	57580.b	17427.	209.2	29579.	86146.	1.149	49505.	4531.	53580.	763.1
20 32	57925.b	12205.	167.6	31647.	86496.	1.132	48941.	4700.	54348.	766.8
26 43	47001.g	9927.	274.8	26489.	93513.	1.000	46757.	5404.	53580.	763.1
26 34	46989.g	9927.	251.8	26484.	93193.	1.000	46597.	5460.	53580.	763.1
3 26	46980.g	9927.	251.7	26479.	93181.	1.000	46591.	5462.	53580.	763.1
26 28	46445.g	9419.	282.5	25969.	92663.	1.000	46332.	5553.	53580.	759.3
27 29	52528.g	6169.	229.1	31629.	91578.	1.000	45789.	5750.	53580.	788.7
2 29	52662.g	19973.	223.8	27119.	91515.	1.000	45758.	5762.	53580.	849.3
24 29	52662.g	19973.	223.3	27119.	91515.	1.000	45758.	5762.	53580.	810.3

Notes b,d,e,k: Fails
g: Weld ISI
h,i: Rupture Location
L: Information

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Vermont Yankee

Version 3.5.1+026 PC-EXE

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 CALCULATION NUMBER 2 CODE SECTION III CLASS 1 ASME-1998
 Vermont Yankee Feedwater PipingSI Fatigue Analysis

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FATIGUE ANALYSIS AT POINT 155, LR ELBOW 155 TO 160

 INDIVIDUAL STRESS RANGE CHECK

 DELTA T1 IN DEGREES
 STRESSES IN PSI

LOAD SET PAIR	SN EQN.10	SE EQN.12	DELTA T1 RANGE	EQN.13	SP EQN.11	KE	SALT EQN.14	ALLOW CYCLES	3*SM	ALLOWABLE FOR DELTA T1 RANGE
29 36	52662.g	19973.	220.6	27119.	91515.	1.000	45758.	5762.	53580.	821.0
29 38	52662.g	19973.	223.8	27119.	91515.	1.000	45758.	5762.	53580.	849.3
22 28	64676.b	5819.	31.9	40719.	64699.	1.414	45748.	5766.	53580.	763.1
26 31	45270.g	9190.	278.4	25496.	91495.	1.000	45747.	5766.	53580.	763.1
29 30	52386.g	6625.	226.2	31032.	91335.	1.000	45668.	5796.	53580.	788.7
4 25	55656.b	18165.	212.0	26916.	84625.	1.077	45590.	5825.	53580.	763.1
20 26	44893.g	15003.	251.8	22898.	91110.	1.000	45555.	5838.	53580.	763.1
29 37	51415.g	19974.	223.8	25871.	90268.	1.000	45134.	6002.	53580.	784.2
5 29	36248.		302.3		90149.	1.000	45075.	6025.	53580.	823.9
4 20	56000.b	12943.	170.5	28984.	84975.	1.061	45071.	6026.	54348.	766.8
19 29	49926.g	11928.	263.5	26952.	88796.	1.000	44398.	6302.	53580.	788.7
25 26	42561.		293.4		88773.	1.000	44386.	6307.	53580.	759.3
2 26	41017.		251.2		87213.	1.000	43607.	6647.	53580.	849.3
24 26	41395.		250.8		87213.	1.000	43607.	6647.	53580.	810.3
26 36	41017.		248.0		87213.	1.000	43607.	6647.	53580.	821.0
26 38	41017.		251.2		87213.	1.000	43607.	6647.	53580.	849.3
1 29	47735.g	19973.	224.1	22191.	86638.	1.000	43319.	6779.	53580.	849.3
22 31	63959.b	6048.	27.7	40246.	63989.	1.354	43310.	6783.	54348.	766.8
23 29	47490.g	15103.	222.4	26817.	86343.	1.000	43172.	6848.	53580.	801.6
16 29	43884.g	16837.	236.9	21476.	85169.	1.000	42585.	7132.	53580.	798.3
22 29	45501.g	13765.	223.2	26165.	84354.	1.000	42177.	7338.	53580.	763.1
26 30	37958.		253.6		84251.	1.000	42125.	7365.	53580.	788.7
29 33	44347.g	7118.	238.8	26905.	84109.	1.000	42055.	7402.	53580.	816.1
5 26	22614.		329.7		83859.	1.000	41930.	7467.	53580.	823.9
29 39	44560.g	14613.	223.8	24377.	83413.	1.000	41706.	7587.	53580.	787.5
26 27	36908.		256.6		83301.	1.000	41651.	7617.	53580.	788.7

 Notes b,d,e,k: Fails
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FATIGUE ANALYSIS AT POINT 155, LR ELBOW 155 TO 160

 INDIVIDUAL STRESS RANGE CHECK

 DELTA T1 IN DEGREES
 STRESSES IN PSI

LOAD SET PAIR	SN EQN.10	SE EQN.12	DELTA T1 RANGE	EQN.13	SP EQN.11	KE	SALT EQN.14	ALLOW CYCLES	3*SM	ALLOWABLE FOR DELTA T1 RANGE
8 29	43884.g	16837.	225.5	21476.	83053.	1.000	41527.	7684.	53580.	798.3
6 29	43884.g	16837.	225.2	21476.	82988.	1.000	41494.	7702.	53580.	798.3
14 29	43884.g	16837.	225.1	21476.	82979.	1.000	41490.	7705.	53580.	798.3
10 29	43884.g	16837.	225.0	21476.	82951.	1.000	41475.	7713.	53580.	798.3
29 41	43884.g	16837.	223.8	21476.	82736.	1.000	41368.	7772.	53580.	798.3
29 40	43760.g	17270.	223.8	20920.	82613.	1.000	41307.	7807.	53580.	798.3
19 26	36292.		290.9		82507.	1.000	41253.	7836.	53580.	788.7
23 26	39055.		249.8		82041.	1.000	41021.	7969.	53580.	801.6
26 37	35796.		251.2		81993.	1.000	40996.	7983.	53580.	784.2
2 4	52124.g	19385.	169.9	27641.	81079.	1.000	40539.	8253.	54348.	851.7
4 24	52124.g	19385.	169.4	27641.	81079.	1.000	40539.	8253.	54348.	813.4
4 36	52124.g	19385.	166.7	27641.	81079.	1.000	40539.	8253.	54348.	824.1
4 38	52124.g	19385.	169.9	27641.	81079.	1.000	40539.	8253.	54348.	851.7
22 43	62228.b	5311.	24.1	39253.	62546.	1.290	40342.	8374.	54348.	766.8
27 32	51926.g	4842.	172.4	32826.	80674.	1.000	40337.	8377.	54348.	792.1
30 32	51784.g	5297.	169.4	32229.	80431.	1.000	40216.	8452.	54348.	792.1
29 42	41341.		253.0		80418.	1.000	40209.	8456.	53580.	818.0
22 34	62217.b	5311.	1.1	39247.	62226.	1.290	40123.	8510.	54348.	766.8
3 22	62208.b	5311.	1.1	39243.	62214.	1.289	40105.	8521.	54348.	766.8
22 26	48016.g	15236.	250.7	27681.	80052.	1.000	40026.	8571.	53580.	763.1
26 33	32701.		266.2		79807.	1.000	39904.	8649.	53580.	816.1
32 37	50812.g	18645.	167.0	27068.	79364.	1.000	39682.	8794.	54348.	788.7
5 32	35646.		245.5		79246.	1.000	39623.	8832.	54348.	827.0
4 27	50002.g	5580.	175.2	30163.	79153.	1.000	39577.	8863.	54348.	792.1
1 26	32672.		251.5		78918.	1.000	39459.	8942.	53580.	849.3
4 30	49860.g	6036.	172.2	29566.	78911.	1.000	39455.	8944.	54348.	792.1

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FATIGUE ANALYSIS AT POINT 155, LR ELBOW 155 TO 160

 INDIVIDUAL STRESS RANGE CHECK

 DELTA T1 IN DEGREES
 STRESSES IN PSI

LOAD SET PAIR	SN EQN.10	SE EQN.12	DELTA T1 RANGE	EQN.13	SP EQN.11	KE	SALT EQN.14	ALLOW CYCLES	3*SM	ALLOWABLE FOR DELTA T1 RANGE
16 26	30250.		264.3		78880.	1.000	39440.	8955.	53580.	798.3
7 29	39141.		222.1		77994.	1.000	38997.	9260.	53580.	798.3
19 32	49324.g	10600.	206.7	28149.	77893.	1.000	38946.	9296.	54348.	792.1
29 45	39023.		223.8		77876.	1.000	38938.	9302.	53580.	814.2
4 37	48890.g	19386.	169.9	24406.	77845.	1.000	38922.	9313.	54348.	788.7
4 5	33721.		248.4		77724.	1.000	38862.	9356.	54348.	827.0
26 40	31239.		251.2		77436.	1.000	38718.	9459.	53580.	798.3
2 32	48721.g	18644.	167.0	24978.	77272.	1.000	38636.	9519.	54348.	851.7
24 32	48721.g	18644.	166.6	24978.	77272.	1.000	38636.	9519.	54348.	813.4
32 36	48721.g	18644.	163.9	24978.	77272.	1.000	38636.	9519.	54348.	824.1
32 38	48721.g	18644.	167.0	24978.	77272.	1.000	38636.	9519.	54348.	851.7
8 26	30250.		252.9		76764.	1.000	38382.	9707.	53580.	798.3
6 26	30250.		252.6		76698.	1.000	38349.	9732.	53580.	798.3
14 26	30250.		252.5		76689.	1.000	38345.	9735.	53580.	798.3
10 26	30250.		252.4		76661.	1.000	38331.	9746.	53580.	798.3
26 41	30250.		251.2		76447.	1.000	38223.	9827.	53580.	798.3
17 29	37547.		189.4		76400.	1.000	38200.	9845.	53580.	798.3
4 19	47399.g	11338.	209.6	25486.	76372.	1.000	38186.	9856.	54348.	792.1
9 29	37405.		222.7		76258.	1.000	38129.	9899.	53580.	798.3
13 29	36537.		227.1		76009.	1.000	38004.	9996.	53580.	822.9
4 23	46952.g	14514.	168.5	27338.	75906.	1.000	37953.	10042.	54348.	804.7
1 32	47132.g	18645.	167.3	23388.	75733.	1.000	37867.	10120.	54348.	851.7
11 29	36537.		222.5		75390.	1.000	37695.	10278.	53580.	798.3
15 29	36537.		198.1		75390.	1.000	37695.	10278.	53580.	798.3
26 39	28939.		251.2		75136.	1.000	37568.	10397.	53580.	787.5
29 35	36132.		189.4		74985.	1.000	37492.	10468.	53580.	798.3

 Notes b,d,e,k: Fails
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FATIGUE ANALYSIS AT POINT 155, LR ELBOW 155 TO 160

INDIVIDUAL STRESS RANGE CHECK

DELTA T1 IN DEGREES
STRESSES IN PSI

LOAD SET PAIR	SN EQN.10	SE EQN.12	DELTA T1 RANGE	EQN.13	SP EQN.11	KE	SALT EQN.14	ALLOW CYCLES	3*SM	ALLOWABLE FOR DELTA T1 RANGE
29 44	36112.		223.8		74965.	1.000	37482.	10477.	53580.	814.2
21 29	35503.		223.8		74356.	1.000	37178.	10772.	53580.	763.1
16 32	43282.		180.1		74266.	1.000	37133.	10817.	54348.	801.4
1 4	45210.g	19385.	170.1	20726.	74214.	1.000	37107.	10843.	54348.	851.7
26 42	27708.		280.4		74129.	1.000	37064.	10885.	53580.	818.0
4 22	44964.g	13177.	169.3	26687.	73917.	1.000	36959.	10992.	54348.	766.8
4 33	43808.g	6529.	184.9	27427.	73672.	1.000	36836.	11117.	54348.	819.2
4 16	41357.		182.9		72745.	1.000	36372.	11607.	54348.	801.4
32 39	43959.g	13286.	167.0	25574.	72510.	1.000	36255.	11735.	54348.	793.6
8 32	43282.		168.7		72150.	1.000	36075.	11936.	54348.	801.4
23 32	43549.g	13774.	165.6	24676.	72100.	1.000	36050.	11964.	54348.	804.7
6 32	43282.		168.4		72085.	1.000	36042.	11972.	54348.	801.4
14 32	43282.		168.3		72076.	1.000	36038.	11978.	54348.	801.4
10 32	43282.		168.2		72048.	1.000	36024.	11993.	54348.	801.4
32 41	43282.		167.0		71833.	1.000	35917.	12116.	54348.	801.4
32 40	43158.		167.0		71710.	1.000	35855.	12187.	54348.	801.4
7 26	25507.		249.5		71704.	1.000	35852.	12190.	53580.	798.3
26 45	25383.		251.2		71580.	1.000	35790.	12262.	53580.	814.2
4 40	42346.		169.9		71301.	1.000	35651.	12426.	54348.	801.4
4 39	42033.		169.9		70988.	1.000	35494.	12614.	54348.	793.6
26 35	24485.		216.8		70682.	1.000	35341.	12801.	53580.	798.3
4 8	41357.		171.6		70629.	1.000	35314.	12834.	54348.	801.4
4 6	41357.		171.2		70563.	1.000	35282.	12874.	54348.	801.4
4 14	41357.		171.2		70554.	1.000	35277.	12880.	54348.	801.4
4 10	41357.		171.0		70526.	1.000	35263.	12897.	54348.	801.4
4 41	41357.		169.9		70312.	1.000	35156.	13032.	54348.	801.4

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FATIGUE ANALYSIS AT POINT 155, LR ELBOW 155 TO 160

 INDIVIDUAL STRESS RANGE CHECK

 DELTA T1 IN DEGREES
 STRESSES IN PSI

LOAD SET PAIR	SN EQN.10	SE EQN.12	DELTA T1 RANGE	EQN.13	SP EQN.11	KE	SALT EQN.14	ALLOW CYCLES	3*SM	ALLOWABLE FOR DELTA T1 RANGE
22 32	41561.		166.5		70111.	1.000	35056.	13159.	54348.	766.8
17 26	23913.		216.8		70110.	1.000	35055.	13160.	53580.	798.3
9 26	23771.		250.1		69968.	1.000	34984.	13251.	53580.	798.3
32 33	40406.		182.1		69867.	1.000	34934.	13316.	54348.	819.2
13 26	22903.		254.6		69719.	1.000	34860.	13413.	53580.	822.9
32 42	40739.		196.2		69515.	1.000	34757.	13548.	54348.	821.2
11 26	22903.		249.9		69100.	1.000	34550.	13826.	53580.	798.3
15 26	22903.		225.6		69100.	1.000	34550.	13826.	53580.	798.3
26 44	22472.		251.2		68669.	1.000	34334.	14124.	53580.	814.2
26 29	29511.		27.4		68364.	1.000	34182.	14340.	53580.	759.3
22 27	58697.b	7598.	5.9	36840.	58896.	1.160	34161.	14370.	54348.	792.1
21 26	21870.		251.2		68067.	1.000	34034.	14554.	53580.	763.1
4 42	38815.		199.0		67994.	1.000	33997.	14607.	54348.	821.2
29 32	29051.		56.8		67904.	1.000	33952.	14674.	53580.	763.1
24 28	58057.b	12029.	31.8	34782.	58080.	1.167	33893.	14760.	53580.	810.3
7 32	38539.		165.3		67091.	1.000	33545.	15288.	54348.	801.4
32 45	38428.		167.0		66979.	1.000	33489.	15375.	54348.	817.3
18 29	28066.		220.6		66919.	1.000	33459.	15422.	53580.	821.0
4 29	27637.		53.9		66490.	1.000	33245.	15763.	53580.	763.1
12 29	27212.		215.9		66064.	1.000	33032.	16111.	53580.	822.9
4 7	36614.		168.2		65569.	1.000	32785.	16530.	54348.	801.4
17 32	36945.		132.6		65496.	1.000	32748.	16592.	54348.	801.4
4 45	36498.		169.9		65453.	1.000	32726.	16630.	54348.	817.3
9 32	36804.		165.9		65355.	1.000	32677.	16715.	54348.	801.4
13 32	35935.		170.4		65106.	1.000	32553.	16934.	54348.	826.0
22 30	57644.b	7142.	2.9	36243.	57742.	1.121	32373.	17257.	54348.	792.1

 Notes b,d,e,k: Fails
 g: Weld ISI
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 L: Information

File No.: VY-16Q-311

Revision: 0

++ DST/PIPESTRESS ++

Vermont Yankee

Version 3.5.1+026 PC-EXE

Release: Jun

 CALCULATION NUMBER 2 CODE SECTION III CLASS 1 ASME-1998
 Vermont Yankee Feedwater PipingSI Fatigue Analysis

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FATIGUE ANALYSIS AT POINT 155, LR ELBOW 155 TO 160

 INDIVIDUAL STRESS RANGE CHECK

 DELTA T1 IN DEGREES
 STRESSES IN PSI

LOAD SET PAIR	SN EQN.10	SE EQN.12	DELTA T1 RANGE	EQN.13	SP EQN.11	KE	SALT EQN.14	ALLOW CYCLES	3*SM	ALLOWABLE FOR DELTA T1 RANGE
4 35	35592.		135.4		64547.	1.000	32273.	17438.	54348.	801.4
11 32	35935.		165.7		64487.	1.000	32243.	17493.	54348.	801.4
15 32	35935.		141.4		64487.	1.000	32243.	17493.	54348.	801.4
32 35	35530.		132.6		64082.	1.000	32041.	17873.	54348.	801.4
32 44	35518.		167.0		64070.	1.000	32035.	17884.	54348.	817.3
4 17	35020.		135.4		63975.	1.000	31987.	17974.	54348.	801.4
4 9	34878.		168.7		63833.	1.000	31917.	18111.	54348.	801.4
24 31	57340.b	12259.	27.6	34309.	57370.	1.110	31844.	18252.	54348.	813.4
26 32	31565.		84.2		63602.	1.000	31801.	18336.	53580.	763.1
4 13	34010.		173.2		63584.	1.000	31792.	18353.	54348.	826.0
21 32	34901.		167.0		63452.	1.000	31726.	18484.	54348.	766.8
4 11	34010.		168.6		62965.	1.000	31483.	18975.	54348.	801.4
4 15	34010.		144.2		62965.	1.000	31483.	18975.	54348.	801.4
4 44	33589.		169.9		62544.	1.000	31272.	19413.	54348.	817.3
4 21	32977.		169.9		61932.	1.000	30966.	20067.	54348.	766.8
18 26	14433.		248.0		60630.	1.000	30315.	21420.	53580.	821.0
23 28	55715.b	7157.	32.7	35896.	55816.	1.080	30132.	21821.	53580.	801.6
4 26	28162.		81.3		60199.	1.000	30100.	21893.	53580.	763.1
12 26	13576.		243.3		59773.	1.000	29886.	22376.	53580.	822.9
24 43	55610.b	11521.	24.0	33315.	55927.	1.046	29262.	23875.	54348.	813.4
24 34	55598.b	11521.	1.0	33309.	55607.	1.046	29083.	24329.	54348.	813.4
3 24	55590.b	11521.	1.0	33305.	55595.	1.046	29068.	24367.	54348.	813.4
12 32	28939.		159.1		57491.	1.000	28745.	25217.	54348.	826.0
4 32	28514.		2.8		57469.	1.000	28735.	25246.	54348.	766.8
18 32	28086.		163.9		56638.	1.000	28319.	26401.	54348.	824.1
23 31	54998.b	7386.	28.5	35423.	55107.	1.024	28212.	26708.	54348.	804.7

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File No.: VY-16Q-311

Revision: 0

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 CALCULATION NUMBER 2 CODE SECTION III CLASS 1 ASME-1998 KRE 2007/07/11 11:35:07
 Vermont Yankee Feedwater PipingSI Fatigue Analysis

FATIGUE ANALYSIS AT POINT 155, LR ELBOW 155 TO 160.

 INDIVIDUAL STRESS RANGE CHECK

 DELTA T1 IN DEGREES
 STRESSES IN PSI

LOAD SET PAIR	SN EQN.10	SE EQN.12	DELTA T1 RANGE	EQN.13	SP EQN.11	KE	SALT EQN.14	ALLOW CYCLES	3*SM	ALLOWABLE FOR DELTA T1 RANGE
4 18	25540.		166.7		54495.	1.000	27247.	29720.	54348.	824.1
4 12	25536.		161.9		54491.	1.000	27245.	29726.	54348.	826.0
23 43	53267.g	6649.	25.0	34429.	53664.	1.000	26832.	31155.	54348.	804.7
23 34	53256.g	6649.	1.9	34423.	53344.	1.000	26672.	31732.	54348.	804.7
3 23	53247.g	6649.	1.9	34419.	53332.	1.000	26666.	31754.	54348.	804.7
22 42	52114.g	9899.	29.7	31901.	52341.	1.000	26170.	33637.	54348.	821.2
24 27	52078.g	13808.	5.8	30903.	52277.	1.000	26138.	33763.	58680.	833.3
5 22	36363.		79.1		51414.	1.000	25707.	35533.	54348.	827.0
24 30	51025.g	13352.	2.8	30306.	51123.	1.000	25561.	36158.	58680.	833.3
20 22	50097.g	363.	1.1	35661.	50120.	1.000	25060.	38425.	54348.	766.8
23 27	49736.g	8936.	6.7	32017.	50014.	1.000	25007.	38676.	58680.	825.8
22 25	49174.g	5006.	42.7	33593.	49190.	1.000	24595.	40698.	53580.	763.1
23 30	48683.g	8480.	3.8	31420.	48860.	1.000	24430.	41549.	58680.	825.8
28 36	46729.g	12029.	34.5	29118.	47343.	1.000	23672.	45771.	53580.	821.0
2 28	46729.g	12029.	31.3	29118.	46750.	1.000	23375.	47577.	53580.	849.3
28 38	46729.g	12029.	31.3	29118.	46750.	1.000	23375.	47577.	53580.	849.3
31 36	46012.g	12259.	30.3	28644.	46633.	1.000	23317.	47944.	54348.	824.1
22 33	45280.g	6649.	15.6	28778.	46192.	1.000	23096.	49364.	54348.	819.2
2 31	46012.g	12259.	27.2	28644.	46040.	1.000	23020.	49865.	54348.	851.7
31 38	46012.g	12259.	27.2	28644.	46040.	1.000	23020.	49865.	54348.	851.7
24 42	45494.		29.6		45720.	1.000	22860.	51538.	60000.	856.2
12 22	45390.g	13603.	7.4	26687.	45392.	1.000	22696.	53409.	54348.	826.0
36 43	44282.g	11521.	26.8	27651.	45190.	1.000	22595.	54602.	54348.	824.1
34 36	44270.g	11521.	3.7	27645.	44871.	1.000	22435.	56559.	54348.	824.1
3 36	44262.g	11521.	3.7	27641.	44859.	1.000	22429.	56635.	54348.	824.1
5 24	29744.		79.0		44794.	1.000	22397.	57040.	60000.	860.7

 Notes b,d,e,k: Fails
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Revision: 0

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Vermont Yankee

Version 3.5.1+026 PC-EXE

Release: Jun

 CALCULATION NUMBER 2 CODE SECTION III CLASS 1 ASME-1998
 Vermont Yankee Feedwater PipingSI Fatigue Analysis

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FATIGUE ANALYSIS AT POINT 155, LR ELBOW 155 TO 160

 INDIVIDUAL STRESS RANGE CHECK

 DELTA T1 IN DEGREES
 STRESSES IN PSI

LOAD SET PAIR	SN EQN.10	SE EQN.12	DELTA T1 RANGE	EQN.13	SP EQN.11	KE	SALT EQN.14	ALLOW CYCLES	3*SM	ALLOWABLE FOR DELTA T1 RANGE
19 22	44599.g	1861.	40.2	32163.	44619.	1.000	22310.	58159.	54348.	792.1
2 43	44282.g	11521.	23.6	27651.	44597.	1.000	22299.	58299.	54348.	851.7
38 43	44282.g	11521.	23.6	27651.	44597.	1.000	22299.	58299.	54348.	851.7
18 22	44533.g	12746.	2.6	26687.	44536.	1.000	22268.	58701.	54348.	824.1
2 34	44270.g	11521.	0.5	27645.	44278.	1.000	22139.	60417.	54348.	851.7
34 38	44270.g	11521.	0.5	27645.	44278.	1.000	22139.	60417.	54348.	851.7
2 3	44262.g	11521.	0.5	27641.	44266.	1.000	22133.	60499.	54348.	851.7
3 38	44262.g	11521.	0.5	27641.	44266.	1.000	22133.	60499.	54348.	851.7
23 42	43153.		30.5		43458.	1.000	21729.	66286.	60000.	849.3
20 24	43368.		1.0		43390.	1.000	21695.	66803.	54348.	813.4
5 23	27401.		79.9		42530.	1.000	21265.	73771.	60000.	853.9
22 37	42394.		0.6		42396.	1.000	21198.	74933.	54348.	788.7
27 36	40750.		8.5		41540.	1.000	20770.	82915.	58680.	841.7
28 37	41509.		31.3		41529.	1.000	20765.	83019.	53580.	784.2
37 43	41048.		23.6		41364.	1.000	20682.	84683.	54348.	788.7
20 23	41038.		2.0		41138.	1.000	20569.	87007.	54348.	804.7
34 37	41036.		0.5		41044.	1.000	20522.	88005.	54348.	788.7
3 37	41028.		0.5		41032.	1.000	20516.	88134.	54348.	788.7
2 27	40750.		5.3		40947.	1.000	20473.	89044.	58680.	868.4
27 38	40750.		5.3		40947.	1.000	20473.	89044.	58680.	868.4
31 37	40792.		27.2		40820.	1.000	20410.	90429.	54348.	788.7
30 36	39697.		5.6		40386.	1.000	20193.	95348.	58680.	841.7
2 30	39697.		2.4		39793.	1.000	19897.	101886.	58680.	868.4
30 38	39697.		2.4		39793.	1.000	19897.	101886.	58680.	868.4
24 33	38661.		15.5		39572.	1.000	19786.	103950.	60000.	854.6
12 24	38769.		7.5		38770.	1.000	19385.	111907.	60000.	860.0

 Notes b,d,e,k: Fails
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Vermont Yankee

Version 3.5.1+026 PC-EXE

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CALCULATION NUMBER 2 CODE SECTION III CLASS 1 ASME-1998
Vermont Yankee Feedwater PipingSI Fatigue Analysis

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FATIGUE ANALYSIS AT POINT 155, LR ELBOW 155 TO 160

INDIVIDUAL STRESS RANGE CHECK

DELTA T1 IN DEGREES
STRESSES IN PSI

LOAD SET PAIR	SN EQN.10	SE EQN.12	DELTA T1 RANGE	EQN.13	SP EQN.11	KE	SALT EQN.14	ALLOW CYCLES	3*SM	ALLOWABLE FOR DELTA T1 RANGE
1 22	38714.		0.8		38766.	1.000	19383.	111954.	54348.	851.7
1 28	38385.		31.1		38406.	1.000	19203.	115784.	53580.	849.3
19 24	37973.		40.1		37992.	1.000	18996.	120392.	58680.	833.3
18 24	37912.		2.7		37914.	1.000	18957.	121286.	60000.	858.4
1 31	37668.		26.9		37696.	1.000	18848.	123834.	54348.	851.7
1 43	37368.		23.3		37683.	1.000	18842.	123981.	54348.	851.7
23 25	37549.		43.5		37644.	1.000	18822.	124447.	53580.	801.6
1 34	37356.		0.3		37364.	1.000	18682.	127846.	54348.	851.7
1 3	37348.		0.3		37352.	1.000	18676.	127996.	54348.	851.7
25 28	37321.		10.8		37342.	1.000	18671.	128119.	53580.	759.3
16 22	34887.		13.6		37322.	1.000	18661.	128359.	54348.	801.4
23 33	36319.		16.4		37309.	1.000	18655.	128517.	60000.	847.8
25 43	36861.		18.6		37176.	1.000	18588.	130181.	53580.	763.1
21 22	37098.		0.6		37100.	1.000	18550.	131149.	54348.	766.8
28 40	36950.		31.3		36971.	1.000	18485.	132812.	53580.	798.3
25 34	36849.		41.6		36857.	1.000	18428.	134297.	53580.	763.1
3 25	36841.		41.6		36845.	1.000	18422.	134456.	53580.	763.1
13 22	36074.		3.9		36696.	1.000	18348.	136430.	54348.	826.0
28 35	30200.		65.8		36655.	1.000	18328.	136979.	53580.	798.3
25 31	36604.		15.0		36632.	1.000	18316.	137291.	53580.	763.1
22 44	36626.		0.6		36628.	1.000	18314.	137342.	54348.	817.3
12 23	36429.		6.5		36509.	1.000	18255.	138959.	60000.	853.1
12 28	34857.		39.3		36357.	1.000	18179.	141064.	53580.	822.9
31 40	36233.		27.2		36261.	1.000	18130.	142421.	54348.	801.4
17 28	29626.		65.8		36082.	1.000	18041.	144986.	53580.	798.3
11 22	36074.		0.7		36077.	1.000	18038.	145056.	54348.	801.4

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Vermont Yankee

Version 3.5.1+026 PC-EXE

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 CALCULATION NUMBER 2 CODE SECTION III CLASS 1 ASME-1998 KRE
 Vermont Yankee Feedwater PipingSI Fatigue Analysis

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FATIGUE ANALYSIS AT POINT 155, LR ELBOW 155 TO 160

 INDIVIDUAL STRESS RANGE CHECK

 DELTA T1 IN DEGREES
 STRESSES IN PSI

LOAD SET PAIR	SN EQN.10	SE EQN.12	DELTA T1 RANGE	EQN.13	SP EQN.11	KE	SALT EQN.14	ALLOW CYCLES	3*SM	ALLOWABLE FOR DELTA T1 RANGE
15 22	36074.		25.1		36077.	1.000	18038.	145056.	54348.	801.4
28 41	35960.		31.3		35981.	1.000	17991.	146447.	53580.	798.3
6 28	35960.		30.0		35981.	1.000	17991.	146449.	53580.	798.3
8 28	35960.		29.6		35981.	1.000	17991.	146449.	53580.	798.3
10 28	35960.		30.2		35981.	1.000	17991.	146449.	53580.	798.3
14 28	35960.		30.0		35981.	1.000	17991.	146449.	53580.	798.3
16 28	35960.		18.3		35981.	1.000	17991.	146449.	53580.	798.3
31 35	29482.		61.6		35944.	1.000	17972.	146991.	54348.	801.4
30 37	35668.		2.4		35764.	1.000	17882.	149673.	58680.	819.8
19 23	35632.		41.1		35730.	1.000	17865.	150191.	58680.	825.8
27 37	35529.		5.3		35726.	1.000	17863.	150253.	58680.	819.8
22 39	35660.		0.6		35662.	1.000	17831.	151224.	54348.	793.6
18 23	35572.		1.8		35653.	1.000	17826.	151370.	60000.	851.6
17 31	28909.		61.6		35371.	1.000	17686.	155755.	54348.	801.4
31 41	35243.		27.2		35272.	1.000	17636.	157347.	54348.	801.4
6 31	35243.		25.8		35271.	1.000	17636.	157350.	54348.	801.4
8 31	35243.		25.5		35271.	1.000	17636.	157350.	54348.	801.4
10 31	35243.		26.0		35271.	1.000	17636.	157350.	54348.	801.4
14 31	35243.		25.9		35271.	1.000	17636.	157350.	54348.	801.4
16 31	35243.		14.1		35271.	1.000	17636.	157350.	54348.	801.4
12 43	33425.		31.5		35220.	1.000	17610.	158176.	54348.	826.0
9 22	35211.		0.6		35213.	1.000	17606.	158293.	54348.	801.4
8 22	34887.		2.3		35206.	1.000	17603.	158403.	54348.	801.4
12 31	33681.		35.1		35189.	1.000	17595.	158680.	54348.	826.0
6 22	34887.		1.9		35141.	1.000	17570.	159466.	54348.	801.4
14 22	34887.		1.9		35132.	1.000	17566.	159612.	54348.	801.4

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Vermont Yankee

Version 3.5.1+026 PC-EXE

Release: Jun

 CALCULATION NUMBER 2 CODE SECTION III CLASS 1 ASME-1998
 Vermont Yankee Feedwater PipingSI Fatigue Analysis

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FATIGUE ANALYSIS AT POINT 155, LR ELBOW 155 TO 160

 INDIVIDUAL STRESS RANGE CHECK

 DELTA T1 IN DEGREES
 STRESSES IN PSI

LOAD SET PAIR	SN EQN.10	SE EQN.12	DELTA T1 RANGE	EQN.13	SP EQN.11	KE	SALT EQN.14	ALLOW CYCLES	3*SM	ALLOWABLE FOR DELTA T1 RANGE
10 22	34887.		1.7		35104.	1.000	17552.	160075.	54348.	801.4
17 22	35068.		33.9		35070.	1.000	17535.	160627.	54348.	801.4
36 42	34166.		32.3		34983.	1.000	17492.	162071.	60000.	864.5
12 34	33413.		8.5		34900.	1.000	17450.	163460.	54348.	826.0
22 41	34887.		0.6		34889.	1.000	17445.	163651.	54348.	801.4
3 12	33405.		8.5		34888.	1.000	17444.	163664.	54348.	826.0
40 43	34503.		23.6		34819.	1.000	17409.	164848.	54348.	801.4
22 40	34759.		0.6		34761.	1.000	17381.	165831.	54348.	801.4
28 39	34652.		31.3		34673.	1.000	17337.	167354.	53580.	787.5
18 28	34000.		34.5		34614.	1.000	17307.	168392.	53580.	821.0
39 43	34193.		23.6		34509.	1.000	17254.	170242.	54348.	793.6
35 43	27756.		58.0		34506.	1.000	17253.	170292.	54348.	801.4
34 40	34491.		0.5		34499.	1.000	17249.	170419.	54348.	801.4
22 35	34495.		33.9		34498.	1.000	17249.	170442.	54348.	801.4
3 40	34483.		0.5		34487.	1.000	17243.	170635.	54348.	801.4
2 42	34166.		29.2		34390.	1.000	17195.	172368.	60000.	893.7
38 42	34166.		29.2		34390.	1.000	17195.	172368.	60000.	893.7
34 39	34181.		0.5		34189.	1.000	17095.	176049.	54348.	793.6
34 35	27745.		35.0		34186.	1.000	17093.	176101.	54348.	801.4
3 39	34173.		0.5		34177.	1.000	17089.	176274.	54348.	793.6
3 35	27736.		35.0		34174.	1.000	17087.	176326.	54348.	801.4
5 36	18416.		81.7		34057.	1.000	17029.	178513.	60000.	869.1
22 45	33979.		0.6		33981.	1.000	16990.	179967.	54348.	817.3
31 39	33935.		27.2		33963.	1.000	16982.	180307.	54348.	793.6
17 43	27181.		58.0		33931.	1.000	16966.	180916.	54348.	801.4
41 43	33514.		23.6		33829.	1.000	16915.	182887.	54348.	801.4

 Notes b,d,e,k: Fails
 g: Weld ISI
 h,i: Rupture Location
 L: Information

File No.: VY-16Q-311

Revision: 0

++ DST/PIPESTRESS ++

Vermont Yankee

Version 3.5.1+026 PC-EXE

Release: Jun

 CALCULATION NUMBER 2 CODE SECTION III CLASS 1 ASME-1998
 Vermont Yankee Feedwater PipingSI Fatigue Analysis

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FATIGUE ANALYSIS AT POINT 155, LR ELBOW 155 TO 160

 INDIVIDUAL STRESS RANGE CHECK

 DELTA T1 IN DEGREES
 STRESSES IN PSI

LOAD SET PAIR	SN EQN.10	SE EQN.12	DELTA T1 RANGE	EQN.13	SP EQN.11	KE	SALT EQN.14	ALLOW CYCLES	3*SM	ALLOWABLE FOR DELTA T1 RANGE
6 43	33514.		22.2		33829.	1.000	16915.	182890.	54348.	801.4
8 43	33514.		21.9		33829.	1.000	16915.	182890.	54348.	801.4
10 43	33514.		22.4		33829.	1.000	16915.	182890.	54348.	801.4
14 43	33514.		22.3		33829.	1.000	16915.	182890.	54348.	801.4
16 43	33514.		10.5		33829.	1.000	16915.	182890.	54348.	801.4
17 34	27170.		35.0		33612.	1.000	16806.	187194.	54348.	801.4
3 17	27161.		35.0		33599.	1.000	16800.	187437.	54348.	801.4
34 41	33502.		0.5		33510.	1.000	16755.	189252.	54348.	801.4
6 34	33502.		0.8		33510.	1.000	16755.	189256.	54348.	801.4
8 34	33502.		1.2		33510.	1.000	16755.	189256.	54348.	801.4
10 34	33502.		0.6		33510.	1.000	16755.	189256.	54348.	801.4
14 34	33502.		0.8		33510.	1.000	16755.	189256.	54348.	801.4
16 34	33502.		12.5		33510.	1.000	16755.	189256.	54348.	801.4
7 22	33497.		1.1		33499.	1.000	16750.	189470.	54348.	801.4
3 41	33494.		0.5		33498.	1.000	16749.	189499.	54348.	801.4
3 6	33494.		0.8		33497.	1.000	16749.	189503.	54348.	801.4
3 8	33494.		1.2		33497.	1.000	16749.	189503.	54348.	801.4
3 10	33494.		0.6		33497.	1.000	16749.	189503.	54348.	801.4
3 14	33494.		0.8		33497.	1.000	16749.	189503.	54348.	801.4
3 16	33494.		12.5		33497.	1.000	16749.	189503.	54348.	801.4
18 43	32568.		26.8		33477.	1.000	16738.	189930.	54348.	824.1
2 5	18416.		78.5		33465.	1.000	16732.	190176.	60000.	905.8
5 38	18416.		78.5		33465.	1.000	16732.	190176.	60000.	905.8
18 31	32824.		30.3		33445.	1.000	16723.	190566.	54348.	824.1
15 28	28620.		57.0		33425.	1.000	16713.	190984.	53580.	798.3
18 34	32556.		3.7		33157.	1.000	16578.	196612.	54348.	824.1

 Notes b,d,e,k: Fails
 g: Weld ISI
 h,i: Rupture Location
 L: Information

File No.: VY-16Q-311

Revision: 0

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Vermont Yankee

Version 3.5.1+026 PC-EXE

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 CALCULATION NUMBER 2 CODE SECTION III CLASS 1 ASME-1998
 Vermont Yankee Feedwater PipingSI Fatigue Analysis

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FATIGUE ANALYSIS AT POINT 155, LR ELBOW 155 TO 160

 INDIVIDUAL STRESS RANGE CHECK

 DELTA T1 IN DEGREES
 STRESSES IN PSI

LOAD SET PAIR	SN. EQN.10	SE EQN.12	DELTA T1 RANGE	EQN.13	SP EQN.11	KE	SALT EQN.14	ALLOW CYCLES	3*SM	ALLOWABLE FOR DELTA T1 RANGE
3 18	32548.		3.7		33145.	1.000	16572.	196871.	54348.	824.1
12 20	31318.		8.5		32817.	1.000	16409.	205131.	54348.	826.0
24 25	32755.		42.6		32771.	1.000	16386.	206456.	53580.	810.3
15 31	27902.		52.8		32714.	1.000	16357.	208114.	54348.	801.4
20 28	32655.		30.7		32675.	1.000	16338.	209234.	53580.	763.1
20 36	32040.		3.8		32653.	1.000	16327.	209888.	54348.	824.1
1 27	32405.		5.1		32602.	1.000	16301.	211385.	58680.	868.4
12 25	30973.		50.1		32467.	1.000	16233.	215442.	53580.	822.9
2 22	32275.		0.6		32278.	1.000	16139.	221264.	54348.	851.7
22 24	32275.		0.1		32278.	1.000	16139.	221264.	54348.	813.4
22 36	32275.		2.6		32278.	1.000	16139.	221264.	54348.	824.1
22 38	32275.		0.6		32278.	1.000	16139.	221264.	54348.	851.7
1 30	31988.		2.1		32084.	1.000	16042.	227424.	58680.	868.4
2 20	32040.		0.6		32060.	1.000	16030.	228208.	54348.	851.7
20 38	32040.		0.6		32060.	1.000	16030.	228208.	54348.	851.7
20 31	31938.		26.6		31966.	1.000	15983.	231302.	54348.	766.8
25 30	31481.		39.8		31577.	1.000	15789.	244590.	53580.	788.7
7 28	31219.		33.0		31557.	1.000	15779.	245285.	53580.	798.3
25 27	31341.		36.8		31538.	1.000	15769.	245964.	53580.	788.7
15 43	26180.		49.2		31279.	1.000	15640.	255410.	54348.	801.4
28 45	31220.		31.3		31241.	1.000	15620.	256841.	53580.	814.2
28 42	31152.		2.2		31173.	1.000	15587.	259394.	53580.	818.0
27 40	30970.		5.3		31167.	1.000	15583.	259631.	58680.	822.5
37 42	30932.		29.2		31156.	1.000	15578.	260037.	60000.	868.4
18 20	30461.		3.8		31074.	1.000	15537.	263206.	54348.	824.1
19 28	31051.		8.4		31071.	1.000	15536.	263297.	53580.	788.7

 Notes b,d,e,k: Fails
 g: Weld ISI
 h,i: Rupture Location
 L: Information

File No.: VY-16Q-311

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Vermont Yankee

Version 3.5.1+026 PC-EXE

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 CALCULATION NUMBER 2 CODE SECTION III CLASS 1 ASME-1998
 Vermont Yankee Feedwater PipingSI Fatigue Analysis

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FATIGUE ANALYSIS AT POINT 155, LR ELBOW 155 TO 160

 INDIVIDUAL STRESS RANGE CHECK

 DELTA T1 IN DEGREES
 STRESSES IN PSI

LOAD SET PAIR	SN EQN.10	SE EQN.12	DELTA T1 RANGE	EQN.13	SP EQN.11	KE	SALT EQN.14	ALLOW CYCLES	3*SM	ALLOWABLE FOR DELTA T1 RANGE
15 34	26168.		26.2		30959.	1.000	15480.	267678.	54348.	801.4
3 15	26159.		26.2		30947.	1.000	15474.	268157.	54348.	801.4
27 35	24217.		39.8		30848.	1.000	15424.	272130.	58680.	822.5
7 31	30502.		28.9		30847.	1.000	15424.	272146.	54348.	801.4
23 37	30758.		1.4		30838.	1.000	15419.	272503.	60000.	838.6
18 25	30116.		45.3		30723.	1.000	15362.	277200.	53580.	821.0
28 33	30567.		16.3		30588.	1.000	15294.	282853.	53580.	816.1
20 43	30208.		23.0		30523.	1.000	15262.	285593.	54348.	766.8
31 45	30493.		27.2		30521.	1.000	15260.	285693.	54348.	817.3
21 24	30480.		0.5		30481.	1.000	15241.	287400.	54348.	813.4
19 31	30333.		12.5		30362.	1.000	15181.	292600.	54348.	792.1
33 37	29424.		15.0		30334.	1.000	15167.	293826.	60000.	860.6
17 27	23644.		39.8		30276.	1.000	15138.	296416.	58680.	822.5
5 37	15182.		78.5		30231.	1.000	15115.	298432.	60000.	892.0
20 34	30196.		0.0		30203.	1.000	15102.	299660.	54348.	766.8
3 20	30188.		0.1		30191.	1.000	15096.	300210.	54348.	766.8
27 41	29980.		5.3		30178.	1.000	15089.	300835.	58680.	822.5
6 27	29980.		4.0		30177.	1.000	15089.	300844.	58680.	822.5
8 27	29980.		3.6		30177.	1.000	15089.	300844.	58680.	822.5
10 27	29980.		4.2		30177.	1.000	15089.	300844.	58680.	822.5
14 27	29980.		4.0		30177.	1.000	15089.	300844.	58680.	822.5
16 27	29980.		7.7		30177.	1.000	15089.	300844.	58680.	822.5
13 24	29455.		3.8		30076.	1.000	15038.	305510.	60000.	860.0
42 43	29721.		5.6		30036.	1.000	15018.	307361.	54348.	821.2
30 40	29917.		2.4		30014.	1.000	15007.	308417.	58680.	822.5
31 42	29977.		2.0		30005.	1.000	15003.	308818.	54348.	821.2

 Notes b,d,e,k: Falls
 g: Weld ISI
 h,i: Rupture Location
 L: Information

File No.: VY-16Q-311

Revision: 0

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Vermont Yankee

Version 3.5.1+026 PC-EXE

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 Vermont Yankee Feedwater PipingSI Fatigue Analysis

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FATIGUE ANALYSIS AT POINT 155, LR ELBOW 155 TO 160

 INDIVIDUAL STRESS RANGE CHECK

 DELTA T1 IN DEGREES
 STRESSES IN PSI

LOAD SET PAIR	SN EQN.10	SE EQN.12	DELTA T1 RANGE	EQN.13	SP EQN.11	KE	SALT EQN.14	ALLOW CYCLES	3*SM	ALLOWABLE FOR DELTA T1 RANGE
24 44	29966.		0.5		29968.	1.000	14984.	310575.	60000.	853.1
9 28	29486.		32.5		29721.	1.000	14861.	322516.	53580.	798.3
34 42	29709.		28.6		29716.	1.000	14858.	322754.	54348.	821.2
3 42	29701.		28.6		29704.	1.000	14852.	323355.	54348.	821.2
30 35	23164.		36.8		29695.	1.000	14847.	323829.	58680.	822.5
11 24	29455.		0.8		29457.	1.000	14728.	335946.	60000.	840.6
15 24	29455.		25.2		29457.	1.000	14728.	335946.	60000.	840.6
33 43	29133.		8.6		29449.	1.000	14724.	336356.	54348.	819.2
31 33	29392.		12.1		29420.	1.000	14710.	337846.	54348.	819.2
7 43	28774.		25.3		29406.	1.000	14703.	338595.	54348.	801.4
33 34	29122.		14.5		29129.	1.000	14565.	353547.	54348.	819.2
17 30	22592.		36.8		29122.	1.000	14561.	353924.	58680.	822.5
3 33	29113.		14.5		29117.	1.000	14559.	354220.	54348.	819.2
43 45	28797.		23.6		29112.	1.000	14556.	354495.	54348.	817.3
7 34	28762.		2.2		29086.	1.000	14543.	355926.	54348.	801.4
3 7	28753.		2.2		29074.	1.000	14537.	356604.	54348.	801.4
30 41	28928.		2.4		29024.	1.000	14512.	359424.	58680.	822.5
6 30	28928.		1.0		29024.	1.000	14512.	359435.	58680.	822.5
8 30	28928.		0.7		29024.	1.000	14512.	359435.	58680.	822.5
10 30	28928.		1.2		29024.	1.000	14512.	359435.	58680.	822.5
14 30	28928.		1.1		29024.	1.000	14512.	359435.	58680.	822.5
16 30	28928.		10.7		29024.	1.000	14512.	359435.	58680.	822.5
9 31	28768.		28.3		29011.	1.000	14505.	360192.	54348.	801.4
2 12	27441.		7.9		28921.	1.000	14460.	365335.	60000.	903.8
12 36	27441.		4.8		28921.	1.000	14460.	365335.	60000.	868.4
12 38	27441.		7.9		28921.	1.000	14460.	365335.	60000.	903.8

 Notes b,d,e,k: Fails
 g: Weld ISI
 h,i: Rupture Location
 L: Information

File No.: VY-16Q-311

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FATIGUE ANALYSIS AT POINT 155, LR ELBOW 155 TO 160

 INDIVIDUAL STRESS RANGE CHECK

 DELTA T1 IN DEGREES
 STRESSES IN PSI

LOAD SET PAIR	SN EQN.10	SE EQN.12	DELTA T1 RANGE	EQN.13	SP EQN.11	KE	SALT EQN.14	ALLOW CYCLES	3*SM	ALLOWABLE FOR DELTA T1 RANGE
19 43	28604.		16.1		28920.	1.000	14460.	365397.	54348.	792.1
30 39	28811.		2.4		28907.	1.000	14454.	366116.	58680.	822.5
11 28	28620.		32.6		28884.	1.000	14442.	367461.	53580.	798.3
27 39	28671.		5.3		28868.	1.000	14434.	368386.	58680.	822.5
20 35	22397.		35.0		28851.	1.000	14425.	369387.	54348.	801.4
33 36	27333.		18.2		28836.	1.000	14418.	370288.	60000.	863.0
20 37	28806.		0.6		28826.	1.000	14413.	370834.	54348.	788.7
34 45	28785.		0.5		28792.	1.000	14396.	372829.	54348.	817.3
3 45	28777.		0.5		28780.	1.000	14390.	373546.	54348.	817.3
27 43	28412.		18.2		28727.	1.000	14364.	376708.	54348.	792.1
13 28	28620.		28.0		28641.	1.000	14321.	381903.	53580.	822.9
19 34	28592.		39.1		28600.	1.000	14300.	384425.	54348.	792.1
9 24	28590.		0.7		28591.	1.000	14296.	384960.	60000.	840.6
3 19	28584.		39.2		28588.	1.000	14294.	385169.	54348.	792.1
25 35	22050.		76.6		28499.	1.000	14249.	390691.	53580.	798.3
28 44	28431.		31.3		28451.	1.000	14226.	393669.	53580.	814.2
17 24	28447.		34.0		28449.	1.000	14224.	393854.	60000.	840.6
27 34	28400.		4.8		28407.	1.000	14204.	396461.	54348.	792.1
3 27	28392.		4.8		28395.	1.000	14198.	397234.	54348.	792.1
5 28	28333.		47.2		28354.	1.000	14177.	399912.	53580.	823.9
2 33	27333.		15.0		28243.	1.000	14121.	407140.	60000.	889.7
33 38	27333.		15.0		28243.	1.000	14121.	407140.	60000.	889.7
21 23	28137.		1.4		28218.	1.000	14109.	408793.	54348.	804.7
11 31	27902.		28.5		28173.	1.000	14086.	411773.	54348.	801.4
28 30	27997.		28.9		28018.	1.000	14009.	422270.	53580.	788.7
13 31	27902.		23.8		27930.	1.000	13965.	428376.	54348.	826.0

 Notes b,d,e,k: Fails.
 g: Weld ISI
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Vermont Yankee

Version 3.5.1+026 PC-EXE

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FATIGUE ANALYSIS AT POINT 155, LR ELBOW 155 TO, 160

 INDIVIDUAL STRESS RANGE CHECK

 DELTA T1 IN DEGREES
 STRESSES IN PSI

LOAD SET PAIR	SN EQN.10	SE EQN.12	DELTA T1 RANGE	EQN.13	SP EQN.11	KE	SALT EQN.14	ALLOW CYCLES	3*SM	ALLOWABLE FOR DELTA T1 RANGE
27 28	27856.		26.0		27877.	1.000	13939.	432081.	53580.	788.7
24 35	27875.		34.0		27877.	1.000	13938.	432125.	60000.	840.6
13 23	27112.		4.7		27812.	1.000	13906.	436721.	60000.	853.1
23 44	27648.		1.4		27729.	1.000	13864.	442732.	60000.	846.3
31 44	27673.		27.2		27701.	1.000	13851.	444748.	54348.	817.3
30 43	27360.		21.2		27675.	1.000	13838.	446650.	54348.	792.1
5 31	27614.		51.4		27642.	1.000	13821.	449150.	54348.	827.0
20 42	27614.		28.6		27634.	1.000	13817.	449757.	54348.	821.2
15 27	22635.		31.0		27616.	1.000	13808.	451074.	58680.	822.5
3 28	27578.		30.8		27599.	1.000	13799.	452344.	53580.	763.1
21 28	27578.		31.3		27599.	1.000	13799.	452344.	53580.	763.1
28 34	27578.		30.8		27599.	1.000	13799.	452344.	53580.	763.1
28 43	27578.		7.7		27599.	1.000	13799.	452344.	53580.	763.1
9 43	27041.		24.7		27572.	1.000	13786.	454392.	54348.	801.4
1 42	27252.		28.9		27476.	1.000	13738.	461639.	60000.	893.7
17 20	20982.		35.0		27436.	1.000	13718.	464734.	54348.	801.4
30 34	27348.		1.8		27356.	1.000	13678.	470989.	54348.	792.1
3 30	27340.		1.9		27344.	1.000	13672.	471943.	54348.	792.1
25 42	27269.		13.0		27283.	1.000	13642.	476732.	53580.	818.0
24 39	27275.		0.5		27276.	1.000	13638.	477271.	60000.	840.6
19 36	26645.		42.9		27255.	1.000	13628.	478974.	58680.	841.7
9 34	27030.		1.7		27252.	1.000	13626.	479249.	54348.	801.4
3 9	27021.		1.7		27240.	1.000	13620.	480224.	54348.	801.4
1 23	27078.		1.7		27208.	1.000	13604.	482793.	60000.	878.3
11 23	27112.		0.1		27193.	1.000	13597.	483997.	60000.	833.8
15 23	27112.		24.3		27193.	1.000	13597.	483997.	60000.	833.8

 Notes b,d,e,k: Fails
 g: Weld ISI
 h,i: Rupture Location
 L: Information

File No.: VY-16Q-311

Revision: 0

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Vermont Yankee

Version 3.5.1+026 FC-EXE

Release: Jun

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FATIGUE ANALYSIS AT POINT 155, LR ELBOW 155 TO 160

 INDIVIDUAL STRESS RANGE CHECK

 DELTA T1 IN DEGREES
 STRESSES IN PSI

LOAD SET PAIR	SN EQN.10	SE EQN.12	DELTA T1 RANGE	EQN.13	SP EQN.11	KE	SALT EQN.14	ALLOW CYCLES	3*SM	ALLOWABLE FOR DELTA T1 RANGE
18 36	26584.		0.0		27177.	1.000	13589.	485280.	60000.	866.8
2 18	26584.		3.2		27177.	1.000	13589.	485286.	60000.	899.7
18 38	26584.		3.2		27177.	1.000	13589.	485286.	60000.	899.7
31 43	26861.		3.6		27177.	1.000	13588.	485321.	54348.	766.8
24 45	27103.		0.5		27105.	1.000	13552.	491227.	60000.	853.1
22 23	27100.		0.8		27103.	1.000	13551.	491420.	54348.	804.7
17 25	20635.		76.6		27084.	1.000	13542.	492958.	53580.	798.3
20 33	27026.		14.4		27046.	1.000	13523.	496131.	54348.	819.2
12 27	25319.		13.3		26996.	1.000	13498.	500727.	58680.	843.2
3 31	26861.		26.6		26889.	1.000	13445.	518868.	54348.	766.8
21 31	26861.		27.2		26889.	1.000	13445.	518868.	54348.	766.8
31 34	26861.		26.6		26889.	1.000	13445.	518868.	54348.	766.8
20 27	26676.		4.8		26872.	1.000	13436.	521787.	54348.	792.1
7 24	26861.		1.2		26862.	1.000	13431.	523582.	60000.	840.6
30 31	26821.		24.8		26850.	1.000	13425.	525802.	54348.	792.1
15 20	21992.		26.2		26795.	1.000	13398.	535458.	54348.	801.4
12 30	25177.		10.3		26753.	1.000	13376.	543158.	58680.	843.2
11 43	26180.		24.9		26738.	1.000	13369.	545912.	54348.	801.4
27 31	26681.		21.8		26709.	1.000	13355.	551258.	54348.	792.1
25 33	26682.		27.1		26696.	1.000	13348.	553668.	53580.	816.1
2 19	26645.		39.7		26662.	1.000	13331.	560023.	58680.	868.4
19 38	26645.		39.7		26662.	1.000	13331.	560023.	58680.	868.4
1 33	25744.		14.8		26654.	1.000	13327.	561655.	60000.	889.7
1 5	11502.		78.2		26550.	1.000	13275.	581619.	60000.	905.8
13 43	26180.		20.3		26495.	1.000	13248.	592663.	54348.	826.0
43 44	26148.		23.6		26464.	1.000	13232.	599029.	54348.	817.3

 Notes b,d,e,k: Fails
 g: Weld ISI
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 L: Information

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Vermont Yankee

Version 3.5.1+026 PC-EXE

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 CALCULATION NUMBER 2 CODE SECTION III CLASS 1 ASME-1998
 Vermont Yankee Feedwater PipingSI Fatigue Analysis

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FATIGUE ANALYSIS AT POINT 155, LR ELBOW 155 TO 160

 INDIVIDUAL STRESS RANGE CHECK

 DELTA T1 IN DEGREES
 STRESSES IN PSI

LOAD SET PAIR	SN EQN.10	SE EQN.12	DELTA T1 RANGE	EQN.13	SP EQN.11	KE	SALT EQN.14	ALLOW CYCLES	3*SM	ALLOWABLE FOR DELTA T1 RANGE
15 30	21583.		28.0		26463.	1.000	13232.	599165.	58680.	822.5
15 25	21645.		67.8		26444.	1.000	13222.	603148.	53580.	798.3
11 34	26168.		1.8		26418.	1.000	13209.	608383.	54348.	801.4
3 11	26159.		1.8		26406.	1.000	13203.	610902.	54348.	801.4
9 23	26247.		0.2		26328.	1.000	13164.	627410.	60000.	833.8
28 31	26306.		4.2		26327.	1.000	13163.	627701.	53580.	763.1
5 43	25895.		54.9		26211.	1.000	13106.	653074.	54348.	827.0
17 23	26105.		33.1		26185.	1.000	13093.	658887.	60000.	833.8
13 34	26168.		2.8		26175.	1.000	13088.	661145.	54348.	826.0
3 13	26159.		2.8		26163.	1.000	13082.	663907.	54348.	826.0
34 44	26137.		0.5		26144.	1.000	13072.	668334.	54348.	817.3
3 44	26128.		0.5		26132.	1.000	13066.	671130.	54348.	817.3
5 25	21935.		36.4		26033.	1.000	13017.	694402.	53580.	823.9
5 34	25884.		78.0		25891.	1.000	12946.	729405.	54348.	827.0
3 5	25875.		78.0		25879.	1.000	12940.	732486.	54348.	827.0
16 23	23283.		14.4		25797.	1.000	12899.	753773.	60000.	833.8
7 27	25238.		7.0		25752.	1.000	12876.	765629.	58680.	822.5
20 30	25623.		1.8		25719.	1.000	12859.	774672.	54348.	792.1
12 37	24207.		7.9		25687.	1.000	12843.	783405.	60000.	888.0
23 35	25532.		33.1		25613.	1.000	12807.	803942.	60000.	833.8
5 12	9038.		86.5		25567.	1.000	12783.	817215.	60000.	870.6
3 43	25130.		23.1		25446.	1.000	12723.	852773.	54348.	766.8
21 43	25130.		23.6		25446.	1.000	12723.	852773.	54348.	766.8
34 43	25130.		23.0		25446.	1.000	12723.	852773.	54348.	766.8
27 45	25182.		5.3		25379.	1.000	12690.	873244.	58680.	836.3
19 27	25071.		34.3		25268.	1.000	12634.	908503.	58680.	813.2

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Vermont Yankee

Version 3.5.1+026 PC-EXE

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FATIGUE ANALYSIS AT POINT 155, LR ELBOW 155 TO 160

 INDIVIDUAL STRESS RANGE CHECK

 DELTA T1 IN DEGREES
 STRESSES IN PSI

LOAD SET PAIR	SN EQN.10	SE EQN.12	DELTA T1 RANGE	EQN.13	SP EQN.11	KE	SALT EQN.14	ALLOW CYCLES	3*SM	ALLOWABLE FOR DELTA T1 RANGE
18 27	24462.		8.5		25252.	1.000	12626.	913613.	58680.	841.7
23 39	25107.		1.4		25188.	1.000	12594.	934874.	60000.	833.8
1 20	25126.		0.3		25146.	1.000	12573.	948882.	54348.	851.7
3 34	25119.		0.0		25126.	1.000	12563.	955635.	54348.	766.8
21 34	25119.		0.5		25126.	1.000	12563.	955635.	54348.	766.8
3 21	25110.		0.5		25114.	1.000	12557.	959796.	54348.	766.8
18 30	24320.		5.5		25009.	1.000	12505.	996650.	58680.	841.7
23 45	24883.		1.4		24963.	1.000	12482.	>1000000.	60000.	846.3
20 25	24601.		41.6		24621.	1.000	12310.	>1000000.	53580.	763.1
40 42	24388.		29.2		24612.	1.000	12306.	>1000000.	60000.	846.7
16 24	22175.		13.5		24609.	1.000	12305.	>1000000.	60000.	840.6
7 23	24523.		0.3		24604.	1.000	12302.	>1000000.	60000.	833.8
7 30	24186.		4.1		24599.	1.000	12300.	>1000000.	58680.	822.5
39 42	24075.		29.2		24300.	1.000	12150.	>1000000.	60000.	846.7
35 42	17634.		63.6		24292.	1.000	12146.	>1000000.	60000.	846.7
30 45	24139.		2.4		24235.	1.000	12118.	>1000000.	58680.	836.3
12 19	22716.		47.6		24214.	1.000	12107.	>1000000.	58680.	843.2
19 30	24018.		37.3		24114.	1.000	12057.	>1000000.	58680.	813.2
18 37	23350.		3.2		23943.	1.000	11972.	>1000000.	60000.	880.1
9 27	23503.		6.5		23914.	1.000	11957.	>1000000.	58680.	822.5
5 18	8182.		81.7		23823.	1.000	11912.	>1000000.	60000.	869.1
17 42	17061.		63.6		23720.	1.000	11860.	>1000000.	60000.	846.7
8 23	23283.		3.1		23681.	1.000	11841.	>1000000.	60000.	833.8
5 40	8626.		78.5		23674.	1.000	11837.	>1000000.	60000.	851.3
41 42	23398.		29.2		23623.	1.000	11811.	>1000000.	60000.	846.7
6 42	23398.		27.8		23622.	1.000	11811.	>1000000.	60000.	846.7

 Notes b,d,e,k: Fails
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FATIGUE ANALYSIS AT POINT 155, LR ELBOW 155 TO 160

 INDIVIDUAL STRESS RANGE CHECK

 DELTA T1 IN DEGREES
 STRESSES IN PSI

LOAD SET PAIR	SN EQN.10	SE EQN.12	DELTA T1 RANGE	EQN.13	SP EQN.11	KE	SALT EQN.14	ALLOW CYCLES	3*SM	ALLOWABLE FOR DELTA T1 RANGE
8 42	23398.		27.5		23622.	1.000	11811.	>1000000.	60000.	846.7
10 42	23398.		28.0		23622.	1.000	11811.	>1000000.	60000.	846.7
14 42	23398.		27.8		23622.	1.000	11811.	>1000000.	60000.	846.7
16 42	23398.		16.1		23622.	1.000	11811.	>1000000.	60000.	846.7
6 23	23283.		2.7		23616.	1.000	11808.	>1000000.	60000.	833.8
14 23	23283.		2.7		23607.	1.000	11803.	>1000000.	60000.	833.8
10 23	23283.		2.5		23579.	1.000	11789.	>1000000.	60000.	833.8
2 17	17119.		34.4		23553.	1.000	11777.	>1000000.	60000.	875.7
17 36	17119.		31.3		23553.	1.000	11777.	>1000000.	60000.	849.0
17 38	17119.		34.4		23553.	1.000	11777.	>1000000.	60000.	875.7
33 39	22569.		15.0		23479.	1.000	11740.	>1000000.	60000.	845.2
19 37	23411.		39.7		23428.	1.000	11714.	>1000000.	58680.	819.8
23 41	23283.		1.4		23364.	1.000	11682.	>1000000.	60000.	833.8
5 39	8313.		78.5		23362.	1.000	11681.	>1000000.	60000.	851.3
24 37	23355.		0.5		23356.	1.000	11678.	>1000000.	60000.	851.8
5 35	1871.		113.0		23354.	1.000	11677.	>1000000.	60000.	851.3
23 40	23146.		1.4		23227.	1.000	11613.	>1000000.	60000.	833.8
27 33	22925.		9.7		23122.	1.000	11561.	>1000000.	58680.	837.9
11 27	22635.		6.6		23075.	1.000	11537.	>1000000.	58680.	822.5
20 21	23025.		0.6		23045.	1.000	11522.	>1000000.	54348.	766.8
2 35	16547.		34.4		22981.	1.000	11491.	>1000000.	60000.	875.7
35 36	16547.		31.3		22981.	1.000	11491.	>1000000.	60000.	849.0
35 38	16547.		34.4		22981.	1.000	11491.	>1000000.	60000.	875.7
2 15	18127.		25.7		22911.	1.000	11456.	>1000000.	60000.	875.7
15 36	18127.		22.5		22911.	1.000	11456.	>1000000.	60000.	849.0
15 38	18127.		25.7		22911.	1.000	11456.	>1000000.	60000.	875.7

 Notes b,d,e,k: Fails
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FATIGUE ANALYSIS AT POINT 155, LR ELBOW 155 TO 160

 INDIVIDUAL STRESS RANGE CHECK

 DELTA T1 IN DEGREES
 STRESSES IN PSI

LOAD SET PAIR	SN EQN.10	SE EQN.12	DELTA T1 RANGE	EQN.13	SP EQN.11	KE	SALT EQN.14	ALLOW CYCLES	3*SM	ALLOWABLE FOR DELTA T1 RANGE
17 33	15555.		49.5		22900.	1.000	11450.	>1000000.	60000.	845.2
13 27	22635.		2.0		22832.	1.000	11416.	>1000000.	58680.	843.2
33 41	21891.		15.0		22801.	1.000	11400.	>1000000.	60000.	845.2
6 33	21891.		13.7		22801.	1.000	11400.	>1000000.	60000.	845.2
8 33	21891.		13.3		22801.	1.000	11400.	>1000000.	60000.	845.2
10 33	21891.		13.9		22801.	1.000	11400.	>1000000.	60000.	845.2
14 33	21891.		13.7		22801.	1.000	11400.	>1000000.	60000.	845.2
16 33	21891.		2.0		22801.	1.000	11400.	>1000000.	60000.	845.2
5 17	1299.		113.0		22783.	1.000	11391.	>1000000.	60000.	851.3
9 30	22450.		3.5		22761.	1.000	11381.	>1000000.	58680.	822.5
21 25	22681.		42.2		22695.	1.000	11348.	>1000000.	53580.	763.1
5 41	7636.		78.5		22685.	1.000	11342.	>1000000.	60000.	851.3
5 6	7636.		77.2		22685.	1.000	11342.	>1000000.	60000.	851.3
5 8	7636.		76.8		22685.	1.000	11342.	>1000000.	60000.	851.3
5 10	7636.		77.4		22685.	1.000	11342.	>1000000.	60000.	851.3
5 14	7636.		77.2		22685.	1.000	11342.	>1000000.	60000.	851.3
5 16	7636.		65.5		22685.	1.000	11342.	>1000000.	60000.	851.3
33 40	21767.		15.0		22677.	1.000	11339.	>1000000.	60000.	845.2
20 44	22548.		0.6		22567.	1.000	11284.	>1000000.	54348.	817.3
5 27	22346.		73.2		22543.	1.000	11271.	>1000000.	58680.	844.0
8 24	22175.		2.2		22493.	1.000	11247.	>1000000.	60000.	840.6
27 44	22291.		5.3		22488.	1.000	11244.	>1000000.	58680.	836.3
18 19	21860.		42.9		22470.	1.000	11235.	>1000000.	58680.	841.7
6 24	22175.		1.8		22428.	1.000	11214.	>1000000.	60000.	840.6
14 24	22175.		1.8		22419.	1.000	11210.	>1000000.	60000.	840.6
10 24	22175.		1.6		22391.	1.000	11195.	>1000000.	60000.	840.6

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FATIGUE ANALYSIS AT POINT 155, LR ELBOW 155 TO 160

 INDIVIDUAL STRESS RANGE CHECK

 DELTA T1 IN DEGREES
 STRESSES IN PSI

LOAD SET PAIR	SN EQN.10	SE EQN.12	DELTA T1 RANGE	EQN.13	SP EQN.11	KE	SALT EQN.14	ALLOW CYCLES	3*SM	ALLOWABLE FOR DELTA T1 RANGE
5 20	22281.		77.9		22301.	1.000	11150.	>1000000.	54348.	827.0
20 40	22243.		0.6		22263.	1.000	11132.	>1000000.	54348.	801.4
11 20	21992.		1.9		22254.	1.000	11127.	>1000000.	54348.	801.4
24 41	22175.		0.5		22176.	1.000	11088.	>1000000.	60000.	840.6
25 44	22161.		42.2		22175.	1.000	11088.	>1000000.	53580.	814.2
1 12	20527.		8.2		22056.	1.000	11028.	>1000000.	60000.	903.8
25 36	21427.		45.3		22035.	1.000	11017.	>1000000.	53580.	821.0
13 20	21992.		2.8		22011.	1.000	11006.	>1000000.	54348.	826.0
30 33	21873.		12.6		21969.	1.000	10984.	>1000000.	58680.	837.9
20 39	21947.		0.6		21967.	1.000	10983.	>1000000.	54348.	793.6
11 30	21583.		3.7		21922.	1.000	10961.	>1000000.	58680.	822.5
11 25	21645.		43.5		21902.	1.000	10951.	>1000000.	53580.	798.3
27 42	21615.		23.8		21812.	1.000	10906.	>1000000.	58680.	839.4
21 27	21600.		5.3		21797.	1.000	10898.	>1000000.	54348.	792.1
35 37	15300.		34.4		21734.	1.000	10867.	>1000000.	60000.	833.7
13 30	21583.		1.0		21679.	1.000	10840.	>1000000.	58680.	843.2
13 25	21645.		38.8		21660.	1.000	10830.	>1000000.	53580.	822.9
30 42	21473.		26.8		21569.	1.000	10784.	>1000000.	58680.	839.4
12 33	19125.		23.0		21514.	1.000	10757.	>1000000.	60000.	864.5
33 35	14141.		49.5		21486.	1.000	10743.	>1000000.	60000.	845.2
2 25	21427.		42.2		21442.	1.000	10721.	>1000000.	53580.	849.3
25 38	21427.		42.2		21442.	1.000	10721.	>1000000.	53580.	849.3
5 30	21294.		76.1		21390.	1.000	10695.	>1000000.	58680.	844.0
9 20	21124.		1.7		21358.	1.000	10679.	>1000000.	54348.	801.4
30 44	21261.		2.4		21358.	1.000	10679.	>1000000.	58680.	836.3
20 41	21254.		0.6		21274.	1.000	10637.	>1000000.	54348.	801.4

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FATIGUE ANALYSIS AT POINT 155, LR ELBOW 155 TO 160

 INDIVIDUAL STRESS RANGE CHECK

 DELTA T1 IN DEGREES
 STRESSES IN PSI

LOAD SET PAIR	SN EQN.10	SE EQN.12	DELTA T1 RANGE	EQN.13	SP EQN.11	KE	SALT EQN.14	ALLOW CYCLES	3*SM	ALLOWABLE FOR DELTA T1 RANGE
6 20	21254.		0.8		21274.	1.000	10637.	>1000000.	54348.	801.4
8 20	21254.		1.1		21274.	1.000	10637.	>1000000.	54348.	801.4
10 20	21254.		0.6		21274.	1.000	10637.	>1000000.	54348.	801.4
14 20	21254.		0.7		21274.	1.000	10637.	>1000000.	54348.	801.4
16 20	21254.		12.5		21274.	1.000	10637.	>1000000.	54348.	801.4
24 40	21195.		0.5		21197.	1.000	10598.	>1000000.	60000.	840.6
15 42	16052.		54.8		21060.	1.000	10530.	>1000000.	60000.	846.7
9 25	20777.		43.3		21006.	1.000	10503.	>1000000.	53580.	798.3
21 30	20547.		2.4		20643.	1.000	10321.	>1000000.	54348.	792.1
12 16	16674.		21.0		20587.	1.000	10294.	>1000000.	60000.	850.5
5 42	15762.		49.4		20384.	1.000	10192.	>1000000.	60000.	866.8
17 37	13885.		34.4		20319.	1.000	10160.	>1000000.	60000.	833.7
1 18	19670.		3.4		20313.	1.000	10156.	>1000000.	60000.	899.7
19 35	13795.		74.1		20247.	1.000	10124.	>1000000.	58680.	822.5
15 33	14547.		40.7		20241.	1.000	10121.	>1000000.	60000.	845.2
5 15	289.		104.2		20122.	1.000	10061.	>1000000.	60000.	851.3
20 45	19910.		0.6		19930.	1.000	9965.	>1000000.	54348.	817.3
5 33	14258.		63.5		19800.	1.000	9900.	>1000000.	60000.	865.3
18 33	18268.		18.2		19771.	1.000	9885.	>1000000.	60000.	863.0
1 19	19731.		39.4		19748.	1.000	9874.	>1000000.	58680.	868.4
21 36	19151.		3.2		19744.	1.000	9872.	>1000000.	54348.	824.1
7 20	19391.		2.3		19728.	1.000	9864.	>1000000.	54348.	801.4
1 24	19674.		0.7		19726.	1.000	9863.	>1000000.	60000.	885.1
15 37	14893.		25.7		19677.	1.000	9839.	>1000000.	60000.	833.7
19 20	19553.		39.1		19573.	1.000	9786.	>1000000.	54348.	792.1
27 30	19370.		3.0		19567.	1.000	9784.	>1000000.	58680.	813.2

 Notes b,d,e,k: Fails
 g: Weld ISI
 h,i: Rupture Location
 L: Information

 File No.: VY-16Q-311
 Revision: 0

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Vermont Yankee

Version 3.5.1+026 PC-EXE

Release: Jun

 CALCULATION NUMBER 2 CODE SECTION III CLASS 1 ASME-1998
 Vermont Yankee Feedwater PipingSI Fatigue Analysis

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FATIGUE ANALYSIS AT POINT 155, LR ELBOW 155 TO 160

 INDIVIDUAL STRESS RANGE CHECK

 DELTA T1 IN DEGREES
 STRESSES IN PSI

LOAD SET PAIR	SN EQN.10	SE EQN.12	DELTA T1 RANGE	EQN.13	SP EQN.11	KE	SALT EQN.14	ALLOW CYCLES	3*SM	ALLOWABLE FOR DELTA T1 RANGE
2 23	19336.		1.4		19416.	1.000	9708.	>1000000.	60000.	878.3
23 24	19336.		0.9		19416.	1.000	9708.	>1000000.	60000.	843.2
23 36	19336.		1.8		19416.	1.000	9708.	>1000000.	60000.	851.6
23 38	19336.		1.4		19416.	1.000	9708.	>1000000.	60000.	878.3
7 25	19041.		43.9		19373.	1.000	9686.	>1000000.	53580.	798.3
13 36	18127.		6.5		19339.	1.000	9670.	>1000000.	60000.	868.4
25 45	19292.		42.2		19306.	1.000	9653.	>1000000.	53580.	814.2
36 44	18638.		3.2		19231.	1.000	9616.	>1000000.	60000.	861.5
19 25	19209.		2.5		19227.	1.000	9613.	>1000000.	53580.	788.7
7 42	18656.		30.9		19197.	1.000	9599.	>1000000.	60000.	846.7
2 21	19151.		0.0		19151.	1.000	9576.	>1000000.	54348.	851.7
21 38	19151.		0.0		19151.	1.000	9576.	>1000000.	54348.	851.7
12 40	17663.		7.9		19143.	1.000	9572.	>1000000.	60000.	850.5
19 42	19013.		10.5		19030.	1.000	9515.	>1000000.	58680.	839.4
16 18	15818.		16.2		18843.	1.000	9422.	>1000000.	60000.	849.0
17 19	12380.		74.1		18832.	1.000	9416.	>1000000.	58680.	822.5
12 39	17350.		7.9		18830.	1.000	9415.	>1000000.	60000.	850.5
42 45	18564.		29.2		18789.	1.000	9394.	>1000000.	60000.	859.2
2 13	18127.		3.3		18746.	1.000	9373.	>1000000.	60000.	903.8
13 38	18127.		3.3		18746.	1.000	9373.	>1000000.	60000.	903.8
11 36	18127.		1.9		18720.	1.000	9360.	>1000000.	60000.	849.0
2 44	18638.		0.0		18638.	1.000	9319.	>1000000.	60000.	888.2
38 44	18638.		0.0		18638.	1.000	9319.	>1000000.	60000.	888.2
8 12	16674.		9.6		18471.	1.000	9235.	>1000000.	60000.	850.5
19 33	18426.		24.7		18443.	1.000	9222.	>1000000.	58680.	837.9
6 12	16674.		9.3		18406.	1.000	9203.	>1000000.	60000.	850.5

 Notes b,d,e,k: Fails
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Vermont Yankee

Version 3.5.1+026 PC-EXE

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 CALCULATION NUMBER 2 CODE SECTION III CLASS 1 ASME-1998
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FATIGUE ANALYSIS AT POINT 155, LR ELBOW 155 TO 160

 INDIVIDUAL STRESS RANGE CHECK

 DELTA T1 IN DEGREES
 STRESSES IN PSI

LOAD SET PAIR	SN EQN.10	SE EQN.12	DELTA T1 RANGE	EQN.13	SP EQN.11	KE	SALT EQN.14	ALLOW CYCLES	3*SM	ALLOWABLE FOR DELTA T1 RANGE
12 14	16674.		9.2		18397.	1.000	9198.	>1000000.	60000.	850.5
7 33	17149.		16.7		18376.	1.000	9188.	>1000000.	60000.	845.2
2 11	18127.		1.3		18370.	1.000	9185.	>1000000.	60000.	875.7
11 38	18127.		1.3		18370.	1.000	9185.	>1000000.	60000.	875.7
10 12	16674.		9.1		18368.	1.000	9184.	>1000000.	60000.	850.5
5 7	2894.		80.2		18259.	1.000	9130.	>1000000.	60000.	851.3
15 19	13390.		65.3		18192.	1.000	9096.	>1000000.	58680.	822.5
12 41	16674.		7.9		18154.	1.000	9077.	>1000000.	60000.	850.5
1 35	11620.		34.7		18104.	1.000	9052.	>1000000.	60000.	875.7
33 45	17108.		15.0		18018.	1.000	9009.	>1000000.	60000.	857.7
5 45	2922.		78.5		17971.	1.000	8985.	>1000000.	60000.	863.8
9 36	17262.		2.0		17855.	1.000	8927.	>1000000.	60000.	849.0
5 19	13679.		38.8		17777.	1.000	8888.	>1000000.	58680.	844.0
2 9	17262.		1.2		17476.	1.000	8738.	>1000000.	60000.	875.7
9 38	17262.		1.2		17476.	1.000	8738.	>1000000.	60000.	875.7
18 40	16807.		3.2		17400.	1.000	8700.	>1000000.	60000.	849.0
9 42	16920.		30.3		17359.	1.000	8679.	>1000000.	60000.	846.7
12 35	10909.		26.5		17343.	1.000	8672.	>1000000.	60000.	850.5
18 39	16494.		3.2		17087.	1.000	8543.	>1000000.	60000.	849.0
19 40	16851.		39.7		16868.	1.000	8434.	>1000000.	58680.	822.5
12 17	10337.		26.5		16772.	1.000	8386.	>1000000.	60000.	850.5
8 18	15818.		4.9		16727.	1.000	8364.	>1000000.	60000.	849.0
1 17	10205.		34.7		16689.	1.000	8344.	>1000000.	60000.	875.7
6 18	15818.		4.5		16662.	1.000	8331.	>1000000.	60000.	849.0
14 18	15818.		4.5		16653.	1.000	8327.	>1000000.	60000.	849.0
10 18	15818.		4.3		16625.	1.000	8312.	>1000000.	60000.	849.0

 Notes b,d,e,k: Fails
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Vermont Yankee

Version 3.5.1+026 PC-EXE

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 CALCULATION NUMBER 2 CODE SECTION III CLASS 1 ASME-1998
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FATIGUE ANALYSIS AT POINT 155, LR ELBOW 155 TO 160

 INDIVIDUAL STRESS RANGE CHECK

 DELTA T1 IN DEGREES
 STRESSES IN PSI

LOAD SET PAIR	SN EQN.10	SE EQN.12	DELTA T1 RANGE	EQN.13	SP EQN.11	KE	SALT EQN.14	ALLOW CYCLES	3*SM	ALLOWABLE FOR DELTA T1 RANGE
16 35	7752.		47.5		16619.	1.000	8309.	>1000000.	60000.	831.2
25 39	16565.		42.2		16580.	1.000	8290.	>1000000.	53580.	787.5
33 42	16341.		14.1		16565.	1.000	8283.	>1000000.	60000.	860.7
19 39	16541.		39.7		16559.	1.000	8280.	>1000000.	58680.	822.5
36 39	15947.		3.2		16540.	1.000	8270.	>1000000.	60000.	849.0
9 33	15414.		16.2		16539.	1.000	8269.	>1000000.	60000.	845.2
11 42	16052.		30.5		16519.	1.000	8259.	>1000000.	60000.	846.7
18 35	10053.		31.3		16487.	1.000	8244.	>1000000.	60000.	849.0
5 9	1158.		79.7		16421.	1.000	8210.	>1000000.	60000.	851.3
18 41	15818.		3.2		16410.	1.000	8205.	>1000000.	60000.	849.0
36 45	15775.		3.2		16368.	1.000	8184.	>1000000.	60000.	861.5
13 42	16052.		25.8		16276.	1.000	8138.	>1000000.	60000.	866.1
7 36	15533.		1.5		16126.	1.000	8063.	>1000000.	60000.	849.0
1 15	11213.		25.9		16047.	1.000	8023.	>1000000.	60000.	875.7
5 44	964.		78.5		16013.	1.000	8007.	>1000000.	60000.	863.8
2 39	15947.		0.0		15947.	1.000	7974.	>1000000.	60000.	875.7
38 39	15947.		0.0		15947.	1.000	7974.	>1000000.	60000.	875.7
21 37	15918.		0.0		15918.	1.000	7959.	>1000000.	54348.	788.7
17 18	9481.		31.3		15915.	1.000	7958.	>1000000.	60000.	849.0
42 44	15657.		29.2		15881.	1.000	7940.	>1000000.	60000.	859.2
19 41	15861.		39.7		15879.	1.000	7940.	>1000000.	58680.	822.5
6 19	15861.		38.3		15879.	1.000	7939.	>1000000.	58680.	822.5
8 19	15861.		38.0		15879.	1.000	7939.	>1000000.	58680.	822.5
10 19	15861.		38.5		15879.	1.000	7939.	>1000000.	58680.	822.5
14 19	15861.		38.4		15879.	1.000	7939.	>1000000.	58680.	822.5
16 19	15861.		26.6		15879.	1.000	7939.	>1000000.	58680.	822.5

 Notes b,d,e,k: Fails
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FATIGUE ANALYSIS AT POINT 155, LR ELBOW. 155 TO 160

INDIVIDUAL STRESS RANGE CHECK

DELTA T1 IN DEGREES
STRESSES IN PSI

LOAD SET PAIR	SN EQN.10	SE EQN.12	DELTA T1 RANGE	SP EQN.13	KE EQN.11	SALT EQN.14	ALLOW CYCLES	3*SM	ALLOWABLE FOR DELTA T1 RANGE
2 7	15533.		1.7	15850.	1.000	7925.	>1000000.	60000.	875.7
7 38	15533.		1.7	15850.	1.000	7925.	>1000000.	60000.	875.7
12 42	14132.		37.1	15836.	1.000	7918.	>1000000.	60000.	866.1
5 21	765.		78.5	15814.	1.000	7907.	>1000000.	54348.	827.0
2 45	15775.		0.0	15775.	1.000	7888.	>1000000.	60000.	888.2
38 45	15775.		0.0	15775.	1.000	7888.	>1000000.	60000.	888.2
11 33	14547.		16.3	15700.	1.000	7850.	>1000000.	60000.	845.2
5 11	289.		79.8	15581.	1.000	7790.	>1000000.	60000.	851.3
13 37	14893.		3.3	15512.	1.000	7756.	>1000000.	60000.	888.0
13 33	14547.		11.7	15457.	1.000	7729.	>1000000.	60000.	864.5
37 44	15404.		0.0	15404.	1.000	7702.	>1000000.	60000.	857.6
25 37	15333.		42.2	15347.	1.000	7673.	>1000000.	53580.	784.2
5 13	289.		75.2	15338.	1.000	7669.	>1000000.	60000.	870.6
21 42	15018.		29.2	15242.	1.000	7621.	>1000000.	54348.	821.2
16 17	6337.		47.5	15204.	1.000	7602.	>1000000.	60000.	831.2
33 44	14277.		15.0	15187.	1.000	7594.	>1000000.	60000.	857.7
11 37	14893.		1.3	15136.	1.000	7568.	>1000000.	60000.	833.7
35 39	8429.		34.4	14863.	1.000	7431.	>1000000.	60000.	831.2
15 16	7347.		38.7	14564.	1.000	7282.	>1000000.	60000.	831.2
8 35	7752.		36.1	14503.	1.000	7251.	>1000000.	60000.	831.2
19 21	14425.		39.7	14443.	1.000	7221.	>1000000.	54348.	792.1
6 35	7752.		35.8	14438.	1.000	7219.	>1000000.	60000.	831.2
25 40	14423.		42.2	14437.	1.000	7218.	>1000000.	53580.	798.3
14 35	7752.		35.8	14429.	1.000	7214.	>1000000.	60000.	831.2
21 33	13510.		15.0	14420.	1.000	7210.	>1000000.	54348.	819.2
10 35	7752.		35.6	14400.	1.000	7200.	>1000000.	60000.	831.2

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FATIGUE ANALYSIS AT POINT 155, LR ELBOW 155 TO 160

 INDIVIDUAL STRESS RANGE CHECK

 DELTA T1 IN DEGREES
 STRESSES IN PSI

LOAD SET PAIR	SN EQN.10	SE EQN.12	DELTA T1 RANGE	EQN.13	SP EQN.11	KE	SALT EQN.14	ALLOW CYCLES	3*SM	ALLOWABLE FOR DELTA T1 RANGE
25 41	14299.		42.2		14314.	1.000	7157.	>1000000.	53580.	798.3
6 25	14299.		40.8		14314.	1.000	7157.	>1000000.	53580.	798.3
8 25	14299.		40.5		14314.	1.000	7157.	>1000000.	53580.	798.3
10 25	14299.		41.0		14314.	1.000	7157.	>1000000.	53580.	798.3
14 25	14299.		40.8		14314.	1.000	7157.	>1000000.	53580.	798.3
16 25	14299.		29.1		14314.	1.000	7157.	>1000000.	53580.	798.3
9 37	14028.		1.2		14242.	1.000	7121.	>1000000.	60000.	833.7
35 41	7752.		34.4		14186.	1.000	7093.	>1000000.	60000.	831.2
12 15	9327.		17.7		14111.	1.000	7056.	>1000000.	60000.	850.5
18 42	13275.		32.3		14092.	1.000	7046.	>1000000.	60000.	864.5
35 40	7628.		34.4		14063.	1.000	7031.	>1000000.	60000.	831.2
19 44	13999.		39.7		14017.	1.000	7008.	>1000000.	58680.	836.3
16 36	10847.		16.2		13873.	1.000	6936.	>1000000.	60000.	849.0
17 40	7326.		34.4		13761.	1.000	6880.	>1000000.	60000.	831.2
11 19	13390.		41.0		13651.	1.000	6825.	>1000000.	58680.	822.5
17 39	7014.		34.4		13448.	1.000	6724.	>1000000.	60000.	831.2
7 12	11932.		6.2		13411.	1.000	6706.	>1000000.	60000.	850.5
13 19	13390.		36.3		13408.	1.000	6704.	>1000000.	58680.	843.2
12 45	11813.		7.9		13293.	1.000	6647.	>1000000.	60000.	863.0
2 16	10847.		13.1		13280.	1.000	6640.	>1000000.	60000.	875.7
16 38	10847.		13.1		13280.	1.000	6640.	>1000000.	60000.	875.7
15 18	8471.		22.5		13255.	1.000	6627.	>1000000.	60000.	849.0
15 40	8336.		25.7		13120.	1.000	6560.	>1000000.	60000.	831.2
1 25	13083.		41.9		13097.	1.000	6549.	>1000000.	53580.	849.3
8 17	6337.		36.2		13088.	1.000	6544.	>1000000.	60000.	831.2
6 17	6337.		35.8		13023.	1.000	6511.	>1000000.	60000.	831.2

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FATIGUE ANALYSIS AT POINT 155, LR ELBOW 155 TO 160

INDIVIDUAL STRESS RANGE CHECK

DELTA T1 IN DEGREES
STRESSES IN PSI

LOAD SET PAIR	SN EQN.10	SE EQN.12	DELTA T1 RANGE	SP EQN.13	EQN.11	KE	SALT EQN.14	ALLOW CYCLES	3*SM	ALLOWABLE FOR DELTA T1 RANGE
14 17	6337.		35.8	13014.		1.000	6507.	>1000000.	60000.	831.2
10 17	6337.		35.6	12986.		1.000	6493.	>1000000.	60000.	831.2
15 39	8023.		25.7	12807.		1.000	6404.	>1000000.	60000.	831.2
17 41	6337.		34.4	12771.		1.000	6386.	>1000000.	60000.	831.2
9 19	12522.		40.8	12755.		1.000	6377.	>1000000.	58680.	822.5
36 37	12027.		3.2	12620.		1.000	6310.	>1000000.	60000.	880.1
7 37	12299.		1.7	12616.		1.000	6308.	>1000000.	60000.	833.7
37 45	12541.		0.0	12541.		1.000	6271.	>1000000.	60000.	857.6
8 15	7347.		27.4	12448.		1.000	6224.	>1000000.	60000.	831.2
6 15	7347.		27.0	12382.		1.000	6191.	>1000000.	60000.	831.2
14 15	7347.		27.0	12373.		1.000	6187.	>1000000.	60000.	831.2
10 15	7347.		26.8	12345.		1.000	6173.	>1000000.	60000.	831.2
1 21	12237.		0.3	12287.		1.000	6144.	>1000000.	54348.	851.7
15 41	7347.		25.7	12131.		1.000	6065.	>1000000.	60000.	831.2
19 45	12049.		39.7	12066.		1.000	6033.	>1000000.	58680.	836.3
2 37	12027.		0.0	12027.		1.000	6013.	>1000000.	60000.	970.1
37 38	12027.		0.0	12027.		1.000	6013.	>1000000.	60000.	970.1
1 13	11213.		3.1	11832.		1.000	5916.	>1000000.	60000.	903.8
1 44	11724.		0.3	11774.		1.000	5887.	>1000000.	60000.	888.2
8 36	10847.		4.9	11757.		1.000	5878.	>1000000.	60000.	849.0
6 36	10847.		4.5	11691.		1.000	5846.	>1000000.	60000.	849.0
14 36	10847.		4.5	11682.		1.000	5841.	>1000000.	60000.	849.0
9 12	10196.		6.8	11676.		1.000	5838.	>1000000.	60000.	850.5
7 18	11075.		1.5	11668.		1.000	5834.	>1000000.	60000.	849.0
10 36	10847.		4.3	11654.		1.000	5827.	>1000000.	60000.	849.0
18 45	10961.		3.2	11553.		1.000	5777.	>1000000.	60000.	861.5

Notes b,d,e,k: Fails
g: Weld ISI
h,i: Rupture Location
L: Information

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++ DST/PIPESTRESS ++

Vermont Yankee

Version 3.5.1+026 PC-EXE

Release: Jun

 CALCULATION NUMBER 2 CODE SECTION III CLASS 1 ASME-1998 KRE
 Vermont Yankee Feedwater PipingSI Fatigue Analysis

2007/07/11 11:35:07

FATIGUE ANALYSIS AT POINT 155, LR ELBOW 155 TO 160

 INDIVIDUAL STRESS RANGE CHECK

 DELTA T1 IN DEGREES
 STRESSES IN PSI

LOAD SET PAIR	SN EQN.10	SE EQN.12	DELTA T1 RANGE	EQN.13	SP EQN.11	KE	SALT EQN.14	ALLOW CYCLES	3*SM	ALLOWABLE FOR DELTA T1 RANGE
1 11	11213.		1.6		11506.	1.000	5753.	>1000000.	60000.	875.7
7 19	11134.		41.4		11469.	1.000	5734.	>1000000.	58680.	822.5
36 41	10847.		3.2		11440.	1.000	5720.	>1000000.	60000.	849.0
12 13	9327.		11.3		11426.	1.000	5713.	>1000000.	60000.	869.9
2 24	11328.		0.5		11330.	1.000	5665.	>1000000.	60000.	885.1
24 36	11328.		2.7		11330.	1.000	5665.	>1000000.	60000.	858.4
24 38	11328.		0.5		11330.	1.000	5665.	>1000000.	60000.	885.1
2 8	10847.		1.7		11164.	1.000	5582.	>1000000.	60000.	875.7
8 38	10847.		1.7		11164.	1.000	5582.	>1000000.	60000.	875.7
2 6	10847.		1.4		11098.	1.000	5549.	>1000000.	60000.	875.7
6 38	10847.		1.4		11098.	1.000	5549.	>1000000.	60000.	875.7
2 14	10847.		1.3		11090.	1.000	5545.	>1000000.	60000.	875.7
14 38	10847.		1.3		11090.	1.000	5545.	>1000000.	60000.	875.7
2 10	10847.		1.2		11061.	1.000	5531.	>1000000.	60000.	875.7
10 38	10847.		1.2		11061.	1.000	5531.	>1000000.	60000.	875.7
2 41	10847.		0.0		10847.	1.000	5423.	>1000000.	60000.	875.7
38 41	10847.		0.0		10847.	1.000	5423.	>1000000.	60000.	875.7
16 21	8383.		13.1		10816.	1.000	5408.	>1000000.	54348.	801.4
11 12	9327.		6.6		10807.	1.000	5404.	>1000000.	60000.	850.5
1 9	10348.		1.4		10612.	1.000	5306.	>1000000.	60000.	875.7
36 40	9867.		3.2		10460.	1.000	5230.	>1000000.	60000.	849.0
12 44	8904.		7.9		10384.	1.000	5192.	>1000000.	60000.	863.0
16 44	7878.		13.1		10311.	1.000	5156.	>1000000.	60000.	843.7
16 37	7613.		13.1		10046.	1.000	5023.	>1000000.	60000.	833.7
11 16	7347.		14.4		10023.	1.000	5011.	>1000000.	60000.	831.2
9 18	9339.		2.0		9932.	1.000	4966.	>1000000.	60000.	849.0

 Notes b,d,e,k: Fails
 g: Weld ISI
 h,i: Rupture Location
 L: Information

File No.: VY-16Q-311

Revision: 0

++ DST/PIPESTRESS ++

Vermont Yankee

Version 3.5.1+026 PC-EXE

Release: Jun

 CALCULATION NUMBER 2 CODE SECTION III CLASS 1 ASME-1998
 Vermont Yankee Feedwater PipingSI Fatigue Analysis

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FATIGUE ANALYSIS AT POINT 155, LR ELBOW 155 TO 160

 INDIVIDUAL STRESS RANGE CHECK

 DELTA T1 IN DEGREES
 STRESSES IN PSI

LOAD SET PAIR	SN EQN.10	SE EQN.12	DELTA T1 RANGE	SP EQN.13	EQN.11	KE	SALT EQN.14	ALLOW CYCLES	3*SM	ALLOWABLE FOR DELTA T1 RANGE
2 40	9867.		0.0		9867.	1.000	4934.	>1000000.	60000.	875.7
38 40	9867.		0.0		9867.	1.000	4934.	>1000000.	60000.	875.7
13 16	7347.		9.7		9780.	1.000	4890.	>1000000.	60000.	850.5
12 21	8294.		7.9		9774.	1.000	4887.	>1000000.	54348.	826.0
13 18	8471.		6.5		9683.	1.000	4841.	>1000000.	60000.	868.4
35 45	3119.		34.4		9554.	1.000	4777.	>1000000.	60000.	843.7
7 35	3009.		32.7		9443.	1.000	4722.	>1000000.	60000.	831.2
21 40	9372.		0.0		9373.	1.000	4686.	>1000000.	54348.	801.4
9 16	6478.		14.2		9126.	1.000	4563.	>1000000.	60000.	831.2
11 18	8471.		1.9		9064.	1.000	4532.	>1000000.	60000.	849.0
21 39	9063.		0.0		9063.	1.000	4531.	>1000000.	54348.	793.6
21 35	2626.		34.4		9060.	1.000	4530.	>1000000.	54348.	801.4
1 36	8346.		3.4		8989.	1.000	4494.	>1000000.	60000.	899.7
1 7	8619.		2.0		8985.	1.000	4493.	>1000000.	60000.	875.7
13 40	8336.		3.3		8955.	1.000	4478.	>1000000.	60000.	850.5
1 45	8861.		0.3		8911.	1.000	4455.	>1000000.	60000.	888.2
40 44	8864.		0.0		8865.	1.000	4432.	>1000000.	60000.	843.7
35 44	2416.		34.4		8850.	1.000	4425.	>1000000.	60000.	843.7
8 21	8383.		1.7		8700.	1.000	4350.	>1000000.	54348.	801.4
18 44	8053.		3.2		8646.	1.000	4323.	>1000000.	60000.	861.5
13 39	8023.		3.3		8643.	1.000	4321.	>1000000.	60000.	850.5
13 35	1582.		37.8		8635.	1.000	4318.	>1000000.	60000.	850.5
6 21	8383.		1.4		8635.	1.000	4317.	>1000000.	54348.	801.4
14 21	8383.		1.3		8626.	1.000	4313.	>1000000.	54348.	801.4
10 21	8383.		1.2		8598.	1.000	4299.	>1000000.	54348.	801.4
11 40	8336.		1.3		8579.	1.000	4289.	>1000000.	60000.	831.2

 Notes b,d,e,k: Fails
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File No.: VY-16Q-311

Revision: 0

++ DST/PIPESTRESS ++

Vermont Yankee

Version 3.5.1+026 PC-EXE

Release: Jun

 CALCULATION NUMBER 2 CODE SECTION III CLASS 1 ASME-1998
 Vermont Yankee Feedwater PipingSI Fatigue Analysis

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FATIGUE ANALYSIS AT POINT 155, LR ELBOW 155 TO 160

 INDIVIDUAL STRESS RANGE CHECK

 DELTA T1 IN DEGREES
 STRESSES IN PSI

LOAD SET PAIR	SN EQN.10	SE EQN.12	DELTA T1 RANGE	EQN.13	SP EQN.11	KE	SALT EQN.14	ALLOW CYCLES	3*SM	ALLOWABLE FOR DELTA T1 RANGE
39 44	8578.		0.0		8579.	1.000	4289.	>1000000.	60000.	843.7
17 21	2051.		34.4		8485.	1.000	4243.	>1000000.	54348.	801.4
1 2	8346.		0.3		8396.	1.000	4198.	>1000000.	60000.	970.1
1 38	8346.		0.3		8396.	1.000	4198.	>1000000.	60000.	970.1
21 41	8383.		0.0		8383.	1.000	4192.	>1000000.	54348.	801.4
11 39	8023.		1.3		8266.	1.000	4133.	>1000000.	60000.	831.2
17 45	1772.		34.4		8206.	1.000	4103.	>1000000.	60000.	843.7
8 44	7878.		1.7		8195.	1.000	4098.	>1000000.	60000.	843.7
17 44	1760.		34.4		8194.	1.000	4097.	>1000000.	60000.	843.7
6 44	7878.		1.4		8130.	1.000	4065.	>1000000.	60000.	843.7
14 44	7878.		1.3		8121.	1.000	4060.	>1000000.	60000.	843.7
10 44	7878.		1.2		8093.	1.000	4046.	>1000000.	60000.	843.7
13 17	1010.		37.8		8064.	1.000	4032.	>1000000.	60000.	850.5
18 21	7438.		3.2		8031.	1.000	4015.	>1000000.	54348.	824.1
7 17	1594.		32.7		8029.	1.000	4014.	>1000000.	60000.	831.2
11 35	1582.		33.1		8016.	1.000	4008.	>1000000.	60000.	831.2
15 35	1582.		8.8		8016.	1.000	4008.	>1000000.	60000.	831.2
13 41	7347.		3.3		7966.	1.000	3983.	>1000000.	60000.	850.5
6 13	7347.		2.0		7966.	1.000	3983.	>1000000.	60000.	850.5
8 13	7347.		1.6		7966.	1.000	3983.	>1000000.	60000.	850.5
10 13	7347.		2.2		7966.	1.000	3983.	>1000000.	60000.	850.5
13 14	7347.		2.0		7966.	1.000	3983.	>1000000.	60000.	850.5
8 37	7613.		1.7		7930.	1.000	3965.	>1000000.	60000.	833.7
8 11	7347.		3.0		7906.	1.000	3953.	>1000000.	60000.	831.2
41 44	7878.		0.0		7878.	1.000	3939.	>1000000.	60000.	843.7
6 37	7613.		1.4		7865.	1.000	3932.	>1000000.	60000.	833.7

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Vermont Yankee

Version 3.5.1+026 PC-EXE

Release: Jun

 CALCULATION NUMBER 2 CODE SECTION III CLASS 1 ASME-1998
 Vermont Yankee Feedwater PipingSI Fatigue Analysis

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FATIGUE ANALYSIS AT POINT 155, LR ELBOW 155 TO 160

 INDIVIDUAL STRESS RANGE CHECK

 DELTA T1 IN DEGREES
 STRESSES IN PSI

LOAD SET PAIR	SN EQN.10	SE EQN.12	DELTA T1 RANGE	EQN.13	SP EQN.11	KE	SALT EQN.14	ALLOW CYCLES	3*SM	ALLOWABLE FOR DELTA T1 RANGE
14 37	7613.		1.3		7856.	1.000	3928.	>1000000.	60000.	833.7
17 35	1415.		0.0		7850.	1.000	3925.	>1000000.	60000.	831.2
6 11	7347.		2.7		7841.	1.000	3921.	>1000000.	60000.	831.2
11 14	7347.		2.6		7832.	1.000	3916.	>1000000.	60000.	831.2
10 37	7613.		1.2		7828.	1.000	3914.	>1000000.	60000.	833.7
10 11	7347.		2.5		7804.	1.000	3902.	>1000000.	60000.	831.2
37 40	7746.		0.0		7746.	1.000	3873.	>1000000.	60000.	833.7
9 35	1273.		33.3		7707.	1.000	3854.	>1000000.	60000.	831.2
9 40	7468.		1.2		7683.	1.000	3841.	>1000000.	60000.	831.2
1 39	7602.		0.3		7652.	1.000	3826.	>1000000.	60000.	875.7
37 41	7613.		0.0		7613.	1.000	3807.	>1000000.	60000.	833.7
11 41	7347.		1.3		7589.	1.000	3795.	>1000000.	60000.	831.2
16 39	5128.		13.1		7561.	1.000	3780.	>1000000.	60000.	831.2
7 16	4742.		14.8		7493.	1.000	3746.	>1000000.	60000.	831.2
16 45	5016.		13.1		7449.	1.000	3725.	>1000000.	60000.	843.7
11 17	1010.		33.1		7444.	1.000	3722.	>1000000.	60000.	831.2
15 17	1010.		8.8		7444.	1.000	3722.	>1000000.	60000.	831.2
15 45	2654.		25.7		7438.	1.000	3719.	>1000000.	60000.	843.7
7 15	2604.		24.0		7388.	1.000	3694.	>1000000.	60000.	831.2
9 39	7155.		1.2		7370.	1.000	3685.	>1000000.	60000.	831.2
8 9	6478.		2.9		7010.	1.000	3505.	>1000000.	60000.	831.2
6 9	6478.		2.5		6945.	1.000	3472.	>1000000.	60000.	831.2
9 14	6478.		2.5		6936.	1.000	3468.	>1000000.	60000.	831.2
37 39	6911.		0.0		6911.	1.000	3456.	>1000000.	60000.	833.7
9 10	6478.		2.3		6908.	1.000	3454.	>1000000.	60000.	831.2
9 41	6478.		1.2		6693.	1.000	3347.	>1000000.	60000.	831.2

 Notes b,d,e,k: Fails
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Vermont Yankee

Version 3.5.1+026 PC-EXE

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 Vermont Yankee Feedwater PipingSI Fatigue Analysis

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FATIGUE ANALYSIS AT POINT 155, LR ELBOW 155 TO 160

 INDIVIDUAL STRESS RANGE CHECK

 DELTA T1 IN DEGREES
 STRESSES IN PSI

LOAD SET PAIR	SN EQN.10	SE EQN.12	DELTA T1 RANGE	EQN.13	SP EQN.11	KE	SALT EQN.14	ALLOW CYCLES	3*SM	ALLOWABLE FOR DELTA T1 RANGE
9 17	143.		33.3		6577.	1.000	3289.	>1000000.	60000.	831.2
1 16	3933.		12.8		6366.	1.000	3183.	>1000000.	60000.	875.7
39 40	6117.		0.0		6118.	1.000	3059.	>1000000.	60000.	831.2
7 40	5732.		1.7		6049.	1.000	3025.	>1000000.	60000.	831.2
40 45	5998.		0.0		5998.	1.000	2999.	>1000000.	60000.	843.7
15 44	1059.		25.7		5843.	1.000	2921.	>1000000.	60000.	843.7
15 21	1049.		25.7		5833.	1.000	2917.	>1000000.	54348.	801.4
39 45	5775.		0.0		5775.	1.000	2887.	>1000000.	60000.	843.7
7 39	5420.		1.7		5737.	1.000	2869.	>1000000.	60000.	831.2
9 15	868.		24.5		5652.	1.000	2826.	>1000000.	60000.	831.2
8 39	5128.		1.7		5445.	1.000	2722.	>1000000.	60000.	831.2
13 15	0.		29.0		5403.	1.000	2702.	>1000000.	60000.	850.5
6 39	5128.		1.4		5380.	1.000	2690.	>1000000.	60000.	831.2
7 8	4742.		3.4		5377.	1.000	2688.	>1000000.	60000.	831.2
14 39	5128.		1.3		5371.	1.000	2685.	>1000000.	60000.	831.2
10 39	5128.		1.2		5342.	1.000	2671.	>1000000.	60000.	831.2
8 45	5016.		1.7		5333.	1.000	2667.	>1000000.	60000.	843.7
6 7	4742.		3.1		5311.	1.000	2656.	>1000000.	60000.	831.2
7 14	4742.		3.0		5302.	1.000	2651.	>1000000.	60000.	831.2
7 10	4742.		2.9		5274.	1.000	2637.	>1000000.	60000.	831.2
6 45	5016.		1.4		5268.	1.000	2634.	>1000000.	60000.	843.7
14 45	5016.		1.3		5259.	1.000	2630.	>1000000.	60000.	843.7
10 45	5016.		1.2		5231.	1.000	2615.	>1000000.	60000.	843.7
39 41	5128.		0.0		5128.	1.000	2564.	>1000000.	60000.	831.2
7 41	4742.		1.7		5060.	1.000	2530.	>1000000.	60000.	831.2
41 45	5016.		0.0		5016.	1.000	2508.	>1000000.	60000.	843.7

 Notes b,d,e,k: Fails
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FATIGUE ANALYSIS AT POINT 155, LR ELBOW 155 TO 160

 INDIVIDUAL STRESS RANGE CHECK

 DELTA T1 IN DEGREES
 STRESSES IN PSI

LOAD SET PAIR	SN EQN.10	SE EQN.12	DELTA T1 RANGE	EQN.13	SP EQN.11	KE	SALT EQN.14	ALLOW CYCLES	3*SM	ALLOWABLE FOR DELTA T1 RANGE
11 15	0.		24.4		4784.	1.000	2392.	>1000000.	60000.	831.2
1 8	3933.		1.4		4250.	1.000	2125.	>1000000.	60000.	875.7
1 6	3933.		1.1		4184.	1.000	2092.	>1000000.	60000.	875.7
1 14	3933.		1.0		4176.	1.000	2088.	>1000000.	60000.	875.7
1 10	3933.		0.9		4147.	1.000	2074.	>1000000.	60000.	875.7
1 40	4066.		0.3		4116.	1.000	2058.	>1000000.	60000.	875.7
1 41	3933.		0.3		3983.	1.000	1991.	>1000000.	60000.	875.7
7 21	3643.		1.7		3960.	1.000	1980.	>1000000.	54348.	801.4
1 37	3680.		0.3		3730.	1.000	1865.	>1000000.	60000.	970.1
21 45	3666.		0.0		3666.	1.000	1833.	>1000000.	54348.	0.0
7 13	2604.		5.0		3540.	1.000	1770.	>1000000.	60000.	850.5
7 44	3221.		1.7		3538.	1.000	1769.	>1000000.	60000.	843.7
44 45	3528.		0.0		3528.	1.000	1764.	>1000000.	60000.	0.0
16 40	990.		13.1		3423.	1.000	1712.	>1000000.	60000.	831.2
13 45	2654.		3.3		3273.	1.000	1636.	>1000000.	60000.	863.0
7 11	2604.		0.4		2921.	1.000	1461.	>1000000.	60000.	831.2
11 45	2654.		1.3		2897.	1.000	1448.	>1000000.	60000.	843.7
16 41	0.		13.1		2433.	1.000	1217.	>1000000.	60000.	831.2
6 16	0.		11.7		2433.	1.000	1217.	>1000000.	60000.	831.2
8 16	0.		11.4		2433.	1.000	1217.	>1000000.	60000.	831.2
10 16	0.		11.9		2433.	1.000	1217.	>1000000.	60000.	831.2
14 16	0.		11.8		2433.	1.000	1217.	>1000000.	60000.	831.2
12 18	857.		4.8		2337.	1.000	1168.	>1000000.	60000.	868.4
9 21	1911.		1.2		2126.	1.000	1063.	>1000000.	54348.	801.4
9 45	1887.		1.2		2102.	1.000	1051.	>1000000.	60000.	843.7
7 9	1736.		0.5		2053.	1.000	1027.	>1000000.	60000.	831.2

 Notes b,d,e,k: Fails
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 Vermont Yankee Feedwater PipingSI Fatigue Analysis

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FATIGUE ANALYSIS AT POINT 155, LR ELBOW 155 TO 160

 INDIVIDUAL STRESS RANGE CHECK

 DELTA T1 IN DEGREES
 STRESSES IN PSI

LOAD SET PAIR	SN EQN.10	SE EQN.12	DELTA T1 RANGE	EQN.13	SP EQN.11	KE	SALT EQN.14	ALLOW CYCLES	3*SM	ALLOWABLE FOR DELTA T1 RANGE
9 44	1642.		1.2		1857.	1.000	929.	>1000000.	60000.	843.7
9 13	868.		4.5		1702.	1.000	851.	>1000000.	60000.	850.5
13 44	1059.		3.3		1678.	1.000	839.	>1000000.	60000.	863.0
13 21	1049.		3.3		1668.	1.000	834.	>1000000.	54348.	826.0
7 45	1057.		1.7		1374.	1.000	687.	>1000000.	60000.	843.7
8 40	990.		1.7		1307.	1.000	654.	>1000000.	60000.	831.2
11 44	1059.		1.3		1301.	1.000	651.	>1000000.	60000.	843.7
11 21	1049.		1.3		1292.	1.000	646.	>1000000.	54348.	801.4
6 40	990.		1.4		1242.	1.000	621.	>1000000.	60000.	831.2
14 40	990.		1.3		1233.	1.000	616.	>1000000.	60000.	831.2
10 40	990.		1.2		1205.	1.000	602.	>1000000.	60000.	831.2
9 11	868.		0.2		1111.	1.000	556.	>1000000.	60000.	831.2
21 44	1018.		0.0		1018.	1.000	509.	>1000000.	54348.	0.0
40 41	990.		0.0		990.	1.000	495.	>1000000.	60000.	831.2
11 13	0.		4.6		862.	1.000	431.	>1000000.	60000.	850.5
2 36	0.		3.2		593.	1.000	296.	>1000000.	60000.	899.7
36 38	0.		3.2		593.	1.000	296.	>1000000.	60000.	899.7
8 41	0.		1.7		317.	1.000	159.	>1000000.	60000.	831.2
6 8	0.		0.4		317.	1.000	159.	>1000000.	60000.	831.2
8 10	0.		0.5		317.	1.000	159.	>1000000.	60000.	831.2
8 14	0.		0.4		317.	1.000	159.	>1000000.	60000.	831.2
6 41	0.		1.4		252.	1.000	126.	>1000000.	60000.	831.2
6 10	0.		0.2		252.	1.000	126.	>1000000.	60000.	831.2
6 14	0.		0.0		252.	1.000	126.	>1000000.	60000.	831.2
14 41	0.		1.3		243.	1.000	121.	>1000000.	60000.	831.2
10 14	0.		0.2		243.	1.000	121.	>1000000.	60000.	831.2

 Notes b,d,e,k: Fails
 g: Weld ISI
 h,i: Rupture Location
 L: Information

File No.: VY-16Q-311

Revision: 0



++ DST/PIPESTRESS ++

Vermont Yankee

Version 3.5.1+026 PC-EXE

Release: Jun

CALCULATION NUMBER 2 CODE SECTION III CLASS 1 ASME-1998
Vermont Yankee Feedwater PipingSI Fatigue Analysis

KRE

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FATIGUE ANALYSIS AT POINT 155, LR ELBOW 155 TO 160

INDIVIDUAL STRESS RANGE CHECK

DELTA T1 IN DEGREES
STRESSES IN PSI

LOAD SET PAIR	SN EQN.10	SE EQN.12	DELTA T1 RANGE	EQN.13	SP EQN.11	KE	SALT EQN.14	ALLOW CYCLES	3*SM	ALLOWABLE FOR DELTA T1 RANGE
10 41	0.		1.2		215.	1.000	107.	>1000000.	60000.	831.2
2 38	0.		0.0		0.	1.000	0.	>1000000.	60000.	0.0

Notes b,d,e,k: Fails
g: Weld ISI
h,i: Rupture Location
L: Information

File No.: VY-16Q-311

Revision: 0

++ DST/PIPESTRESS ++

Vermont Yankee

Version 3.5.1+026 PC-EXE

Release: Jun

 CALCULATION NUMBER 2 CODE SECTION III CLASS 1 ASME-1998
 Vermont Yankee Feedwater PipingSI Fatigue Analysis

KRE

2007/07/11 11:35:07 11

FATIGUE ANALYSIS AT POINT 155, LR ELBOW 155 TO 160

 INDIVIDUAL STRESS RANGE CHECK

 DELTA T1 IN DEGREES
 STRESSES IN PSI

LOAD SET PAIR	SALT EQN.14	OCCURENCES		NUMBER USED	SETS ELIMINATED DYNAM.	NO. CYCLES TO FAILURE	USAGE FACTOR	REMARKS
I J		NI	NJ					
28 29	66456.	10 0	10 0	10	28,29	1808.	0.0055	
31 32	54146.	10 0	10 0	10	31,32	3406.	0.0029	
4 43	49733.	610 310	300 0	300	43	4465.	0.0672	
4 34	49533.	310 300	10 0	10	34	4523.	0.0022	
3 4	49512.	300 0	300 0	300	3, 4	4529.	0.0662	
20 26	45555.	300 290	10 0	10	26	5838.	0.0017	
22 27	34161.	300 290	10 0	10	27	14370.	0.0007	
22 30	32373.	290 280	10 0	10	30	17257.	0.0006	
22 42	26170.	280 0	300 20	280	22	33637.	0.0083	
24 42	22860.	300 280	20 0	20	42	51538.	0.0004	
5 24	22397.	599 319	280 0	280	24	57040.	0.0049	
5 23	21265.	319 19	300 0	300	23	73771.	0.0041	
5 36	17029.	19 18	1 0	1	36	178513.	0.0000	
2 5	16732.	120 102	18 0	18	5	190176.	0.0001	
12 20	16409.	10 0	290 280	10	12	205131.	0.0000	

File No.: VY-16Q-311

Revision: 0

++ DST/PIPESTRESS ++

Vermont Yankee

Version 3.5.1+026 PC-EXE

Release: Jun

 CALCULATION NUMBER 2 CODE SECTION III CLASS 1 ASME-1998 KRE
 Vermont Yankee Feedwater PipingSI Fatigue Analysis

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FATIGUE ANALYSIS AT POINT 155, LR ELBOW 155 TO 160

 INDIVIDUAL STRESS RANGE CHECK

 DELTA T1 IN DEGREES
 STRESSES IN PSI

LOAD SET PAIR I J	SALT EQN.14	OCCURENCES		NUMBER USED	SETS ELIMINATED DYNAM.	NO. CYCLES TO FAILURE	USAGE FACTOR	REMARKS
		NI	NJ					
2 20	16030.	102 0	280 178	102	2	228208.	0.0004	
20 38	16030.	178 177	1 0	1	38	228208.	0.0000	
18 20	15537.	289 112	177 0	177	20	263206.	0.0007	
18 25	15362.	112 102	10 0	10	25	277200.	0.0000	
33 37	15167.	10 9	1 0	1	37	293826.	0.0000	
1 33	13327.	120 111	9 0	9	33	561655.	0.0000	
18 19	11235.	102 0	300 198	102	18	>1000000.	0.0000	
19 35	10124.	198 197	1 0	1	35	>1000000.	0.0000	
1 19	9874.	111 0	197 86	111	1	>1000000.	0.0000	
17 19	9416.	289 203	86 0	86	19	>1000000.	0.0000	
16 17	7602.	70 0	203 133	70	16	>1000000.	0.0000	
17 40	6880.	133 0	289 156	133	17	>1000000.	0.0000	
15 40	6560.	70 0	156 86	70	15	>1000000.	0.0000	
21 40	4686.	300 214	86 0	86	40	>1000000.	0.0000	
21 39	4531.	214 0	289 75	214	21	>1000000.	0.0000	

File No.: VY-16Q-311

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Vermont Yankee

Version 3.5.1+026 PC-EXE

Release: Ju

 CALCULATION NUMBER 2 CODE SECTION III CLASS 1 ASME-1998
 Vermont Yankee Feedwater PipingSI Fatigue Analysis

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FATIGUE ANALYSIS AT POINT 155, LR ELBOW 155 TO 160

 INDIVIDUAL STRESS RANGE CHECK

 DELTA T1 IN DEGREES
 STRESSES IN PSI

LOAD SET PAIR	SALT	OCCURENCES		NUMBER	SETS	NO. CYCLES	USAGE	REMARKS
I J EQN.14		NI	NJ	USED	ELIMINATED DYNAM.	TO FAILURE	FACTOR	
13 39	4321.	10	75	10	13	>1000000.	0.0000	
		0	65					
39 44	4289.	65	5	5	44	>1000000.	0.0000	
		60	0	45				
11 39	4133.	310	60	60	39	>1000000.	0.0000	
		250	0					
8 11	3953.	10000	250	250	11	>1000000.	0.0000	
		9750	0					
8 9	3505.	9750	2000	2000	9	>1000000.	0.0000	
		7750	0					
7 8	2688.	10000	7750	7750	8	>1000000.	0.0000	
		2250	0					
6 7	2656.	599	2250	599	6	>1000000.	0.0000	
		0	1651					
7 14	2651.	1651	10	10	14	>1000000.	0.0000	
		1641	0					
7 10	2637.	1641	2000	1641	7	>1000000.	0.0000	
		0	359					
10 45	2615.	359	5	5	45	>1000000.	0.0000	
		354	0	45				
44 45	1764.	45	45	45	45 44	>1000000.	0.0000	DYN. RANGE OF EVENT.NO.
		0	0					
10 41	107.	354	289	289	41	>1000000.	0.0000	
		65	0					

TOTAL USAGE FACTOR = 0.1661j

File No.: VY-16Q-311

Revision: 0

REDACTED VERSION

Report No.: SIR-07-130-NPS
Revision No.: 0
Project No.: VY-16Q
File No.: VY-16Q-401
July 2007

**Environmental Fatigue Analysis
for the
Vermont Yankee
Reactor Pressure Vessel
Feedwater Nozzles**

NOTE

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in this document where those references are used are identified so that this information can be treated
in accordance with applicable vendor proprietary agreements.~~

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Client: Entergy Nuclear Vermont Yankee, LLC

SI Project Number: VY-16Q

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2.0	2-1 – 2-4			
3.0	3-1 – 3-34			
4.0	4-1 – 4-11			
5.0	5-1 – 5-2			
6.0	6-1			
7.0	7-1 – 7-3			

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1.0 INTRODUCTION

In Table 4.3-3 of the Vermont Yankee (VY) License Renewal Application (LRA), the 60-year cumulative usage factor (CUF) value for the reactor pressure vessel (RPV) feedwater nozzle (FW) is reported as 0.750. Application of an environmentally assisted fatigue (EAF) multiplier, as required for the license renewal period, resulted in an unacceptable EAF CUF value of 2.86. Therefore, further refined analysis was necessitated to show acceptable EAF CUF results for this component.

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The VY FW nozzles were re-evaluated in detail by SI in 2004 for EPU and 60 years of operation. However, that analysis used conservative transient definitions and cyclic projections for 60 years of operation that have since been updated as a part of LRA development.

This report documents a refined fatigue evaluation for the VY FW nozzle. The intent of this evaluation is to use refined transient definitions and the revised cyclic transient counts for 60 years for a computation of CUF, including EAF effects, that is more refined than previously performed fatigue analyses. The fatigue-limiting locations in the FW nozzle and safe end are included in the evaluation, to be consistent with NUREG/CR-6260 [16] needs for EAF evaluation for license renewal. The resulting fatigue results will be used as a replacement to the values previously reported in the VY LRA.

The refined evaluation summarized in this report included development of a detailed finite element model of the FW nozzle, including relevant portions of the safe end, thermal sleeve, and the RPV wall. Thermal and pressure stress histories were developed for relevant transients affecting the FW nozzle, including any effects of EPU, as specified by the VY RPV Design Specification [3], the VY EPU Design Specification [17] and other boiling water reactor (BWR) operating experience. The thermal and pressure stress histories were used to determine total stress and primary plus secondary stress for use in a subsequent fatigue evaluation. Stresses were also included due to loads from the attached piping for application in the stress/fatigue analysis based on the bounding reaction loads obtained from the relevant design documents. The revised fatigue calculation was performed using Section III methodology from the 1998 Edition, 2000 Addenda of the ASME Code [15], and was performed using actual cycles from past plant operation projected out to 60 years of operation.

1.1 Green's Function Methodology

In order to provide an overall approach and strategy for evaluating the feedwater nozzle, the Green's Function methodology and associated ASME Code stress and fatigue analyses are described in this section.

Revised stress and fatigue analyses are being performed for the feedwater nozzle using ASME Code, Section III methodology. These analyses are being performed to address license renewal requirements to evaluate environmental fatigue for this component in response to Generic Aging Lessons Learned (GALL) Report [22] requirements. The revised analysis is being performed to refine the fatigue usage so that an environmental fatigue factor can be determined for subsequent license renewal efforts.

Two sets of rules are available under ASME Code, Section III, Class 1 [15]. Subparagraph NB-3600 of Section III provides simplified rules for analysis of piping components, and NB-3200 allows for more detailed analysis of vessel components. The NB-3600 piping equations combine by absolute sum the stresses due to pressure, moments and through wall thermal gradient effects, regardless of where within the pipe cross-section the maximum value of the components of stress

are located. By considering stress signs, affected surface (inside or outside) and azimuthal position, the stress ranges can be significantly reduced. In addition, NB-3600 assigns stress indices by which the stresses are multiplied to conservatively incorporate the effects of geometric discontinuities. In NB-3200, these are not required, as the stresses are calculated by finite element analysis and any applicable stress concentration factors. This generally results in a net reduction of the stress ranges and consequently, in the fatigue usage. Article 4 [27] methodology was originally used to evaluate the feedwater nozzle. NB-3200 methodology, which is the modern day equivalent to Article 4, is used in this analysis to be consistent with the Section III design bases for this component, as well as to allow a more detailed analysis of this component. In addition, several of the conservatisms originally used in the original feedwater nozzle evaluation (such as grouping of transients) are removed in the current evaluation so as to achieve as accurate a CUF as reasonably achievable.

For the feedwater nozzle evaluated as a part of this work, stress histories will be computed by a time integration of the product of a pre-determined Green's Function and the transient data. This Green's Function integration scheme is similar in concept to the well-known Duhamel theory used in structural dynamics. A detailed derivation of this approach and examples of its application to specific plant locations is contained in Reference [4]. A general outline is provided in this section.

The steps involved in the evaluation are as follows:

- Develop finite element model
- Develop heat transfer coefficients and boundary conditions for the finite element model
- Develop Green's Functions
- Develop thermal transient definitions
- Perform stress analysis to determine stresses for all thermal transients
- Perform fatigue analysis

A Green's Function is derived by using finite-element methods to determine the transient stress response of the component to a step change in loading (usually a thermal shock). The critical location in the component is identified based on the maximum stress, and the thermal stress response over time is extracted for this location. This response to the input thermal step is the "Green's Function." Figure 1-1 shows a typical set of two Green's Functions, each for a different set of heat transfer coefficients (representing different flow rate conditions).

To compute the thermal stress response for an arbitrary transient, the loading parameter (usually local fluid temperature) is deconstructed into a series of step-loadings. By using the Green's Function, the response to each step can be quickly determined. By the principle of superposition, these can be added (algebraically) to determine the response to the original load history. The result is demonstrated in Figure 1-2. The input transient temperature history contains five step-changes of varying size, as shown in the upper plot in Figure 1-2. These five step changes produce the five successive stress responses in the second plot shown in Figure 1-2. By adding all five response curves, the real-time stress response for the input thermal transient is computed.

The Green's Function methodology produces identical results compared to running the input transient through the finite element model. The advantage of using Green's Functions is that many individual transients can be run with a significant reduction of effort compared to running all transients through the finite element model. The trade-off in this process is that the Green's Functions are based on constant material properties and heat transfer coefficients. Therefore, these parameters are chosen to bound all transients that constitute the majority of fatigue usage, i.e., the heat transfer coefficients at 300°F bound the cold water injection transient. In addition, the instantaneous value for the coefficient of thermal expansion is used instead of the mean value for the coefficient of thermal expansion. This conservatism is more than offset by the benefit of not having to analyze every transient, which was done in the VY reactor feedwater nozzle evaluation.

Once the stress history is obtained for all transients using the Green's Function approach, the remainder of the fatigue analysis is carried out using traditional methodologies in accordance with ASME Code, Section III requirements.

Fatigue calculations are performed in accordance with ASME Code, Section III, Subsection NB-3200 methodology. Fatigue analysis is performed for the three limiting locations (two in the safe end and one in the nozzle forging, representing the three materials of the nozzle assembly) using the Green's Functions developed for the three feedwater flow conditions and 60-year projected cycle counts.

Three Structural Integrity utility computer programs are used to facilitate the fatigue analysis process: STRESS.EXE, P.V.EXE, and FATIGUE.EXE. The first program, STRESS.EXE, calculates a stress history in response to a thermal transient using a Green's Function. The second program, P-V.EXE, reduces the stress history to peaks and valleys, as required by ASME Code fatigue evaluation methods. The third program, FATIGUE.EXE, calculates fatigue from the reduced peak and valley history using ASME Code, Section III range-pair methodology. All three programs are explained in detail and have been independently verified for generic use in the Reference [14] calculation.

In order to perform the fatigue analysis, Green's Functions are developed using the finite element model. Then, input files with the necessary data are prepared and the three utility computer programs are run. The first program (STRESS.EXE) requires the following three input files:

- Input file "GREEN.DAT": This file contains the Green's Function for the location being evaluated. For each flow condition, two Green's Functions are determined: a membrane plus bending stress intensity Green's Function and a total stress intensity Green's Function. This allows computation of total stress, as well as membrane plus bending stress, which is necessary to compute K_e per ASME Code, Section III requirements.

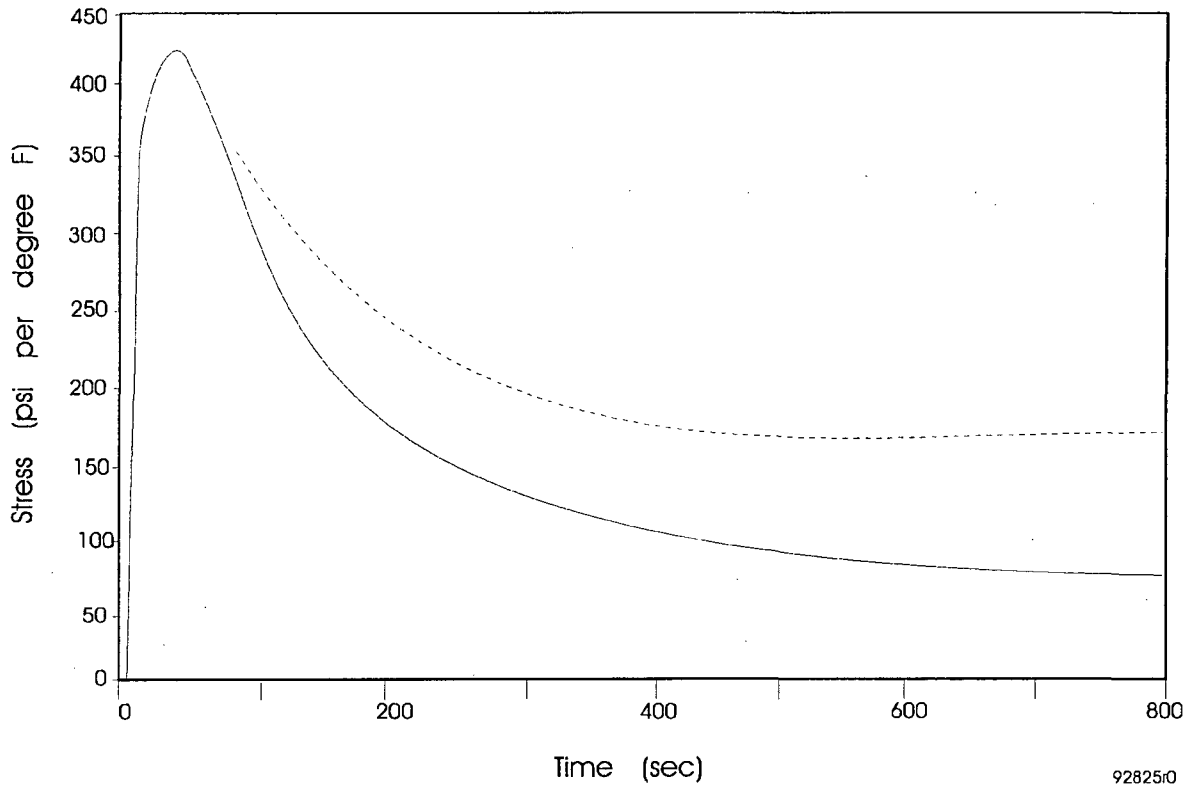
- Input file "GREEN.CFG": This file is a configuration file containing parameters that define the Green's Function (i.e., number of points, temperature drop analyzed, etc.).
- Input file "TRANSNT.INP": This file contains the input transient history for all thermal transients to be analyzed for the location being evaluated.

Pressure and piping stress intensities are also included for each transient case, based on pressure stress results from finite element analysis and attached piping load calculations.

The second program (P-V.EXE) simply extracts only the maxima and minima stress (i.e., the peaks and valleys) from the stress histories generated by program STRESS.EXE.

The third program (FATIGUE.EXE) performs the ASME Code peak event-pairing required to calculate a fatigue usage value. The input data consists of the output peak and valley history from program P-V.EXE and a configuration input file that provides ASME Code configuration data relevant to the fatigue analysis (i.e., K_e parameters, S_m , Young's modulus, etc.). The output is the final fatigue calculation for the location being evaluated.

The Green's Function methodology described above uses standard industry stress and fatigue analysis practices, and is the same as the methodology used in typical stress reports. Special approval for the use of this methodology is therefore not required.



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Note: A typical set of two Green's Functions is shown, each for a different set of heat transfer coefficients (representing different flow rate conditions).

Figure 1-1. Typical Green's Functions for Thermal Transient Stress

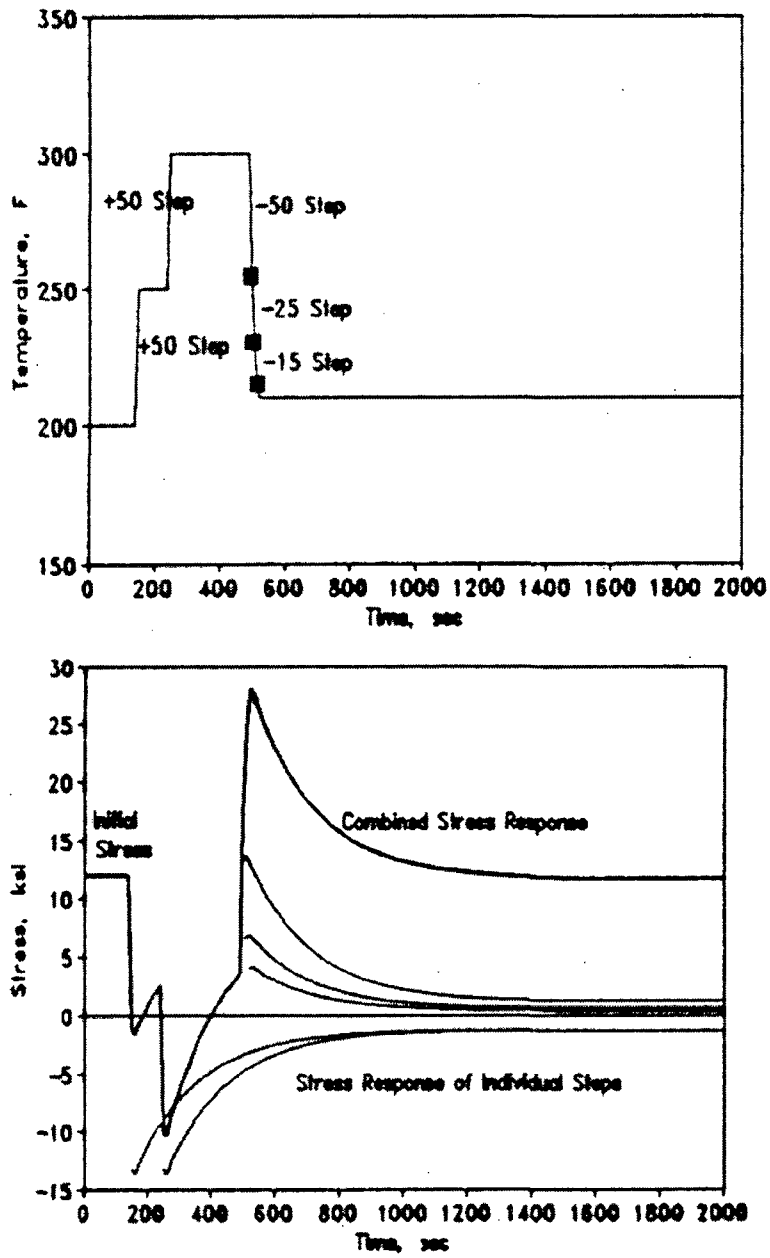


Figure 1-2. Typical Stress Response Using Green's Functions

2.0 FINITE ELEMENT MODEL

A previously generated ANSYS [5] finite element model (FEM) of the VY feedwater nozzle and safe end was used to perform the updated stress and fatigue analyses. The details of the model development are documented in the Reference [6] calculation.

A few key points with respect to model development are as follows:

- The model is identical to the geometry and mesh of the model previously developed for feedwater nozzle fracture mechanics work performed for VY [7].
- The boundary condition corresponding to the location of the start of the thermal sleeve in the FEM are consistent with Reference [8].

The materials of the various components of the model are listed below:

- Reactor Pressure Vessel – SA533 Grade B
- Reactor Pressure Vessel Cladding – Stainless Steel
- Nozzle Forging – ASTM A508 Class II
- Safe End Forging – ASTM A508 Class I
- Feedwater Piping – ASTM A106 Grade B

The FEM model the radius of RPV was increased by a factor of two to account for the fact that the vessel portion of the finite element model is a sphere and the actual geometry is a cylinder.

Material properties were based upon the 1998 ASME Code, Section II, Part D, with 2000 Addenda [9], and are shown in Table 2-1. The properties were evaluated at an average temperature of 300°F. This average temperature is based on a thermal shock of 500°F to 100°F which was applied to the FEM model for Green's Function development.

The finite element model is shown in Figures 2-1 and 2-2.

Table 2-1. Material Properties @ 300°F ⁽¹⁾

Material Ident.	Young's Modulus, $E \times 10^6$ (psi)	Instantaneous Coefficient of Thermal Expansion, $\alpha \times 10^{-6}$ (in/in-°F)	Density, ρ (lb/in ³) (assumed)	Conductivity, k (BTU/hr-ft-°F)	Diffusivity, d (ft ² /hr)	Specific Heat, c_p (BTU/lbm-°F) (see Note 5)	Poisson's Ratio (assumed)
SA533 Grade B, A508 Class II (see Note 2)	26.7	7.3	0.283	23.4	0.401	0.119	0.3
SS Clad (see Note 3)	27.0	9.8	0.283	9.8	0.160	0.125	0.3
A508 Class I (see Note 4)	28.1	7.3	0.283	32.3	0.561	0.118	0.3
A106 Grade B (see Note 4)	28.3	7.3	0.283	32.3	0.561	0.118	0.3

- Notes
1. The material properties applied in the analyses are taken from ASME Section II Part D 1998 Edition with 2000 Addenda. This is consistent with information provided in the Design Input Record (page 13 of VY EC No. 1773, SI File No. VY-16Q-209). The use of a later code edition than that used for the original design code is acceptable since later editions typically reflect more accurate material properties than was published in prior Code editions. Material Properties are evaluated at 300°F from the 1998 ASME Code, 2000 Addenda, Section II, Part D [9], except for density and Poisson's ratio, which are assumed typical values.
 2. Properties of A508 Class II are used (3/4Ni-1/2Mo-1/3Cr-V).
 3. Properties of 18Cr - 8Ni austenitic stainless steel are used.
 4. Composition = C-Si.
 5. Calculated as $k/(\rho d)/12^3$.

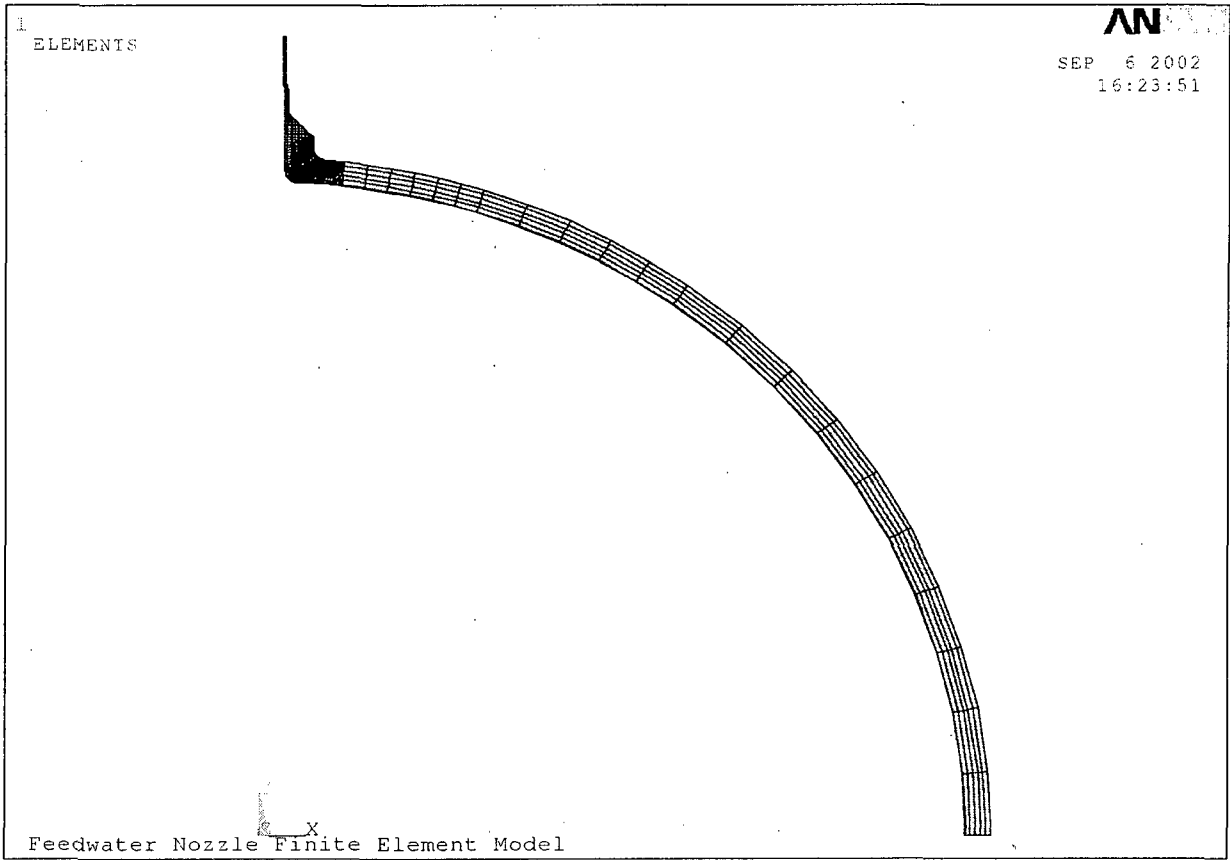


Figure 2-1: VY Feedwater Nozzle FEM

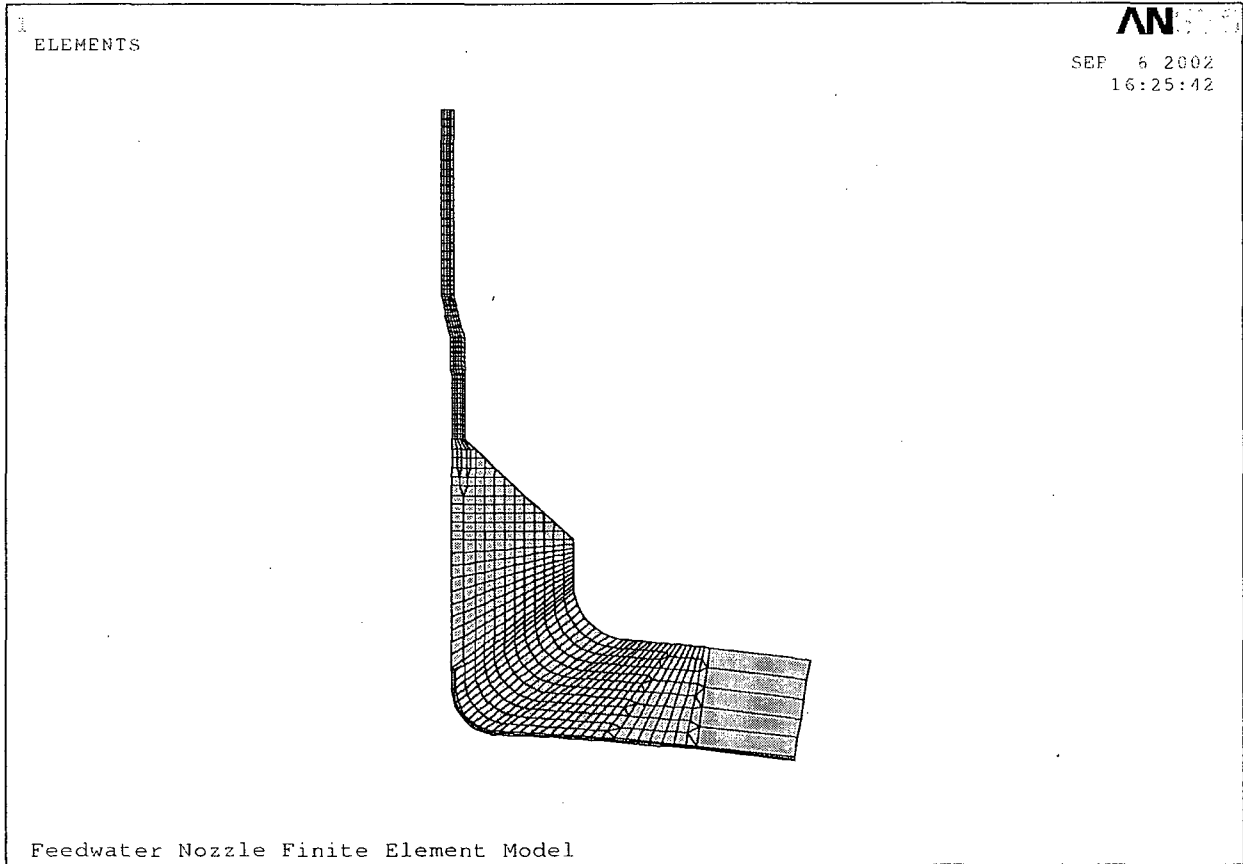


Figure 2-2: VY Feedwater Nozzle FEM – Safe End/Nozzle Region

3.0 LOAD DEFINITIONS

The pressure and thermal stresses for the feedwater nozzle for the revised fatigue evaluation were developed using the axisymmetric FEM model described in Section 2.0 of this report. The details of the Green's function development and associated stress evaluation are documented in the Reference [10] calculation.

3.1 Thermal Loading

Thermal loads are applied to the feedwater nozzle model. The heat transfer coefficients after power uprate were determined in Reference [10]. These values were determined for various regions of the finite element model and for 100% (4,590 GPM), 40% (1836 GPM) and 25% (1,148 GPM) [10]. The annulus leakage flow rate is assumed to be 25 GPM for non-EPU conditions and 31 GPM for EPU conditions. The 25 GPM value is calculated by scaling the 23 GPM [Page 6, 13] value up by approximately 9%. The 23 GPM value is scaled up to provide some conservatism and allow for inaccuracies in the determination of leakage flow. The 31 GPM value is calculated by multiplying the 25 GPM value by 1.25 [Page 6, 13]. Based on this, the annulus leakage flow rate is assumed to be 8 GPM for EPU conditions with 25% flow rate and 13 GPM for EPU condition with 40% flow rate. The temperatures used are based upon a thermal shock from 500°F to 100°F.

3.1.1 Heat Transfer Coefficients and Boundary Fluid Temperatures

Referring to Figure 3-4, heat transfer coefficients were applied as follows:

- The heat transfer coefficient for the outside surfaces of the FEM (Region 8) was a constant value of 0.2 BTU/hr-ft²-°F (3.858x10⁻⁷ BTU/sec-in²-°F).
- Table 3-3 shows a sampling of the heat transfer coefficient calculations for Region 1 for the 40% flow case.

For all Green's Functions, a 500°F to 100°F thermal shock was run to determine the stress response.

The applied heat transfer coefficients and the initial temperatures for all regions are contained in Reference [10].

3.1.2 *Green's Function's*

Three flow dependent thermal load cases were run on the FEM model with the heat transfer coefficients and the fluid temperature conditions listed above. Two locations were selected for analysis (see Figures 3-5 and 3-6):

1. The critical safe end location was chosen as the node with the highest stress intensity due to thermal loading under high flow conditions. The highest stress intensity due to thermal loading occurred at Node 192 (see Figure 3-5), on the inside diameter of the nozzle safe end, and therefore, this node was selected for analysis. Because the safe end stress response is affected by flow, three flow conditions were analyzed (100%, 40% and 25%).
2. The critical blend radius location was chosen, based upon the highest pressure stress. Conservatively assuming the cladding has cracked, the critical location is selected as node 657 at base metal of the nozzle, as shown in Figure 3-6. Because the blend radius stress response is affected by flow, three flow conditions were analyzed (100%, 40% and 25%).

Two stress intensity time history were developed for each location and each flow case: (1) total stress intensity, and (2) membrane plus bending stress intensity. The stress time histories for the safe end location, where the maximum stress was obtained for each of the flow conditions, are shown in Figures 3-7 through 3-12. The stress time histories for the blend radius location, where the maximum stress was obtained for each of the flow conditions, are shown in Figures 3-13 through 3-18.

3.1.3 Thermal Transients (for program STRESS.EXE)

The program STRESS.EXE requires the following three input files for analyzing an individual transient:

- Green.dat. There are 12 stress history functions obtained from Reference [10]. They represent the membrane plus bending and total stress intensities at the blend radius and safe end locations. Both of the blend radius and the safe end have two stress history functions for each of the following flow conditions; 100%, 40%, and 25% flow.
- Green.cfg is configured as described in Reference [14].
- Transnt.inp. These files are created to represent the transients shown on the thermal cycle diagrams and redefined by power uprate. Note that transients 12, 13, and 15 are nearly identical on the thermal cycle diagram [19] and the results from running transient 12 will be used for all three transients. Transient 16, 17 and 18 will not be considered since there is no temperature change. Tables 3-4 and 3-5 show the thermal history used to represent each transient. Based upon the thermal cycle diagram for the feedwater nozzle [19], the transients are split into the following groups based upon flow rate:
 - Transients 3, 20, 20A, and 21-23 are run at 25% flow. Although Reference [19] shows 15% flow rate, it is conservative to use 25% flow rate for these transients. Transient 20, Hot Standby, is split up into two parts. The first portion is “Heatup portion” and the second portion is “Feedwater Injection portion” that are defined from Reference [19].
 - Transient 11 is run at 40% flow. Transient 11 starts off and ends at 100% flow.
 - Transients 5, 6, 9, 10, and 19 are run at 100% flow.
 - Transient 4 is run at 100% flow only to obtain the last stress point. The remainder of the stress points for transient 4 is obtained from the 25% flow stress results. The results are pulled from the two flow case results based upon the flow rates defined in the thermal cycle diagram [19].
 - Transients 12, 13, 14 and 15 were run at 100% flow. Heat transfer coefficients were not re-calculated for the 1 minute intervals each of these transients is at 110% flow. The effect of this small flow rate increase for such a relatively short duration should be minor.

- Transients 1, 2, 24, and 25 are set as no thermal stress due to very small temperature changes (70°F to 100°F) at these transients.

3.2 Pressure Loading

A uniform pressure of 1,000 psi was applied along the inside surface of the feedwater nozzle and the vessel wall. A pressure load of 1,000 psi was used because it is easily scaled up or down to account for different pressures that occur during transients. In addition, a cap load was applied to the piping at the end of the nozzle. The nodal forces shown in Table 3-1 [10] are defined by the following equation:

$$F_{element} = \pi(IR)^2 P \cdot \frac{\pi(R_o^2 - R_i^2)}{\pi(OR^2 - IR^2)}$$

where:

P	=	unit pressure load = 1,000 psi
IR	=	inner pipe radius = 4.8345 in
OR	=	outer pipe radius = 5.42 in
R _i	=	inside radius of element that node is attached to
R _o	=	outside radius of element that node is attached to
F _{node}	=	average of the element forces on either side of the node.

Note: The force on the innermost and outermost nodes is calculated as one half of the force on the element that they are attached to.

The calculated nodal forces were applied as positive values so they would exert tension on the end of the model. Figures 3-1, 3-2, and 3-3 show the internal pressure distribution, cap load, and symmetry condition applied to the vessel end of the model, respectively.

The pressure stress associated with a 1000 psi internal pressure was determined in Reference [10]. These values are as follows:

Pressure stress for the safe end:

- 8693 psi membrane plus bending stress intensity.
- 8891 psi total linearized stress intensity.

Pressure stress for the blend radius:

- 36653 psi membrane plus bending stress intensity.
- 37733 psi total linearized stress intensity.

These pressure stress values for each location were linearly scaled with pressure. The actual pressure for column 6 of Tables 4-1 and 4-2 is obtained from Tables 3-4 and 3-5. The scaled pressure stress values are shown in columns 7 and 8 of Tables 4-1 and 4-2.

The pressure stress is combined with the thermal and piping loads to calculate the final stress values used for fatigue analysis. The piping load sign is set as the same as the thermal stress sign.

3.3 Piping Loading

Additionally, the piping stress intensity (stress caused by the attached piping) was determined. These piping forces and moments are determined as shown in Figure 3-36.

The following formulas are used to determine the maximum stress intensity in the nozzle at the two locations of interest. From engineering statics, the piping loads at the end of the model can be translated to the first and second cut locations using the following equations:

$$\begin{aligned} \text{For Cut I:} \quad (M_x)_1 &= M_x - F_y L_1 \\ (M_y)_1 &= M_y + F_x L_1 \end{aligned}$$

$$\begin{aligned} \text{For Cut II:} \quad (M_x)_2 &= M_x - F_y L_2 \\ (M_y)_2 &= M_y + F_x L_2 \end{aligned}$$

The total bending moment and shear loads are obtained using the equations below:

$$\begin{aligned} \text{For Cut I: } M_{xy} &= \sqrt{(M_x)_1^2 + (M_y)_1^2} \\ F_{xy} &= \sqrt{(F_x)_1^2 + (F_y)_1^2} \end{aligned}$$

$$\begin{aligned} \text{For Cut II: } M_{xy} &= \sqrt{(M_x)_2^2 + (M_y)_2^2} \\ F_{xy} &= \sqrt{(F_x)_2^2 + (F_y)_2^2} \end{aligned}$$

The distributed loads for a thin-walled cylinder are obtained using the equations below:

$$\begin{aligned} N_z &= \frac{1}{\pi R_N} \left[\frac{1}{2} F_z + \frac{M_{xy}}{R_N} \right] \\ q_N &= \frac{1}{\pi R_N} \left[F_{xy} - \frac{M_z}{2R_N} \right] \end{aligned}$$

To determine the primary stresses, P_M , due to internal pressure and piping loads, the following equations are used.

For Cut I, using thin-walled equations:

$$(P_M)_z = \frac{Pa_N}{2t_N} + \frac{Nz}{t_N}$$

$$(P_M)_\theta = \frac{Pa_N}{t_N}$$

$$(P_M)_R = -P$$

$$\tau_M = \frac{q_N}{t_N}$$

$$SI_{MAX} = 2\sqrt{\left(\frac{(P_M)_\theta - (P_M)_R}{2}\right)^2 + (\tau_M)_{z\theta}^2}$$

or

$$SI_{MAX} = 2\sqrt{\left(\frac{(P_M)_z - (P_M)_R}{2}\right)^2 + (\tau_M)_{z\theta}^2}$$

- where:
- L_1 = The length from the end of the nozzle where the piping loads are applied to the location of interest in the safe end.
 - L_2 = The length from the end of the nozzle where the piping loads are applied to the location of interest in the blend radius.
 - M_{xy} = The maximum bending moment in the xy plane.
 - F_{yx} = The maximum shear force in the xy plane.
 - N_z = The normal force per inch of circumference applied to the end of the nozzle in the z direction.
 - q_N = The shear force per inch of circumference applied to the nozzle.
 - R_N = The mid-wall nozzle radius.

Because pressure was not considered in this analysis, the equations used for Cut I are valid for Cut II. Furthermore, since the pressure was not considered in this analysis, the equations can be simplified as follows:

$$(P_M)_z = \frac{Nz}{t_N}$$

$$(P_M)_\theta = 0$$

$$(P_M)_R = 0$$

$$\tau_M = \frac{q_N}{t_N}$$

$$SI_{MAX} = 2(\tau_M)_{z,\theta}$$

OR

$$SI_{MAX} = 2\sqrt{\left(\frac{Nz}{2t_N}\right)^2 + (\tau_M)_{z,\theta}^2}$$

Per Reference [11], the feedwater nozzle piping loads are as follows:

$$F_x = 3,000 \text{ lbs}$$

$$M_x = 28,000 \text{ ft-lb} = 336,000 \text{ in-lb}$$

$$F_y = 15,000 \text{ lbs}$$

$$M_y = 13,000 \text{ ft-lb} = 156,000 \text{ in-lb}$$

$$F_z = 3,200 \text{ lbs}$$

$$M_z = 40,000 \text{ ft-lb} = 480,000 \text{ in-lb}$$

The loads are applied at the connection of the piping and safe end. Therefore, the L_1 is equal to 12.0871 inches and the L_2 is equal to 27.572 inches. The calculations for the safe end and blend radius are shown in Table 3-2. The first cut location is the same as the Green's Function cross section per [10] at the safe end, and the second cut is from Node 645 (outside) to Node 501 (inside). The maximum stress intensities due to piping loads are 5707.97 psi at the safe end and 265.47 psi at the blend radius, respectively.

These piping stress values are scaled assuming no stress occurs at an ambient temperature of 70°F and the full values are reached at reactor design temperature, 575°F. The scaled piping stress values are shown in columns 9 and 10 of Tables 4-1 and 4-2. Columns 11 and 12 of Tables 4-1 and 4-1 show the summation of all stresses for each thermal peak and valley stress point.

Table 3-1: Nodal Force Calculation for End Cap Load

Node Number	Element Number	Radius (in)	Δ Radius (in)	$R_o^2 - R_i^2$ (in ²)	$F_{element}$ (lb)	F_{node} (lb)
1		5.42				7678.0
	1022		0.1171	1.25565	15356.1	
2		5.3029				15188.4
	1021		0.1171	1.22823	15020.7	
3		5.1858				14853.0
	1020		0.1171	1.20080	14685.3	
4		5.0687				14517.6
	1019		0.1171	1.17338	14349.9	
5		4.9516				14182.2
	1018		0.1171	1.14595	14014.5	
6		4.8345				7007.3

Table 3-2: Maximum Piping Stress Intensity Calculations

Safe End External Piping Loads		
Parameters		
$F_x =$	3.00	kips
$F_y =$	15.00	kips
$F_z =$	3.20	kips
$M_x =$	336.00	in-kips
$M_y =$	156.00	in-kips
$M_z =$	480.00	in-kips
OD=	11.86	in
ID=	10.409	in
$R_N =$	5.57	in
L =	12.09	in
$t_N =$	0.72	in
$(M_x)_1 =$	154.69	in-kips
$(M_y)_1 =$	192.26	in-kips
$M_{xy} =$	246.77	in-kips
$F_{xy} =$	15.30	kips
$N_z =$	2.63	kips/in
$q_N =$	-1.59	kips/in
Primary Membrane Stress Intensity		
$PM_z =$	3.63	ksi
$\tau =$	-2.20	ksi
$SI_{max} =$	5.71	ksi
$SI_{max} =$	5707.97	psi

Blend Radius External Piping Loads		
Parameters		
$F_x =$	3.00	kips
$F_y =$	15.00	kips
$F_z =$	3.20	kips
$M_x =$	336.00	in-kips
$M_y =$	156.00	in-kips
$M_z =$	480.00	in-kips
OD=	22.67	in
ID=	10.750	in
$R_N =$	8.35	in
L =	27.57	in
$t_N =$	5.96	in
$(M_x)_2 =$	-77.58	in-kips
$(M_y)_2 =$	238.72	in-kips
$M_{xy} =$	251.01	in-kips
$F_{xy} =$	15.30	kips
$N_z =$	1.21	kips/in
$q_N =$	-0.51	kips/in
Primary Membrane Stress Intensity		
$PM_z =$	0.20	ksi
$\tau =$	-0.09	ksi
$SI_{max} =$	0.27	ksi
$SI_{max} =$	265.47	psi

Note: The locations for Cut I and Cut II were defined in Reference [10] for safe end and blend radius paths, respectively.

Table 3-4: Blend Radius Transients

Transient Number	Time (s)	Temp (°F)	Time Step (s)	Pressure (psia)	Transient Number	Time (s)	Temp (°F)	Time Step (s)	Pressure (psia)	Transient Number	Time (s)	Temp (°F)	Time Step (s)	Pressure (psia)
1. Soft-up 123 Cycles	9	70	10	0	10. FW Heater Bypass	0	392	1010	1010	14. SRV Blowdown	0	392	1010	1010
2. Design HYD Test 120 Cycles	0	70	10	0	60	265	50	1010	1010	60	275	60	895	895
	1880	100	3600	1100	1890	265	1000	1010	1010	960	100	900	50	50
	5280	100	3600	1100	2070	392	180	1010	1010	5960	100	5000	50	50
	5880	100	3600	50	7070	392	5000	1010	1010	0	392	1010	1010	1010
	10880	100	5000	50	11. Loss of FW Pumps	0	392	1	1010	1010	1800	265	1600	1010
3. Startup 300 Cycles	0	100	50	1010	1	565	1	1010	1010	6800	265	5000	1010	1010
LF 25	16164	549	16164	1010	3.5	565	2.5	1100	1100	0	265	5000	1010	1010
	21164	549	5000	1010	13.5	50	0	1135	1135	20. Hot Standby (Heatup Portion)	1	440	1	1010
4. Turbine Flall and increased to Rated Power	0	549	1010	1010	184.5	50	171	1135	1135	390 Cycles	3925	549	3924	1010
	1801	100	1	1010	1564.5	440	1380	1135	1135	LF 25	8925	549	5000	1010
	1802	100	1	1010	1565.5	565	1	1135	1135	20A. Hot Standby (FW Injection Portion)	0	549	1010	1010
	3602	392	1800	1010	2165.5	565	600	1135	1135	1	100	1	1010	1010
	3602	392	1800	1010	2346.5	50	180	1135	1135	181	100	180	1010	1010
	8602	392	5000	1010	2446.5	50	1	1135	1135	390 Cycles	241	290	60	1010
	0	392	5000	1010	5406.5	440	3050	1055	1055	LF 25	451	549	210	1010
5. Daily Reduction 75% Power 10,000 Cycles	0	392	1010	1010	5407.5	565	1	1055	1055	5451	549	5000	1010	1010
HF 100	900	310	900	1010	6727.5	565	1320	1135	1135	0	549	1010	1010	1010
	2700	310	1800	1010	6728.5	50	1	1135	1135	21-23. Shutdown 300 Cycles	6954	315	6264	50
	3600	392	900	1010	7148.5	59	420	675	675	LF 25	6954	330	630	50
	8600	392	5000	1010	7448.5	300	300	675	675	15144	100	8280	50	50
6. Weekly Reduct. 50% Power 2,000 Cycles	0	392	1010	1010	11048.5	400	3600	232	232	20144	100	5000	50	50
HF 100	1800	280	1800	1010	16411.5	549	5363	885	885	0	100	50	50	50
	3500	280	1800	1010	16412.5	549	1	1010	1010	600	100	600	1563	1563
	5400	392	1800	1010	18212.5	549	1800	1010	1010	1200	100	600	50	50
	10400	392	5000	1010	18213.5	100	1	1010	1010	1800	100	600	50	50
9. Turbine Trip at 25% Power	0	392	1010	1010	20013.5	100	1800	1010	1010	2400	100	600	50	50
	1800	265	1800	1010	20014.5	260	1	1010	1010	0	100	0	0	0
	2340	90	360	1010	21814.5	392	1800	1010	1010	1080	70	1680	0	0
	2520	50	1010	1010	28814.5	392	5000	1010	1010	5080	70	5000	0	0
10 Cycles HF 100	3420	265	900	1010	12. Turbine Generator Trip	0	392	1010	1010	0	392	1010	1010	1010
	3800	180	1010	1010	10	392	10	1135/1375	1135/1375	10	392	10	1135/1375	1135/1375
	5400	392	1800	1010	15	392	5	1135/1375	1135/1375	66 Cycles	15	392	5	1135/1375
	10400	392	5000	1010	13. Reactor Overpressure	30	392	15	940	30	392	15	940	940
	0	392	1010	1010	90	275	60	940	940	1 Cycles	990	100	900	940
	2790	100	1800	940	15. Other SCRAMs	2790	100	1800	940	2790	100	1800	940	940
	2791	260	1	940	228 cycles	2280	260	1	1010	2280	260	1	1010	1010
	4591	392	1381	1010	HF 100	4591	392	1381	1010	4591	392	1381	1010	1010
	5591	392	5000	1010	5591	392	5000	1010	1010	5591	392	5000	1010	1010

Note: 1. The indicated time or pressure was assumed.
2. 1375 psi is for Transient 13 only.

Table 3-5: Safe End Transient

Transient Number	Time (s)	Temp (°F)	Time Step (s)	Pressure (psia)	Transient Number	Time (s)	Temp (°F)	Time Step (s)	Pressure (psia)	Transient Number	Time (s)	Temp (°F)	Time Step (s)	Pressure (psia)
1. Start-up 120 Cycles	0 10	70 70	10 10	0 0	10. FW Hstator By-pass 70 Cycles HF 100	0 90 1800 2070 2570	392 265 265 392 392	10 90 1800 180 500	1010 1010 1010 1010 1010	14. SRV Blowdown 1 Cycles HF 100	0 60 100 1400	392 275 100 392	10 60 900 500	1010 285 50 50
2. Design HYD Test 120 Cycles	0 1980 1680 5280 5880 6390	70 100 100 100 100	0 1080 100 3600 600 500	0 1100 1100 50 50	11. Loss of FW Pumps 10 Cycles MF 40 HF 100	0 1 3.5 4.5 13.5 184.5 1564.5 1565.5 2165.5 2346.5 5406.5	392 565 50 50 50 50 440 565 600 50 180 440	0 1 1 1 6 171 1380 1 1 180 3600	1010 1010 1010 1185 1135 1135 1135 1135 1135 1055	19. Reduction in CR Power 300 Cycles HF 100	0 1800 2300	392 265 265	10 1800 500	1010 1010 1010
3. Startup 300 Cycles LF 25	0 18184 18664	100 549 549	0 16164 500	30 1010 1010	12. Turbine Generator Trip 60 Cycles 15 Reactor Overpressure 1 Cycles 5 Other SORAMS 228 cycles HF 100	0 10 15 30 90 990 2790 2791 3210 4591 5091	392 392 392 392 275 100 265 291 392 392	0 10 5 15 60 100 1800 1 1 419 1381 500	1010 1135/1375 1135/1375 940 940 940 540 940 1010 1010 1010	20. Fuel Stowage (Startup Position) 300 Cycles LF 25	0 3925 4425	265 440 549 549	1010 1010 1010 1010	
4. Turbine Roll and increased to Rated Power 300 Cycles LF 25 HF 120	0 1 1801 1902 3302 4102	100 100 100 200 392 392	0 1 1800 1 1800 500	1010 1010 1010 1010 1010 1010	21. Hydrostatic Test 1 Cycles	0 600 1200 1800 2400	549 100 100 100 100	0 1 1 1 1	1010 1010 1010 1010 1010	21-22. Shutdown 300 Cycles LF 25	0 6204 6854 15144 15644	549 275 330 100 100	0 6204 600 8280 500	1010 50 50 50 50
5. Daily Reduction 75% Power 10,000 Cycles HF 100	0 900 2700 3600 4100	392 310 310 392 392	0 900 1800 900 500	1010 1010 1010 1010 1010	24. Unbolt 120 Cycles	0 1000 1000	392 100 100	0 1 1	1010 1010 1010	24. Hydrostatic Test 1 Cycles	0 600 1200 1800 2400	100 100 100 100	0 600 600 600 600	1010 1563 1563 50 50
6. Weekly Recycle 50% Power 2,000 Cycles HF 100	0 1900 3600 5400 5800	392 280 280 392 392	0 1800 1800 1800 500	1010 1010 1010 1010 1010	25. Unbolt 120 Cycles	0 1000 1000	392 100 100	0 1 1	1010 1010 1010	25. Unbolt 120 Cycles	0 1000 1000	100 100	0 1000 1000	0 0 0
7. Turbine Trip at 25% Power 10 Cycles HF 100	0 2340 2520 3420 3500 5400 5500	392 90 90 265 265 392 392	0 300 90 900 150 1800 500	1010 1010 1010 1010 1010 1010	13.735 psi	1135/1375	1010	1010	1010	25. Unbolt 120 Cycles	0 1000 1000	100 100	0 1000 1000	0 0 0

- Note: 1. These transients are the same as in Table 3-4 with the exception of the 500 second steady state time increment that is used. The transients in Table 3-4 are plotted using a 5000 second steady state increment. The difference is due to the length of the Green's Function for the safe end which is shorter compared to the blend Radius.
2. The indicated time or pressure was assumed.
3. 1375 psi is for Transient 13 only.

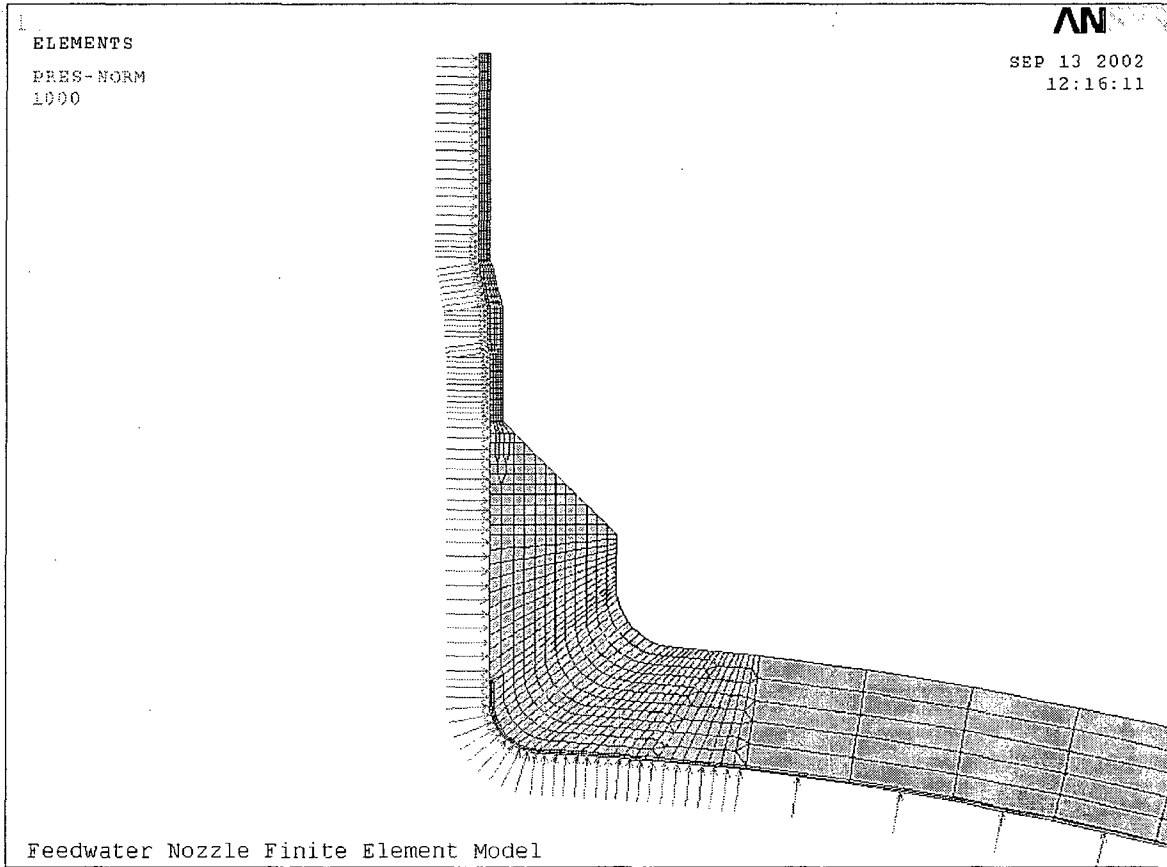


Figure 3-1: Feedwater Nozzle Internal Pressure Distribution

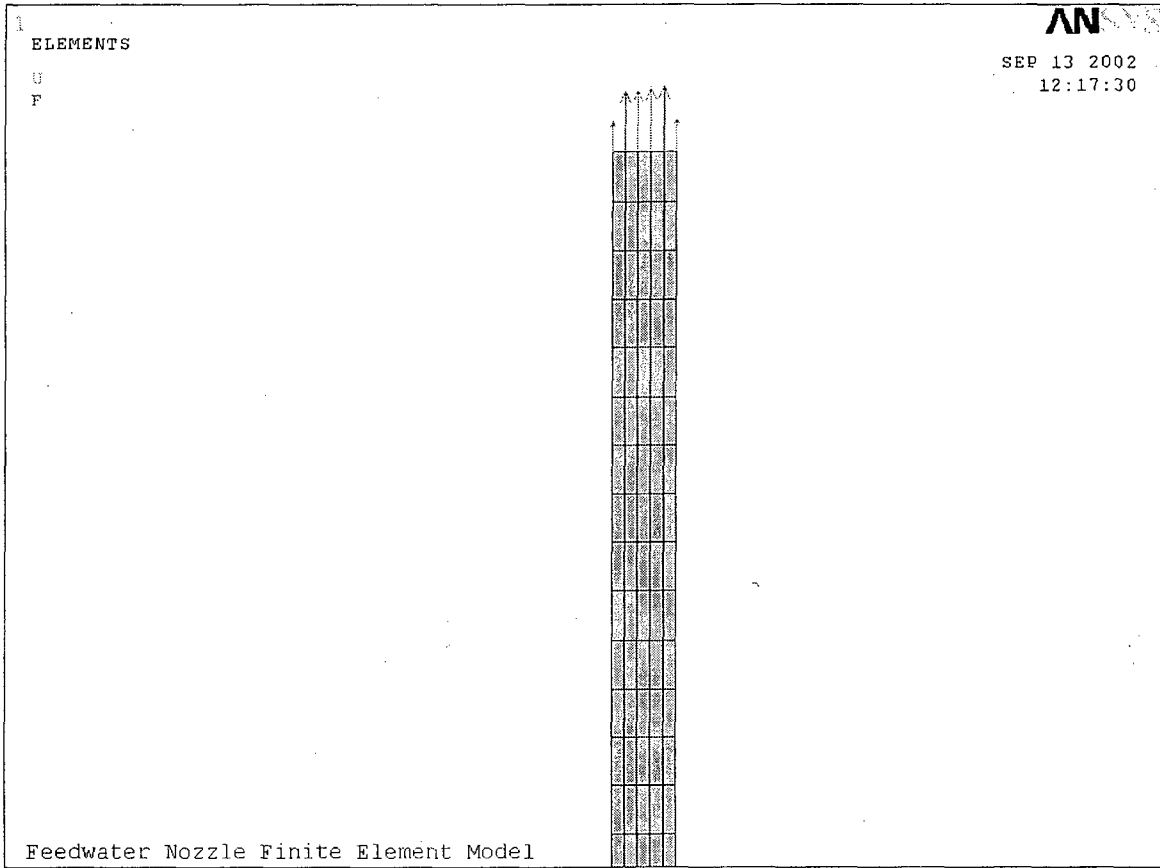


Figure 3-2: Feedwater Nozzle Pressure Cap Load

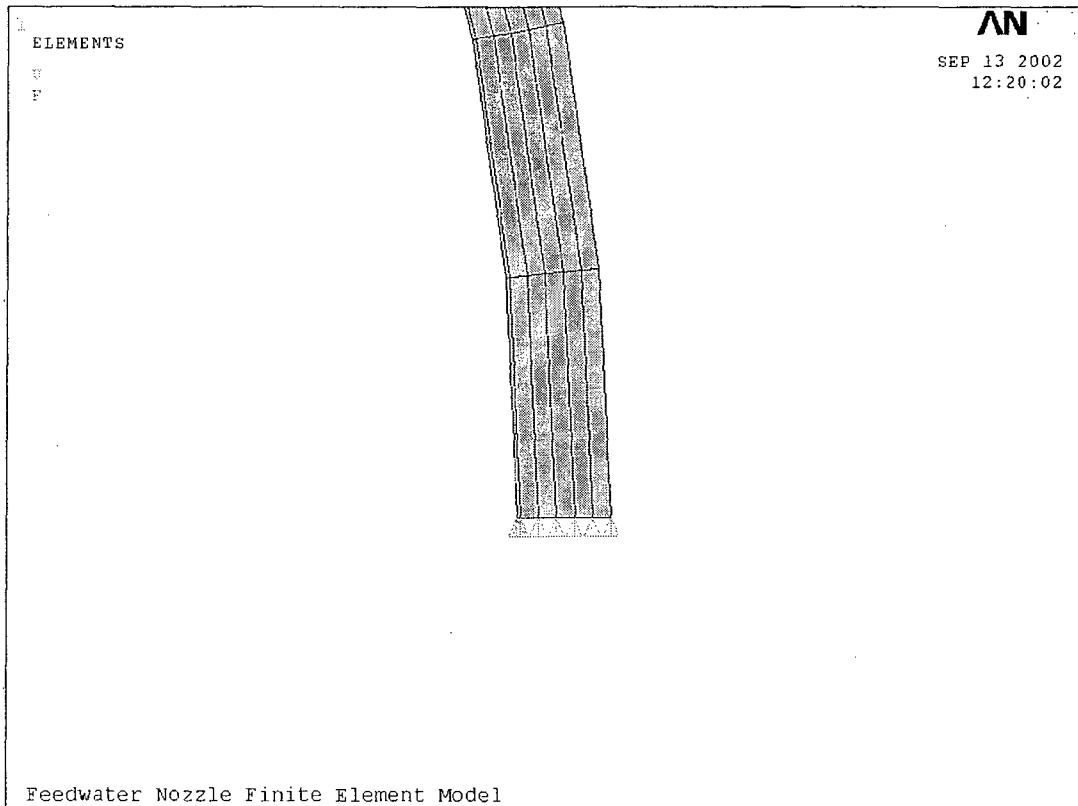
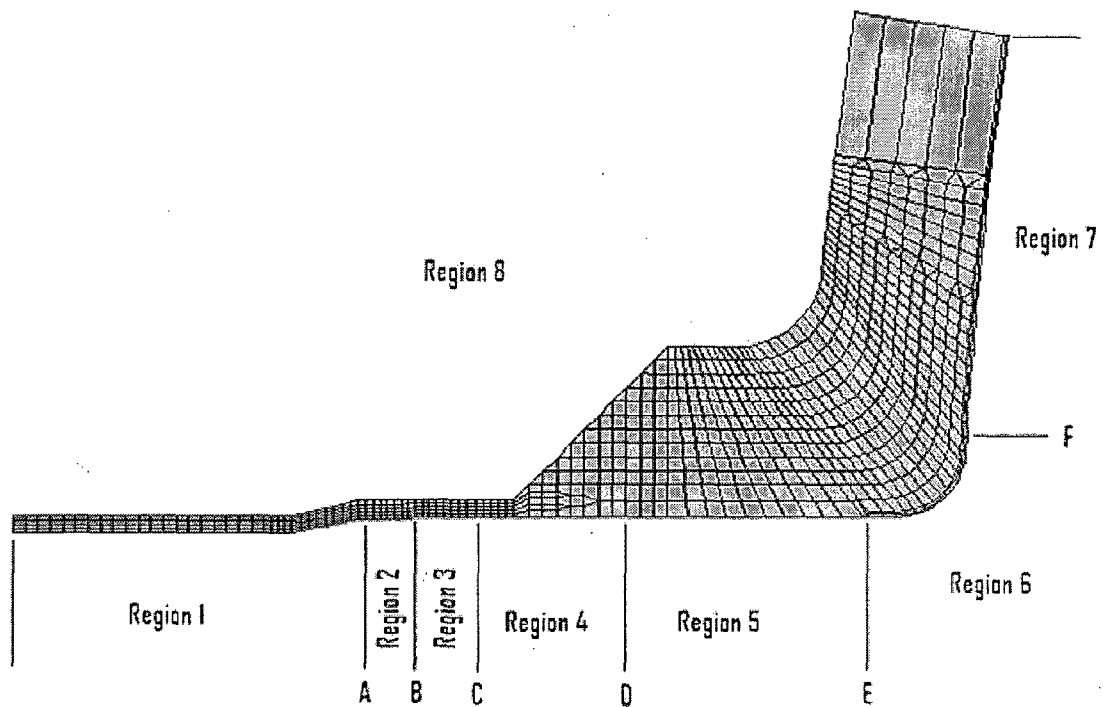


Figure 3-3: Feedwater Nozzle Vessel Boundary Condition



- Notes:
- Point A: End of thermal sleeve = Node 204 = 0.25" from feedwater inlet side of thermal sleeve flat per Reference [8].
 - Point B: Beginning of annulus = Node 252.
 - Point C: Beginning of thermal sleeve transition = approximately 4.0" from Point A per Reference [8] = Node 294.
 - Point D: End of thermal sleeve transition = approximately 9.5" from Point A per Reference [8] = Node 387.
 - Point E: End of inner blend radius (nozzle side) = Node 553.
 - Point F: End of inner blend radius (vessel wall side) = Node 779.

Figure 3-4: Thermal Regions

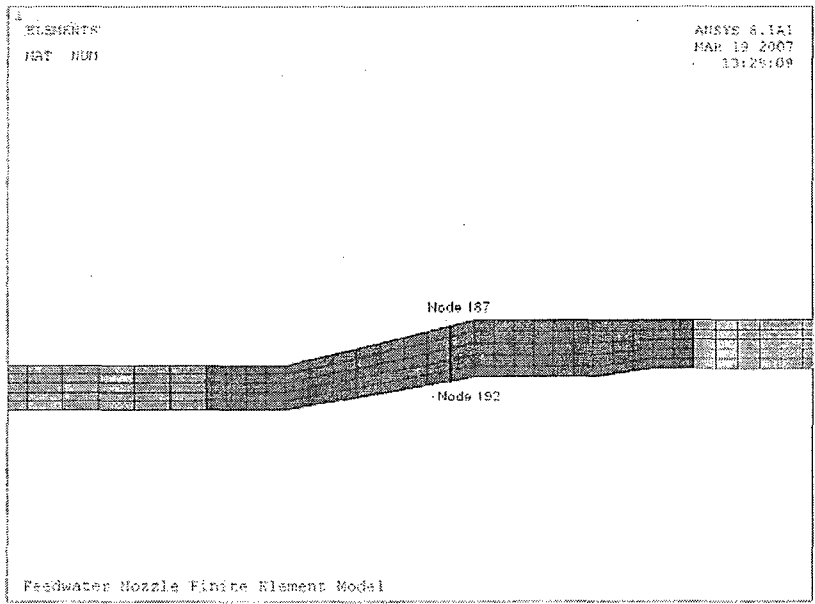
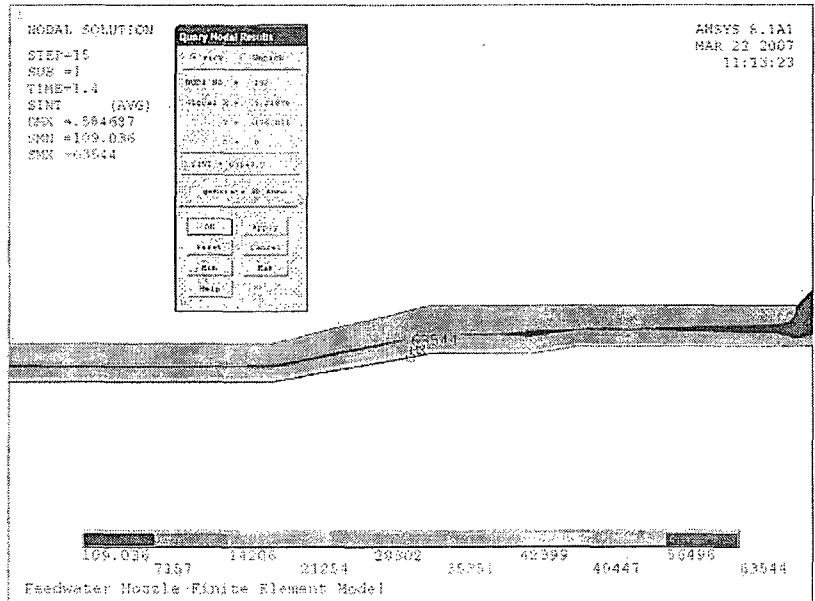


Figure 3-5: Safe End Critical Thermal Stress Location and Linearized Stress Paths

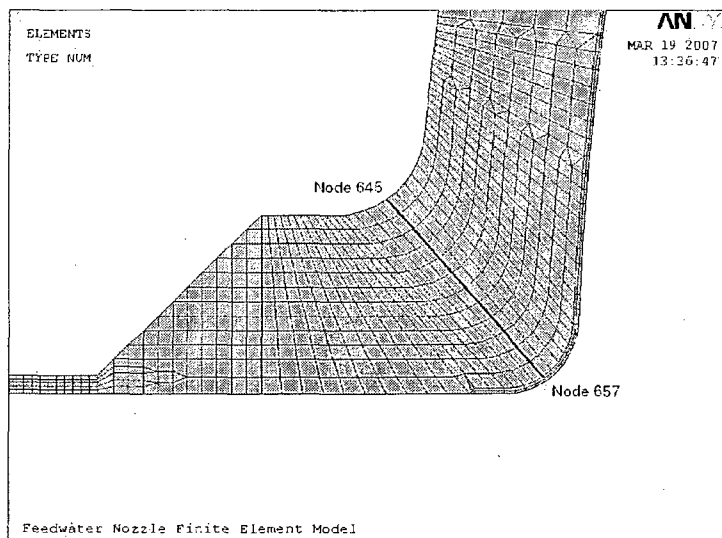
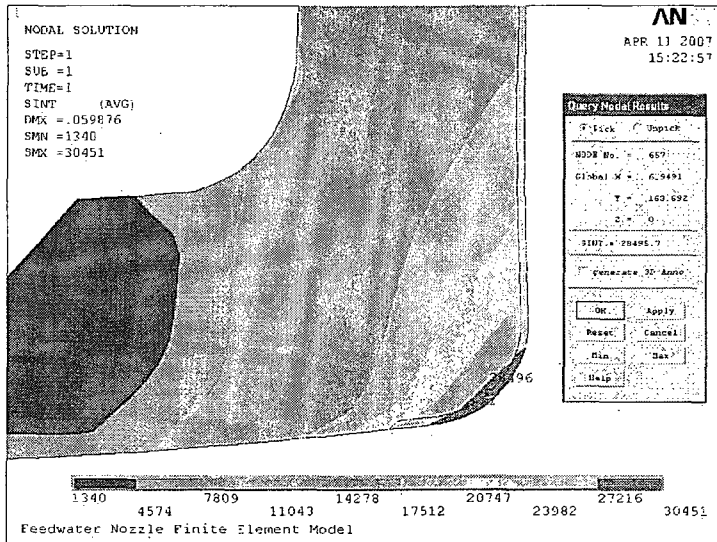


Figure 3-6: Brand Radius Critical Thermal Stress Location and Linearized Stress Paths

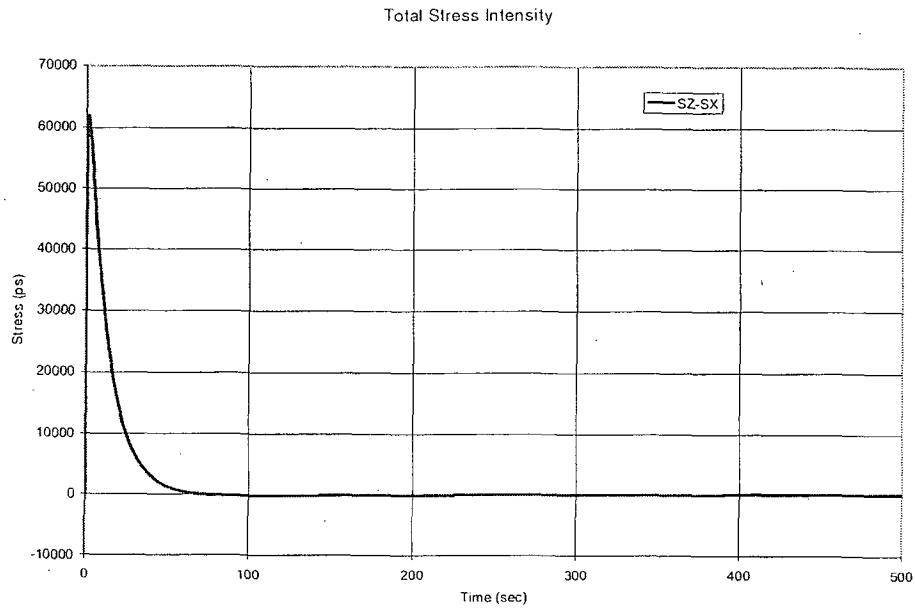


Figure 3-7: Safe End Total Stress History for 100% Flow

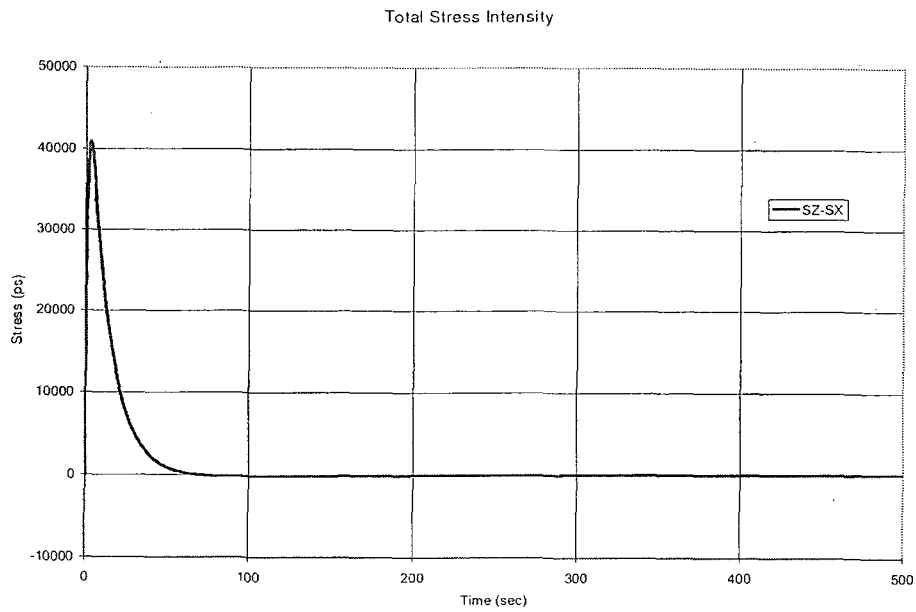


Figure 3-8: Safe End Membrane Plus Bending Stress History for 100% Flow

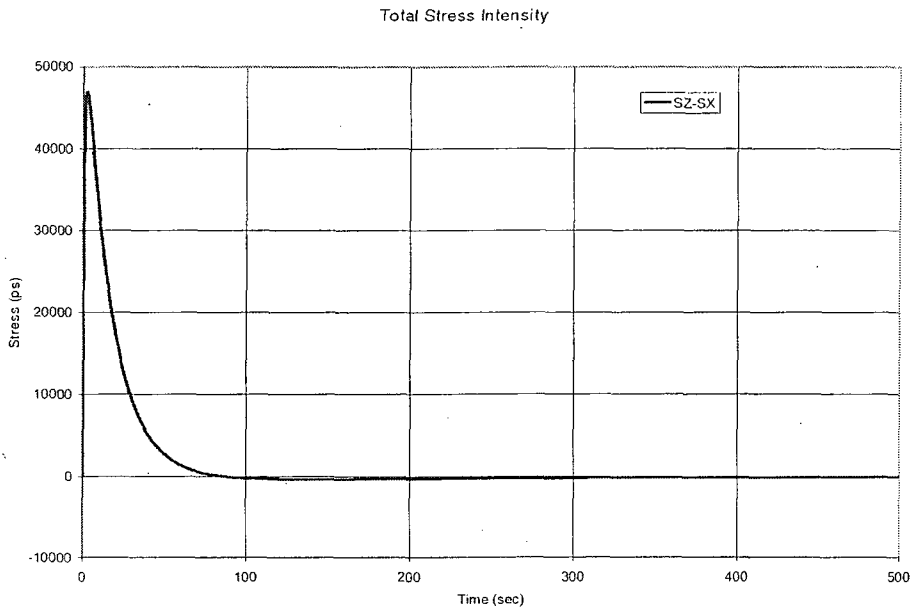


Figure 3-9: Safe End Total Stress History for 40% Flow

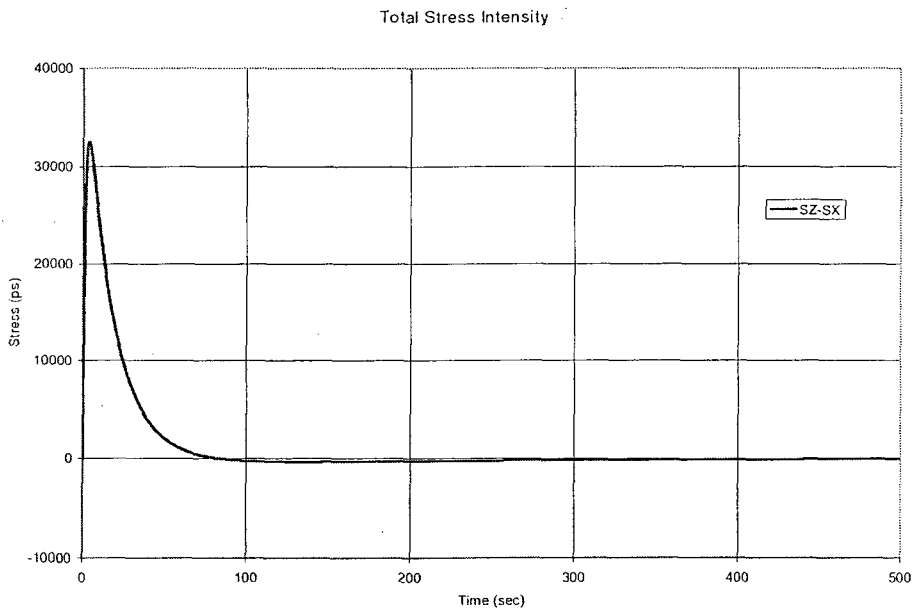


Figure 3-10: Safe End Membrane Plus Bending Stress History for 40% Flow

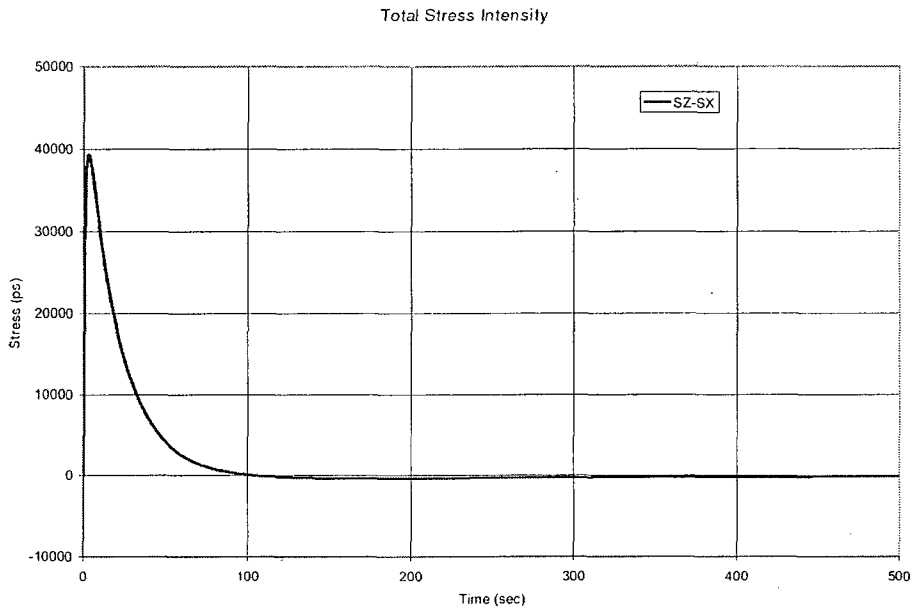


Figure 3-11: Safe End Total Stress History for 25% Flow

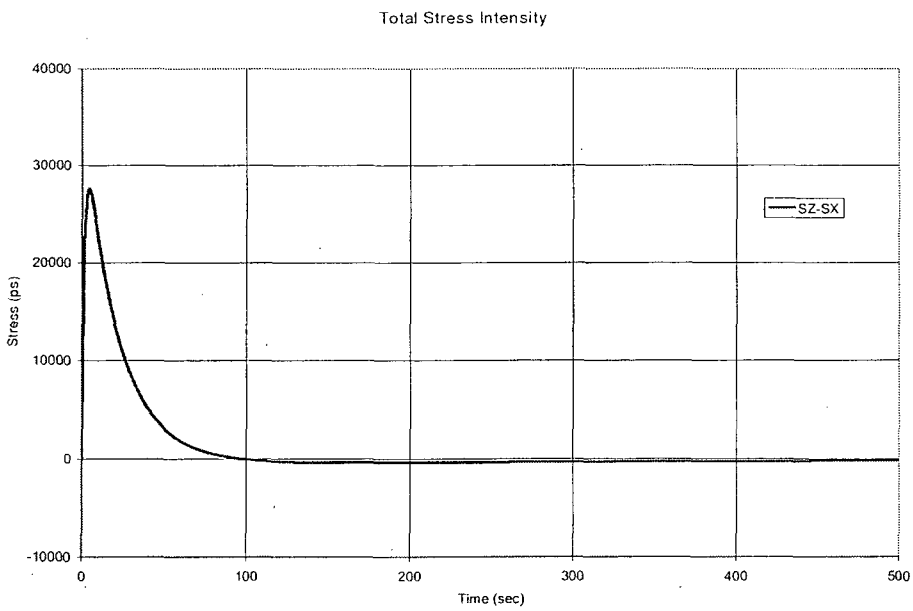


Figure 3-12: Safe End Membrane Plus Bending Stress History for 25% Flow

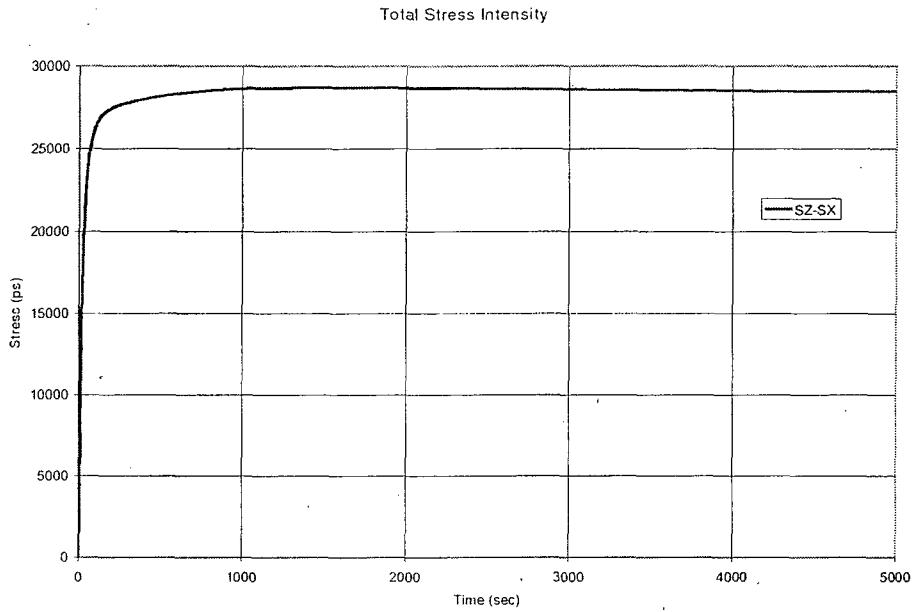


Figure 3-13: Blend Radius Total Stress History for 100% Flow

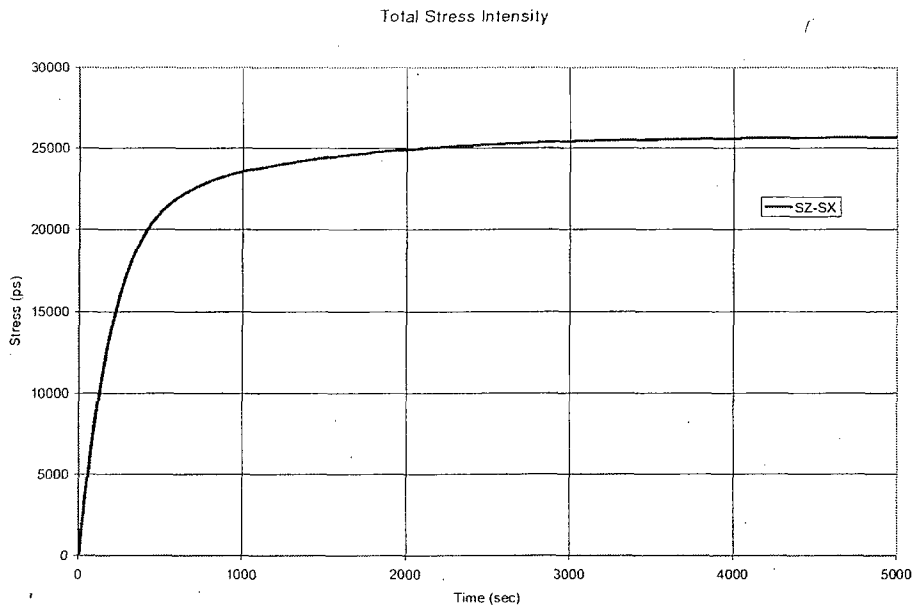


Figure 3-14: Blend Radius Membrane Plus Bending Stress History for 100% Flow

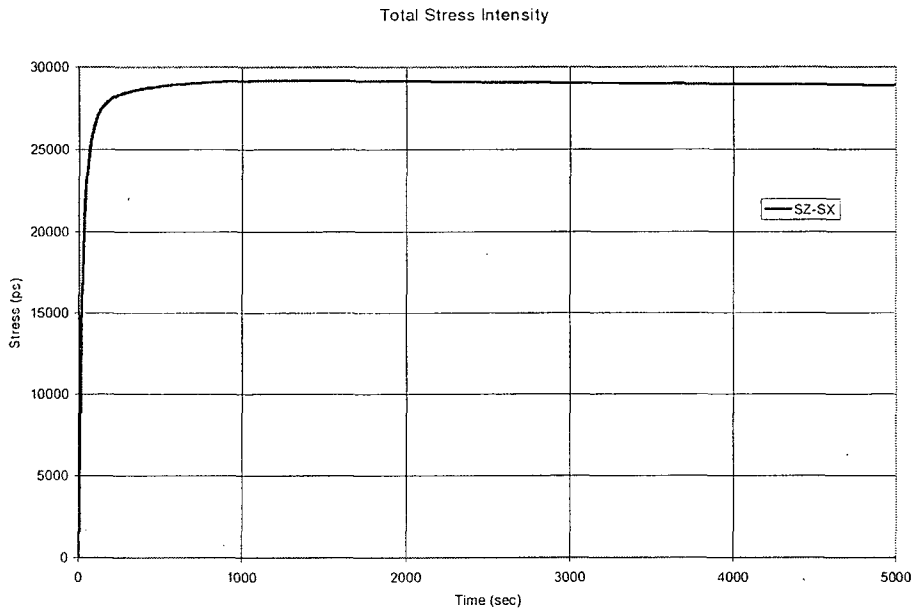


Figure 3-15: Blend Radius Total Stress History for 40% Flow

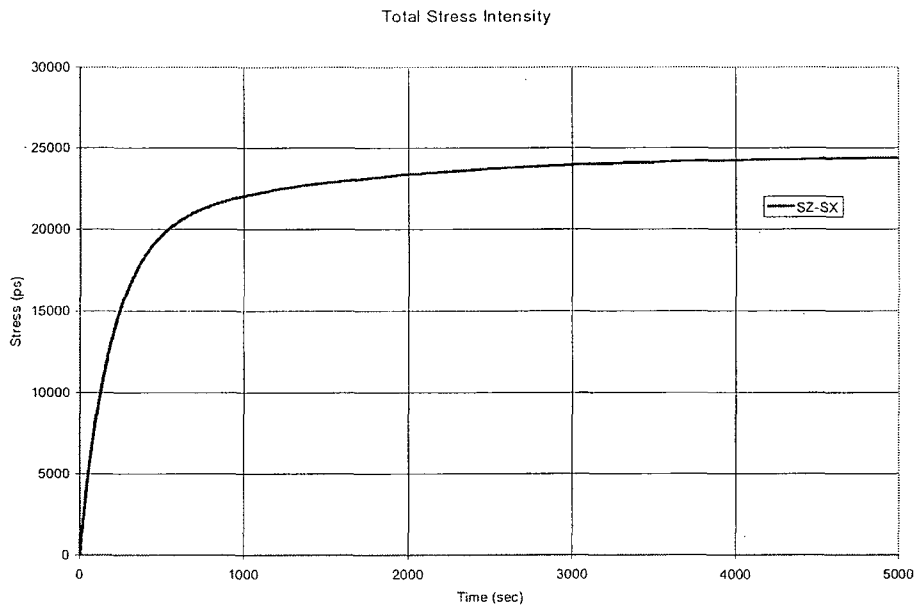


Figure 3-16: Blend Radius Membrane Plus Bending Stress History for 40% Flow

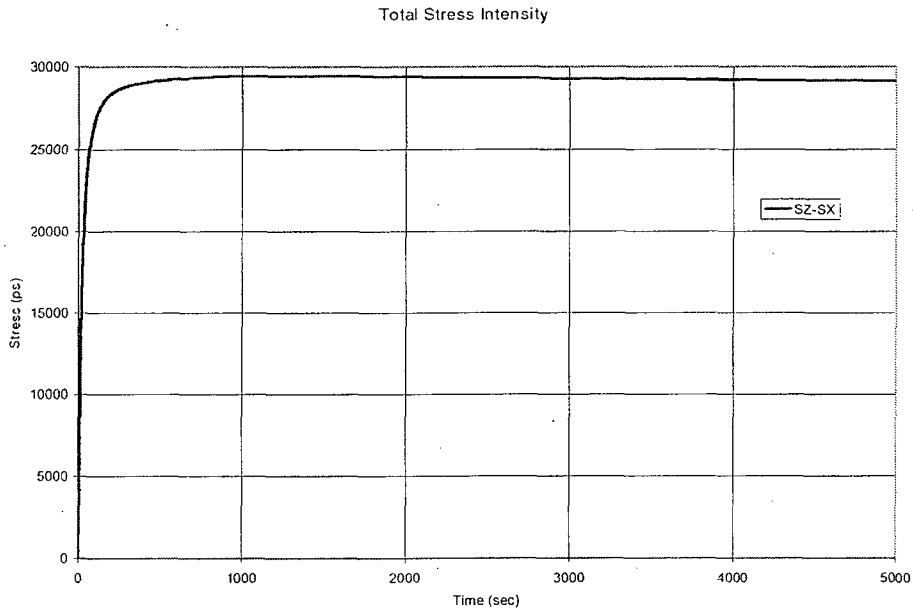


Figure 3-17: Blend Radius Total Stress History for 25% Flow

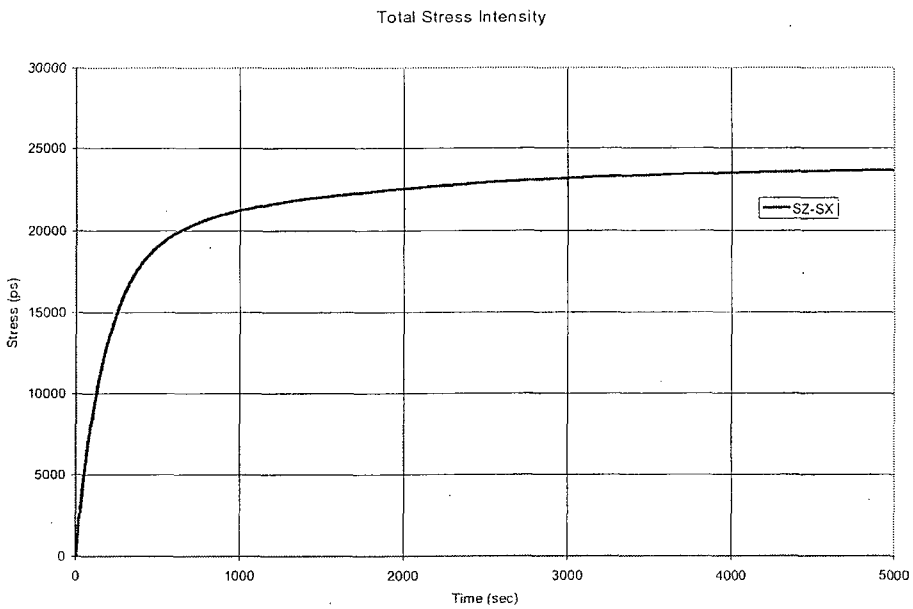


Figure 3-18: Blend Radius Membrane Plus Bending Stress History for 25% Flow

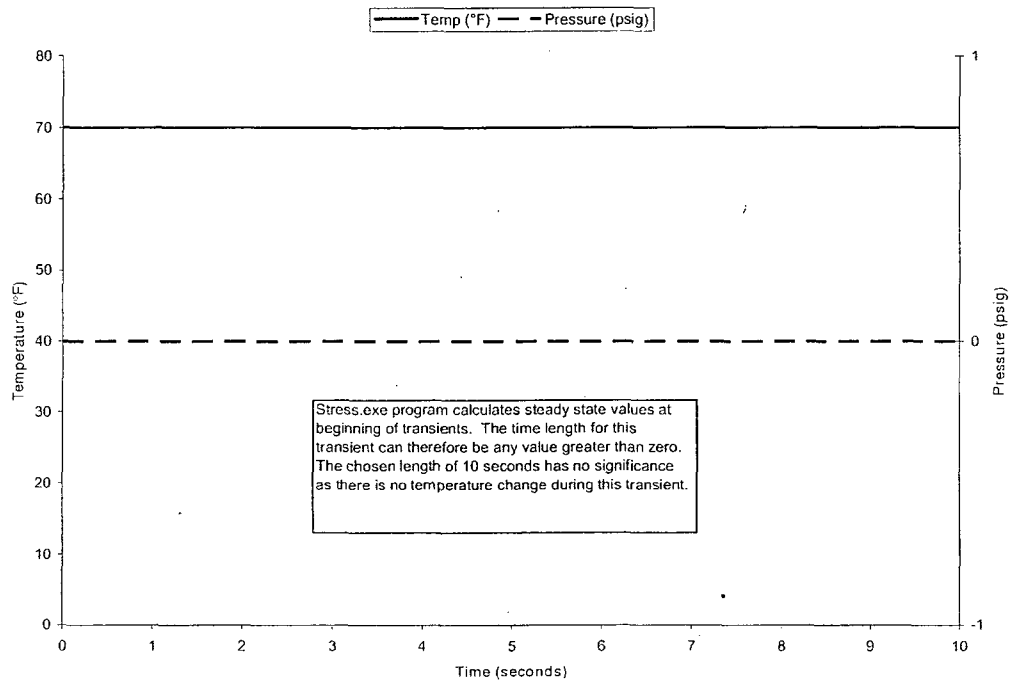


Figure 3-19: Transient 1, Bolt-up

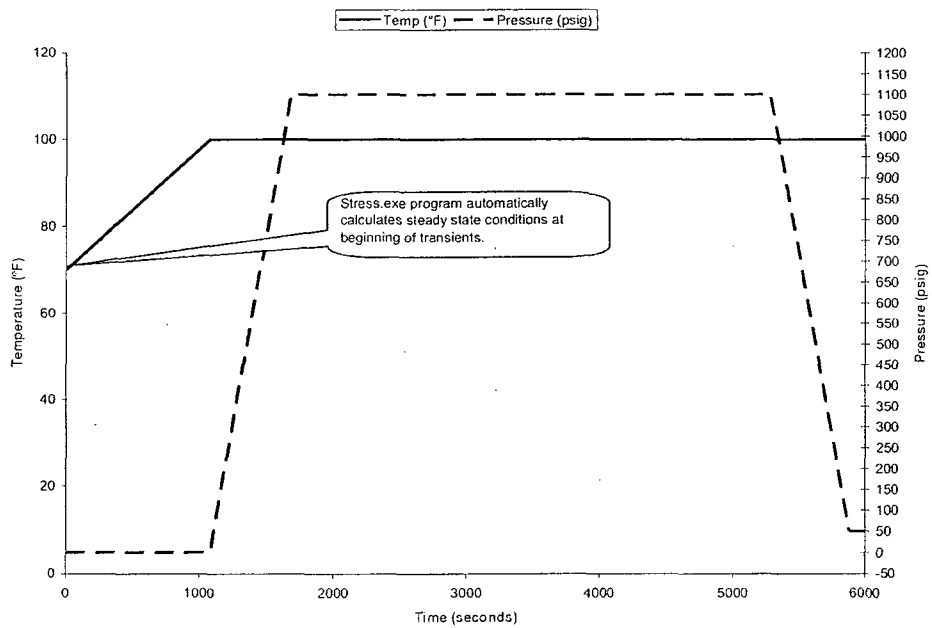


Figure 3-20: Transient 2, Design HYD Test

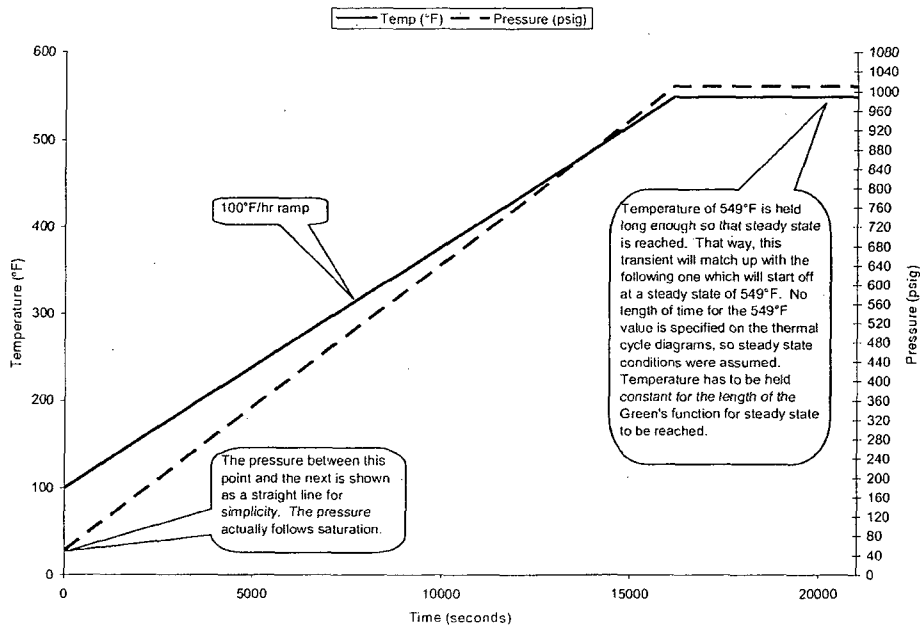


Figure 3-21: Transient 3, Startup

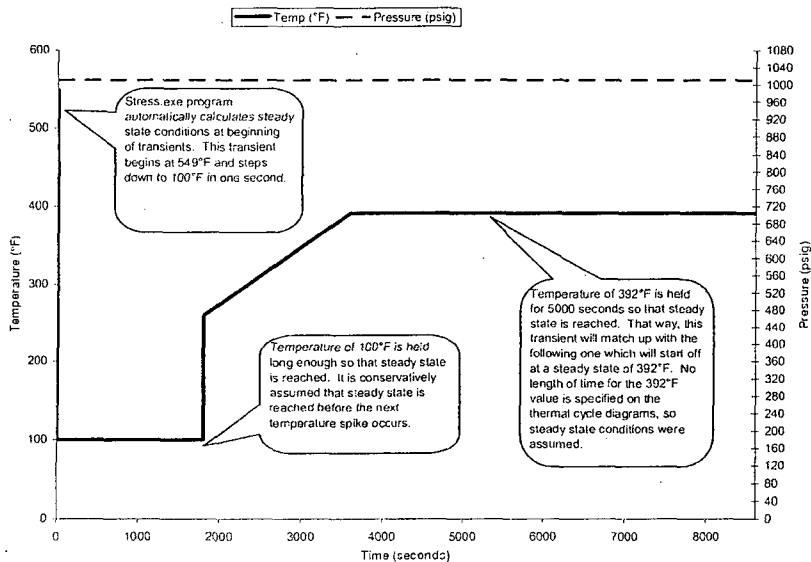


Figure 3-22: Transient 4, Turbine Roll and Increased to Rated Power

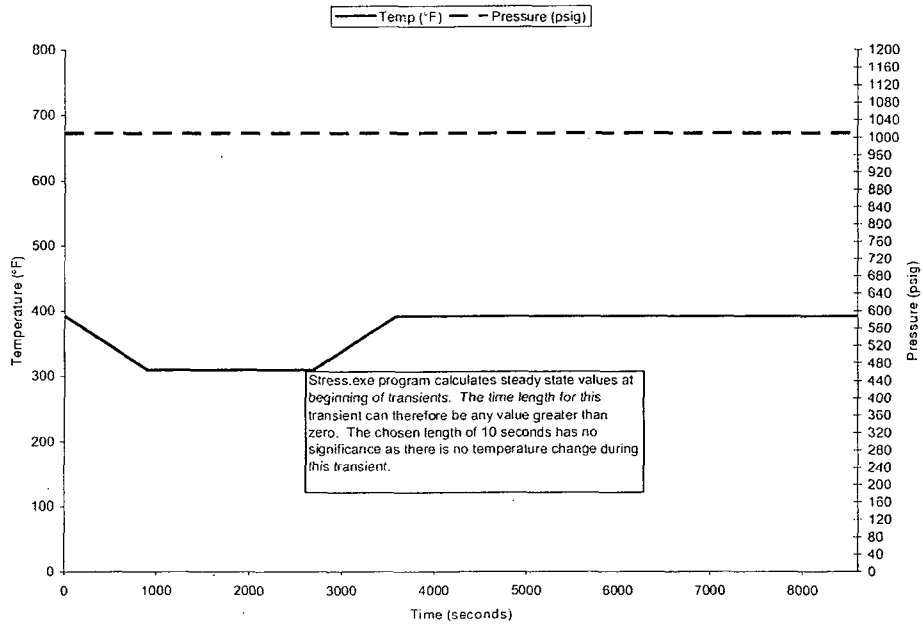


Figure 3-23: Transient 5, Daily Reduction 75% Power

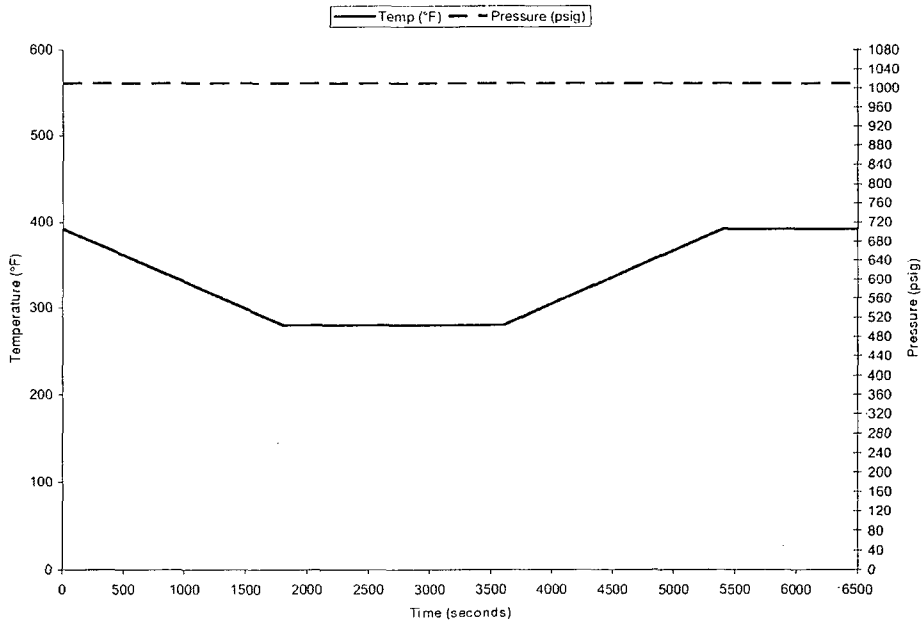


Figure 3-24: Transient 6, Weekly Reduction 50% Power

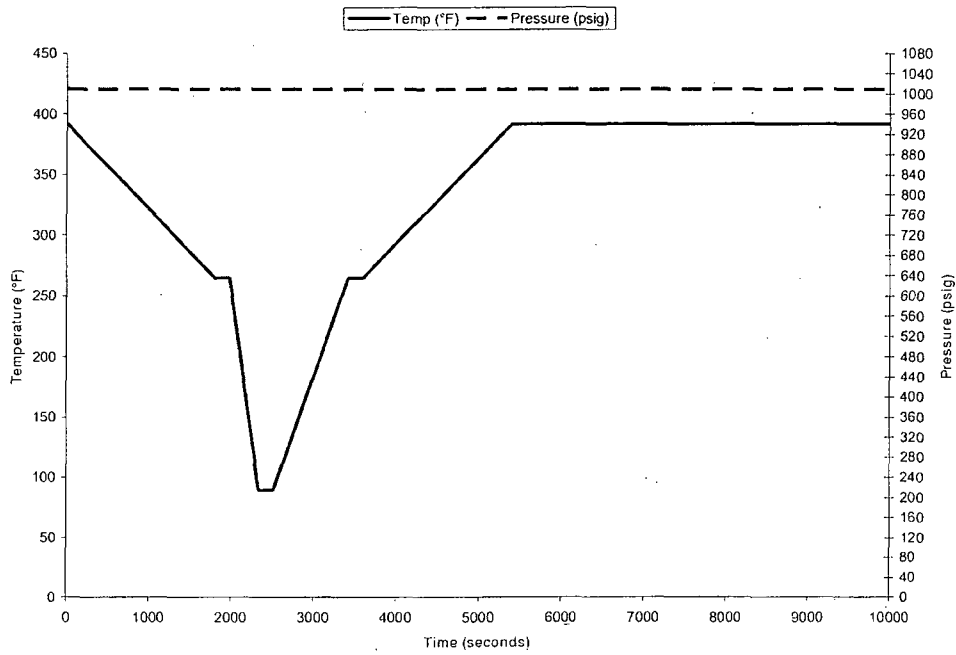


Figure 3-25: Transient 9, Turbine Trip at 25% Power

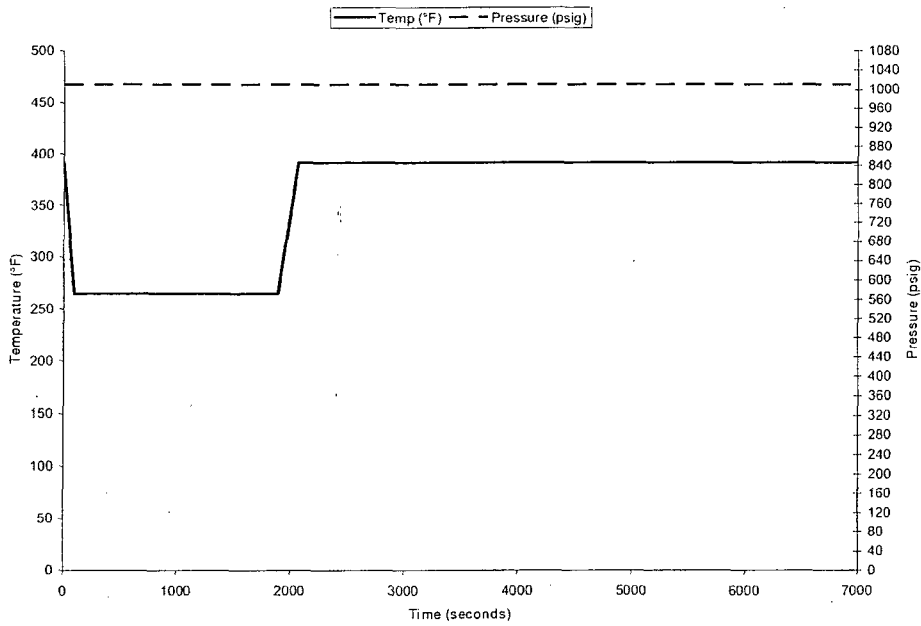


Figure 3-26: Transient 10, Feedwater Bypass

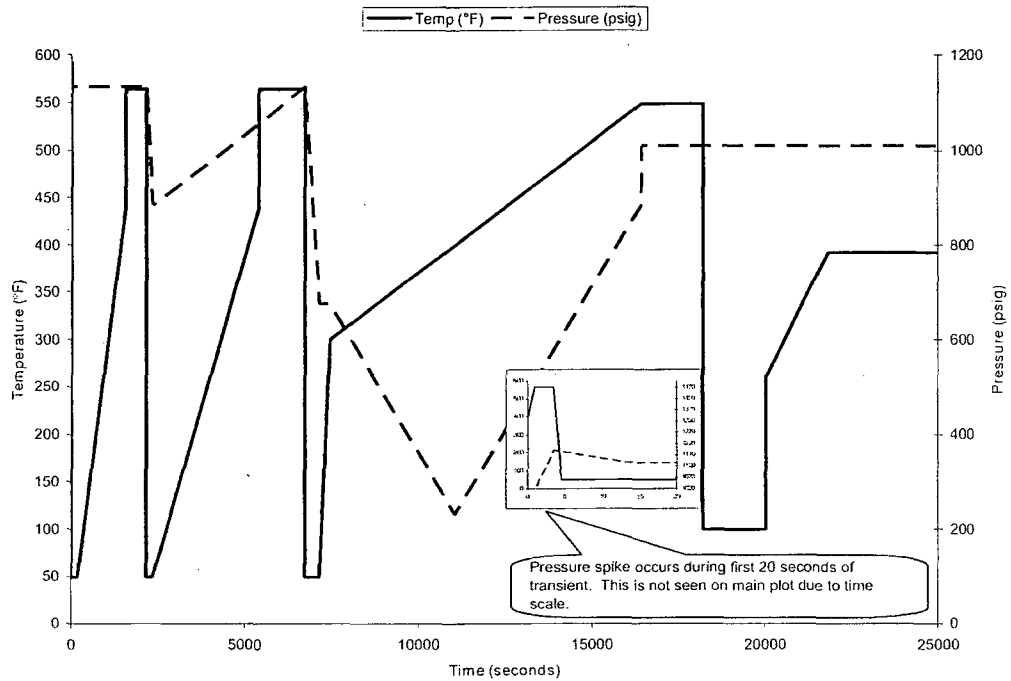


Figure 3-27: Transient 11, Loss of Feedwater Pumps

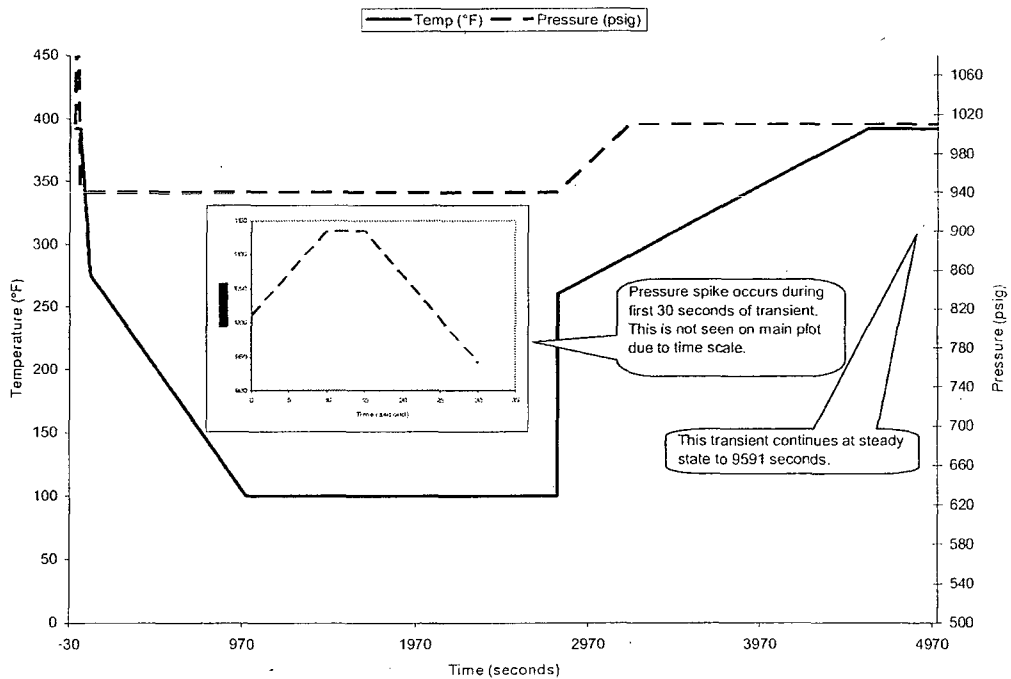


Figure 3-28: Transient 12, Turbine Generator Trip

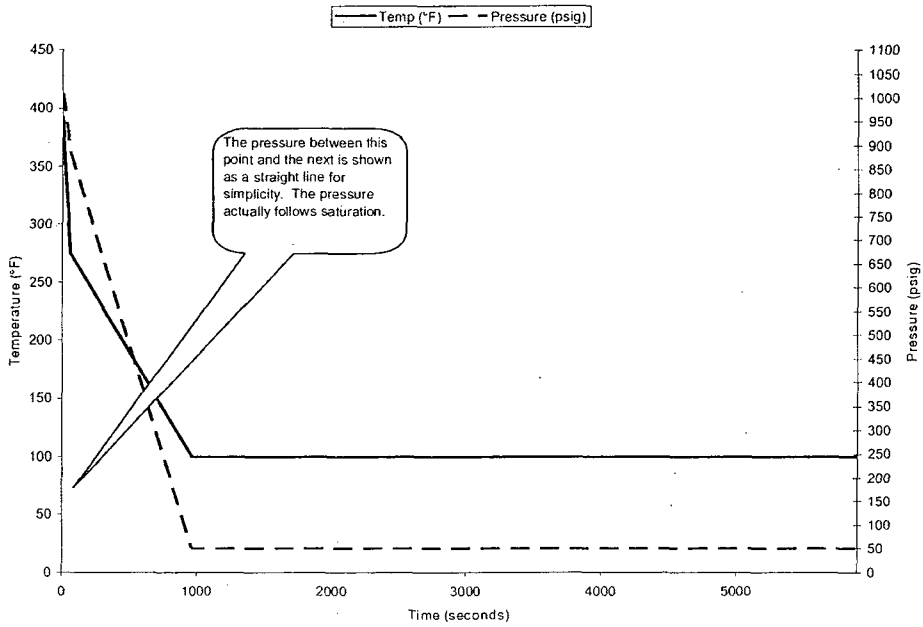


Figure 3-29: Transient 14, SRV Blowdown

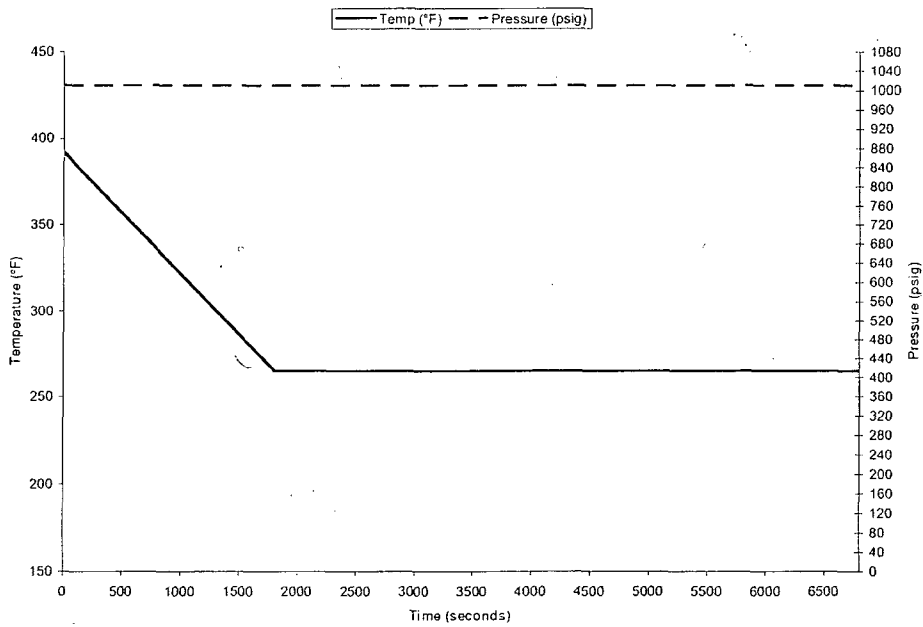


Figure 3-30: Transient 19, Reduction to 0% Power

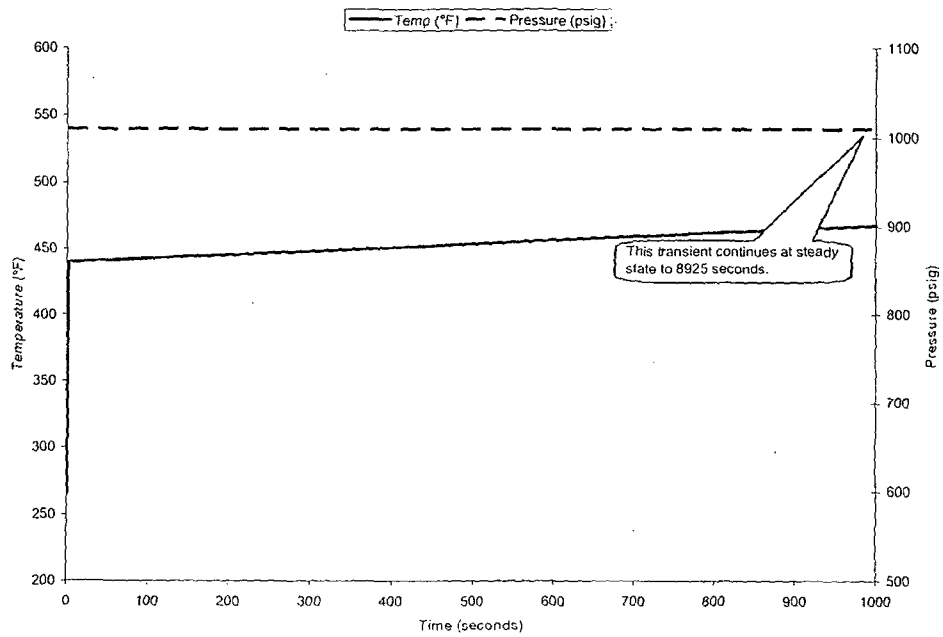


Figure 3-31: Transient 20, Hot Standby (Heatup Portion)

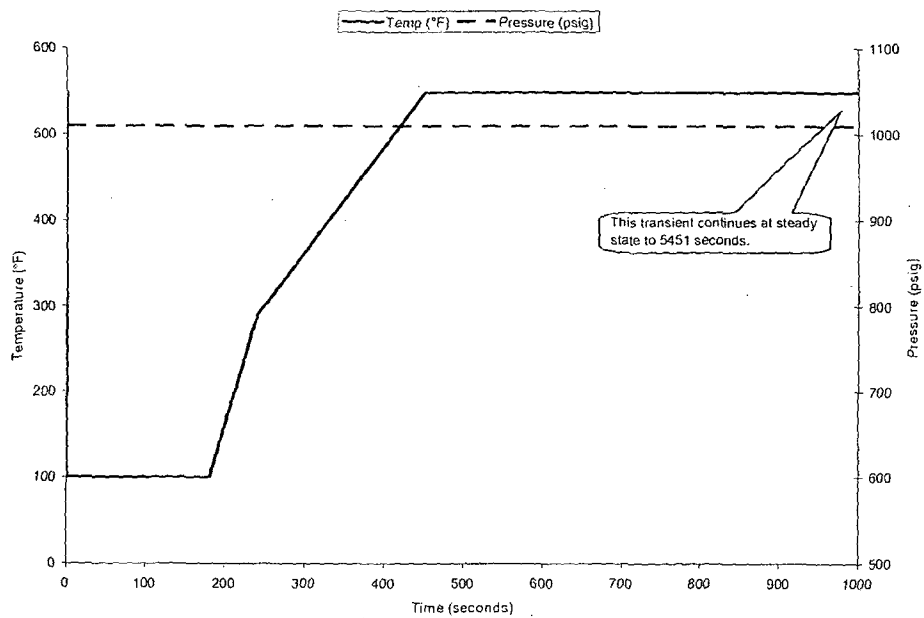


Figure 3-32: Transient 20A, Hot Standby (Feedwater Injection Portion)

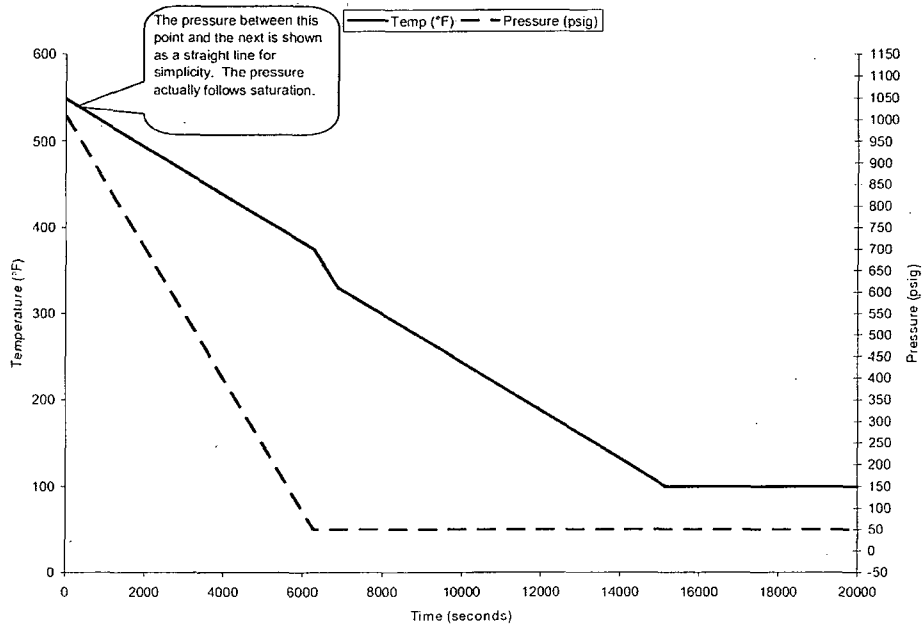


Figure 3-33: Transient 21-23, Shutdown

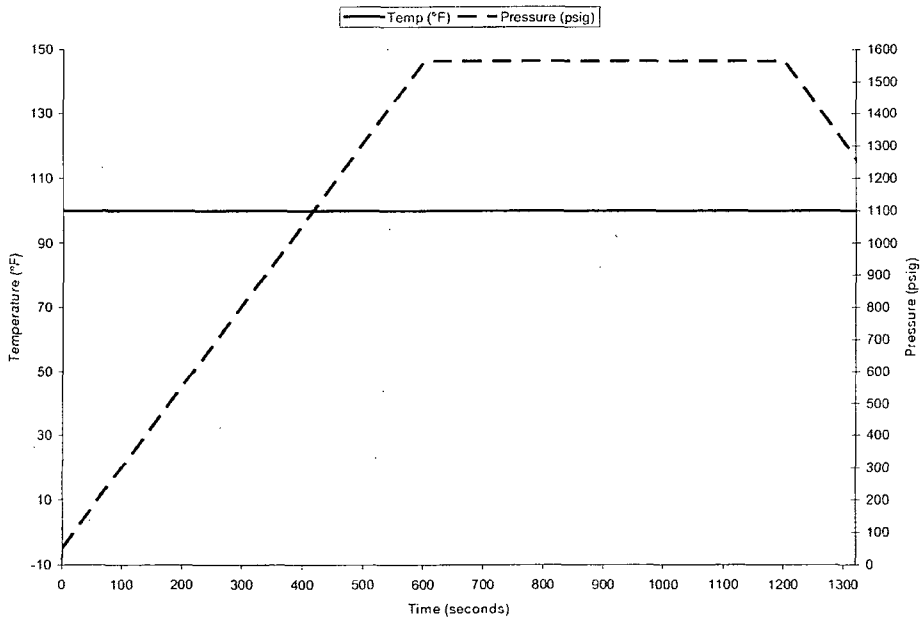


Figure 3-34: Transient 24, Hydrostatic Test

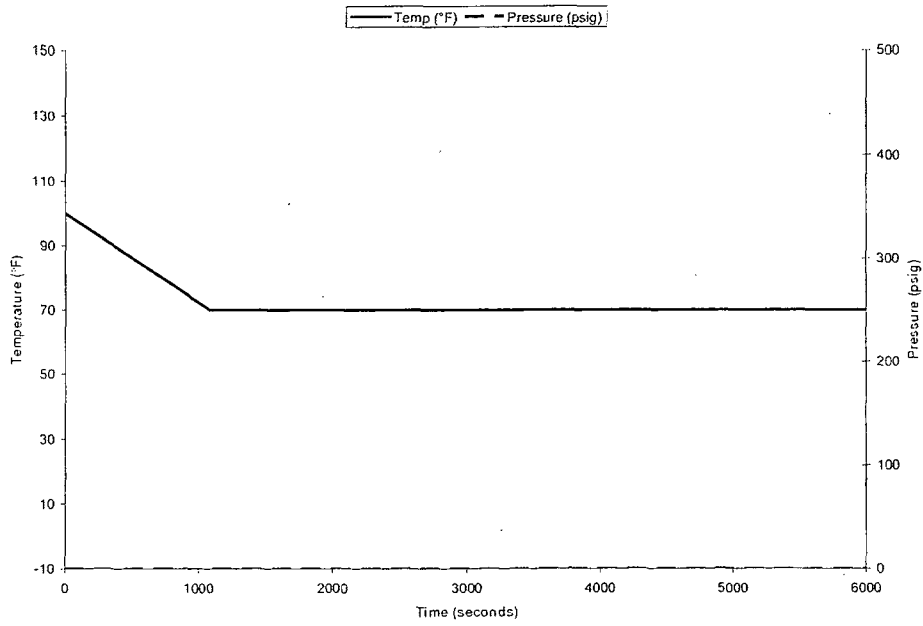


Figure 3-35: Transient 25, Unbolt

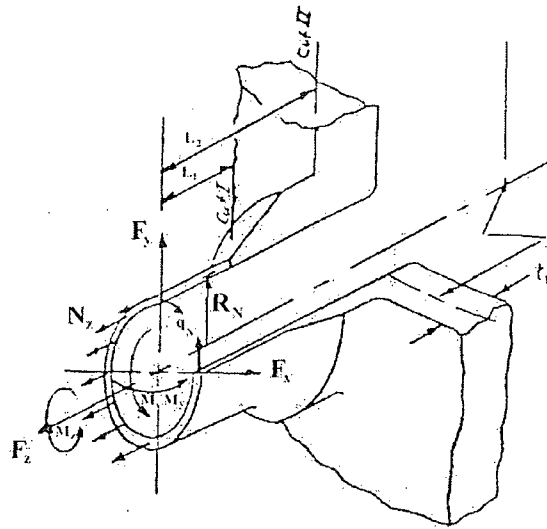


Figure 3-36: External Forces and Moments on the Feedwater Nozzle

4.0 STRESS AND FATIGUE ANALYSIS RESULTS

Fatigue calculations for the VY FW nozzle were performed in accordance with ASME Code, Section III, Subsection NB-3200 methodology (1998 Edition, 2000 Addenda) [15]. Fatigue analysis was performed in the Reference [23] calculation for the two locations identified in Section 3.1.2 using the Green's Functions developed for these two locations and the 60-year projected cycle counts from Reference [19].

Tables 4-1 and 4-2 show the stresses for each location that were used in the fatigue analysis. Columns 2 through 5 of Table 4-1 (for the blend radius) and Table 4-2 (for the safe end) show the final thermal peak and valley output. The pressure values for Column 6 in each table were determined from the transient pressures specified in Tables 3-4 and 3-5. The pressure stress intensities from Section 3.2 were scaled appropriately for each transient case. The scaled piping stress values are shown in Columns 9 and 10 of Tables 4-1 and 4-2. The piping stress intensities from Section 3.3 were scaled based on the transient case RPV fluid temperature and assuming no stress occurs at an ambient temperature of 70°F. Both of these stress intensities were then added to the thermal stress intensity peak and valley points to calculate the final stress values used for the fatigue analysis. In the case of the piping load stress intensities, the sign of the stress intensity was conservatively set to the same sign as the thermal stress intensity to ensure bounding fatigue usage results. Columns 11 and 12 of Tables 4-1 and 4-2 show the summation of all stresses for each thermal peak and valley stress point. The last column shows the number of cycles associated with each peak or valley based on the cycle counts shown in Tables 3-4 and 3-5.

The program FATIGUE.EXE performs the ASME Code peak event-pairing required to calculate a fatigue usage value. The input data for the configuration input file for FATIGUE.EXE, which is named FATIGUE.CFG, is shown in Table 4-3.

The results of the fatigue analysis are presented in Tables 4-4 and 4-5 for the safe end and blend radius for 60 years, respectively. The blend radius cumulative usage factor (CUF) from system cycling is 0.0636 for 60 years. The safe end CUF is 0.1471 for 60 years.

Table 4-1: Feedwater Nozzle Blend Radius Stress Summary

1	2	3	4	5	6	7	8	9	10	11	12	13
Transient Number	Time (s)	Total Stress (psi)	M+B Stress (psi)	Temperature F	Pressure (psig)	Total Pressure Stress (psi)	M+B Pressure Stress (psi)	Total Piping Stress (psi)	M+B Piping Stress (psi)	Total Total Stress (psi)	Total M+B Stress (psi)	Number of Cycles (60 years)
1	0	0	0	70	0	0	0	0	0	0.00	0.00	123
	0	0	0	70	0	0	0	0	0	0.00	0.00	120
2	1680	0	0	100	1100	41506.3	40318.3	15.77042	15.77042	41522.07	40334.07	120
	10880	0	0	100	50	1886.65	1832.65	15.77042	15.77042	1902.42	1848.42	120
	0	29166	23676	100	50	1886.65	1832.65	15.77042	15.77042	31068.42	25524.42	300
3	16782.8	-3577	-3138	549	1010	38110.33	37019.53	-251.801	-251.801	34281.53	33629.73	300
	21164	-3532	-3138	549	1010	38110.33	37019.53	-251.801	-251.801	34326.53	33629.73	300
	0	-3530	-3158	549	1010	38110.33	37019.53	-251.801	-251.801	34328.53	33609.73	300
4	1801.9	29465	22266	244.004	1010	38110.33	37019.53	91.47053	91.47053	67666.80	59377.00	300
	8602	7720	6749	392	1010	38110.33	37019.53	169.2692	169.2692	45999.60	43940.80	300
	0	7720	6752	392	1010	38110.33	37019.53	169.2692	169.2692	45999.60	43940.80	10000
5	2229.8	13598	11941	311.002	1010	38110.33	37019.53	126.6901	126.6901	51835.02	49087.22	10000
	8600	7720	6752	392	1010	38110.33	37019.53	169.2692	169.2692	45999.60	43940.80	10000
	0	7720	6752	392	1010	38110.33	37019.53	169.2692	169.2692	45999.60	43940.80	2000
6	2820.3	15742	13892	280.691	1010	38110.33	37019.53	110.7562	110.7562	53963.09	51022.29	2000
	10400	7720	6752	392	1010	38110.33	37019.53	169.2692	169.2692	45999.60	43940.80	2000
	0	7720	6752	392	1010	38110.33	37019.53	169.2692	169.2692	45999.60	43940.80	10
9	2524	29006	23417	118.311	1010	38110.33	37019.53	25.39616	25.39616	67141.73	60461.93	10
	10400	7720	6752	392	1010	38110.33	37019.53	169.2692	169.2692	45999.60	43940.80	10
	0	7720	6752	392	1010	38110.33	37019.53	169.2692	169.2692	45999.60	43940.80	70
10	1632.4	16828	14701	267.399	1010	38110.33	37019.53	103.7688	103.7688	55042.10	51824.30	70
	7070	7720	6752	392	1010	38110.33	37019.53	169.2692	169.2692	45999.60	43940.80	70
	0	7720	6752	392	1010	38110.33	37019.53	169.2692	169.2692	45999.60	43940.80	10
	3.5	6620	6632	565	1190	44902.27	43617.07	260.2119	260.2119	51782.48	50509.28	10
	4.5	6190	6608	50	1185	44713.61	43433.81	10.51361	10.51361	50914.12	50052.32	10
	194.5	31720	21067	109.348	1135	42826.96	41601.16	20.68448	20.68448	74567.64	62688.84	10
	2166.3	-4761	-1859	513.483	972	36676.48	35626.72	-233.1304	-233.1304	31682.35	33534.59	10
11	2362.5	31268	22070	102.255	1010	38110.33	37019.53	16.95583	16.95583	69395.29	59106.49	10
	6728.3	-4913	-3149	513.448	1010	38110.33	37019.53	-233.112	-233.112	32964.22	33637.42	10
	7149.9	32114	21472	83.333	1010	38110.33	37019.53	7.0089	7.0089	70231.34	58498.54	10
	18213.3	-3565	-3162	503.978	1010	38110.33	37019.53	-228.1338	-228.1338	34317.20	33629.40	10
	19122.6	29156	23083	100.048	1010	38110.33	37019.53	15.79565	15.79565	67282.13	60118.33	10
	26814.5	7720	6410	392	1010	38110.33	37019.53	169.2692	169.2692	45999.60	43598.80	10
	0	7720	6752	392	1010	38110.33	37019.53	169.2692	169.2692	45999.60	43940.80	60
	10	7720	6752	392	1135	42826.96	41601.16	169.2692	169.2692	50716.22	48522.42	60
	30	7720	6752	392	940	35469.02	34453.82	169.2692	169.2692	43358.29	41375.09	60
	2033.7	28648	25301	132.007	940	35469.02	34453.82	32.59588	32.59588	64149.62	59787.42	60
	9591	7720	6752	392	1010	38110.33	37019.53	169.2692	169.2692	45999.60	43940.80	60
	0	7720	6752	392	1010	38110.33	37019.53	169.2692	169.2692	45999.60	43940.80	1
	10	7720	6752	392	1375	51882.88	50397.88	169.2692	169.2692	59772.14	57319.14	1
	30	7720	6752	392	940	35469.02	34453.82	169.2692	169.2692	43358.29	41375.09	1
	2033.7	28648	25301	132.007	1010	38110.33	37019.53	32.59588	32.59588	66790.93	62353.13	1
	9591	7720	6752	392	1010	38110.33	37019.53	169.2692	169.2692	45999.60	43940.80	1
	0	7720	6752	392	1010	38110.33	37019.53	169.2692	169.2692	45999.60	43940.80	1
14	0	7720	6752	392	1010	38110.33	37019.53	169.2692	169.2692	45999.60	43940.80	1
	5960	28487	25650	100	50	1886.65	1832.65	15.77042	15.77042	30389.42	27498.42	1
	0	7720	6752	392	1010	38110.33	37019.53	169.2692	169.2692	45999.60	43940.80	228
	10	7720	6752	392	1135	42826.96	41601.16	169.2692	169.2692	50716.22	48522.42	228
	30	7720	6752	392	940	35469.02	34453.82	169.2692	169.2692	43358.29	41375.09	228
	2033.7	28648	25301	132.007	1010	38110.33	37019.53	32.59588	32.59588	66790.93	62353.13	228
	9591	7720	6752	392	1010	38110.33	37019.53	169.2692	169.2692	45999.60	43940.80	228
	0	7720	6752	392	1010	38110.33	37019.53	169.2692	169.2692	45999.60	43940.80	300
19	6800	16752	14971	265	1010	38110.33	37019.53	102.5077	102.5077	54964.84	52093.04	300
	0	17151	13815	265	1010	38110.33	37019.53	102.5077	102.5077	55363.84	50937.04	300
	8925	-3531	-3146	549	1010	38110.33	37019.53	-251.801	-251.801	34327.53	33621.73	300
	0	-3530	-3158	549	1010	38110.33	37019.53	-251.801	-251.801	34328.53	33609.73	300
20A	183	28102	12153	233	1010	38110.33	37019.53	85.68595	85.68595	66298.02	49258.22	300
	5451	-3530	-3158	549	1010	38110.33	37019.53	-251.801	-251.801	34328.53	33609.73	300
	0	-3530	-3158	549	1010	38110.33	37019.53	-251.801	-251.801	34328.53	33609.73	300
21-23	20144	29168	23656	100	50	1886.65	1832.65	15.77042	15.77042	31070.42	25504.42	300
	0	0	0	100	50	1886.65	1832.65	15.77042	15.77042	1902.42	1848.42	1
	600	0	0	100	1563	58976.68	57288.64	15.77042	15.77042	58992.45	57304.41	1
	2400	0	0	100	50	1886.65	1832.65	15.77042	15.77042	1902.42	1848.42	1
	0	0	0	100	0	0	0	15.77042	15.77042	15.77	15.77	123
25	1580	0	0	70	0	0	0	0	0	0.00	0.00	123

For notes, see last page of table...

Table 4-1: Feedwater Nozzle Blend Radius Stress Summary (continued)

- NOTES:
- Column 1: Transient number identification.
 - Column 2: Time during transient where a maxima or minima stress intensity occurs from P-V.OUT output file.
 - Column 3: Maxima or minima total stress intensity from P-V.OUT output file.
 - Column 4: Maxima or minima membrane plus bending stress intensity from P-V.OUT output file.
 - Column 5: Temperature per total stress intensity.
 - Column 6: Pressure per Table 3-4.
 - Column 7: Total pressure stress intensity from the quantity $(\text{Column 6} \times 37733)/1000$ [Table 3, 10].
 - Column 8: Membrane plus bending pressure stress intensity from the quantity $(\text{Column 6} \times 36653)/1000$ [Table 3, 10].
 - Column 9: Total external stress from calculation in Table 3-2, $265.47 \text{ psi} \cdot (\text{Column 5} - 70^\circ\text{F}) / (575^\circ\text{F} - 70^\circ\text{F})$.
 - Column 10: Same as Column 9, but for M+B stress.
 - Column 11: Sum of total stresses (Columns 3, 7, and 9).
 - Column 12: Sum of membrane plus bending stresses (Columns 4, 8, and 10).
 - Column 13: Number of cycles for the transient (60 years).

Table 4-2: Feedwater Nozzle Safe End Stress Summary

1	2	3	4	5	6	7	8	9	10	11	12	13
Transient Number	Time (s)	Total Stress (psi)	M+B Stress (psi)	Temperature F	Pressure (psig)	Total Pressure Stress (psi)	M+B Pressure Stress (psi)	Total Piping Stress (psi)	M+B Piping Stress (psi)	Total Total Stress (psi)	Total M+B Stress (psi)	Number of Cycles (60 years)
1	0	0	0	70	0	0	0	0	0	0.00	0.00	123
	0	0	0	70	0	0	0	0	0	0.00	0.00	120
2	1680	0	0	100	1100	9780.1	9562.3	339.0875	339.0875	10119.19	9901.39	120
	6960	0	0	100	50	444.55	434.65	339.0875	339.0875	783.64	773.74	120
	0	-170	-165	100	50	444.55	434.65	-339.0875	-339.0875	-64.54	-69.44	300
3	153.2	-235	-212	104.256	50	444.55	434.65	-387.1927	-387.1927	-177.64	-164.54	300
	16328.2	2	3	549	1010	8979.91	8779.93	5414.097	5414.097	14396.01	14197.03	300
	16664	-1	0	549	1010	8979.91	8779.93	-5414.097	5414.097	3564.81	14194.03	300
	0	-3	-2	549	1010	8979.91	8779.93	-5414.097	-5414.097	3562.81	3363.83	300
4	3.6	44060	30988	100	1010	8979.91	8779.93	339.0875	339.0875	53379.00	40107.02	300
	1804.6	-15889	-11224	260.286	1010	8979.91	8779.93	-2150.787	-2150.787	-9059.88	-4594.86	300
	4102	21	23	392	1010	8979.91	8779.93	3639.539	3639.539	12640.45	12442.47	300
5	0	22	23	392	1010	8979.91	8779.93	3639.539	3639.539	12641.45	12442.47	10000
	900.1	244	189	310	1010	8979.91	8779.93	2712.7	2712.7	11936.61	11681.63	10000
	3600	-169	-110	392	1010	8979.91	8779.93	-3639.539	-3639.539	5171.37	5030.39	10000
	3684.4	33	35	392	1010	8979.91	8779.93	3639.539	3639.539	12652.45	12454.47	10000
	4100	22	23	392	1010	8979.91	8779.93	3639.539	3639.539	12641.45	12442.47	10000
6	0	22	23	392	1010	8979.91	8779.93	3639.539	3639.539	12641.45	12442.47	2000
	1800.1	196	159	280	1010	8979.91	8779.93	2373.612	2373.612	11549.52	11312.54	2000
	5400.2	-108	-68	392	1010	8979.91	8779.93	-3639.539	-3639.539	5232.37	5072.39	2000
	5496.6	29	31	392	1010	8979.91	8779.93	3639.539	3639.539	12648.45	12450.47	2000
	5900	22	23	392	1010	8979.91	8779.93	3639.539	3639.539	12641.45	12442.47	2000
7	0	22	23	392	1010	8979.91	8779.93	3639.539	3639.539	12641.45	12442.47	10
	97.3	180	137	385.135	1010	8979.91	8779.93	3561.945	3561.945	12721.85	12478.87	10
	1884.1	63	65	265	1010	8979.91	8779.93	2204.069	2204.069	11246.98	11049.00	10
	2059.2	1161	859	226.597	1010	8979.91	8779.93	1770.003	1770.003	11910.91	11408.93	10
	3420.1	-334	-211	265	1010	8979.91	8779.93	-2204.069	-2204.069	6441.84	6364.86	10
	3490.2	97	98	265	1010	8979.91	8779.93	2204.069	2204.069	11280.98	11082.00	10
	5400.1	-126	-80	392	1010	8979.91	8779.93	-3639.539	-3639.539	5214.37	5060.39	10
	5470.6	31	32	392	1010	8979.91	8779.93	3639.539	3639.539	12650.45	12451.47	10
	5900	22	23	392	1010	8979.91	8779.93	3639.539	3639.539	12641.45	12442.47	10
8	0	23	22	392	1010	8979.91	8779.93	3639.539	3639.539	12642.45	12441.47	70
	77.1	2308	3188	285.461	1010	8979.91	8779.93	2435.338	2435.338	13723.25	14403.27	70
	169.4	-12	-13	265	1010	8979.91	8779.93	-2204.069	-2204.069	6763.84	6562.86	70
	1890	74	72	265	1010	8979.91	8779.93	2204.069	2204.069	11257.98	11056.00	70
	1968.2	-1069	-1511	322.362	1010	8979.91	8779.93	-2852.427	-2852.427	5058.48	4416.50	70
	2147.2	91	90	392	1010	8979.91	8779.93	3639.539	3639.539	12710.45	12509.47	70
	2570	23	22	392	1010	8979.91	8779.93	3639.539	3639.539	12642.45	12441.47	70
9	0	-29	-27	392	1010	8979.91	8779.93	-3639.539	-3639.539	5311.37	5113.39	10
	2.9	-20317	-13859	565	1147	10197.98	9970.871	-5594.944	-5594.944	-15713.97	-9483.07	10
	6.8	42852	29563	565	1172	10420.25	10188.2	5594.944	5594.944	58867.20	45346.14	10
	1567.4	-15216	-10526	565	1135	10091.29	9866.555	-5594.944	-5594.944	-10719.66	-6254.39	10
	2168.4	60377	41773	50	1134	10082.39	9857.862	-226.0583	-226.0583	70233.34	51404.80	10
	5409.4	-14924	-10329	565	1054	9371.114	9162.422	-5594.944	-5594.944	-11147.83	-6761.52	10
	6730.4	60377	41773	50	1133	10073.5	9849.169	-226.0583	-226.0583	70224.44	51396.11	10
	7243.2	-1965	-1434	128.917	675	6001.425	5867.775	-665.9339	-665.9339	3370.49	3767.84	10
	18215.4	52636	36417	100	1010	8979.91	8779.93	339.0875	339.0875	61955.00	45536.02	10
	20015.5	-24511	-16189	260.183	1010	8979.91	8779.93	-2149.623	-2149.623	-17680.71	-9558.69	10
	22314.5	22	23	392	937	8330.867	8145.341	3639.539	3639.539	11992.41	11807.88	10
10	0	23	22	392	1010	8979.91	8779.93	3639.539	3639.539	12642.45	12441.47	60
	10	23	22	392	1135	10091.29	9866.555	3639.539	3639.539	13753.82	13528.09	60
	30	23	22	392	940	8357.54	8171.42	3639.539	3639.539	12020.08	11832.96	60
	90	3174	4383	275	940	8357.54	8171.42	2317.098	2317.098	13848.64	14871.52	60
	2793.5	-16189	-24511	260.183	941	8366.431	8180.113	-2149.623	-2149.623	-9972.19	-18480.51	60
	5091	23	22	392	1010	8979.91	8779.93	3639.539	3639.539	12642.45	12441.47	60
11	0	23	22	392	1010	8979.91	8779.93	3639.539	3639.539	12642.45	12441.47	1
	10	23	22	392	1375	12225.13	11952.88	3639.539	3639.539	15887.66	15614.41	1
	30	23	22	392	940	8357.54	8171.42	3639.539	3639.539	12020.08	11832.96	1
	90	3174	4383	275	940	8357.54	8171.42	2317.098	2317.098	13848.64	14871.52	1
	2793.5	-16189	-24511	260.183	941	8366.431	8180.113	-2149.623	-2149.623	-9972.19	-18480.51	1
	5091	23	22	392	1010	8979.91	8779.93	3639.539	3639.539	12642.45	12441.47	1

For notes, see last page of table...

Table 4-2: Feedwater Nozzle Safe End Stress Summary (continued)

1	2	3	4	5	6	7	8	9	10	11	12	13
Transient Number	Time (s)	Total Stress (psi)	M+B Stress (psi)	Temperature F	Pressure (psig)	Total Pressure Stress (psi)	M+B Pressure Stress (psi)	Total Piping Stress (psi)	M+B Piping Stress (psi)	Total Total Stress (psi)	Total M+B Stress (psi)	Number of Cycles (60 years)
14	0	22	23	392	1010	8979.91	8779.93	3639.539	3639.539	12641.45	12442.47	1
	60	4383	3174	275	885	7868.535	7693.305	2317.098	2317.098	14568.63	13184.40	1
	148	420	300	258.492	803	7139.473	6980.479	2130.509	2130.509	9689.98	9410.99	1
	960	544	424	100	50	444.55	434.65	339.0875	339.0875	1327.64	1197.74	1
	1460	137	139	100	50	444.55	434.65	339.0875	339.0875	920.64	912.74	1
15	0	23	22	392	1010	8979.91	8779.93	3639.539	3639.539	12642.45	12441.47	228
	10	23	22	392	1135	10091.29	9866.555	3639.539	3639.539	13753.82	13528.09	228
	30	23	22	392	940	8357.54	8171.42	3639.539	3639.539	12020.08	11832.96	228
	90	3174	4383	275	940	8357.54	8171.42	2317.098	2317.098	13848.64	14871.52	228
	2793.5	-16189	-24511	260.183	941	8366.431	8180.113	-2149.623	-2149.623	-9972.19	-18480.51	228
	5091	23	22	392	1010	8979.91	8779.93	3639.539	3639.539	12642.45	12441.47	228
19	0	22	23	392	1010	8979.91	8779.93	3639.539	3639.539	12641.45	12442.47	300
	1800	219	177	265	1010	8979.91	8779.93	2204.069	2204.069	11402.98	11161.00	300
	2300	72	74	265	1010	8979.91	8779.93	2204.069	2204.069	11255.98	11058.00	300
20	0	-109	-105	265	1010	8979.91	8779.93	-2204.069	-2204.069	6666.84	6470.86	300
	4	-17288	-12189	440.106	1010	8979.91	8779.93	-4183.277	-4183.277	-12491.37	-7592.35	300
	4425	-2	-1	549	1010	8979.91	8779.93	-5414.097	-5414.097	3563.81	3364.83	300
20A	0	-3	-2	549	1010	8979.91	8779.93	-5414.097	-5414.097	3562.81	3363.83	300
	4	44060	30988	100	1010	8979.91	8779.93	339.0875	339.0875	53379.00	40107.02	300
	241	-7461	-5525	290.247	1010	8979.91	8779.93	-2489.433	-2489.433	-970.52	765.50	300
	572	128	132	549	1010	8979.91	8779.93	5414.097	5414.097	14522.01	14326.03	300
	951	-3	-2	549	1010	8979.91	8779.93	-5414.097	-5414.097	3562.81	3363.83	300
21-23	0	-3	-2	549	1010	8979.91	8779.93	-5414.097	-5414.097	3562.81	3363.83	300
	138	62	45	545.167	989	8793.199	8597.377	5370.773	5370.773	14225.97	14013.15	300
	6264	-5	-20	374.97	50	444.55	434.65	-3447.05	-3447.05	-3007.50	-3032.40	300
	6390	104	59	366.172	50	444.55	434.65	3347.607	3347.607	3896.16	3841.26	300
	15644	-173	-167	100	50	444.55	434.65	-339.0875	-339.0875	-67.54	-71.44	300
24	0	0	0	100	50	444.55	434.65	339.0875	339.0875	783.64	773.74	1
	600	0	0	100	1563	13896.63	13587.16	339.0875	339.0875	14235.72	13926.25	1
	2400	0	0	100	50	444.55	434.65	339.0875	339.0875	783.64	773.74	1
25	0	0	0	100	0	0	0	339.0875	339.0875	339.09	339.09	123
	1580	0	0	70	0	0	0	0	0	0.00	0.00	123

- NOTES: Column 1: Transient number identification.
 Column 2: Time during transient where a maxima or minima stress intensity occurs from P-V.OUT output file.
 Column 3: Maxima or minima total stress intensity from P-V.OUT output file.
 Column 4: Maxima or minima membrane plus bending stress intensity from P-V.OUT output file.
 Column 5: Temperature per total stress intensity.
 Column 6: Pressure per Table 3-5.
 Column 7: Total pressure stress intensity from the quantity (Column 6 x 8891)/1000 [Table 3, 10].
 Column 8: Membrane plus bending pressure stress intensity from the quantity (Column 6 x 8693)/1000 [Table 3, 10].
 Column 9: Total external stress from calculation in Table 3-2, 5707.97 psi*(Column 5-70°F)/(575°F-70°F).
 Column 10: Same as Column 9, but for M+B stress.
 Column 11: Sum of total stresses (Columns 3, 7, and 9).
 Column 12: Sum of membrane plus bending stresses (Columns 4, 8, and 10).
 Column 13: Number of cycles for the transient (60 years).

Table 4-3: Fatigue Parameters Used in the Feedwater Nozzle Fatigue Analysis

	Blend Radius	Safe End
Parameters m and n for Computing K_e	2.0 & 0.2 (low alloy steel) [15]	3.0 & 0.2 (carbon steel) [15]
Design Stress Intensity Values, S_m	26700 psi [9] @ 600°F	17800 psi [9] @ 600°F
Elastic Modulus from Applicable Fatigue Curve	30.0×10^6 psi [15]	30.0×10^6 psi [15]
Elastic Modulus Used in Finite Element Model	26.7×10^6 psi [10]	28.1×10^6 psi [10]
The Geometric Stress Concentration Factor K_t	1.0	1.34 [2, page 35 of S4]

Table 4-4: Fatigue Results for Feedwater Nozzle Blend Radius

LOCATION = LOCATION NO. 2 -- BLEND RADIUS
 FATIGUE CURVE = 1 (1 = CARBON/LOW ALLOY, 2 = STAINLESS STEEL)
 m = 2.0
 n = .2
 Sm = 26700. psi
 Ecurve = 3.000E+07 psi
 Eanalysis = 2.670E+07 psi
 Kt = 1.00

MAX	MIN	RANGE	MEM+BEND	Ke	Salt	Napplied	Nallowed	U
74568.	0.	74568.	62689.	1.000	41892.	1.000E+01	7.488E+03	.0013
70231.	0.	70231.	58499.	1.000	39456.	1.000E+01	8.944E+03	.0011
69395.	0.	69395.	59106.	1.000	38986.	1.000E+01	9.268E+03	.0011
67667.	0.	67667.	59377.	1.000	38015.	9.300E+01	9.988E+03	.0093
67667.	0.	67667.	59377.	1.000	38015.	1.200E+02	9.988E+03	.0120
67667.	0.	67667.	59377.	1.000	38015.	8.700E+01	9.988E+03	.0087
67282.	0.	67282.	60118.	1.000	37799.	1.000E+01	1.018E+04	.0010
67142.	0.	67142.	60462.	1.000	37720.	1.000E+01	1.025E+04	.0010
66791.	0.	66791.	62353.	1.000	37523.	1.000E+00	1.044E+04	.0001
66791.	0.	66791.	62353.	1.000	37523.	1.500E+01	1.044E+04	.0014
66791.	16.	66775.	62337.	1.000	37514.	1.230E+02	1.045E+04	.0118
66791.	1902.	64889.	60505.	1.000	36454.	9.000E+01	1.152E+04	.0078
66298.	1902.	64396.	47410.	1.000	36177.	3.000E+01	1.182E+04	.0025
66298.	1902.	64396.	47410.	1.000	36177.	1.000E+00	1.182E+04	.0001
66298.	1902.	64396.	47410.	1.000	36177.	1.000E+00	1.182E+04	.0001
66298.	30389.	35909.	21760.	1.000	20173.	1.000E+00	9.581E+04	.0000
66298.	31068.	35230.	23734.	1.000	19792.	2.670E+02	1.038E+05	.0026
64150.	31068.	33081.	34263.	1.000	18585.	3.300E+01	1.303E+05	.0003
64150.	31070.	33079.	34283.	1.000	18584.	2.700E+01	1.303E+05	.0002
59772.	31070.	28702.	31815.	1.000	16125.	1.000E+00	2.222E+05	.0000
58992.	31070.	27922.	31800.	1.000	15687.	1.000E+00	2.519E+05	.0000
55364.	31070.	24293.	25433.	1.000	13648.	2.710E+02	4.757E+05	.0006
55364.	31682.	23681.	17402.	1.000	13304.	1.000E+01	5.703E+05	.0000
55364.	32964.	22400.	17300.	1.000	12584.	1.000E+01	9.414E+05	.0000
55364.	34282.	21082.	17307.	1.000	11844.	9.000E+00	1.912E+06	.0000
55042.	34282.	20761.	18195.	1.000	11663.	7.000E+01	2.231E+06	.0000
54965.	34282.	20683.	18463.	1.000	11620.	2.210E+02	2.310E+06	.0001
54965.	34317.	20648.	18464.	1.000	11600.	1.000E+01	2.348E+06	.0000
54965.	34327.	20638.	18463.	1.000	11595.	6.900E+01	2.358E+06	.0000
53963.	34327.	19637.	17393.	1.000	11032.	2.310E+02	3.757E+06	.0001
53963.	34328.	19636.	17401.	1.000	11031.	3.000E+02	3.758E+06	.0001
53963.	34329.	19635.	17413.	1.000	11031.	3.000E+02	3.760E+06	.0001
53963.	34329.	19635.	17413.	1.000	11031.	3.000E+02	3.760E+06	.0001
53963.	34329.	19635.	17413.	1.000	11031.	3.000E+02	3.760E+06	.0001
53963.	34329.	19635.	17413.	1.000	11031.	3.000E+02	3.760E+06	.0001
53963.	41522.	12441.	10688.	1.000	6989.	1.200E+02	1.000E+20	.0000
53963.	43358.	10605.	9647.	1.000	5958.	6.000E+01	1.000E+20	.0000
53963.	43358.	10605.	9647.	1.000	5958.	1.000E+00	1.000E+20	.0000
53963.	43358.	10605.	9647.	1.000	5958.	8.800E+01	1.000E+20	.0000
51835.	43358.	8477.	7712.	1.000	4762.	1.400E+02	1.000E+20	.0000
51835.	46000.	5835.	5149.	1.000	3278.	3.000E+02	1.000E+20	.0000

51835.	46000.	5835.	5146.	1.000	3278.	9.560E+03	1.000E+20	.0000
51782.	46000.	5783.	6568.	1.000	3249.	1.000E+01	1.000E+20	.0000
50914.	46000.	4915.	6112.	1.000	2761.	1.000E+01	1.000E+20	.0000
50716.	46000.	4717.	4582.	1.000	2650.	6.000E+01	1.000E+20	.0000
50716.	46000.	4717.	4582.	1.000	2650.	2.280E+02	1.000E+20	.0000
46000.	46000.	0.	0.	1.000	0.	1.320E+02	1.000E+20	.0000
46000.	46000.	0.	0.	1.000	0.	1.000E+04	1.000E+20	.0000
46000.	46000.	0.	0.	1.000	0.	2.000E+03	1.000E+20	.0000
46000.	46000.	0.	0.	1.000	0.	2.000E+03	1.000E+20	.0000
46000.	46000.	0.	0.	1.000	0.	1.000E+01	1.000E+20	.0000
46000.	46000.	0.	0.	1.000	0.	1.000E+01	1.000E+20	.0000
46000.	46000.	0.	0.	1.000	0.	7.000E+01	1.000E+20	.0000
46000.	46000.	0.	0.	1.000	0.	7.000E+01	1.000E+20	.0000
46000.	46000.	0.	0.	1.000	0.	1.000E+01	1.000E+20	.0000
46000.	46000.	0.	0.	1.000	0.	1.000E+01	1.000E+20	.0000
46000.	46000.	0.	0.	1.000	0.	6.000E+01	1.000E+20	.0000
46000.	46000.	0.	0.	1.000	0.	6.000E+01	1.000E+20	.0000
46000.	46000.	0.	0.	1.000	0.	1.000E+00	1.000E+20	.0000
46000.	46000.	0.	0.	1.000	0.	1.000E+00	1.000E+20	.0000
46000.	46000.	0.	0.	1.000	0.	1.000E+00	1.000E+20	.0000
46000.	46000.	0.	0.	1.000	0.	2.280E+02	1.000E+20	.0000
46000.	46000.	0.	0.	1.000	0.	2.280E+02	1.000E+20	.0000

=====
TOTAL USAGE FACTOR = .0636

Table 4-5: Fatigue Results for the Feedwater Nozzle Safe End

LOCATION = LOCATION NO. 1 -- SAFE END
 FATIGUE CURVE = 1 (1 = CARBON/LOW ALLOY, 2 = STAINLESS STEEL)
 m = 3.0
 n = .2
 Sm = 17800. psi
 Ecurve = 3.000E+07 psi
 Eanalysis = 2.810E+07 psi
 Kt = 1.34

MAX	MIN	RANGE	MEM+BEND	Ke	Salt	Napplied	Nallowed	U
70233.	-17681.	87914.	60963.	1.283	74422.	1.000E+01	1.338E+03	.0075
70224.	-15714.	85938.	60879.	1.280	72869.	1.000E+01	1.415E+03	.0071
61955.	-12491.	74446.	53128.	1.000	49383.	1.000E+01	4.568E+03	.0022
58867.	-12491.	71359.	52938.	1.000	47700.	1.000E+01	5.094E+03	.0020
53379.	-12491.	65870.	47699.	1.000	43819.	2.800E+02	6.552E+03	.0427
53379.	-11148.	64527.	46869.	1.000	42951.	1.000E+01	6.953E+03	.0014
53379.	-10720.	64099.	46361.	1.000	42631.	1.000E+01	7.109E+03	.0014
53379.	-9972.	63351.	58588.	1.194	53087.	6.000E+01	3.628E+03	.0165
53379.	-9972.	63351.	58588.	1.194	53087.	1.000E+00	3.628E+03	.0003
53379.	-9972.	63351.	58588.	1.194	53087.	2.280E+02	3.628E+03	.0628
53379.	-9060.	62439.	44702.	1.000	41444.	1.100E+01	7.731E+03	.0014
15888.	-9060.	24948.	20209.	1.000	16985.	1.000E+00	1.802E+05	.0000
14569.	-9060.	23629.	17779.	1.000	15840.	1.000E+00	2.410E+05	.0000
14522.	-9060.	23582.	18921.	1.000	16022.	2.870E+02	2.287E+05	.0013
14522.	-3008.	17530.	17358.	1.000	12508.	1.300E+01	9.944E+05	.0000
14396.	-3008.	17404.	17229.	1.000	12417.	2.870E+02	1.083E+06	.0003
14396.	-971.	15367.	13432.	1.000	10641.	1.300E+01	5.165E+06	.0000
14236.	-971.	15206.	13161.	1.000	10506.	1.000E+00	5.563E+06	.0000
14226.	-971.	15196.	13248.	1.000	10516.	2.860E+02	5.531E+06	.0001
14226.	-178.	14404.	14178.	1.000	10262.	1.400E+01	6.379E+06	.0000
13849.	-178.	14026.	15036.	1.000	10216.	6.000E+01	6.547E+06	.0000
13849.	-178.	14026.	15036.	1.000	10216.	1.000E+00	6.547E+06	.0000
13849.	-178.	14026.	15036.	1.000	10216.	2.250E+02	6.547E+06	.0000
13849.	-68.	13916.	14943.	1.000	10141.	3.000E+00	6.837E+06	.0000
13754.	-68.	13821.	13600.	1.000	9846.	6.000E+01	8.117E+06	.0000
13754.	-68.	13821.	13600.	1.000	9846.	2.280E+02	8.117E+06	.0000
13723.	-68.	13791.	14475.	1.000	9989.	9.000E+00	7.465E+06	.0000
13723.	-65.	13788.	14473.	1.000	9987.	6.100E+01	7.474E+06	.0000
12722.	-65.	12786.	12548.	1.000	9103.	1.000E+01	1.729E+07	.0000
12710.	-65.	12775.	12579.	1.000	9102.	7.000E+01	1.730E+07	.0000
12652.	-65.	12717.	12524.	1.000	9061.	1.590E+02	1.833E+07	.0000
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12652.	784.	11869.	11681.	1.000	8456.	1.200E+02	4.952E+07	.0000
12652.	784.	11869.	11681.	1.000	8456.	1.000E+00	4.952E+07	.0000
12652.	784.	11869.	11681.	1.000	8456.	1.000E+00	4.952E+07	.0000
12652.	921.	11732.	11542.	1.000	8357.	1.000E+00	5.462E+07	.0000
12652.	1328.	11325.	11257.	1.000	8088.	1.000E+00	7.100E+07	.0000
12652.	3370.	9282.	8687.	1.000	6531.	1.000E+01	1.000E+20	.0000
12652.	3563.	9090.	9091.	1.000	6502.	3.000E+02	1.000E+20	.0000

12652.	3563.	9090.	9091.	1.000	6502.	3.000E+02	1.000E+20	.0000
12652.	3563.	9090.	9091.	1.000	6502.	3.000E+02	1.000E+20	.0000
12652.	3563.	9090.	9091.	1.000	6502.	3.000E+02	1.000E+20	.0000
12652.	3564.	9089.	9090.	1.000	6501.	3.000E+02	1.000E+20	.0000
12652.	3565.	9088.	-1740.	1.000	4535.	3.000E+02	1.000E+20	.0000
12652.	3896.	8756.	8613.	1.000	6237.	3.000E+02	1.000E+20	.0000
12652.	5058.	7594.	8038.	1.000	5513.	7.000E+01	1.000E+20	.0000
12652.	5171.	7481.	7424.	1.000	5341.	7.048E+03	1.000E+20	.0000
12650.	5171.	7479.	7421.	1.000	5339.	1.000E+01	1.000E+20	.0000
12648.	5171.	7477.	7420.	1.000	5338.	2.000E+03	1.000E+20	.0000
12642.	5171.	7471.	7411.	1.000	5333.	7.000E+01	1.000E+20	.0000
12642.	5171.	7471.	7411.	1.000	5333.	7.000E+01	1.000E+20	.0000
12642.	5171.	7471.	7411.	1.000	5333.	6.000E+01	1.000E+20	.0000
12642.	5171.	7471.	7411.	1.000	5333.	6.000E+01	1.000E+20	.0000
12642.	5171.	7471.	7411.	1.000	5333.	1.000E+00	1.000E+20	.0000
12642.	5171.	7471.	7411.	1.000	5333.	1.000E+00	1.000E+20	.0000
12642.	5171.	7471.	7411.	1.000	5333.	2.280E+02	1.000E+20	.0000
12642.	5171.	7471.	7411.	1.000	5333.	2.280E+02	1.000E+20	.0000
12641.	5171.	7470.	7412.	1.000	5333.	2.240E+02	1.000E+20	.0000
12641.	5214.	7427.	7382.	1.000	5304.	1.000E+01	1.000E+20	.0000
12641.	5232.	7409.	7370.	1.000	5293.	2.000E+03	1.000E+20	.0000
12641.	5311.	7330.	7329.	1.000	5243.	1.000E+01	1.000E+20	.0000
12641.	6442.	6200.	6078.	1.000	4412.	1.000E+01	1.000E+20	.0000
12641.	6667.	5975.	5972.	1.000	4273.	3.000E+02	1.000E+20	.0000
12641.	6764.	5878.	5880.	1.000	4205.	7.000E+01	1.000E+20	.0000
12641.	9690.	2951.	3031.	1.000	2126.	1.000E+00	1.000E+20	.0000
12641.	10119.	2522.	2541.	1.000	1808.	1.200E+02	1.000E+20	.0000
12641.	11247.	1394.	1393.	1.000	997.	1.000E+01	1.000E+20	.0000
12641.	11256.	1385.	1384.	1.000	991.	3.000E+02	1.000E+20	.0000
12641.	11258.	1383.	1386.	1.000	990.	7.000E+01	1.000E+20	.0000
12641.	11281.	1360.	1360.	1.000	973.	1.000E+01	1.000E+20	.0000
12641.	11403.	1238.	1281.	1.000	894.	3.000E+02	1.000E+20	.0000
12641.	11550.	1092.	1130.	1.000	788.	2.000E+03	1.000E+20	.0000
12641.	11911.	731.	1034.	1.000	578.	1.000E+01	1.000E+20	.0000
12641.	11937.	705.	761.	1.000	514.	4.555E+03	1.000E+20	.0000
12641.	11937.	705.	761.	1.000	514.	5.445E+03	1.000E+20	.0000
12641.	11992.	649.	635.	1.000	462.	1.000E+01	1.000E+20	.0000
12641.	12020.	621.	610.	1.000	442.	6.000E+01	1.000E+20	.0000
12641.	12020.	621.	610.	1.000	442.	1.000E+00	1.000E+20	.0000
12641.	12020.	621.	610.	1.000	442.	2.280E+02	1.000E+20	.0000
12641.	12640.	1.	0.	1.000	1.	3.000E+02	1.000E+20	.0000
12641.	12641.	0.	0.	1.000	0.	3.956E+03	1.000E+20	.0000
12641.	12641.	0.	0.	1.000	0.	2.000E+03	1.000E+20	.0000
12641.	12641.	0.	0.	1.000	0.	2.000E+03	1.000E+20	.0000
12641.	12641.	0.	0.	1.000	0.	1.000E+01	1.000E+20	.0000
12641.	12641.	0.	0.	1.000	0.	1.000E+01	1.000E+20	.0000
12641.	12641.	0.	0.	1.000	0.	1.000E+00	1.000E+20	.0000

=====
TOTAL USAGE FACTOR = .1471

5.0 ENVIRONMENTAL FATIGUE ANALYSIS

In the response to NRC request for additional information (RAI) 4.3-H-02 [19], VYNPS states that they have conservatively assumed that fatigue cracks may be present in the clad. VYNPS manages this cracking by performing periodic inspections that were implemented in response to Generic Letters 80-095 and 81-11, and NUREG-0619. The inspection frequency is based on the calculated fatigue crack growth of a postulated flaw in the nozzle inner blend radius. The VYNPS fatigue crack growth calculation uses methods in compliance with GE BWR Owners Group Topical Report "Alternate BWR Feedwater Nozzle Inspection Requirements", GE-NE-523-A71-0594, Revision 1, August 1999 and the associated NRC Final Safety Evaluation (TAC No. MA6787) dated March 10, 2000. The NRC has reviewed and approved this approach to handling FW nozzle inner blend radius cracking (Letter D.H. Dorman (USNRC) to D.A. Reid (VYNPC), Subject: Evaluation of Request for Relief from NUREG-0619 for VYNPS dated 2/6/95, (TAC No. M88803)).

The analysis performed for the feedwater nozzle calculated fatigue in the blend radius base metal, not the clad. This is consistent with the VYNPS position stated in the response to RAI 4.3-H-02, and is also consistent with ASME Code methodology since cladding is structurally neglected in fatigue analyses, per ASME Code, Section III, NB-3122.3 [15].

Environmental fatigue multipliers were computed for both normal water chemistry (NWC) and hydrogen water chemistry (HWC) conditions in Reference [21] for various regions of the VY RPV and attached piping. Based on VY-specific dates for plant startup and HWC implementation, as well as past and future predicted HWC system availability, it was determined that overall HWC availability is 47% over the sixty year operating period for VY. Therefore, for the purposes of the EAF assessment of the FW nozzle, it was assumed that HWC conditions exist for 47% of the time, and NWC conditions exist for 53% of the time over the 60-year operating life of the plant. RPV upper region chemistry was assumed for the FW nozzle blend radius location, since this location experiences reactor conditions for all times. FW line chemistry was assumed for the FW nozzle safe end location, since this location experiences feedwater conditions for all times.

For the safe end location, the environmental fatigue factors for pre-HWC and post-HWC are both 1.74 from Table 3 of Reference [21] for the RPV FW line. This results in an EAF adjusted CUF as follows:

$$\text{60-Year CUF, } U_{60} = 0.1470 \text{ (from Table 4-5)}$$

$$\text{Overall EAF multiplier, } F_{en} = 1.74$$

$$\text{60-Year EAF CUF, } U_{60-env} = 0.14709 \times 1.74 = 0.2560$$

The EAF CUF value of 0.2560 for 60 years for the safe end is acceptable (i.e., less than the allowable value of 1.0).

The fatigue calculation documented in Section 4.0 for the blend radius location was performed for the nozzle base material since cladding is structurally neglected in modern-day fatigue analyses, per ASME Code, Section III, NB-3122.3 [15]. This is also consistent with Sections 5.7.1 and 5.7.4 of NUREG/CR-6260 [16]. Therefore, the cladding was neglected and EAF assessment of the nozzle base material was performed for the blend radius location.

For the blend radius location, the environmental fatigue factors for pre-HWC and post-HWC are 11.14 and 8.82, respectively, from Table 4 of Reference [21] for the RPV upper region. This results in an EAF adjusted CUF as follows:

$$\text{60-Year CUF, } U_{60} = 0.0636 \text{ (from Table 4-4)}$$

$$\text{Overall EAF multiplier, } F_{en} = (11.14 \times 53\% + 8.82 \times 47\%) = 10.05$$

$$\text{60-Year EAF CUF, } U_{60-env} = 0.0636 \times 10.05 = 0.6392$$

The EAF CUF value of 0.6392 for 60 years for the blend radius is acceptable (i.e., less than the allowable value of 1.0).

6.0 CONCLUSIONS

This report documents a refined fatigue evaluation for the VY FW nozzle. The intent of this evaluation is to use refined transient definitions and the revised cyclic transient counts for 60 years for a computation of CUF, including EAF effects, that is more refined than previously performed fatigue analyses. The fatigue-limiting locations in the FW nozzle and safe end are included in the evaluation, to be consistent with NUREG/CR-6260 [16] needs for EAF evaluation for license renewal. The final fatigue results are considered to be a replacement to the values previously reported in the VY LRA.

The fatigue calculations for the VY FW nozzle were performed in accordance with ASME Code, Section III, Subsection NB-3200 methodology (1998 Edition, 2000 Addenda) [15]. The stress evaluation is summarized in Section 3.0, and the fatigue analysis is summarized in Section 4.0. The results in Section 4.0 reveal that the CUF for the limiting safe end location is 0.1470, and the CUF for the limiting blend radius location is 0.0636. Both of these values represent 60 years of plant operation, including all relevant EPU effects.

EAF calculations for the VY FW nozzle were also performed, as summarized in Section 5.0. The results in Section 5.0 reveal that the EAF CUF for the limiting safe end location is 0.2560, and the EAF CUF for the limiting blend radius location is 0.6392. Both of these values represent 60 years of plant operation, including all relevant EPU effects.

All fatigue allowables, both with and without EAF effects, are met, thus demonstrating acceptability for 60 years of operation.

7.0 REFERENCES

1. REDACTED

2. CB&I RPV Stress Report, Sections T4 and S4, "Feedwater Nozzle, Vermont Yankee Reactor Vessel, CB&I Contract 9-6201," SI File No. VY-05Q-238.
3. GE Design Specification No. 21A1115, Revision 4, "Vermont Yankee Reactor Pressure Vessel," October 21, 1969, SI File No. VY-05Q-210.
4. Kuo, A. Y., Tang, S. S., and Riccardella, P. C., "An On-Line Fatigue Monitoring System for Power Plants, Part I - Direct Calculation of Transient Peak Stress Through Transfer Matrices and Green's Functions," ASME PVP Conference, Chicago, 1986.
5. ANSYS, Release 8.1 (w/Service Pack 1), ANSYS, Inc., June 2004.
6. Structural Integrity Associates Calculation No. VY-10Q-301, Revision 0, "Feedwater Nozzle Finite Element Model and Heat Transfer Coefficients."
7. Structural Integrity Associates Calculation No. YAEC-13Q-303, Revision 0, "Thermal Transient Analysis."
8. VY Drawing No. 5920-9057, Sheet I, Revision 1, SI File No. VY-05Q-215.
9. American Society of Mechanical Engineers, Boiler and Pressure Vessel Code, Section II, Part D, 1998 Edition, 2000 Addenda.

10. Structural Integrity Associates Calculation No. VY-16Q-301, Revision 0, "Feedwater Stress History Development for Nozzle Green's Function."
11. Vermont Yankee Drawing 5920-00024, Rev. 11, GE Drawing No. 919D294, Revision 11, Sheet No. 7, "Reactor Vessel," SI File No. VY-05Q-241.
12. CB&I Addenda to RPV Stress Report, "Certification of Addenda to the Stress Report for Vermont Yankee Reactor Vessel," July 9, 1971, SI File No. VY-05Q-238.
13. VY Calculation Change Notice (CCN), CCN Number 1 for Calculation VYC1005 Revision 2, "This CCN Provides a Basis for the Power Uprate Safety Analysis Report being submitted as part of the power uprate project. The 50.59 assessment will be handled by the EPU design change and NRC SER for this submittal." SI File Number VY-05Q-208.
14. Structural Integrity Associates Calculation No. SW-SPVF-01Q-301, Revision 0, "STRESS.EXE, P-V.EXE, and FATIGUE.EXE Software Verification."
15. American Society of Mechanical Engineers, Boiler and Pressure Vessel Code, Section III Subsection NB, 1998 Edition, 2000 Addenda.
16. NUREG/CR-6260 (INEL-95/0045), "Application of NUREG/CR-5999 Interim Fatigue Curves to Selected Nuclear Power Plant Components," March 1995.
17. GE Certified Design Specification No. 26A6019, Revision 1, "Reactor Vessel – Extended Power Uprate," August 29, 2003, SI File No. VY-05Q-236.
18. General Electric Stress Report No. DC22A5583, Revision 0, Section T, "Thermal Analysis FitzPatrick Feedwater Nozzle Modification," SI File No. NYPA-53Q-212.

19. Entergy Design Input Record (DIR) Revision 1, EC No. 1773, Revision 0
"Environmental Fatigue Analysis for Vermont Yankee Nuclear Power Station,"
7/26/2007, SI File No. VY-16Q-209.
20. Chicago Bridge & Iron Company Contractor 9-6201, Revision 2, "Section S4, Stress
Analysis Feedwater Nozzle Vermont Yankee Reactor Vessel," SI File No. VY-05Q-238.
21. Structural Integrity Associates Calculation No. VY-16Q-303, Revision 0,
"Environmental Fatigue Evaluation of Reactor Recirculation Inlet Nozzle and Vessel
Shell Bottom Head."
22. NUREG-1801, Revision 1, "Generic Aging Lessons Learned (GALL) Report," U. S.
Nuclear Regulatory Commission, September 2005.
23. Structural Integrity Associates Calculation No. VY-16Q-302, Revision 0, "Fatigue
Analysis of Feedwater Nozzle."
24. J. P. Holman, "Heat Transfer," 4th Edition, McGraw-Hill, 1976.
25. J. P. Holman, "Heat Transfer," 5th Edition, McGraw-Hill, 1981.
26. N. P. Chermisinoff, "Heat Transfer Pocket Handbook," Gulf Publishing Co, 1984.
27. American Society of Mechanical Engineers, Boiler and Pressure Vessel Code, Section
III, Subsection A, Article 4, 1965 Edition with Winter 1966 Addenda.

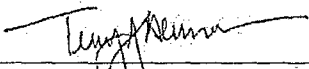
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Revision No.: 0
Project No.: VY-16Q
File No.: VY-16Q-402
July 2007

**Environmental Fatigue Analysis
for the
Vermont Yankee
Reactor Pressure Vessel
Reactor Recirculation Outlet Nozzle**

Prepared for:
Entergy Nuclear Operations, Inc.
Contract Number: 10150394

Prepared by:
Structural Integrity Associates, Inc.
Centennial, CO

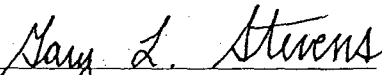
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Date: 7/26/2007

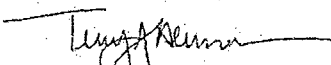
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1.0 INTRODUCTION

In Table 4.3-3 of the Vermont Yankee License Renewal Application (LRA), the 60-year cumulative usage factor (CUF) value for the reactor pressure vessel (RPV) reactor recirculation outlet nozzle is reported as 0.810. Application of environmentally assisted fatigue (EAF) multipliers, as required for the license renewal period, resulted in an unacceptable EAF CUF value of 1.98. Therefore, further refined analysis is necessary to show acceptable EAF CUF results for this component.

This report documents a refined fatigue evaluation for the VY reactor recirculation outlet nozzle. The intent of this evaluation is to use refined transient definitions and the revised cyclic transient counts for 60 years for a computation of CUF, including EAF effects, that is more refined than previously performed fatigue analyses. The fatigue-limiting locations in the reactor recirculation outlet nozzle are included in the evaluation, to be consistent with NUREG/CR-6260 [1] needs for EAF evaluation for license renewal. The resulting fatigue results will be used as a replacement to the value previously reported in the VY LRA.

The refined evaluation summarized in this report included development of a detailed finite element model of the reactor recirculation outlet nozzle, including relevant portions of the safe end, the nozzle forging, a portion of the vessel shell, and cladding as shown in the applicable drawings [2, 3]. Thermal and pressure stress histories were developed for relevant transients affecting the reactor recirculation outlet nozzle, including any effects of Extended Power Uprate (EPU), as specified by the VY RPV Design Specification [4], the VY EPU Design Specification [5] and other boiling water reactor (BWR) operating experience. The thermal and pressure stress histories were used to determine total stress intensities and primary plus secondary stress intensities for use in a subsequent fatigue evaluation [11]. Stress intensities were also included due to loads from the attached piping for application in the stress/fatigue analysis based on the bounding reaction loads obtained from the relevant design document. The revised fatigue calculation was performed using Section III methodology from the 1998 Edition, 2000 Addenda of the ASME Code [17], and was performed using actual cycles from past plant operation projected out to 60 years of operation.

1.1 Green's Function Methodology

For the reactor recirculation outlet nozzle evaluated as a part of this work, stress intensity histories were computed by a time integration of the product of a pre-determined Green's Function and the transient data. This Green's Function integration scheme is similar in concept to the Duhamel theory used in structural dynamics. A detailed derivation of this approach and examples of its application to specific plant locations is contained in Reference [6]. A general outline is provided in this section.

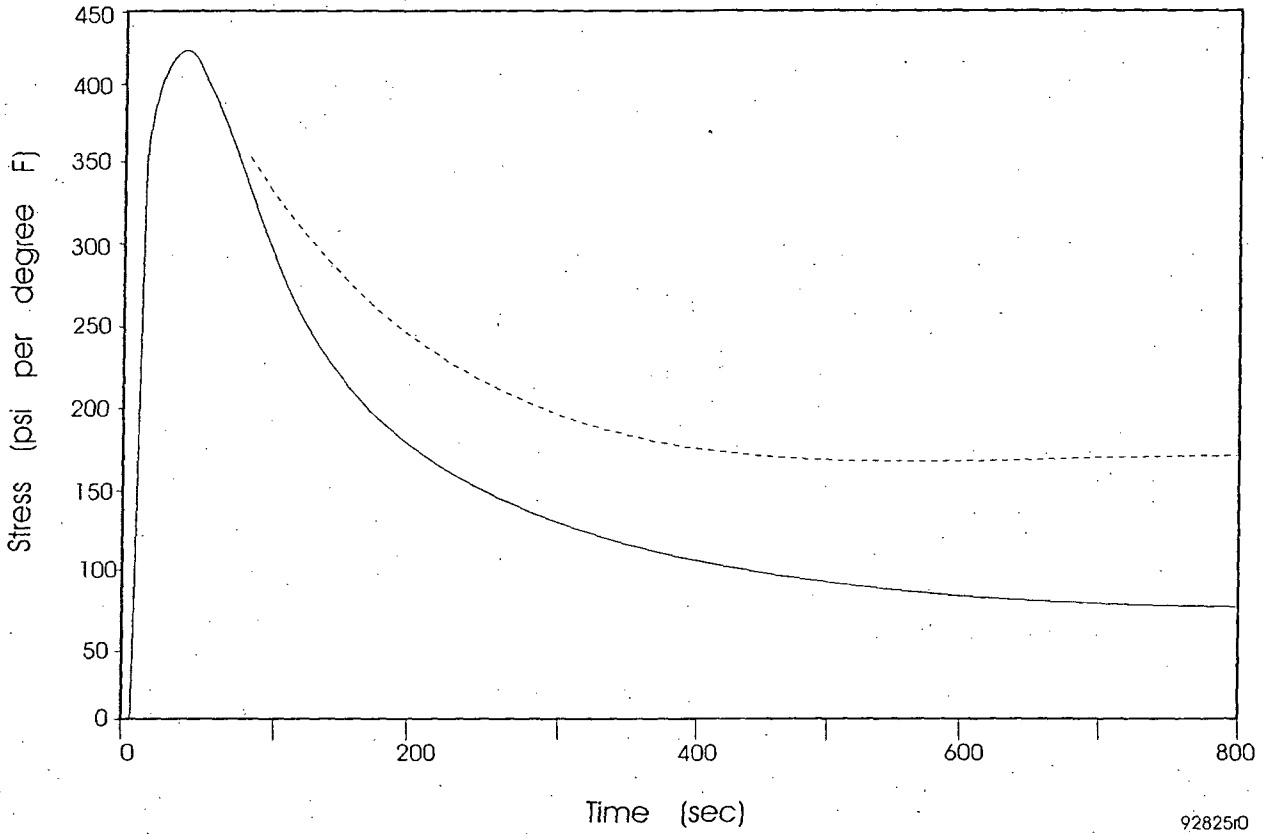
A Green's Function is derived by using finite element methods to determine the transient stress response of the component to a step change in loading (usually a thermal shock). The critical location in the component is identified based on the maximum stress intensity, and the thermal stress intensity response over time is extracted for this location. This response to the input thermal step is the "Green's Function." Figure 1-1 shows a typical set of two Green's Functions, each for a different set of heat transfer coefficients (representing different flow rate conditions).

To compute the thermal stress response for an arbitrary transient, the loading parameter (usually local fluid temperature) is deconstructed into a series of step-loadings. By using the Green's Function, the response to each step can be quickly determined. By the principle of superposition, these can be added (algebraically) to determine the response to the original load history. The result is demonstrated in Figure 1-2. The input transient temperature history contains five step-changes of varying size, as shown in the upper plot in Figure 1-2. These five step changes produce the five successive stress responses in the second plot shown in Figure 1-2. By adding all five response curves, the real-time stress response for the input thermal transient is computed:

The Green's Function methodology produces identical results compared to running the input transient through the finite element model. The advantage of using Green's Functions is that many individual transients can be run with a significant reduction of effort compared to running all transients through the finite element model. The trade-off in this process is that the Green's Functions are based on constant material properties and heat transfer coefficients. Therefore, these parameters are chosen to bound all transients that constitute the majority of fatigue usage, i.e., the heat transfer coefficients at 300°F bound

the cold water injection transient. In addition, the instantaneous value for the coefficient of thermal expansion is used instead of the mean value for the coefficient of thermal expansion. This conservatism is more than offset by the benefit of not having to analyze every transient, which was done in the VY reactor recirculation outlet nozzle evaluation.

Once the stress history is obtained for all transients using the Green's Function approach, the remainder of the fatigue analysis is carried out using traditional methodologies in accordance with ASME Code, Section III requirements [17].



Note: A typical set of two Green's Functions is shown, each for a different set of heat transfer coefficients (representing different flow rate conditions).

Figure 1-1. Typical Green's Functions for Thermal Transient Stress

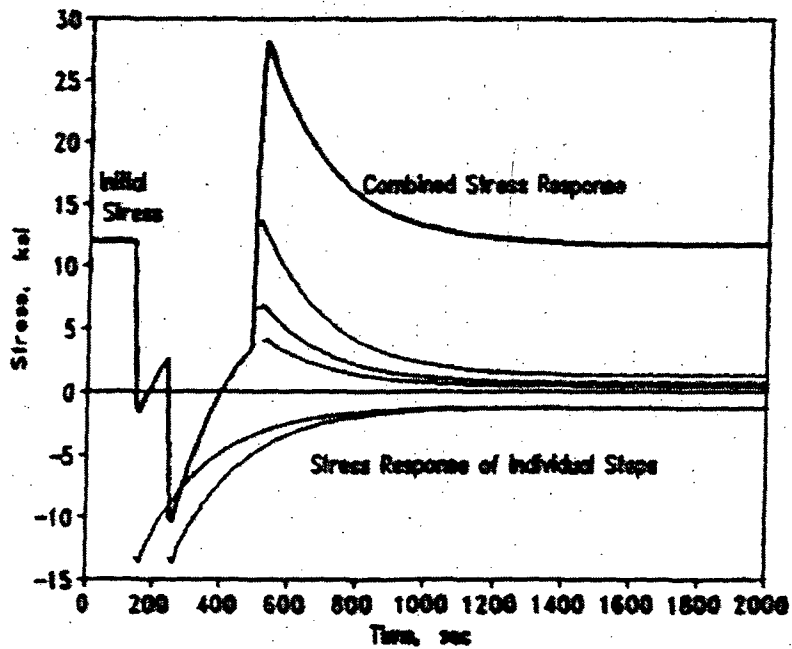
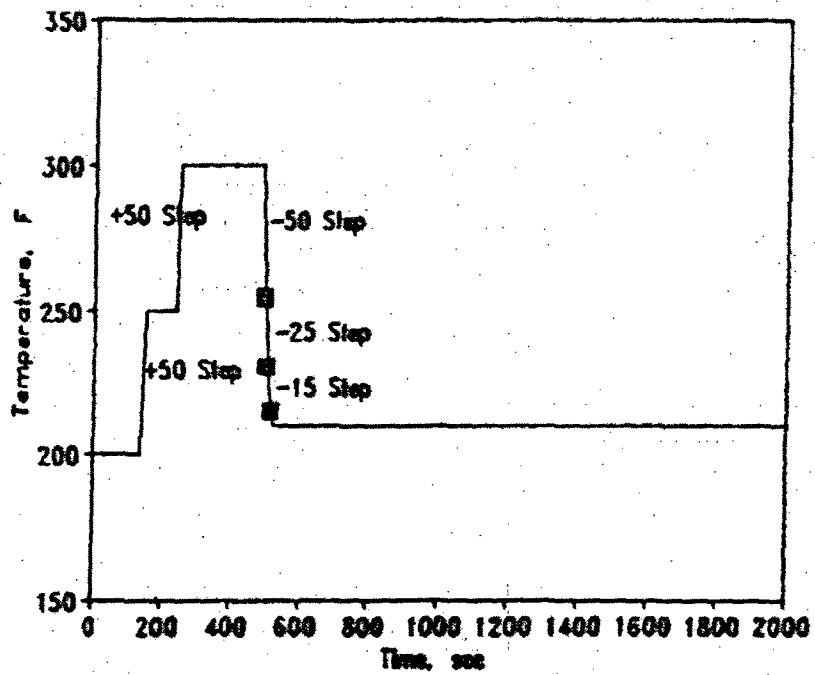


Figure 1-2. Typical Stress Response Using Green's Functions

2.0 FINITE ELEMENT MODEL

An ANSYS [7] finite element model (FEM) of the VY reactor recirculation outlet nozzle configuration was developed and used to perform the updated stress and fatigue analyses. The details of the model development are documented in the Reference [8] calculation.

The materials of the various components of the model are listed below:

- Safe End – SA182 F316 (16Cr-12Ni-2Mo)
- Piping – SA376 TP316 (16Cr-12Ni-2Mo)
- Nozzle Forging – SA508 Class 2 (3/4Ni-1/2Mo-1/3Cr-V)
- Vessel – SA533 Grade B (Mn-1/2Mo-1/2Ni)
- Cladding – SA240 Type 304 (18Cr-8Ni)

The radius of the RPV was increased by a factor of two to account for the fact that the vessel portion of the two dimensional axisymmetric finite element model is a sphere and the actual geometry is a cylinder.

Material properties were based upon the 1998 ASME Code, Section II, Part D, with 2000 Addenda [9], and are shown in Table 2-1. The properties for the Green's Functions were evaluated at an average temperature of 300°F. This average temperature is based on a thermal shock of 500°F to 100°F which was applied to the FEM model for Green's Function development. Due to the thermal shocks having a different temperature range at the nozzle blend radius and safe end for the Improper Start transient (Transient 9), this transient was run separately. Material properties were evaluated at 400°F for all locations except for the safe end and piping, which were evaluated using material properties at 300°F since the temperature is lower at this location and the properties at 300°F are closest to the average temperature of 330°F.

The finite element model is shown in Figure 2-1.

Table 2-1. Material Properties

For the Green's Functions (evaluated at 300°F):

Material	SA533 Gr. B Mn-1/2Mo- 1/2Ni	SA508 Cl. 2 3/4Ni-1/2Mo- 1/3Cr-V	SA240 TP 304 18Cr-8Ni	SA18 SA370 16Cr-1
Modulus of Elasticity, e+6 psi	28.0	26.7	27.0	2
Coefficient of Thermal Expansion, e-6, in/in/°F	7.7	7.3	9.8	9
Thermal Conductivity, Btu/hr-ft-°F	23.4	23.4	9.8	9
Thermal Diffusivity, ft ² /hr	0.401	0.401	0.160	0.
Specific Heat, Btu/lb-°F	0.119	0.119	0.125	0.
Density, lb/in ³	0.283	0.283	0.283	0.
Poisson's Ratio	0.3	0.3	0.3	(

For Transient 9 (evaluated at 400°F – except for the safe end and piping, which are evaluated at 300°F):

Property	SA533 Gr. B Mn-1/2Mo- 1/2Ni	SA508 Cl. 2 3/4Ni-1/2Mo- 1/3Cr-V	SA240 TP 304 18Cr-8Ni	SA18 SA370 16Cr-1 @:
Modulus of Elasticity, e+6 psi	27.4	26.1	26.5	2
Coefficient of Thermal Expansion, e-6, in/in/°F	8.0	7.7	10.2	
Thermal Conductivity, Btu/hr-ft-°F	23.1	23.1	10.4	
Thermal Diffusivity, ft ² /hr	0.378	0.378	0.165	0
Specific Heat, Btu/lb-°F	0.125	0.125	0.129	0
Density, lb/in ³	0.283	0.283	0.283	0.
Poisson's Ratio	0.3	0.3	0.3	

Note: 1. Material properties are taken from the 1998 ASME Code, Section II, Part D, with 2000 Addenda [9], except for density which are assumed typical values and specific heat calculated as $[k/pd] / 12^3$.

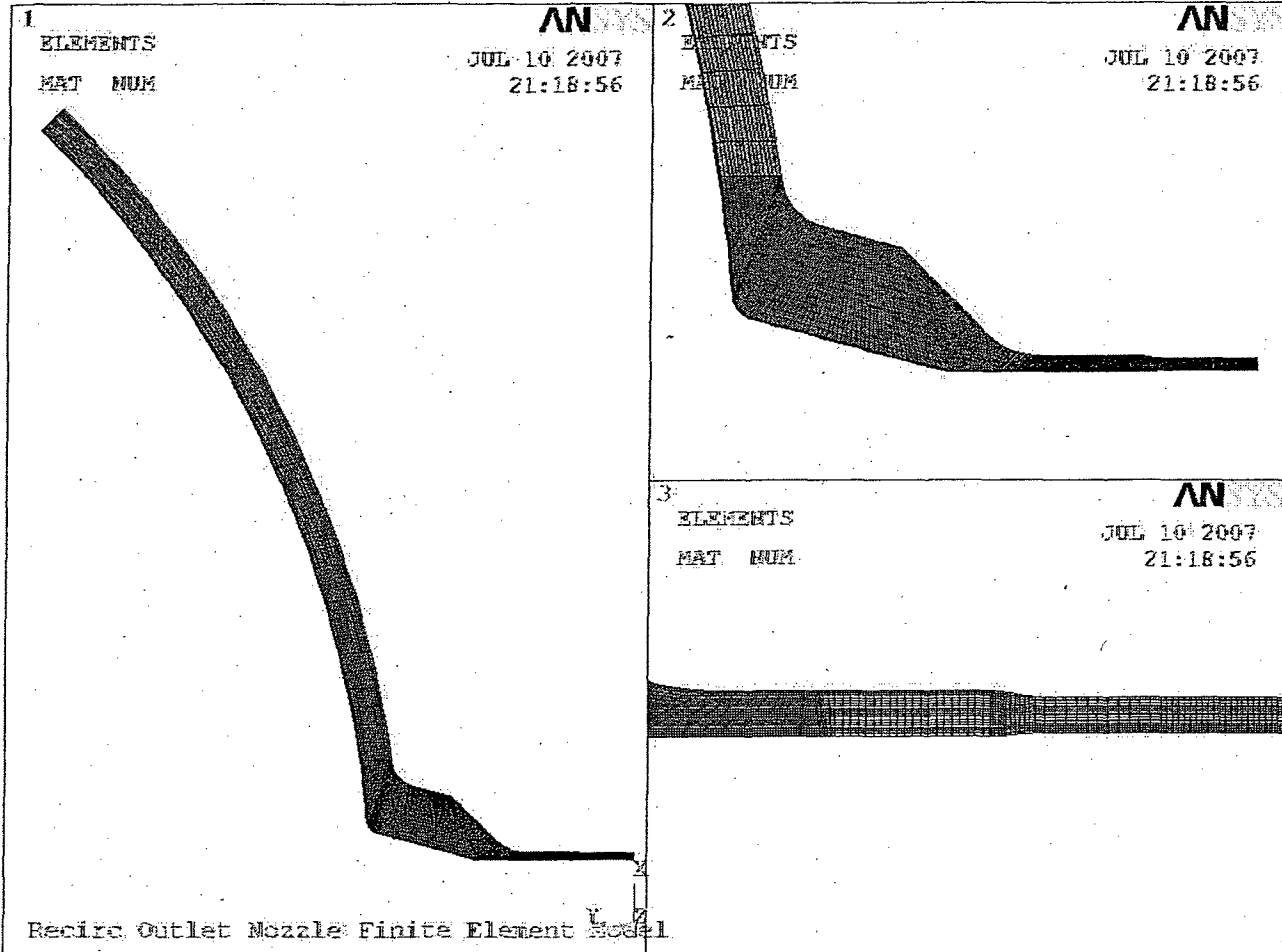


Figure 2-1. VY Reactor Recirculation Outlet Nozzle FEM

3.0 LOAD DEFINITIONS

The pressure and thermal stresses for the reactor recirculation outlet nozzle for the revised fatigue evaluation were developed using the axisymmetric FEM model described in Section 2.0 of this report. The details of the Green's Function development and associated stress and fatigue evaluations are documented in the Reference [10] and [11] calculations.

3.1 Thermal Loading

Thermal loads were applied to the recirculation outlet nozzle model to generate the Green's Functions. As a first step in the Green's Function process, heat transfer coefficients were determined for various regions of the reactor recirculation outlet nozzle FEM for three different flow cases: (1) 100% reactor recirculation outlet nozzle flow, (2) 50% reactor recirculation outlet nozzle flow and (3) 0% reactor recirculation outlet nozzle flow.

The 100% flow case simulates the operational condition of the reactor recirculation outlet nozzle (i.e., normal recirculation system flow). The heat transfer coefficients for the high flow case are for forced convection. The applied boundary fluid temperature is changed to simulate a thermal shock from 500°F to 100°F to develop the stress response on the reactor recirculation outlet nozzle due to normal operating conditions.

The 50% flow case simulates a reduced flow condition of the reactor recirculation outlet nozzle (i.e., during power ascension). The heat transfer coefficients for the reduced flow case are also for forced convection. The applied boundary fluid temperature is changed to simulate a thermal shock from 500°F to 100°F to develop the stress response on the reactor recirculation outlet nozzle under reduced flow conditions.

The 0% flow case simulates a stagnant condition of the reactor recirculation outlet nozzle when recirculation flow is stopped and the entire reactor recirculation outlet nozzle is at the same temperature as the RPV fluid. The heat transfer coefficients for the 0% flow case are for free

convection (stagnant) conditions. The applied boundary fluid temperature is changed to simulate a thermal shock from 500°F to 100°F to develop the stress response on the reactor recirculation outlet nozzle in the stagnant condition.

The temperature on the exterior of the reactor, nozzle, safe end and pipe was assumed to be 120°F (ambient). Figure 3-1 shows the heat transfer coefficient regions assumed for the reactor recirculation outlet nozzle FEM. The applied heat transfer coefficients and the fluid temperatures are summarized in the sections that follow.

3.1.1 Heat Transfer Coefficients and Boundary Fluid Temperatures

Referring to Figure 3-1, heat transfer coefficients for the Green's Functions were applied as follows:

For Green's Functions:

Region 1

The heat transfer coefficient, h , for 100% flow is 3,577.8 BTU/hr-ft²-°F at 300°F.

The heat transfer coefficient, h , for 50% flow is 2,054.9 BTU/hr-ft²-°F at 300°F.

The heat transfer coefficient, h , for 0% flow is 112.34 BTU/hr-ft²-°F at 300°F.

Region 2

The heat transfer coefficient for Region 2 was linearly transitioned from the value of the heat transfer coefficient used in Region 1 to the value used in Region 3.

Region 3

The heat transfer coefficient, h , for 100% flow is 3,361 BTU/hr-ft²-°F at 300°F.

The heat transfer coefficient, h , for 50% flow is 1,930.9 BTU/hr-ft²-°F at 300°F.

The heat transfer coefficient, h , for 0% flow is 112.34 BTU/hr-ft²-°F at 300°F.

Region 4

The heat transfer coefficient for Region 4 (nozzle blend radius) was linearly transitioned from the value of the heat transfer coefficient used in Region 3 to the value used in Region 5.

Region 5

The heat transfer coefficient, h , for 100% flow is 1,788.9 BTU/hr-ft²-°F at 300°F.

The heat transfer coefficient, h , for 50% flow is 1,027.4 BTU/hr-ft²-°F at 300°F.

The heat transfer coefficient, h , for 0% flow is 101 BTU/hr-ft²-°F at 300°F.

Region 6

The heat transfer coefficient, h , is 0.4 BTU/hr-ft²-°F.

For all three Green's functions, a 500°F to 100°F thermal shock was evaluated in Regions 1 through 5 to determine the stress response. For Region 6, a constant temperature of 120°F was used.

Referring to Figure 3-2, heat transfer coefficients for Transient 9 were applied as follows:

For Transient 9:

Region 1

The heat transfer coefficient, h , for 12% flow is 672.8 BTU/hr-ft²-°F at 500°F.

The heat transfer coefficient, h , for 12% flow is 308.2 BTU/hr-ft²-°F at 100°F.

The fluid temperature shock is: $T = 526^\circ\text{F} - 130^\circ\text{F} - 526^\circ\text{F}$.

Region 2

The heat transfer coefficient, h , for 12% flow is 632.2 BTU/hr-ft²-°F at 500°F.

The heat transfer coefficient, h , for 12% flow is 616.6 BTU/hr-ft²-°F at 300°F.

The fluid temperature shock is: $T = 526^\circ\text{F} - 268^\circ\text{F} - 526^\circ\text{F}$.

Region 3

The heat transfer coefficient, h , for 12% flow is 336.4 BTU/hr-ft²-°F at 500°F.

The heat transfer coefficient, h , for 12% flow is 328.0 BTU/hr-ft²-°F at 300°F.

The fluid temperature shock is:

Case 1: $T = 526^\circ\text{F} - 268^\circ\text{F} - 526^\circ\text{F}$

Case 2: $T = 526^\circ\text{F}$

Region 4

The heat transfer coefficient, h , is 0.4 BTU/hr-ft²-°F.

The temperature, T , is 120°F.

Transition Regions

The heat transfer coefficient was linearly transitioned from the value of the heat transfer coefficient used in adjacent regions.

3.1.2 Green's Functions

The three flow-dependent thermal load cases outlined in the previous section were run on the reactor recirculation outlet nozzle FEM with the heat transfer coefficients and the fluid temperature conditions listed in Section 3.1.1. Two locations were selected for analysis (see Figures 3-3 and 3-4):

1. The critical safe end location was chosen as the node with the highest stress intensity due to thermal loading under high flow conditions. The highest stress intensity due to thermal loading occurred at Node 6395 (see Figure 3-3), on the inside diameter of the nozzle safe end. Therefore, this node was selected for analysis.
2. The critical blend radius location was chosen based upon the highest pressure stress intensity, neglecting the cladding. The critical location is selected as Node 3829, as shown in Figure 3-4.

Twelve stress intensity Green's Functions were developed (i.e., total stress intensity and membrane plus bending stress intensity for each location and each flow case). The total stress intensity Green's Functions for the safe end location at 100%, 50% and 0% flow are shown in Figures 3-5, 3-6 and 3-7, respectively. The total stress intensity Green's Function for the blend radius location at 100%, 50% and 0% flow are shown in Figures 3-8, 3-9 and 3-10, respectively.

3.1.3 Thermal Transients

The transients analyzed for the reactor recirculation outlet nozzle were developed based on the definitions in the original RPV Design Specification [4], as modified for EPU [5], as well as more recent definitions based on BWR operating experience. For BWR operating experience, the transients described in the thermal cycle diagrams for a BWR-4 plant similar in design and vintage to VY were obtained, and plant-specific data from VY was applied to each transient. The resulting thermal cycle diagrams are shown in References [12 and 13]. The final transients evaluated in the stress and fatigue analyses are shown in Figures 3-11 through 3-20.

The number of cycles projected for the 60-year operating life was used for each transient [14]. Tables 3-1 and 3-2 summarize the thermal transients for the safe end and blend radius locations, respectively.

3.2 Pressure Loading

A uniform pressure of 1,000 psi was applied along the inside surface of the reactor recirculation outlet nozzle and the RPV wall. A pressure load of 1,000 psi was used because it is easily scaled up or down to account for different pressures that occur during transients. In addition, a cap load was applied to the piping at the end of the FEM. This cap load was calculated as follows:

$$P_{\text{cap}} = P \cdot \frac{(R_i^2)}{(R_o^2 - R_i^2)}$$

where:

P_{cap}	=	end cap pressure load (psi)
P	=	unit pressure load = 1,000 psi
R_i	=	Inner Radius = 12.969 in
R_o	=	Outer Radius = 14.188 in

The calculated pressure cap load of 5,081.7 psi was applied as a negative value so that it would exert tension on the end of the model. The nodes on the end of the FEM were coupled in the axial direction to ensure mutual displacement of the end of the FEM due to the attached piping. Figures 3-21, 3-22, and 3-23 show the internal pressure distribution, cap load, and symmetry condition applied to the RPV end of the model, respectively.

The internal pressure load case for the blend radius resulted in a total stress intensity of 31,300 psi, and for the safe-end resulted in a total stress intensity of 11,490 psi. The membrane plus bending stress intensity at the blend radius is 33,640 psi and at the safe end is 11,350 psi, respectively.

3.3 Piping Loading

The piping stress intensities (stress caused by the attached piping) were determined for the two evaluated reactor recirculation outlet nozzle locations. The design piping reactions that were used in the stress and fatigue evaluation are defined on the Reference [15] drawing. These loads represent shear and moment loadings on the nozzle resulting from thermal expansion of the attached piping and seismic loads. The loads are as shown in Figure 3-24. The stresses resulting from these loads were calculated by hand using classical structural mechanics formulas, as documented in Reference [11], and are shown in Tables 3-3 and 3-4 for the safe end and blend radius locations, respectively.

Table 3-1: Safe End Transients

Transient Number	Time (s)	Temp (°F)	Time Step (s)	Pressure (psig)	Flow Rate (GPM)	Transient Number	Time (s)	Temp (°F)	Time Step (s)	Pressure (psig)	Flow Rate (GPM)
1. Normal Startup with Heatup at 100°F/hr 300 Cycles	0	100		0	14147.0	6. Reactor Overpressure 1 Cycle	0	526		1010	28294
	16164	549	16164	1010	(50%)		2	526	2	1375	(100%)
	16864	549	700	1010			32	526	30	940	
2. Turbine Roll and Increase to Rated Power 300 Cycles	0	549		1010	28294		1832	526	1800	940	
	1	542	1	1010	(100%)		2252	549	420	1010	
	601	542	600	1010			2312	549	60	1010	
	602	526	1	1010			2313	542	1	1010	
	1302	526	700	1010			2913	542	600	1010	
3. Loss of Feedwater Heaters Turbine Trip 25% Power 10 Cycles	0	526		1010	28294		2914	526	1	1010	
	1800	542	1800	1010	(100%)		3614	526	700	1010	
	2100	542	300	1010			7. SRV Blowdown 1 Cycle	0	526	1010	28294
	2460	526	360	1010				600	375	600	170
	3060	526	600	1010		11580		70	10980	50	
	3960	542	900	1010		12280	70	700	50		
	4260	542	300	1010		8. SCRAM Other 228 Cycles	0	526	1010	28294	
	6060	526	1800	1010			15	526	15	940	(100%)
6760	526	700	1010		1815		526	1800	940		
4. Loss of Feedwater Pumps 10 Cycles	0	526		1010	0		2235	549	420	1010	
	3	526	3	1190	(0%)		2295	549	60	1010	
	13	526	10	1135			2296	542	1	1010	
	233	300	220	1135		2356	542	60	1010		
	2213	500	1980	1135		2357	526	1	1010		
	2393	300	180	885		3057	526	700	1010		
	6773	500	4380	1135		9. Improper Startup 1 Cycle	0	526	1010	3395	
	7193	300	420	675	14147		1	130 ⁽³⁾	1	1010	(12%)
	7493	300	300	675	(50%)		27	130 ⁽³⁾	26	1010	
	11093	400	3600	240			28	526	1	1010	
16457	549	5364	1010		728	526	700	1010			
16517	549	60	1010		10. Shutdown 300 Cycles	0	549	1010	14147		
16518	542	1	1010	28294		6264	375	6264	170	(50%)	
17118	542	600	1010	(100%)		6864	330	600	88		
17119	526	1	1010			16224	70	9360	50		
17819	526	700	1010		16924	70	700	50			
5. Turbine Generator Trip 60 Cycles	0	526		1010	28294	11. Design Hydrostatic Test 120 Cycles	—	100	—	0	1981
	10	526	10	1135	(100%)		—	—	—	1100	(7%)
	15	526	5	1135		12. Hydrostatic Test 1 Cycle	—	100	—	50	1981
	30	526	15	940			—	—	—	50	(7%)
	1830	526	1800	940			—	—	—	1563	(7%)
	2250	549	420	1010			—	—	—	50	
	2310	549	60	1010							
	2311	542	1	1010							
	2911	542	600	1010							
	2912	526	1	1010							
3612	526	700	1010								

- Notes:
1. The instant temperature change is assumed as 1 second time step.
 2. The number of cycles is for 60 years [14].
 3. 130°F is the safe end temperature for this transient. The blend radius has a different temperature for Transient 9. [13]

Note: These transients are the same as in Table 3-2 with the exception of the 700 second steady state time increment that is used. The transients in Table 2 are plotted using a 6000 second steady state increment. The difference is due to the length of the Green's Function for the safe end which is shorter compared to the blend radius.

Table 3-2: Blend Radius Transients

Transient Number	Time (s)	Temp (°F)	Time Step (s)	Pressure (psig)	Flow Rate (GPM)	Transient Number	Time (s)	Temp (°F)	Time Step (s)	Pressure (psig)	Flow Rate (GPM)
1. Normal Startup with Heatup at 100°F/hr 300 Cycles	0	100		0	14147.0	6. Reactor Overpressure 1 Cycle	0	526		1010	28294
	16164	549	16164	1010	(50%)		2	526	2	1375	(100%)
	22164	549	6000	1010			32	526	30	940	
2. Turbine Roll and Increase to Rated Power 300 Cycles	0	549		1010	28294		1832	526	1800	940	
	1	542	1	1010	(100%)		2252	549	420	1010	
	601	542	600	1010			2312	549	60	1010	
	602	526	1	1010			2313	542	1	1010	
3. Loss of Feedwater Heaters Turbine Trip 25% Power 10 Cycles	0	526		1010	28294		2913	542	600	1010	
	1800	542	1800	1010	(100%)		2914	526	1	1010	
	2100	542	300	1010			8914	526	6000	1010	
	2460	526	360	1010			7. SRV Blowdown 1 Cycle	0	526		1010
	3060	526	600	1010		600		375	600	170	(100%)
	3960	542	900	1010		11580		70	10980	50	
	4260	542	300	1010		17580		70	6000	50	
6060	526	1800	1010		8. SCRAM Other 228 Cycles	0		526		1010	28294
12060	526	6000	1010			15	526	15	940	(100%)	
4. Loss of Feedwater Pumps 10 Cycles	0	526		1010		0	1815	526	1800	940	
	3	526	3	1190		(0%)	2235	549	420	1010	
	13	526	10	1135			2295	549	60	1010	
	233	300	220	1135		2296	542	1	1010		
	2213	500	1980	1136		2356	542	60	1010		
	2393	300	180	885		2357	526	1	1010		
	6773	500	4380	1135		8357	526	6000	1010		
	7193	300	420	675		9. Improper Startup 1 Cycle	0	526		1010	3395
	7493	300	300	675			1	268 ⁽³⁾	1	1010	(12%)
	10793	400	3600	240			27	268 ⁽³⁾	26	1010	
	12862	549	5369	1010			28	526	1	1010	
	12922	549	60	1010		6028	526	6000	1010		
	12923	542	1	1010		10. Shutdown 300 Cycles	0	549		1010	14147
	13523	542	600	1010			6264	375	6264	170	(50%)
13524	526	1	1010		6864		330	600	88		
19524	526	6000	1010		16224		70	9360	50		
5. Turbine Generator Trip 60 Cycles	0	526		1010	28294	22224	70	6000	50		
	10	526	10	1135	(100%)	11. Design Hydrostatic Test 120 cycles	—	100	—	50	1981
	15	526	5	1135			—	—	—	1563	(7%)
	30	526	15	940		12. Hydrostatic Test 1 Cycle	—	100	—	0	1981
	1830	526	1800	940			—	—	—	1100	(7%)
	2250	549	420	1010			—	—	—	50	
	2310	549	60	1010							
	2311	542	1	1010							
	2911	542	600	1010							
	2912	526	1	1010							
8912	526	6000	1010								

Notes:

1. The instant temperature change is assumed as 1 second time step.
2. The number of cycles is for 60 years.[14].
3. 268°F is the blend radius temperature for this transient. The safe end has a different temperature for Transient 9. [13]

Table 3-3: Stresses Due to Piping Loads for Safe End Location

Safe End External Piping Loads		
Parameters		
$F_x =$	20.00	kips
$F_y =$	20.00	kips
$F_z =$	30.00	kips
$M_x =$	2004.00	in-kips
$M_y =$	3000.00	in-kips
$M_z =$	2004.00	in-kips
OD=	28.38	in
ID=	25.938	in
$R_N =$	13.58	in
L =	4.25	in
$t_N =$	1.22	in
$(M_x)_2 =$	1919.00	in-kips
$(M_y)_2 =$	3085.00	in-kips
$M_{xy} =$	3633.15	in-kips
$F_{xy} =$	28.28	kips
$N_z =$	6.62	kips/in
$q_N =$	-1.07	kips/in
Primary Membrane Stress Intensity		
$PM_z =$	5.43	ksi
$\tau =$	-0.88	ksi
$SI_{max} =$	5.71	ksi
$SI_{max} =$	5708.89	psi

Table 3-4: Stresses Due to Piping Loads for Blend Radius Location

Blend Radius External Piping Loads		
Parameters		
$F_x =$	20.00	kips
$F_y =$	20.00	kips
$F_z =$	30.00	kips
$M_x =$	2004.00	in-kips
$M_y =$	3000.00	in-kips
$M_z =$	2004.00	in-kips
OD=	55.88	in
ID=	37.368	in
$R_N =$	23.31	in
L =	42.77	in
$t_N =$	9.25	in
$(M_x)_2 =$	1148.54	in-kips
$(M_y)_2 =$	3855.46	in-kips
$M_{xy} =$	4022.90	in-kips
$F_{xy} =$	28.28	kips
$N_z =$	2.56	kips/in
$q_N =$	-0.20	kips/in
Primary Membrane Stress Intensity		
$PM_z =$	0.28	ksi
$\tau =$	-0.02	ksi
$SI_{max} =$	0.28	ksi
$SI_{max} =$	280.16	psi

AREAS
MAT NUM

Region 5

Region 6

Region 4

Region 3

Region 2

Region 1



Recirc Outlet Nozzle Finite Element Model

Figure 3-1: Thermal Regions for Green's Functions

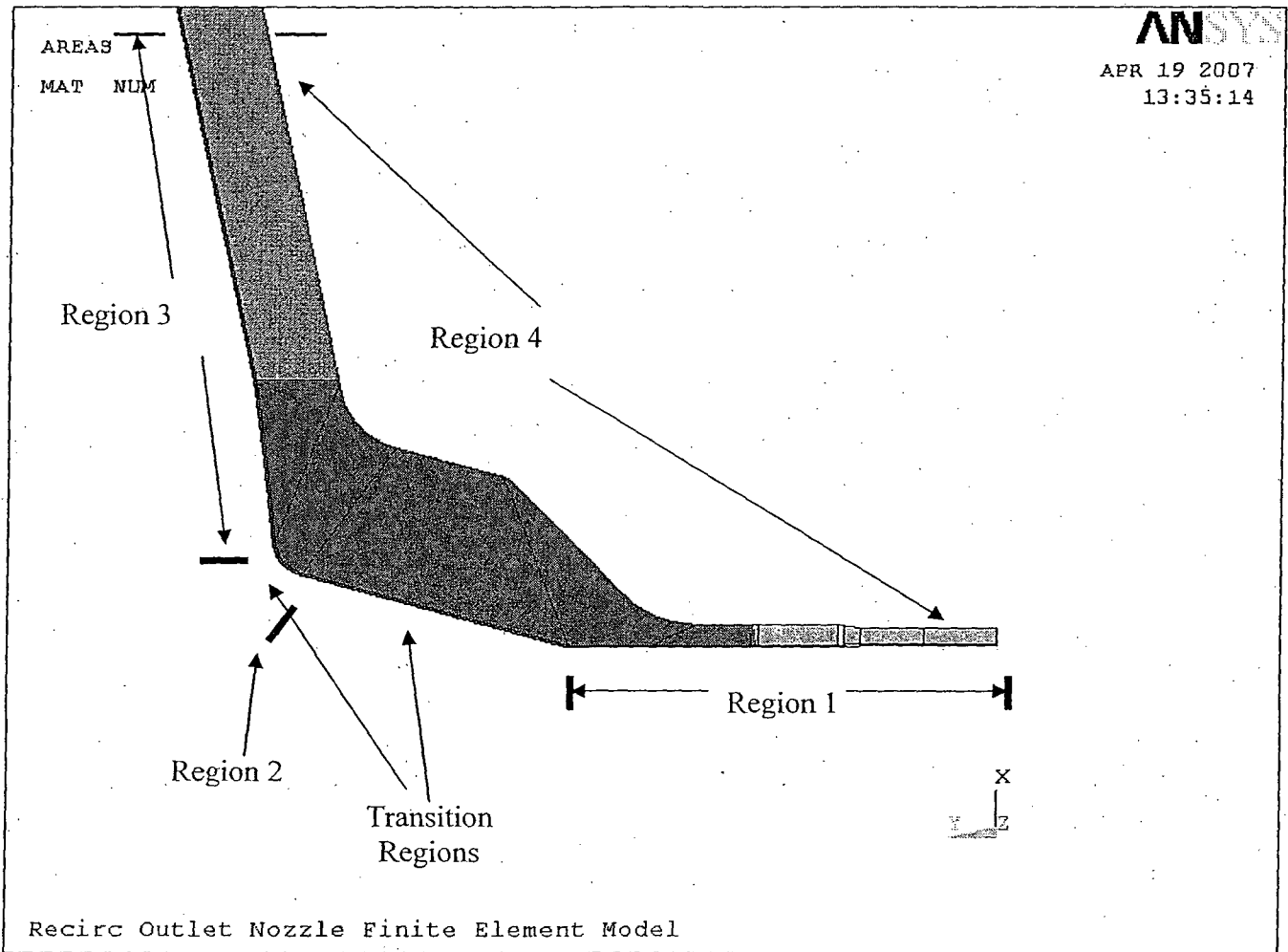


Figure 3-2: Thermal Regions for Transient 9

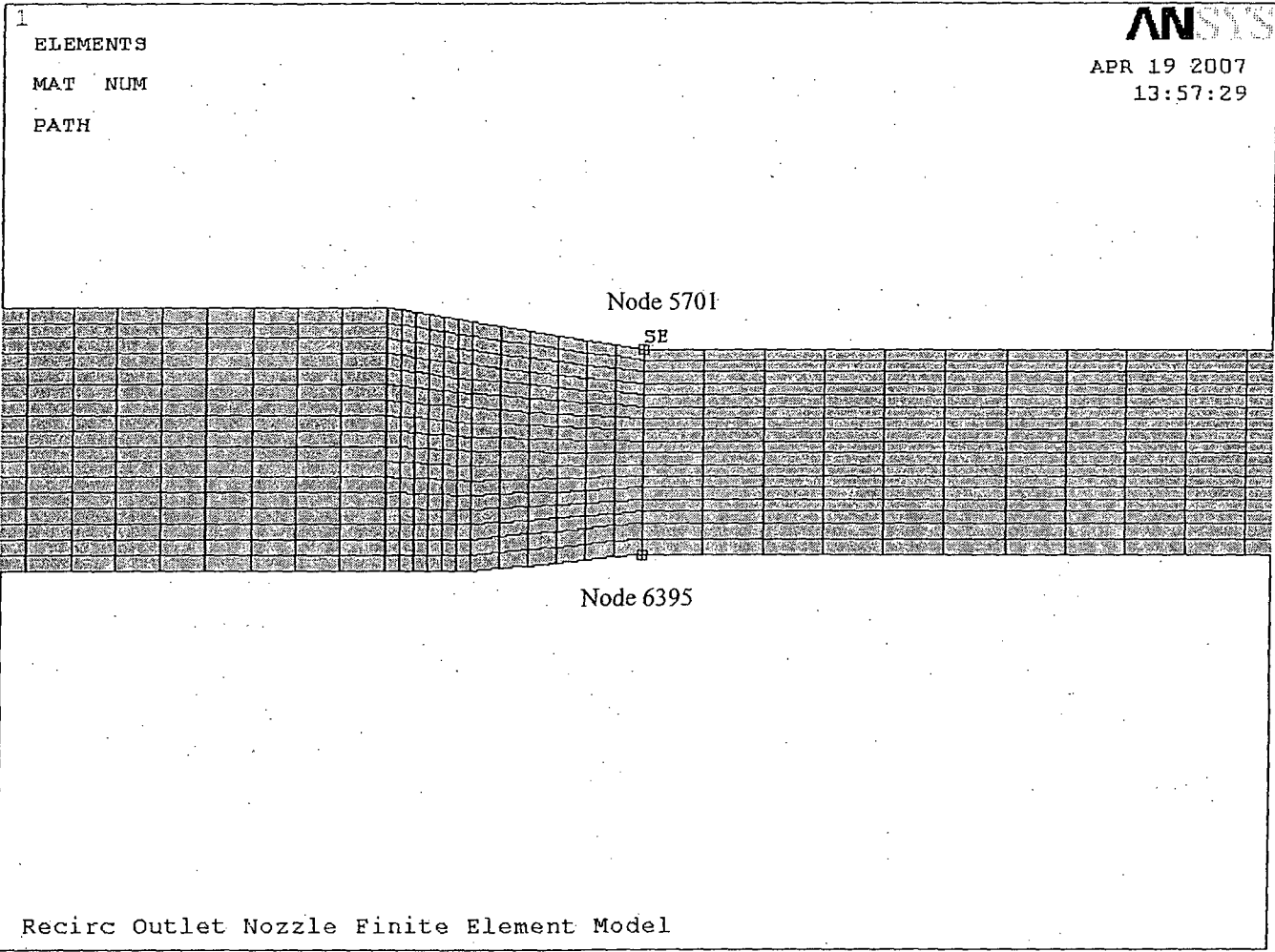


Figure 3-3: Safe End Critical Thermal Stress Location

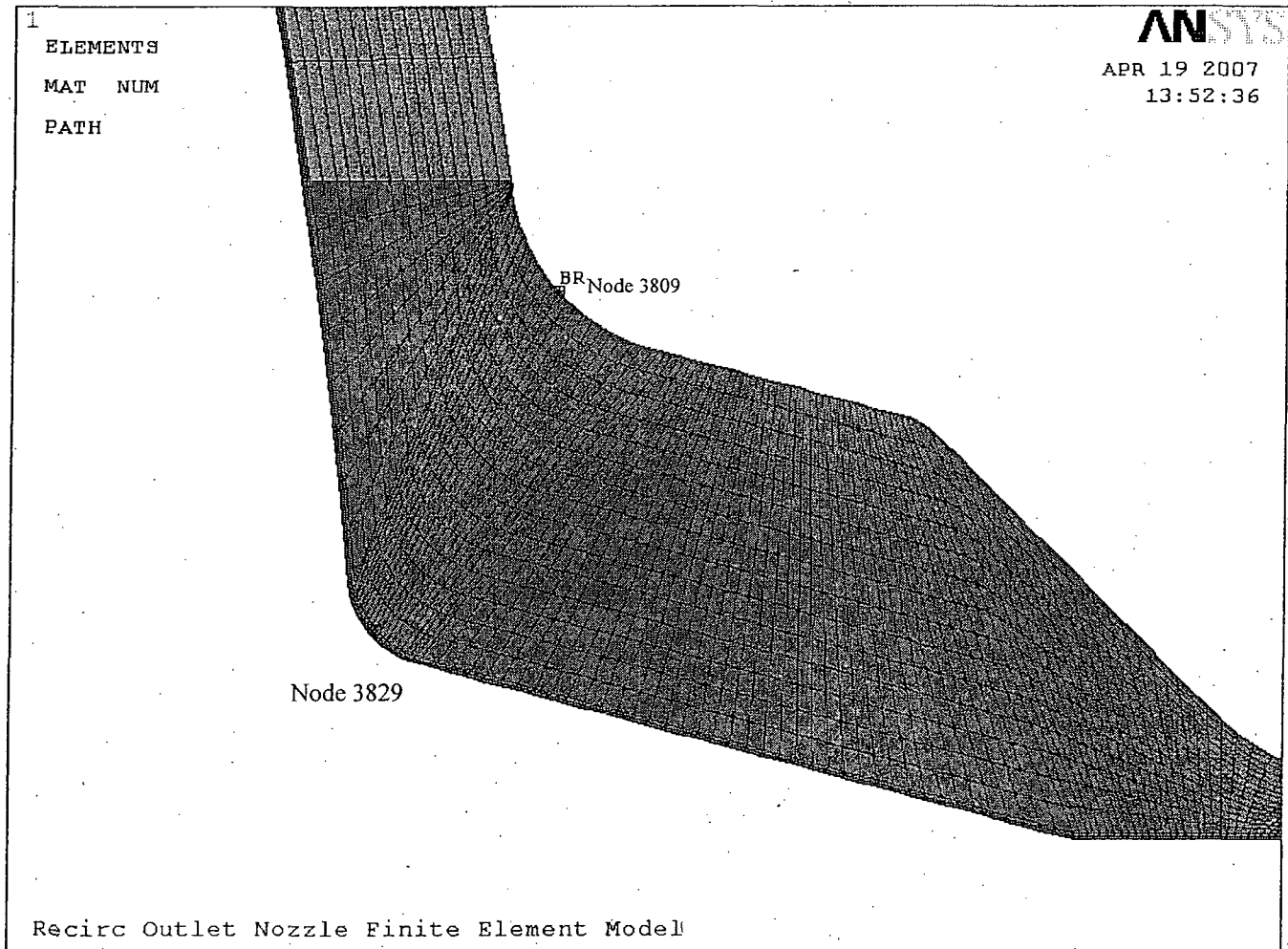


Figure 3-4: Blend Radius Critical Thermal Stress Location

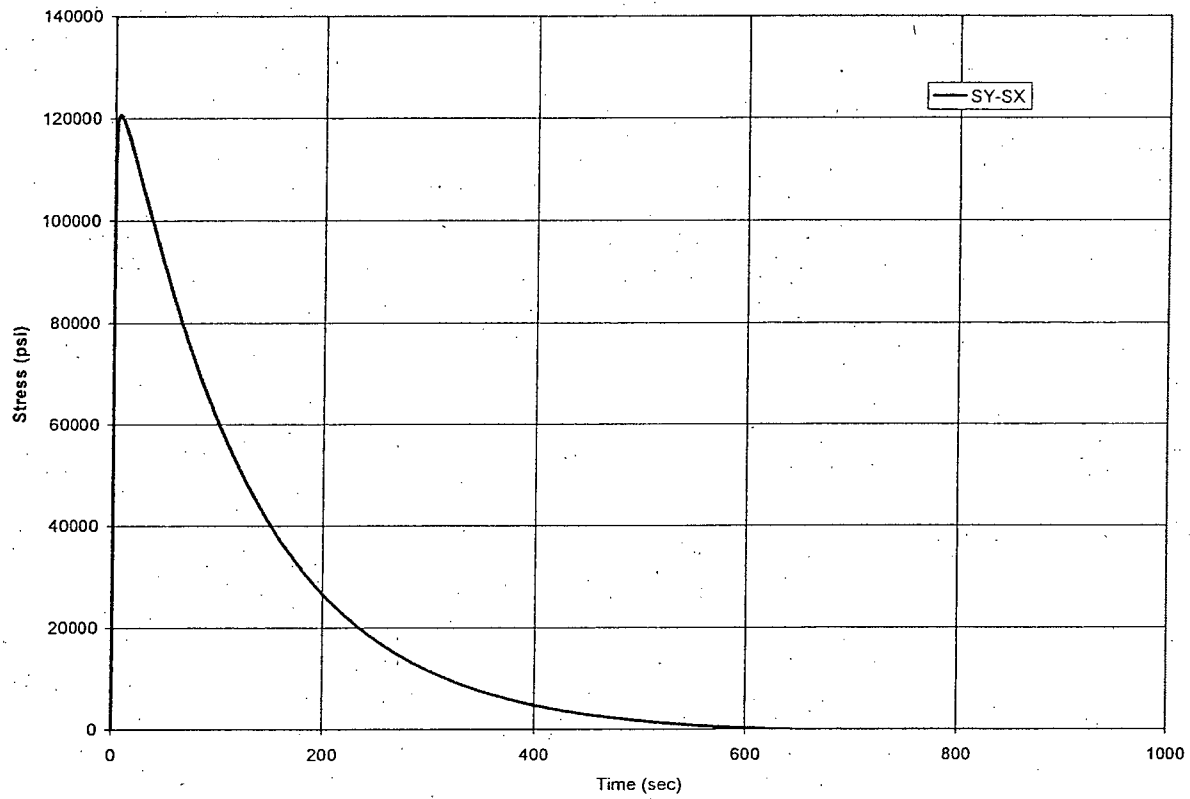


Figure 3-5: Safe End Total Stress Intensity Green's Function for 100% Flow

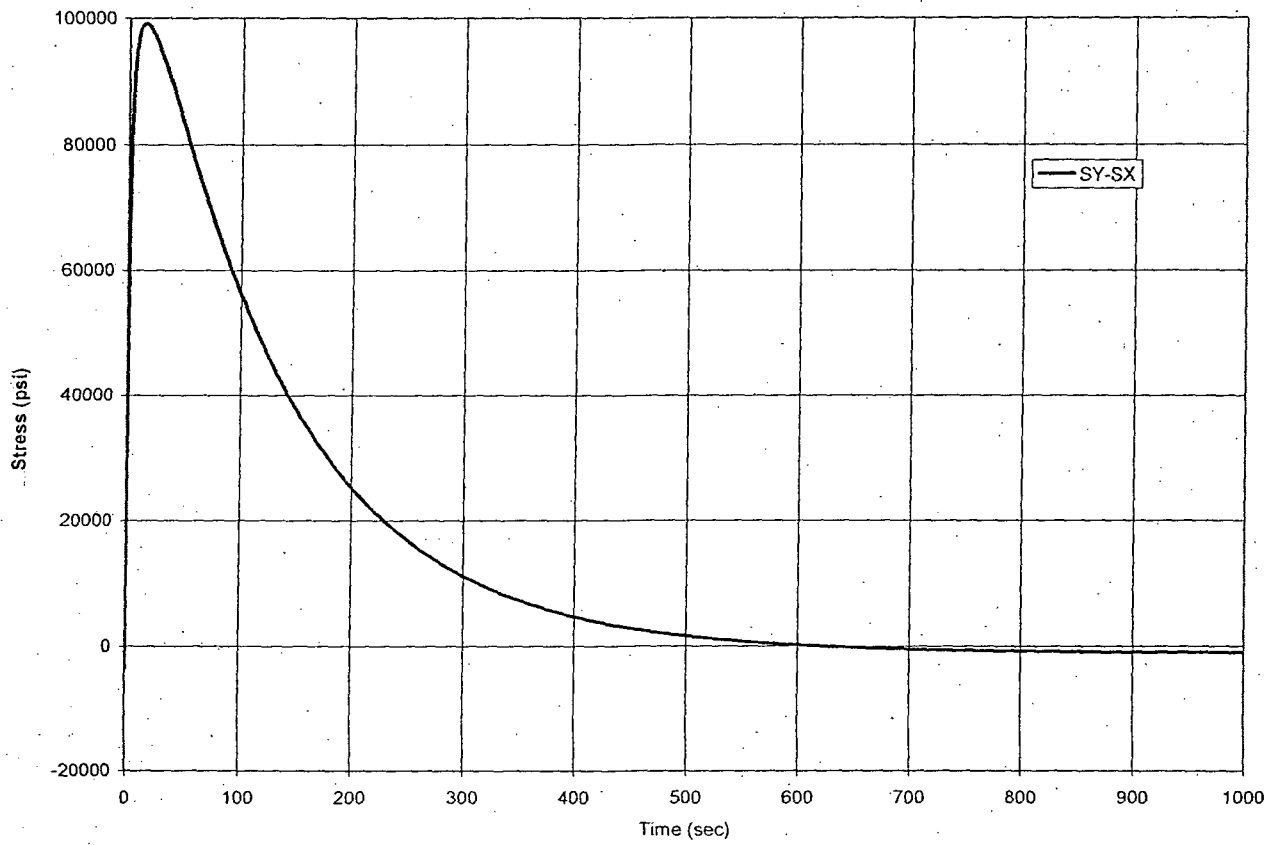


Figure 3-6: Safe End Total Stress Intensity Green's Function for 50% Flow

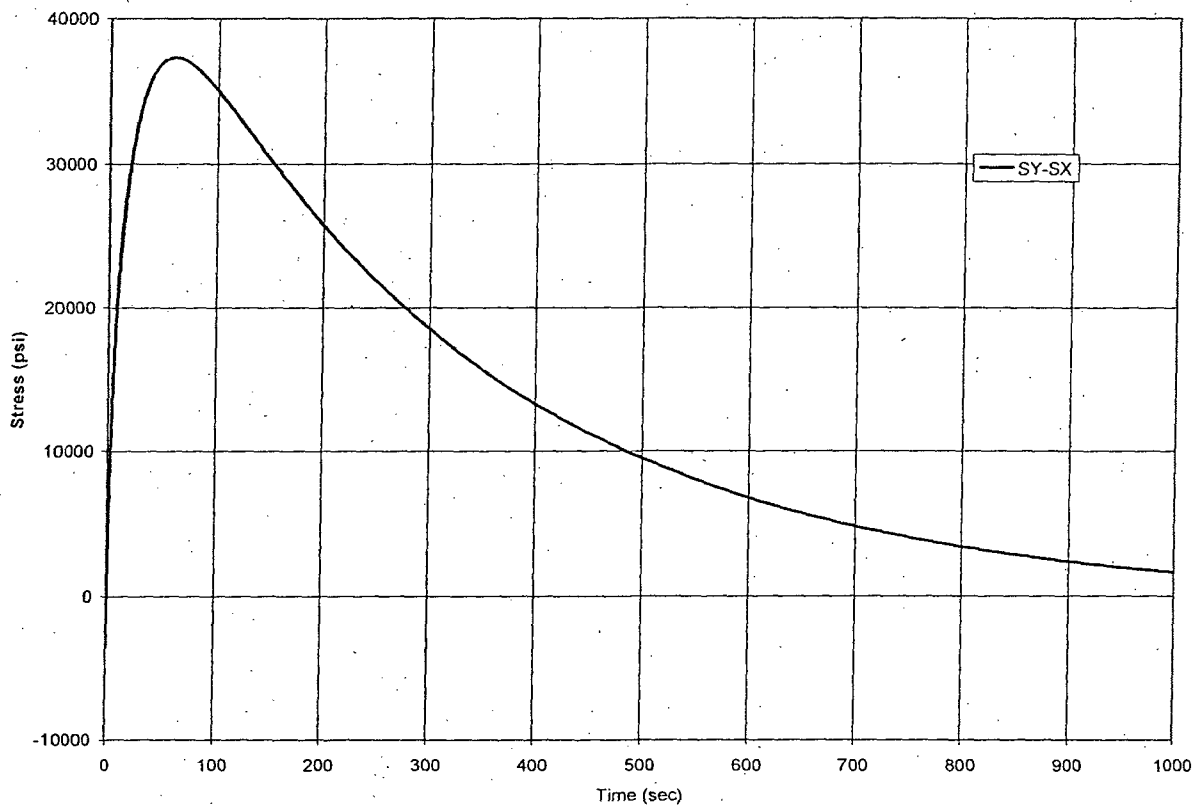


Figure 3-7: Safe End Total Stress Intensity Green's Function for 0% Flow

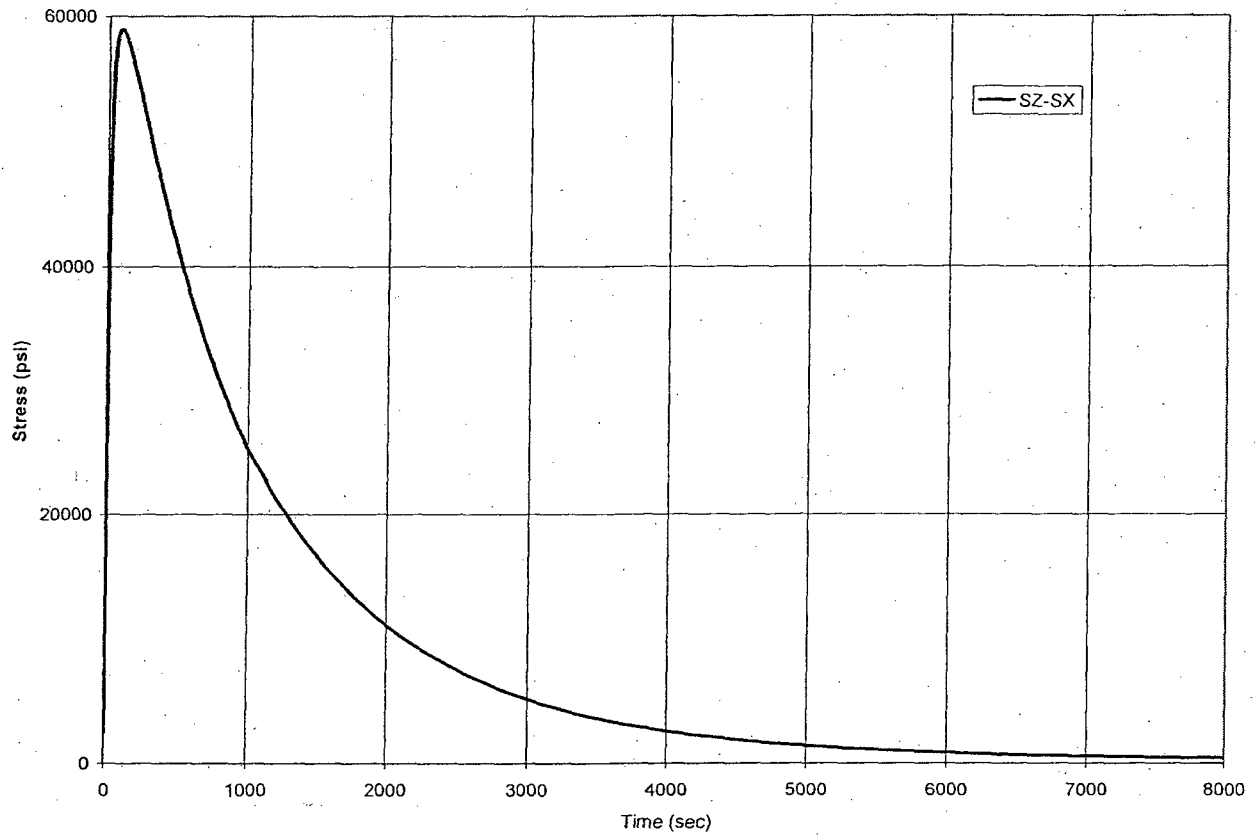


Figure 3-8: Blend Radius Total Stress Intensity Green's Function for 100% Flow

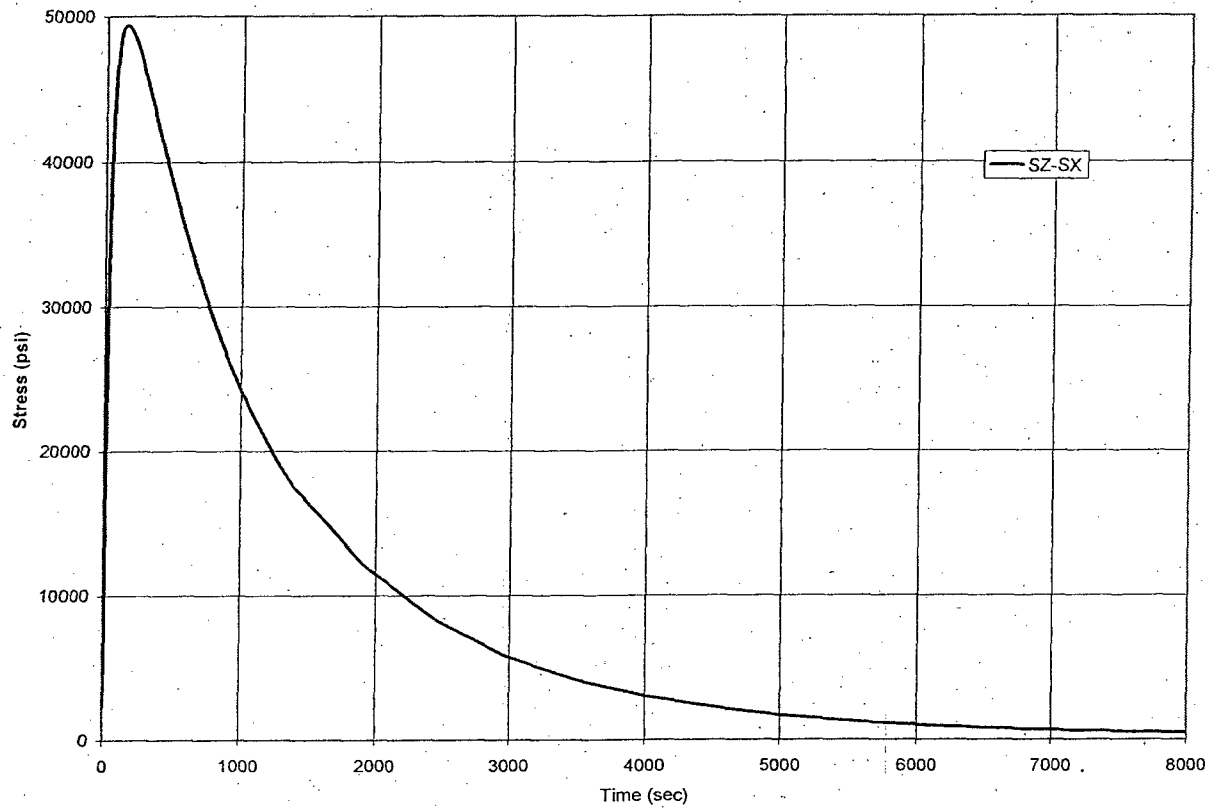


Figure 3-9: Blend Radius Total Stress Intensity Green's Function for 50% Flow

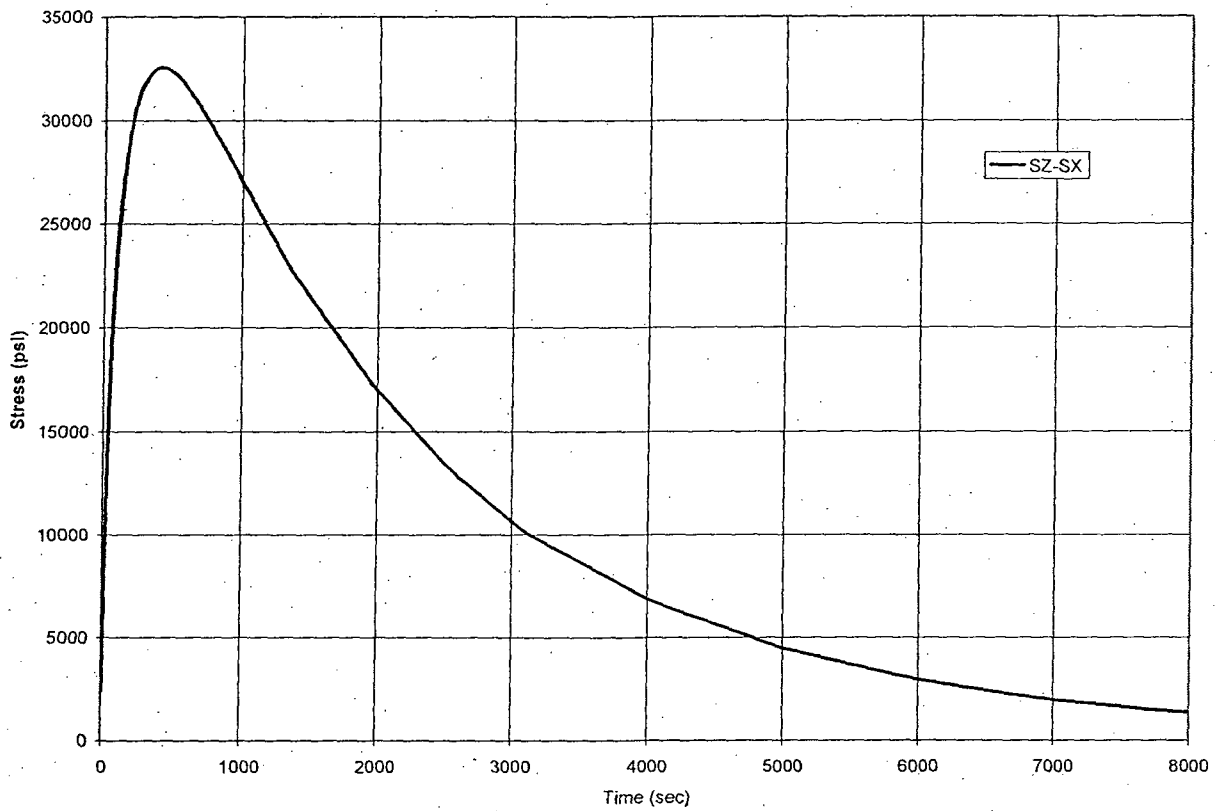


Figure 3-10: Blend Radius Total Stress Intensity Green's Function for 0% Flow

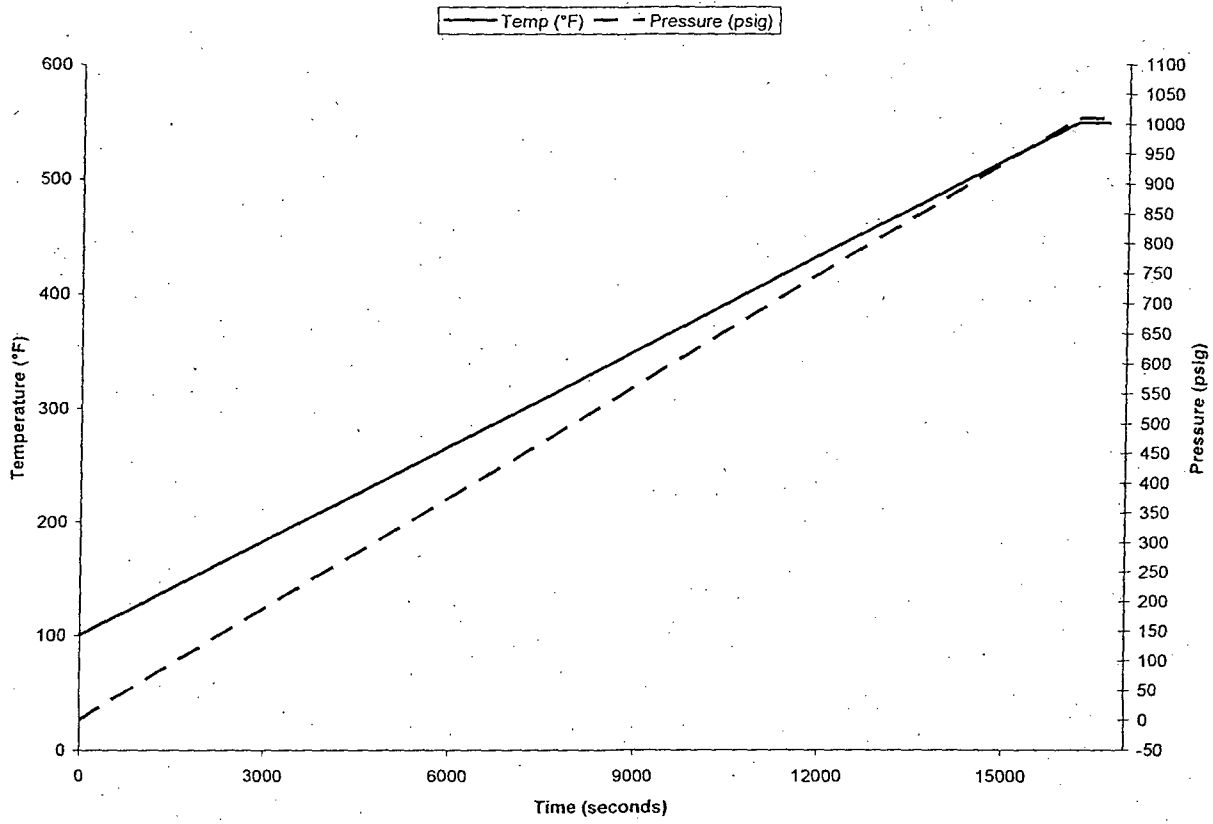


Figure 3-11: Transient 1: Normal Startup at 100°F/hr

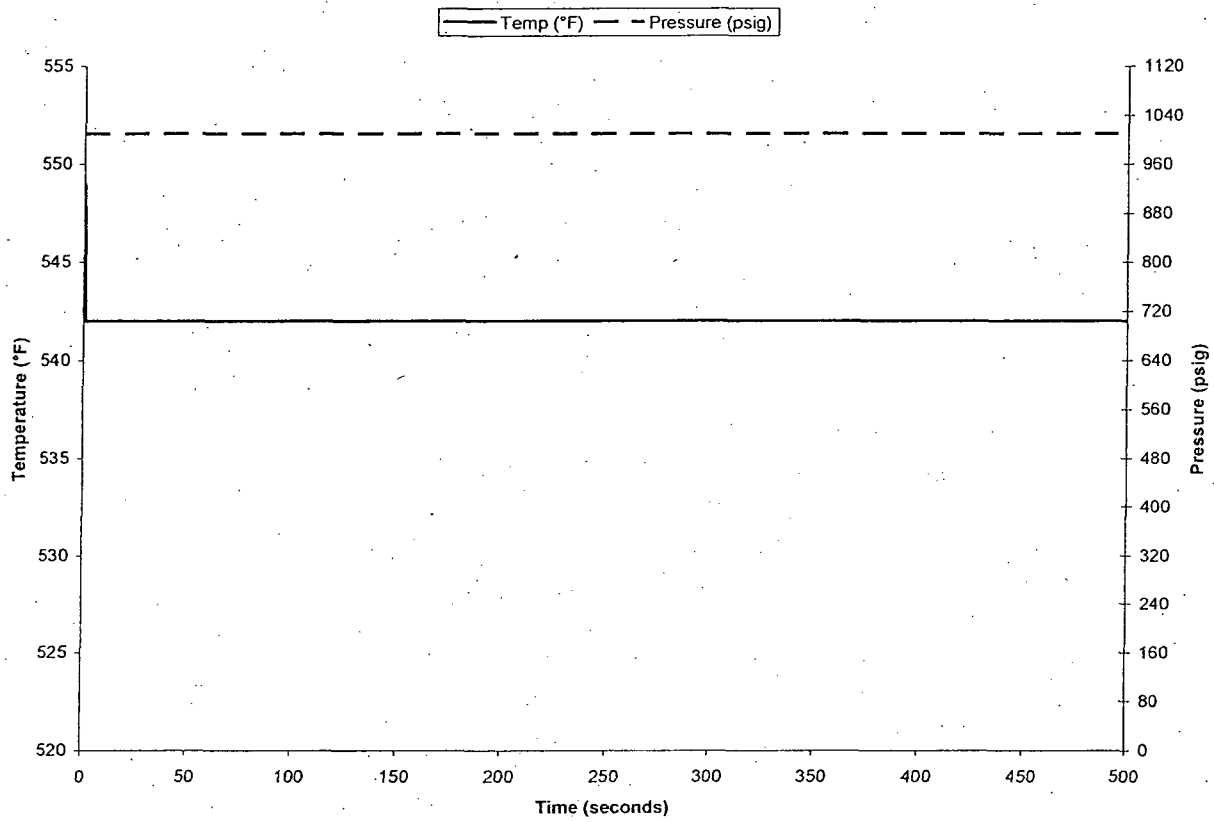


Figure 3-12: Transient 2: Turbine Roll and Increase to Rated Power

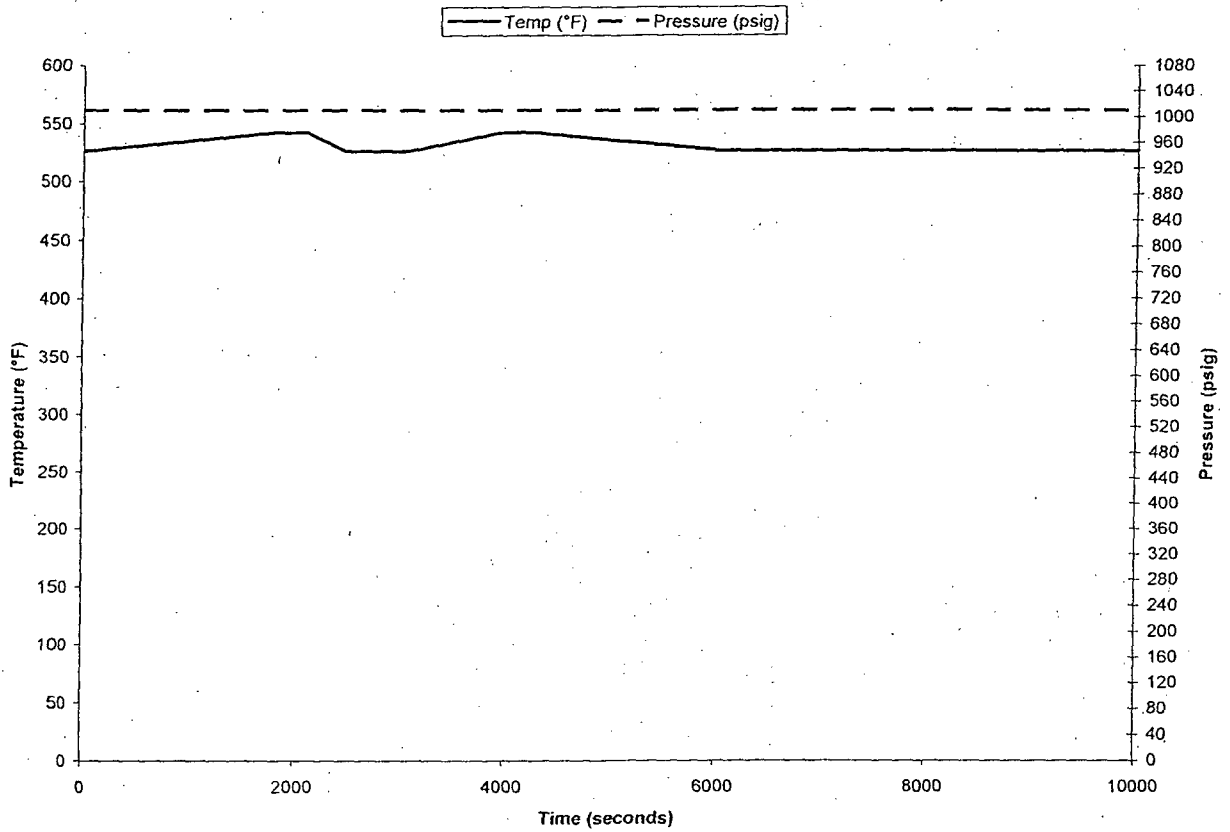


Figure 3-13: Transient 3: Loss of Feedwater Heaters and Turbine Trip at 25% Power

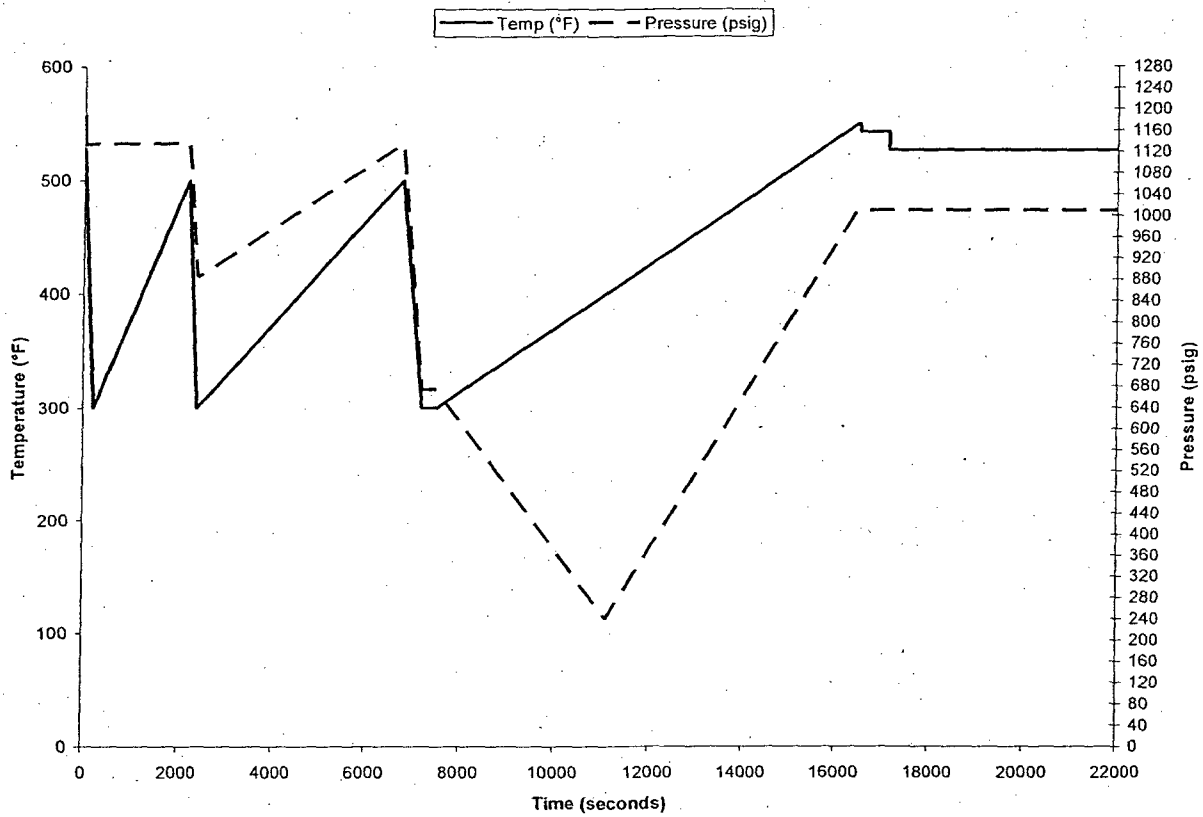


Figure 3-14: Transient 4: Loss of Feedwater Pumps

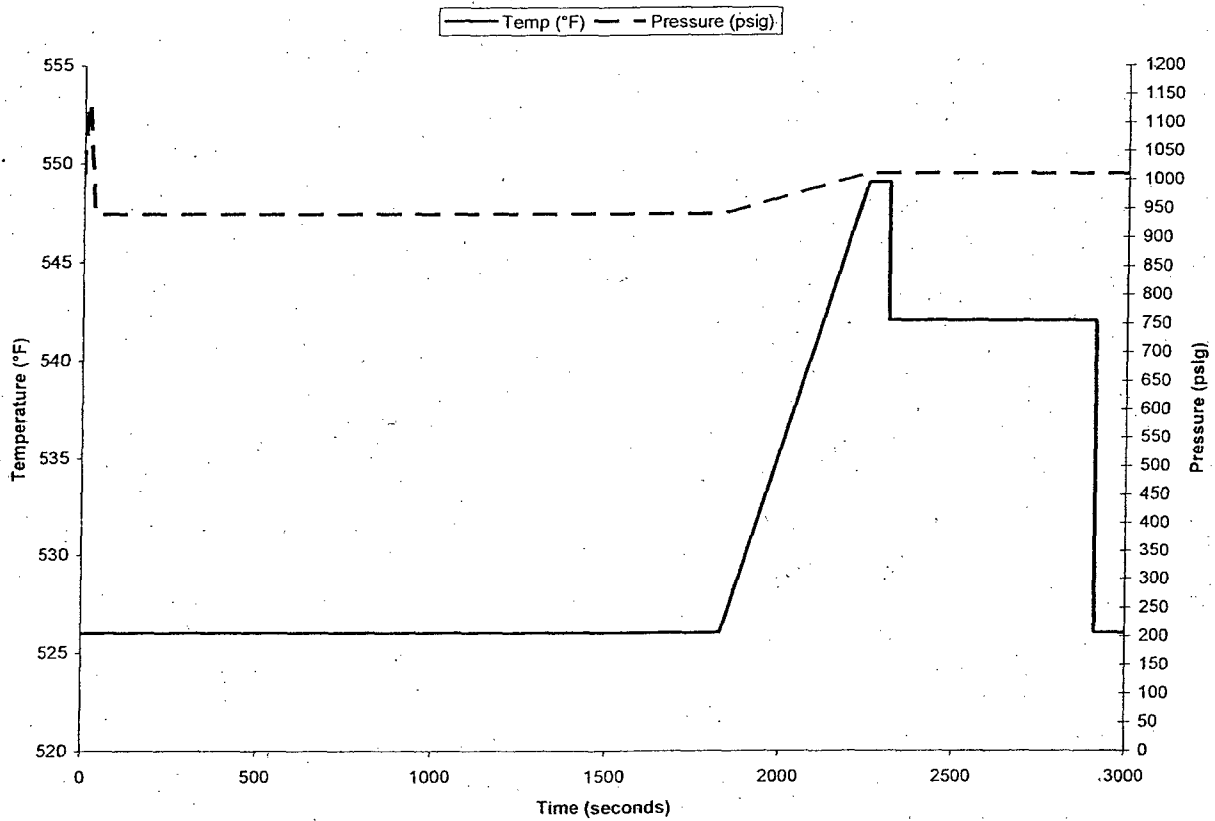


Figure 3-15: Transient 5: Turbine Generator Trip

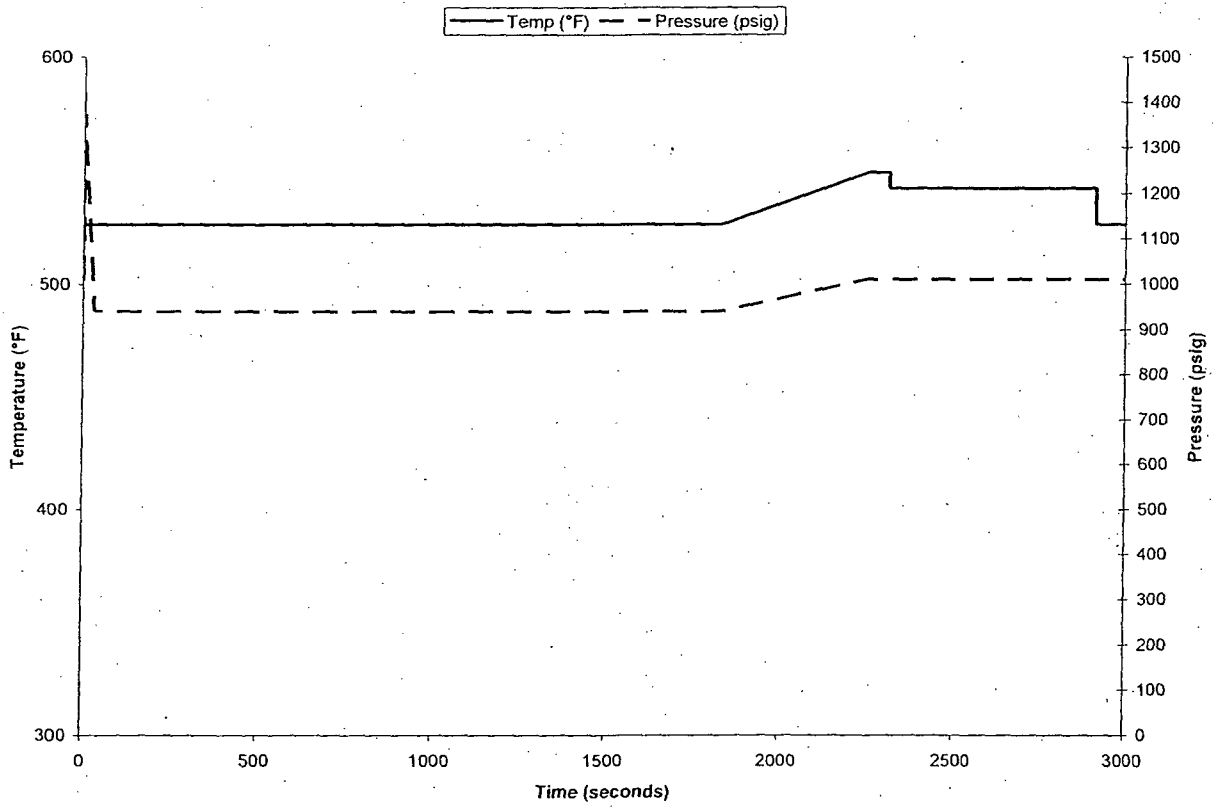


Figure 3-16: Transient 6: Reactor Overpressure

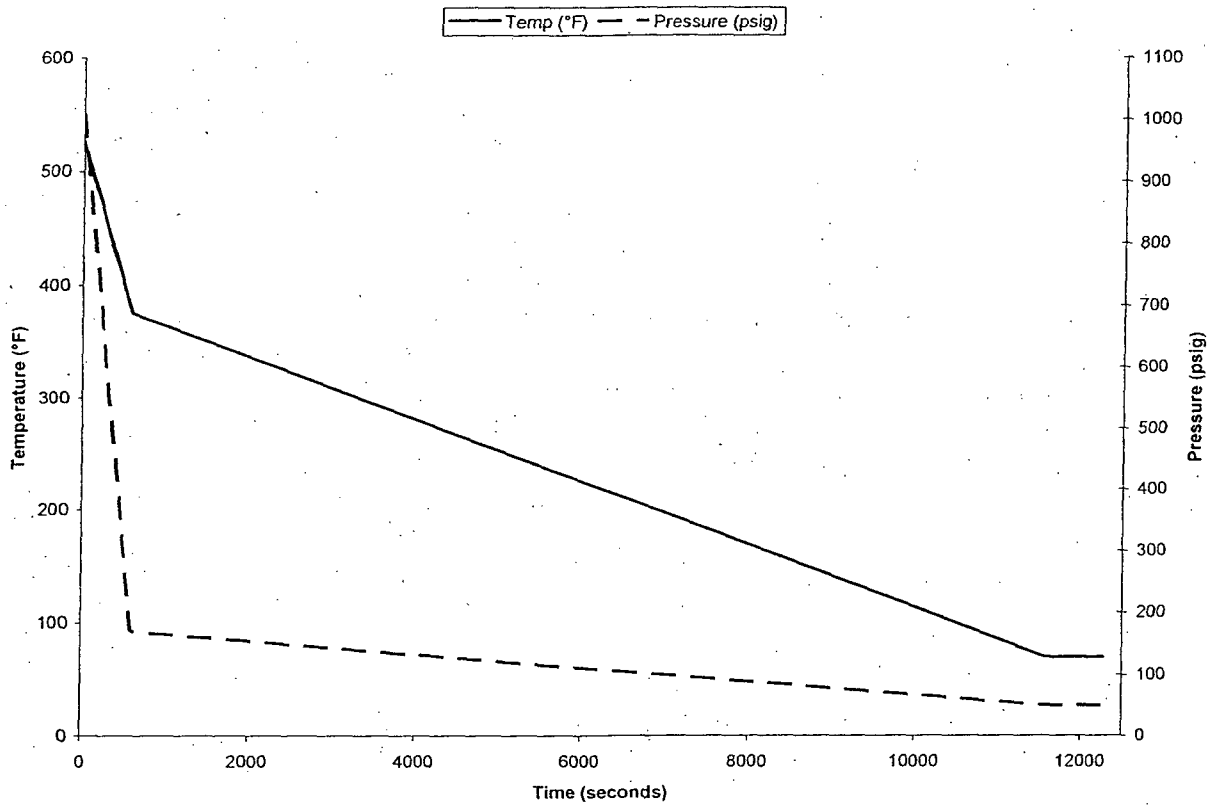


Figure 3-17: Transient 7: SRV Blowdown

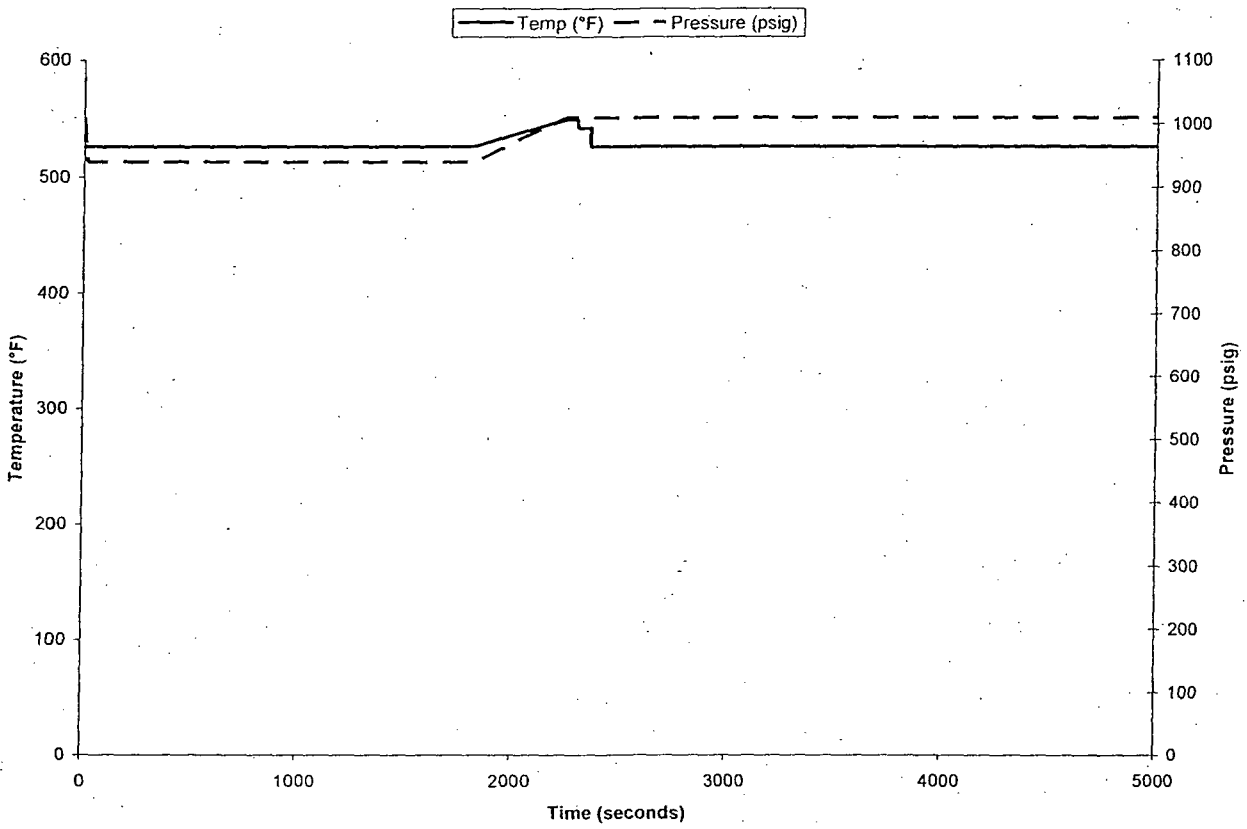


Figure 3-18: Transient 8: Scram - Other

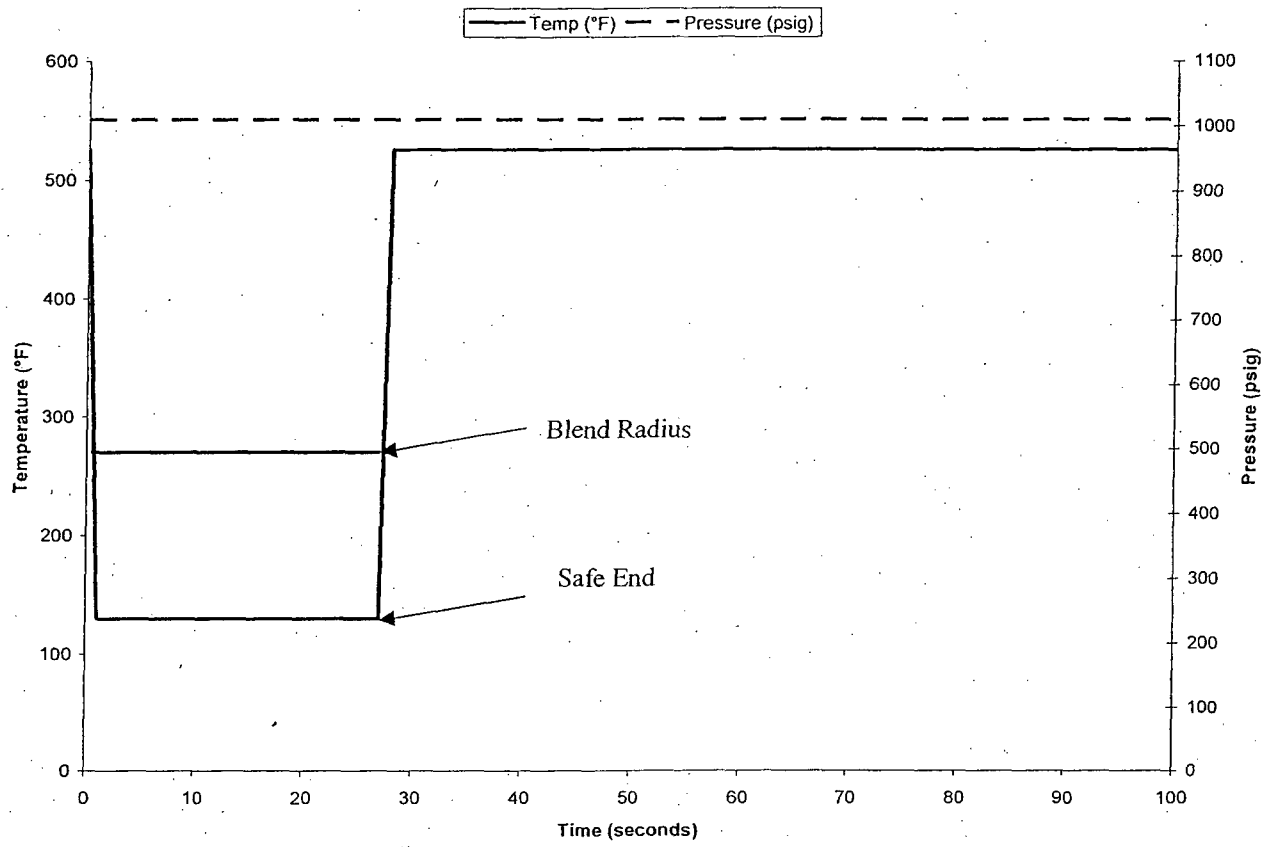


Figure 3-19: Transient 9: Improper Startup

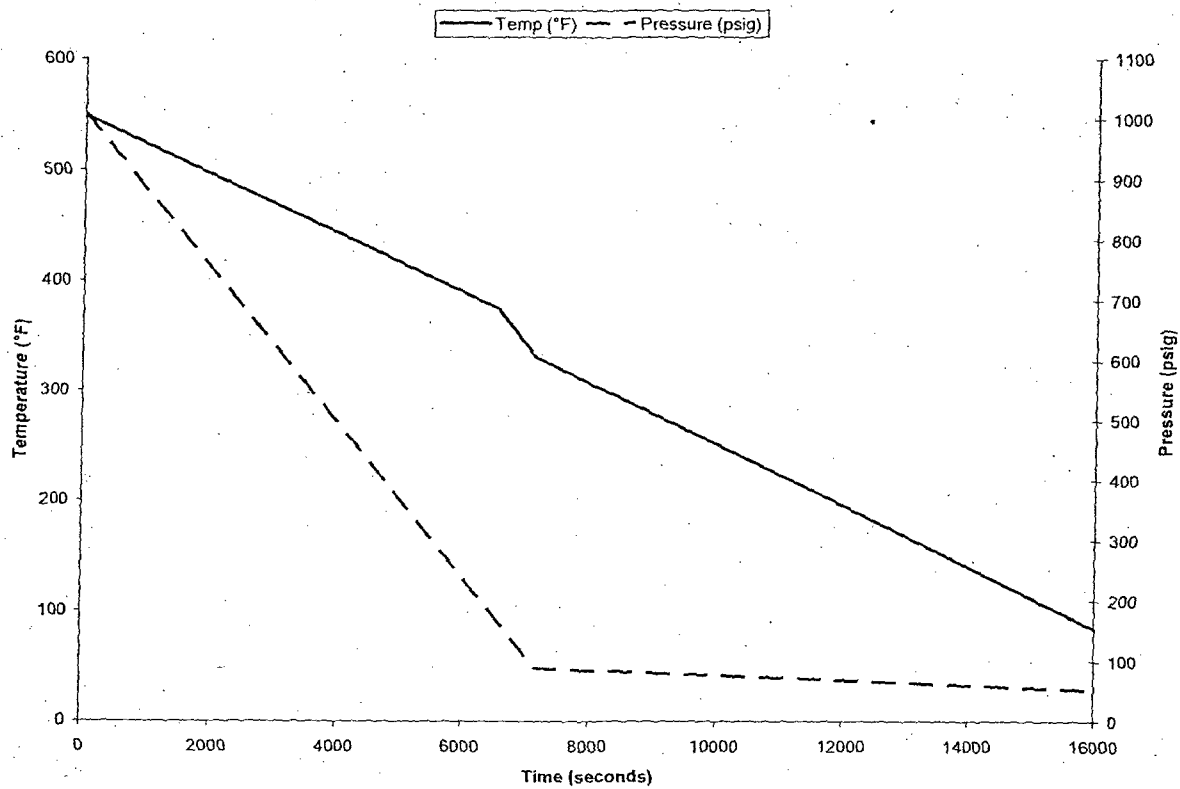


Figure 3-20: Transient 10: Shutdown

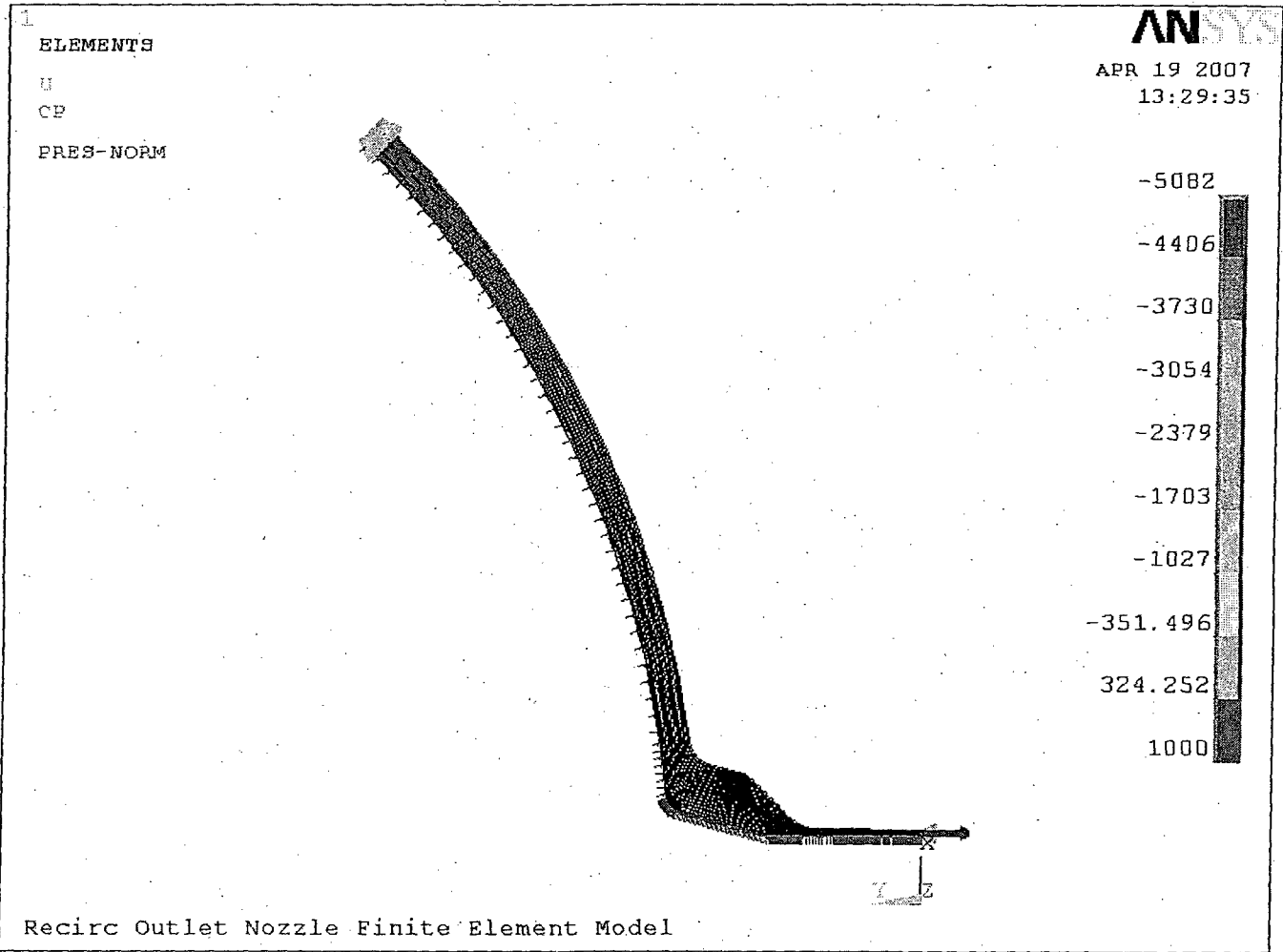


Figure 3-21: Reactor Recirculation Outlet Nozzle Internal Pressure Distribution

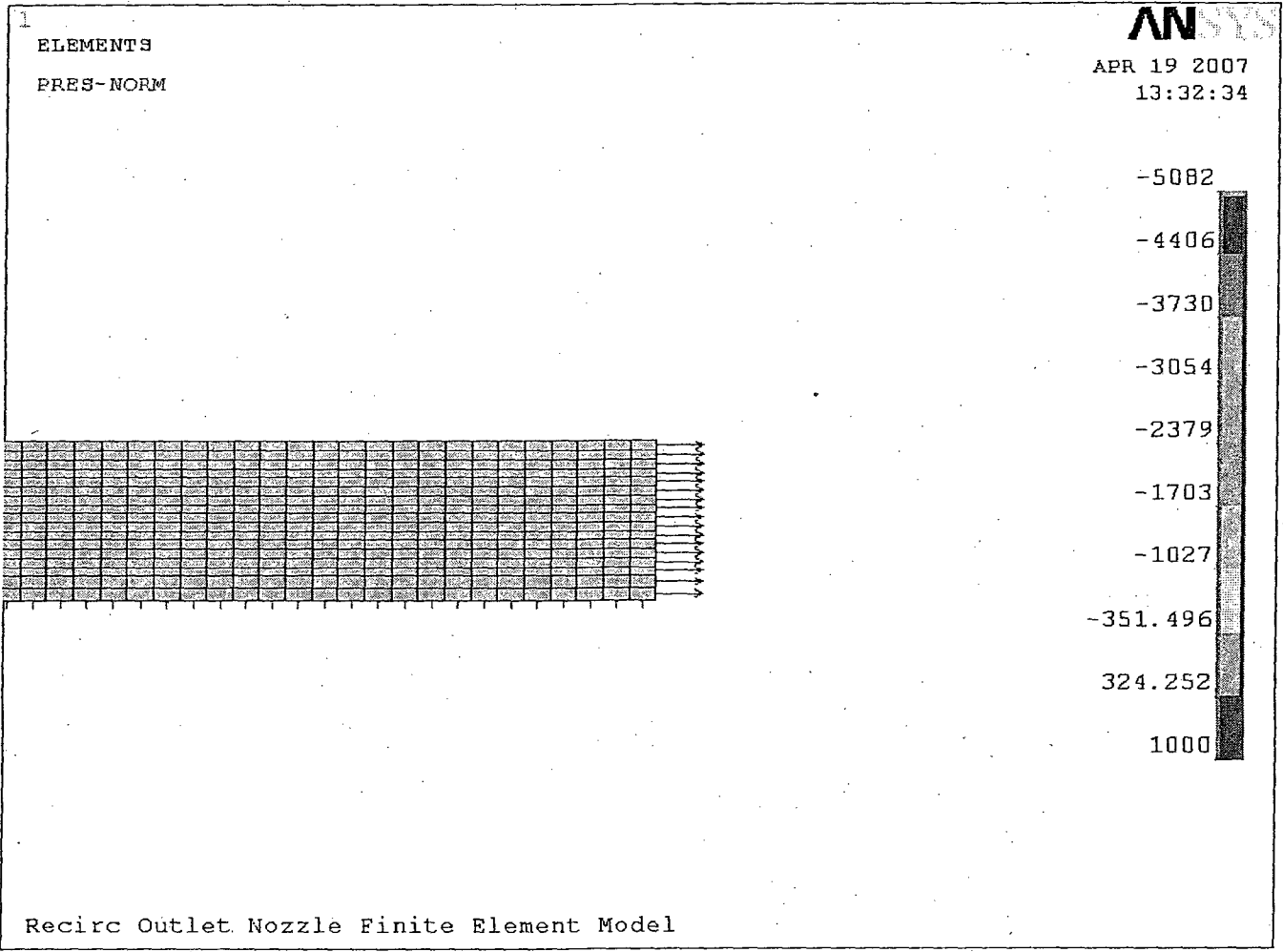


Figure 3-22: Reactor Recirculation Outlet Nozzle Pressure Cap Load

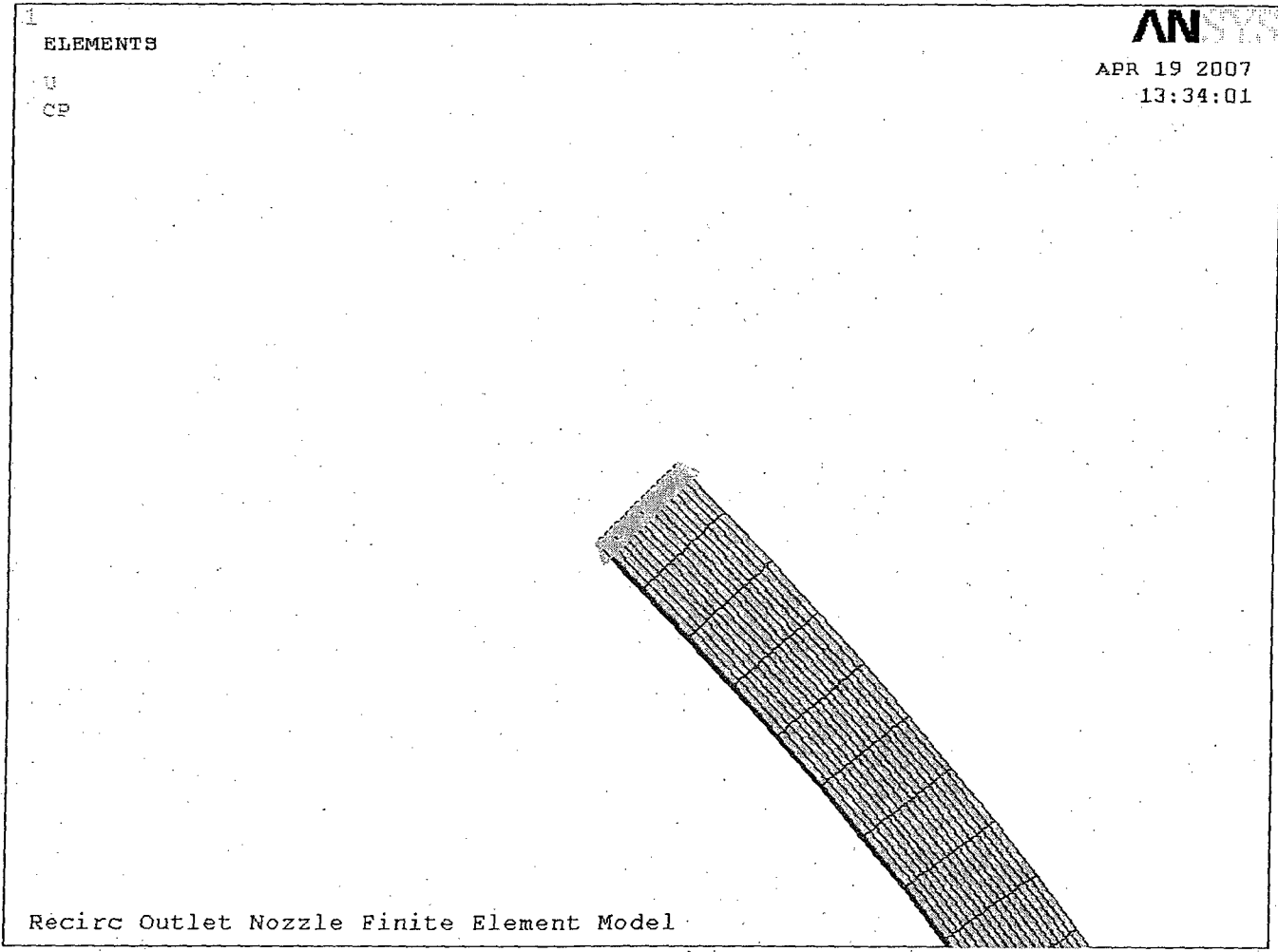


Figure 3-23: Reactor Recirculation Outlet Nozzle Vessel Boundary Condition

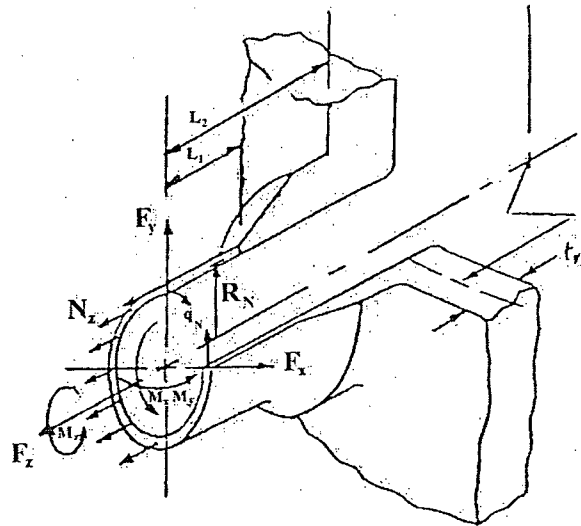


Figure 3-24: Pipe Reactions

4.0 STRESS AND FATIGUE ANALYSIS RESULTS

Fatigue calculations for the VY reactor recirculation outlet nozzle were performed in accordance with ASME Code, Section III, Subsection NB-3200 methodology (1998 Edition, 2000 Addenda) [17]. Fatigue analysis was performed in the Reference [11] calculation for the two locations identified in Section 3.1.2 using the Green's Functions developed for these two locations and the 60-year projected cycle counts from Reference [14].

Three computer programs were used to facilitate the fatigue analysis process: STRESS.EXE, P-V.EXE, and FATIGUE.EXE. The first program, STRESS.EXE, calculates a stress history in response to a thermal transient using a Green's Function. The second program, P-V.EXE, reduces the stress history to peaks and valleys. The third program, FATIGUE.EXE, calculates fatigue from the reduced peak and valley history using ASME Code, Section III methodology. All three programs are explained in detail and were independently verified for use in the Reference [16] calculation.

In order to perform the fatigue analysis, input files with the necessary data were prepared and the three analysis programs were run. The program STRESS.EXE required the following three input files:

- **Green.dat:** This file contains the Green's Function. As discussed above, the reactor recirculation outlet nozzle analyses utilize twelve Green's Functions: a membrane plus bending stress intensity Green's Function and a total stress intensity Green's Function for both the safe end and blend radius locations for each of three flow conditions.
- **Green.cfg:** A configuration file containing parameters that describe the Green's Function.
- **Transnt.inp:** This file contains the input transient history defined in Tables 3-1 and 3-2.

Tables 4-1 and 4-2 show the stresses for each location that were used in the fatigue analysis. Columns 2 through 5 of Table 4-1 (for the safe end) and Table 4-2 (for the blend radius) show the final peak and valley output after stress history reduction. The pressure values for Columns 6 through 8 in each table were determined from the transient pressures specified in Tables 3-1 and 3-2. The pressure stress intensities from Section 3.2 were scaled appropriately for each transient case. The scaled piping stress values are shown in Columns 9 and 10 of Tables 4-1 and 4-2. The piping stress intensities from Section 3.3 were scaled based on the transient case RPV fluid temperature and assuming no stress occurs at an ambient temperature of 70°F. Both of these stress intensities were then added to the thermal stress intensity peak and valley points to calculate the final stress values used for the fatigue analysis. In the case of the piping load stress intensities, the sign of the stress intensity was conservatively set to the same sign as the thermal stress intensity to ensure bounding fatigue usage results. Columns 11 and 12 of Tables 4-1 and 4-2 show the summation of all stresses for each thermal peak and valley stress point. The last column shows the number of cycles associated with each peak or valley based on the cycle counts shown in Tables 3-1 and 3-2.

The program FATIGUE.EXE performs the ASME Code peak event-pairing required to calculate a fatigue usage value. The input data for the configuration input file for FATIGUE.EXE, which is named FATIGUE.CFG, is shown in Table 4-3.

The results of the fatigue analysis are presented in Tables 4-4 and 4-5 for the safe end and the blend radius for 60 years, respectively.

Table 4-1: Reactor Recirculation Outlet Nozzle Safe End Stress Summary

1	2	3	4	5	6	7	8	9	10	11	12	13
Transient Number	Time (s)	Total Stress (psi)	M+B Stress (psi)	Temperature F	Pressure (psig)	Total Pressure Stress (psi)	M+B Pressure Stress (psi)	Total Piping Stress (psi)	M+B Piping Stress (psi)	Total Total Stress (psi)	Total M+B Stress (psi)	Number of Cycles (60 years)
1	0	-925	-949	100.00	0	0	0	-339.1419	-339.1419	-1264.14	-1288.14	300
	16164	-4814	-4433	549.00	1010	11604.9	11463.5	-5414.966	-5414.966	1375.93	1615.53	300
	16864	-3749	-3705	549.00	1010	11604.9	11463.5	-5414.966	-5414.966	2440.93	2343.53	300
2	0	-3838	-3665	549	1010	11604.9	11463.5	-5414.966	-5414.966	2351.93	2383.53	300
	6	-1664	-2263	542	1010	11604.9	11463.5	-5335.833	-5335.833	4605.07	3864.67	300
	601	-3773	-3607	542	1010	11604.9	11463.5	-5335.833	-5335.833	2496.07	2520.67	300
	606.6	1196	-403	526	1010	11604.9	11463.5	5154.958	-5154.958	17955.86	5905.54	300
	1302	-3670	-3509	526	1010	11604.9	11463.5	-5154.958	-5154.958	2779.94	2799.54	300
3	0	-3688	-3522	526	1010	11604.9	11463.5	-5154.958	-5154.958	2761.94	2786.54	10
	1800.1	-4165	-3904	542	1010	11604.9	11463.5	-5335.833	-5335.833	2104.07	2223.67	10
	2460.2	-1932	-2200	526	1010	11604.9	11463.5	-5154.958	-5154.958	4517.94	4108.54	10
	3960.2	-4537	-4185	542	1010	11604.9	11463.5	-5335.833	-5335.833	1732.07	1942.67	10
	6060.2	-3315	-3241	526	1010	11604.9	11463.5	-5154.958	-5154.958	3134.94	3067.54	10
	6760	-3687	-3522	526	1010	11604.9	11463.5	-5154.958	-5154.958	2762.94	2786.54	10
4	0.00	-3756	-3716	526	1020	11719.8	11577	-5154.958	-5154.958	2808.84	2706.04	10
	3.00	-3756	-3716	526	1190	13673.1	13506.5	-5154.958	-5154.958	4762.14	4635.54	10
	13.00	-3756	-3716	526	1135	13041.15	12882.25	-5154.958	-5154.958	4130.19	4011.29	10
	242.30	15878	10049	302.374	1135	13041.15	12882.25	2626.926	2626.926	31546.08	25558.18	10
	2213.10	-6389	-5428	499.889	1135	13041.15	12882.25	-4859.78	-4859.78	1793.37	2594.47	10
	2408.60	13203	8265	301.443	885	10168.65	10044.75	2616.401	2616.401	25988.05	20926.15	10
	6773.40	-4763	-4312	499.809	1135	13041.15	12882.25	-4858.875	-4858.875	3419.27	3711.37	10
	7193.10	15374	9801	300	675	7755.75	7661.25	2600.088	2600.088	25729.84	20062.34	10
	16457.50	-4812	-5032	549	240	2757.6	2724	-5414.966	-5414.966	-7469.37	-7722.97	10
	16524.70	-2358	-2725	542	1010	11604.9	11463.5	-5335.833	-5335.833	3911.07	3402.67	10
	17118.00	-3778	-3610	541.996	1010	11604.9	11463.5	-5335.788	-5335.788	2491.11	2517.71	10
	17123.60	1192	-406	526	1010	11604.9	11463.5	5154.958	-5154.958	17951.86	5902.54	10
	17819.00	-3670	-3509	526	1010	11604.9	11463.5	-5154.958	-5154.958	2779.94	2799.54	10
5	0.00	-3688	-3522	526	1010	11604.9	11463.5	-5154.958	-5154.958	2761.94	2786.54	60
	10.00	-3688	-3522	526	1135	13041.15	12882.25	-5154.958	-5154.958	4198.19	4205.29	60
	30.00	-3688	-3522	526	940	10800.6	10669	-5154.958	-5154.958	1957.64	1992.04	60
	2250.10	-6054	-5337	549	1010	11604.9	11463.5	-5414.966	-5414.966	135.93	711.53	60
	2319.90	-2977	-3123	542	1010	11604.9	11463.5	-5335.833	-5335.833	3292.07	3004.67	60
	2911.00	-3782	-3613	541.999	1010	11604.9	11463.5	-5335.822	-5335.822	2487.08	2514.68	60
	2916.70	1188	-408	526	1010	11604.9	11463.5	5154.958	-5154.958	17947.86	5900.54	60
	3612.00	-3670	-3509	526	1010	11604.9	11463.5	-5154.958	-5154.958	2779.94	2799.54	60
6	0.00	-3688	-3522	5.26E+02	1010	11604.9	11463.5	-5154.958	-5154.958	2761.94	2786.54	1
	2.00	-3688	-3522	5.26E+02	1375	15798.75	15606.25	-5154.958	-5154.958	6955.79	6929.29	1
	32.00	-3688	-3522	5.26E+02	940	10800.6	10669	-5154.958	-5154.958	1957.64	1992.04	1
	2252.10	-6054	-5337	5.49E+02	1010	11604.9	11463.5	-5414.966	-5414.966	135.93	711.53	1
	2322.20	-2977	-3123	5.42E+02	1010	11604.9	11463.5	-5335.833	-5335.833	3292.07	3004.67	1
	2913.00	-3782	-3613	5.42E+02	1010	11604.9	11463.5	-5335.822	-5335.822	2487.08	2514.68	1
	2918.70	1188	-408	5.26E+02	1010	11604.9	11463.5	5154.958	-5154.958	17947.86	5900.54	1
	3614.00	-3670	-3509	5.26E+02	1010	11604.9	11463.5	-5154.958	-5154.958	2779.94	2799.54	1
7	0	-3688	-3522	526	1010	11604.9	11463.5	-5154.958	-5154.958	2761.94	2786.54	1
	600	7773	5336	375	170	1953.3	1929.5	3447.943	3447.943	13174.24	10713.44	1
	1367.9	-1390	-1567	354.172	162	1861.38	1838.7	-3212.488	-3212.488	-2741.11	-2940.79	1
	11580.1	454	190	70	50	574.5	567.5	0	0	1028.50	757.50	1
	12280	-707	-689	70	50	574.5	567.5	0	0	-132.50	-121.50	1
8	0.00	-3688	-3522	526	1010	11604.9	11463.5	-5154.958	-5154.958	2761.94	2786.54	228
	15.00	-3688	-3522	526	940	10800.6	10669	-5154.958	-5154.958	1957.64	1992.04	228
	2235.10	-6054	-5337	549	1010	11604.9	11463.5	-5414.966	-5414.966	135.93	711.53	228
	2305.20	-2977	-3123	542	1010	11604.9	11463.5	-5335.833	-5335.833	3292.07	3004.67	228
	2356.00	-3183	-3151	541.999	1010	11604.9	11463.5	-5335.822	-5335.822	3086.08	2976.68	228
	2361.50	1761	-28	526	1010	11604.9	11463.5	5154.958	-5154.958	18520.86	6280.54	228
	3057.00	-3667	-3506	526	1010	11604.9	11463.5	-5154.958	-5154.958	2782.94	2802.54	228
9	0	-2968	-2837	525.7	1010	11604.9	11463.5	-5151.566	-5151.566	3485.22	3474.82	1
	27	68473	45303	291.3	1010	11604.9	11463.5	2501.737	2501.737	82579.74	59268.34	1
	80.7	-11546	-8877	518.4	1010	11604.9	11463.5	-5069.042	-5069.042	-5010.04	-2482.14	1
	5200	-2967	-2832	525.7	1010	11604.9	11463.5	-5151.566	-5151.566	3486.21	3479.78	1
10	0	-3745	-3709	549	1010	11604.9	11463.5	-5414.966	-5414.966	2444.93	2339.53	300
	6864.2	501	-405	329.994	170	1953.3	1929.5	2939.162	-2939.162	5393.46	-1414.66	300
	7455.5	-1183	-1528	314.325	88	1011.12	998.8	-2762.029	-2762.029	-2933.91	-3291.23	300
	16224.1	334	-35	70	50	574.5	567.5	0	0	908.50	532.50	300
	16924	-731	-763	70	50	574.5	567.5	0	0	-156.50	-195.50	300
11	0	0	0	100	0	0	0	339.1419	339.1419	339.14	339.14	120
	0	0	0	100	1100	12639	12485	339.1419	339.1419	12978.14	12824.14	120
	0	0	0	100	50	574.5	567.5	339.1419	339.1419	913.64	906.64	120
12	0	0	0	100	50	574.5	567.5	339.1419	339.1419	913.64	906.64	1
	0	0	0	100	1563	17958.87	17740.05	339.1419	339.1419	18298.01	18079.19	1
	0	0	0	100	50	574.5	567.5	339.1419	339.1419	913.64	906.64	1

For notes, see next page.

Table 4-1: Reactor Recirculation Outlet Nozzle Safe End Stress Summary (concluded)

- NOTES:
- Column 1: Transient number identification.
 - Column 2: Time during transient where maximum or minimum stress intensity occurs from P-V.OUT output file.
 - Column 3: Maximum or minimum total stress intensity from P-V.OUT output file.
 - Column 4: Maximum or minimum membrane plus bending stress intensity from P-V.OUT output file.
 - Column 5: Temperature per total stress intensity.
 - Column 6: Pressure per Table 3-1.
 - Column 7: Total pressure stress intensity from the quantity (Column 6 x 11490)/1000.
 - Column 8: Membrane plus bending pressure stress intensity from the quantity (Column 6 x 11350)/1000.
 - Column 9: Total external stress, $5707.89 \text{ psi} * (\text{Column 5} - 70^\circ\text{F}) / (549^\circ\text{F} - 70^\circ\text{F})$.
 - Column 10: Same as Column 9, but for M+B stress.
 - Column 11: Sum of total stresses (Columns 3, 7, and 9).
 - Column 12: Sum of membrane plus bending stresses (Columns 4, 8, and 10).
 - Column 13: Number of cycles for the transient (60 years).

Table 4-2: Reactor Recirculation Outlet Nozzle Blend Radius Stress Summary

1	2	3	4	5	6	7	8	9	10	11	12	13
Transient Number	Time (s)	Total Stress (psi)	M+B Stress (psi)	Temperature F	Pressure (psig)	Total Pressure Stress (psi)	M+B Pressure Stress (psi)	Total Piping Stress (psi)	M+B Piping Stress (psi)	Total Total Stress (psi)	Total M+B Stress (psi)	Number of Cycles (60 years)
1	0	459	388	100.00	0	0	0	16.64312	16.64312	475.64	404.64	300
	4303	-3417	-1594	219.53	1010	31613	33976.4	-82.95209	-82.95209	28113.05	32299.45	300
	22164	2713	2306	549.00	1010	31613	33976.4	265.7352	265.7352	34591.74	36548.14	300
2	0.00	3094	1934	549	1010	31613	33976.4	265.7352	265.7352	34972.74	36176.14	300
	94.30	4079	2481	542	1010	31613	33976.4	261.8518	261.8518	35953.85	36719.25	300
	601.70	3683	2435	538.8	1010	31613	33976.4	260.0765	260.0765	35556.08	36671.48	300
3	680.10	5891	3489	526	1010	31613	33976.4	252.9754	252.9754	37756.98	37718.38	300
	6602.00	2977	1859	526	1010	31613	33976.4	252.9754	252.9754	34842.98	36088.38	300
	0.00	2959	1849	526	1010	31613	33976.4	252.9754	252.9754	34824.98	36078.38	10
4	1807.20	1834	1043	542	1010	31613	33976.4	261.8518	261.8518	33708.85	35281.25	10
	2491.50	4425	2667	526	1010	31613	33976.4	252.9754	252.9754	36290.98	36896.38	10
	3974.40	1706	1060	542	1010	31613	33976.4	261.8518	261.8518	33580.85	35298.25	10
5	6070.80	3971	2551	526	1010	31613	33976.4	252.9754	252.9754	35836.98	36780.38	10
	12060.00	2965	1852	526	1010	31613	33976.4	252.9754	252.9754	34830.98	36081.38	10
	0	2465	-703	526.00	1010	31613	33976.4	252.9754	-252.9754	34330.98	33020.42	10
6	3	2465	-703	526.00	1190	37247	40031.6	252.9754	-252.9754	39964.98	39075.62	10
	13	2465	-703	526.00	1135	35525.5	38181.4	252.9754	-252.9754	38243.48	37225.42	10
	435.6	18138	9690	356.38	1135	35525.5	38181.4	158.8774	158.8774	53822.38	48030.28	10
7	2222.5	-1169	-2598	489.44	1135	35525.5	38181.4	-232.6952	-232.6952	34123.80	35350.70	10
	2665.5	12763	6695	328.40	885	27700.5	29771.4	143.3539	143.3539	40606.85	36609.75	10
	6779.2	-4008	-2829	497.05	1010	31613	33976.4	-236.9137	-236.9137	27368.09	30910.49	10
8	7243.8	19275	9965	302.91	1010	31613	33976.4	129.2122	129.2122	51017.21	44070.61	10
	13996	-2135	34	542.00	1010	31613	33976.4	-261.8518	261.8518	29216.15	34272.25	10
	17247	3413	2074	526.00	1010	31613	33976.4	252.9754	252.9754	35278.98	36303.38	10
9	23119	2971	1855	526.00	1010	31613	33976.4	252.9754	252.9754	34836.98	36084.38	10
	0.00	2959	1849	526	1010	31613	33976.4	252.9754	252.9754	34824.98	36078.38	60
	10.00	2959	1849	526	1135	35525.5	38181.4	252.9754	252.9754	38737.48	40283.38	60
10	15.00	2959	1849	526	940	29422	31621.6	252.9754	252.9754	32633.98	33723.58	60
	2269.50	111	295	549	1010	31613	33976.4	265.7352	265.7352	31989.74	34537.14	60
	3010.10	4407	2579	526	1010	31613	33976.4	252.9754	252.9754	36272.98	36808.38	60
11	8912.00	2968	1854	526	1010	31613	33976.4	252.9754	252.9754	34833.98	36083.38	60
	0.00	2959	1849	526.00	1010	31613	33976.4	252.9754	252.9754	34824.98	36078.38	1
	2.00	2959	1849	526.00	1375	43037.5	46255	252.9754	252.9754	46249.48	48356.98	1
12	32.00	2959	1849	526.00	940	29422	31621.6	252.9754	252.9754	32633.98	33723.58	1
	2271.50	111	295	549.00	1010	31613	33976.4	265.7352	265.7352	31989.74	34537.14	1
	3022.00	4407	2579	526.00	1010	31613	33976.4	252.9754	252.9754	36272.98	36808.38	1
13	8914.00	2968	1854	526.00	1010	31613	33976.4	252.9754	252.9754	34833.98	36083.38	1
	0.00	2959	1849	526	1010	31613	33976.4	252.9754	252.9754	34824.98	36078.38	1
	615.10	20260	12980	374.581	170	5321	5718.8	168.9726	168.9726	25749.97	18867.77	1
14	17580.00	279	179	70	50	1565	1682	0	0	1844.00	1861.00	1
	0.00	2959	1849	526	1010	31613	33976.4	252.9754	252.9754	34824.98	36078.38	228
	15.00	2959	1849	526	940	29422	31621.6	252.9754	252.9754	32633.98	33723.58	228
15	2254.50	111	295	549	1010	31613	33976.4	265.7352	265.7352	31989.74	34537.14	228
	2491.20	3792	2234	526	1010	31613	33976.4	252.9754	252.9754	35657.98	36463.38	228
	8357.00	2963	1851	526	1010	31613	33976.4	252.9754	252.9754	34828.98	36080.38	228
16	0	2058	961	525.8	1010	31613	33976.4	252.8645	252.8645	33923.86	35190.26	1
	0.52	1956	734	525.6	1010	31613	33976.4	252.7535	252.7535	33821.75	34963.15	1
	28	23747	3188	504.5	1010	31613	33976.4	241.0479	241.0479	55601.05	37405.45	1
17	425	1520	611	525.5	1010	31613	33976.4	252.698	252.698	33385.70	34840.10	1
	12400	2058	879	525.8	1010	31613	33976.4	252.8645	252.8645	33923.86	35108.26	1
	0	2767	2176	549	1010	31613	33976.4	265.7352	265.7352	34645.74	36418.14	300
18	4240.8	6643	4158	445.775	441	13803.3	14835.24	208.469	208.469	20654.77	19201.71	300
	6268	6498	3675	374.7	170	5321	5718.8	169.0386	169.0386	11988.04	9562.84	300
	6891.8	9282	5241	329.228	88	2754.4	2960.32	143.8121	143.8121	12180.21	8345.13	300
19	22224	361	120	70	50	1565	1682	0	0	1926.00	1802.00	300
	0	0	0	100	0	0	0	16.64312	16.64312	16.64	16.64	120
	0	0	0	100	1100	34430	37004	16.64312	16.64312	34446.64	37020.64	120
20	0	0	0	100	50	1565	1682	16.64312	16.64312	1581.64	1698.64	120
	0	0	0	100	50	1565	1682	16.64312	16.64312	1581.64	1698.64	1
	0	0	0	100	1563	48921.9	52579.32	16.64312	16.64312	48938.54	52595.96	1
21	0	0	0	100	50	1565	1682	16.64312	16.64312	1581.64	1698.64	1

For notes, see next page.

Table 4-2: Reactor Recirculation Outlet Nozzle Blend Radius Stress Summary (concluded)

- NOTES:
- Column 1: Transient number identification.
 - Column 2: Time during transient where maximum or minimum stress intensity occurs from P-V.OUT output file.
 - Column 3: Maximum or minimum total stress intensity from P-V.OUT output file.
 - Column 4: Maximum or minimum membrane plus bending stress intensity from P-V.OUT output file.
 - Column 5: Temperature per total stress intensity.
 - Column 6: Pressure per Table 3-2.
 - Column 7: Total pressure stress intensity from the quantity $(\text{Column 6} \times 31300)/1000$.
 - Column 8: Membrane plus bending pressure stress intensity from the quantity $(\text{Column 6} \times 33640)/1000$.
 - Column 9: Total external stress, $280.16 \text{ psi} \cdot (\text{Column 5} - 70^\circ\text{F}) / (549^\circ\text{F} - 70^\circ\text{F})$.
 - Column 10: Same as Column 9, but for M+B stress.
 - Column 11: Sum of total stresses (Columns 3, 7, and 9).
 - Column 12: Sum of membrane plus bending stresses (Columns 4, 8, and 10).
 - Column 13: Number of cycles for the transient (60 years).

Table 4-3: Fatigue Parameters Used in the Recirculation Outlet Nozzle Fatigue Analysis

Parameter	Blend Radius	Safe End
Parameters <i>m</i> and <i>n</i> for Computing K_e	2.0 & 0.2 (low alloy steel) [17]	1.7 & 0.3 (stainless steel) [17]
Design Stress Intensity Values, S_m	26700 psi [9] @ 600°F	17000 psi [9] @ 600°F
Elastic Modulus from Applicable Fatigue Curve	30.0×10^6 psi [17]	28.3×10^6 psi [17]
Elastic Modulus Used in Finite Element Model	26.7×10^6 psi [10]	27.0×10^6 psi [10]
The Geometric Stress Concentration Factor K_t	1.0	1.53 [20]

Table 4-4: Fatigue Results for Reactor Recirculation Outlet Nozzle Safe End

LOCATION = LOCATION NO. 1 -- SAFE END
 FATIGUE CURVE = 2 (1 = CARBON/LOW ALLOY, 2 = STAINLESS STEEL)
 m = 1.7
 n = .3
 Sm = 17000. psi
 Ecurve = 2.830E+07 psi
 Eanalysis = 2.700E+07 psi
 Kt = 1.53

MAX	MIN	RANGE	MEM+BEND	Ke	Salt	Napplied	Nallowed	U
82580.	-7469.	90049.	66991.	2.045	134573.	1.000E+00	6.765E+02	.0015
31546.	-7469.	39015.	33281.	1.000	29691.	9.000E+00	6.857E+05	.0000
31546.	-5010.	36556.	28040.	1.000	26947.	1.000E+00	1.160E+06	.0000
25988.	-2934.	28922.	24217.	1.000	21884.	1.000E+01	2.383E+06	.0000
25730.	-2934.	28664.	23354.	1.000	21509.	1.000E+01	2.566E+06	.0000
18521.	-2934.	21455.	9572.	1.000	13903.	2.280E+02	9.710E+08	.0000
18298.	-2934.	21232.	21370.	1.000	17063.	1.000E+00	7.876E+06	.0000
17956.	-2934.	20890.	9197.	1.000	13502.	5.100E+01	1.000E+20	.0000
17956.	-2741.	20697.	8846.	1.000	13304.	1.000E+00	1.000E+20	.0000
17956.	-1264.	19220.	7194.	1.000	12071.	2.480E+02	1.000E+20	.0000
17952.	-1264.	19216.	7191.	1.000	12068.	1.000E+01	1.000E+20	.0000
17948.	-1264.	19212.	7189.	1.000	12065.	4.200E+01	1.000E+20	.0000
17948.	-157.	18104.	6096.	1.000	11181.	1.800E+01	1.000E+20	.0000
17948.	-157.	18104.	6096.	1.000	11181.	1.000E+00	1.000E+20	.0000
13174.	-157.	13331.	10909.	1.000	10016.	1.000E+00	1.000E+20	.0000
12978.	-157.	13135.	13020.	1.000	10500.	1.200E+02	1.000E+20	.0000
6956.	-157.	7112.	7125.	1.000	5706.	1.000E+00	1.000E+20	.0000
5393.	-157.	5550.	-1219.	1.000	2570.	1.590E+02	1.000E+20	.0000
5393.	-133.	5526.	-1293.	1.000	2537.	1.000E+00	1.000E+20	.0000
5393.	136.	5258.	-2126.	1.000	2165.	6.000E+01	1.000E+20	.0000
5393.	136.	5258.	-2126.	1.000	2165.	1.000E+00	1.000E+20	.0000
5393.	136.	5258.	-2126.	1.000	2165.	7.900E+01	1.000E+20	.0000
4762.	136.	4626.	3924.	1.000	3514.	1.000E+01	1.000E+20	.0000
4605.	136.	4469.	3153.	1.000	3218.	1.390E+02	1.000E+20	.0000
4605.	339.	4266.	3526.	1.000	3215.	1.200E+02	1.000E+20	.0000
4605.	909.	3697.	3332.	1.000	2863.	4.100E+01	1.000E+20	.0000
4518.	909.	3609.	3576.	1.000	2885.	1.000E+01	1.000E+20	.0000
4198.	909.	3290.	3673.	1.000	2744.	6.000E+01	1.000E+20	.0000
4130.	909.	3222.	3479.	1.000	2655.	1.000E+01	1.000E+20	.0000
3911.	909.	3003.	2870.	1.000	2371.	1.000E+01	1.000E+20	.0000
3486.	909.	2578.	2947.	1.000	2170.	1.000E+00	1.000E+20	.0000
3485.	909.	2577.	2942.	1.000	2168.	1.000E+00	1.000E+20	.0000
3419.	909.	2511.	3179.	1.000	2199.	1.000E+01	1.000E+20	.0000
3292.	909.	2384.	2472.	1.000	1936.	6.000E+01	1.000E+20	.0000
3292.	909.	2384.	2472.	1.000	1936.	1.000E+00	1.000E+20	.0000
3292.	909.	2384.	2472.	1.000	1936.	9.600E+01	1.000E+20	.0000
3292.	914.	2378.	2098.	1.000	1829.	1.200E+02	1.000E+20	.0000
3292.	914.	2378.	2098.	1.000	1829.	1.000E+00	1.000E+20	.0000
3292.	914.	2378.	2098.	1.000	1829.	1.000E+00	1.000E+20	.0000

**Table 4-4: Fatigue Results for Reactor Recirculation Outlet Nozzle Safe End
(concluded)**

MAX	MIN	RANGE	MEM+BEND	Ke	Salt	Napplied	Nallowed	U
3292.	1029.	2264.	2247.	1.000	1810.	1.000E+00	1.000E+20	.0000
3292.	1376.	1916.	1389.	1.000	1390.	9.000E+00	1.000E+20	.0000
3135.	1376.	1759.	1452.	1.000	1325.	1.000E+01	1.000E+20	.0000
3086.	1376.	1710.	1361.	1.000	1274.	2.280E+02	1.000E+20	.0000
2809.	1376.	1433.	1091.	1.000	1054.	1.000E+01	1.000E+20	.0000
2783.	1376.	1407.	1187.	1.000	1067.	4.300E+01	1.000E+20	.0000
2783.	1732.	1051.	860.	1.000	790.	1.000E+01	1.000E+20	.0000
2783.	1793.	990.	208.	1.000	576.	1.000E+01	1.000E+20	.0000
2783.	1958.	825.	811.	1.000	658.	6.000E+01	1.000E+20	.0000
2783.	1958.	825.	811.	1.000	658.	1.000E+00	1.000E+20	.0000
2783.	1958.	825.	811.	1.000	658.	1.040E+02	1.000E+20	.0000
2780.	1958.	822.	808.	1.000	655.	1.240E+02	1.000E+20	.0000
2780.	2104.	676.	576.	1.000	514.	1.000E+01	1.000E+20	.0000
2780.	2352.	428.	416.	1.000	340.	1.660E+02	1.000E+20	.0000
2780.	2352.	428.	416.	1.000	340.	1.000E+01	1.000E+20	.0000
2780.	2352.	428.	416.	1.000	340.	6.000E+01	1.000E+20	.0000
2780.	2352.	428.	416.	1.000	340.	1.000E+00	1.000E+20	.0000
2763.	2352.	411.	403.	1.000	327.	1.000E+01	1.000E+20	.0000
2762.	2352.	410.	403.	1.000	327.	1.000E+01	1.000E+20	.0000
2762.	2352.	410.	403.	1.000	327.	4.300E+01	1.000E+20	.0000
2762.	2441.	321.	443.	1.000	291.	1.700E+01	1.000E+20	.0000
2762.	2441.	321.	443.	1.000	291.	1.000E+00	1.000E+20	.0000
2762.	2441.	321.	443.	1.000	291.	1.000E+00	1.000E+20	.0000
2762.	2441.	321.	443.	1.000	291.	2.280E+02	1.000E+20	.0000
2496.	2441.	55.	177.	1.000	78.	5.300E+01	1.000E+20	.0000
2496.	2445.	51.	181.	1.000	77.	2.470E+02	1.000E+20	.0000
2491.	2445.	46.	178.	1.000	74.	1.000E+01	1.000E+20	.0000
2487.	2445.	42.	175.	1.000	71.	4.300E+01	1.000E+20	.0000
2487.	2487.	0.	0.	1.000	0.	1.700E+01	1.000E+20	.0000

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TOTAL USAGE FACTOR = .0015

Table 4-5: Fatigue Results for the Reactor Recirculation Outlet Nozzle Blend Radius

LOCATION = LOCATION NO. 2 -- BLEND RADIUS
 FATIGUE CURVE = 1 (1 = CARBON/LOW ALLOY, 2 = STAINLESS STEEL)
 m = 2.0
 n = .2
 Sm = 26700. psi
 Ecurve = 3.000E+07 psi
 Eanalysis = 2.670E+07 psi
 Kt = 1.00

MAX	MIN	RANGE	MEM+BEND	Ke	Salt	Napplied	Nallowed	U
55601.	17.	55584.	37389.	1.000	31227.	1.000E+00	1.951E+04	.0001
53822.	17.	53806.	48014.	1.000	30228.	1.000E+01	2.161E+04	.0005
51017.	17.	51001.	44054.	1.000	28652.	1.000E+01	2.547E+04	.0004
48939.	17.	48922.	52579.	1.000	27484.	1.000E+00	2.894E+04	.0000
46249.	17.	46233.	48340.	1.000	25974.	1.000E+00	3.443E+04	.0000
40607.	17.	40590.	36593.	1.000	22803.	1.000E+01	5.217E+04	.0002
39965.	17.	39948.	39059.	1.000	22443.	1.000E+01	5.647E+04	.0002
38737.	17.	38721.	40267.	1.000	21753.	6.000E+01	6.592E+04	.0009
38243.	17.	38227.	37209.	1.000	21476.	1.000E+01	7.025E+04	.0001
37757.	17.	37740.	37702.	1.000	21202.	7.000E+00	7.486E+04	.0001
37757.	476.	37281.	37314.	1.000	20945.	2.930E+02	7.954E+04	.0037
36291.	476.	35815.	36492.	1.000	20121.	7.000E+00	9.705E+04	.0001
36291.	1582.	34709.	35198.	1.000	19500.	3.000E+00	1.096E+05	.0000
36273.	1582.	34691.	35110.	1.000	19490.	6.000E+01	1.098E+05	.0005
36273.	1582.	34691.	35110.	1.000	19490.	1.000E+00	1.098E+05	.0000
35954.	1582.	34372.	35021.	1.000	19310.	5.600E+01	1.135E+05	.0005
35954.	1582.	34372.	35021.	1.000	19310.	1.000E+00	1.135E+05	.0000
35954.	1582.	34372.	35021.	1.000	19310.	1.000E+00	1.135E+05	.0000
35954.	1844.	34110.	34858.	1.000	19163.	1.000E+00	1.167E+05	.0000
35954.	1926.	34028.	34917.	1.000	19117.	2.410E+02	1.177E+05	.0020
35837.	1926.	33911.	34978.	1.000	19051.	1.000E+01	1.191E+05	.0001
35658.	1926.	33732.	34661.	1.000	18951.	4.900E+01	1.214E+05	.0004
35658.	11988.	23670.	26901.	1.000	13298.	1.790E+02	5.728E+05	.0003
35556.	11988.	23568.	27109.	1.000	13240.	1.210E+02	5.955E+05	.0002
35556.	12180.	23376.	28326.	1.000	13133.	1.790E+02	6.411E+05	.0003
35279.	12180.	23099.	27958.	1.000	12977.	1.000E+01	7.138E+05	.0000
34973.	12180.	22793.	27831.	1.000	12805.	1.110E+02	8.050E+05	.0001
34973.	20655.	14318.	16974.	1.000	8044.	1.890E+02	7.421E+07	.0000
34843.	20655.	14188.	16887.	1.000	7971.	1.110E+02	7.983E+07	.0000
34843.	25750.	9093.	17221.	1.000	5108.	1.000E+00	1.000E+20	.0000
34843.	27368.	7475.	5178.	1.000	4199.	1.000E+01	1.000E+20	.0000
34843.	28113.	6730.	3789.	1.000	3781.	1.780E+02	1.000E+20	.0000
34837.	28113.	6724.	3785.	1.000	3777.	1.000E+01	1.000E+20	.0000
34834.	28113.	6721.	3784.	1.000	3776.	6.000E+01	1.000E+20	.0000
34834.	28113.	6721.	3784.	1.000	3776.	1.000E+00	1.000E+20	.0000
34831.	28113.	6718.	3782.	1.000	3774.	1.000E+01	1.000E+20	.0000
34829.	28113.	6716.	3781.	1.000	3773.	4.100E+01	1.000E+20	.0000
34829.	29216.	5613.	1808.	1.000	3153.	1.000E+01	1.000E+20	.0000
34829.	31990.	2839.	1543.	1.000	1595.	6.000E+01	1.000E+20	.0000

**Table 4-5: Fatigue Results for Reactor Recirculation Outlet Nozzle Blend Radius
(concluded)**

MAX	MIN	RANGE	MEM+BEND	Ke	Salt	Napplied	Nallowed	U
34829.	31990.	2839.	1543.	1.000	1595.	1.000E+00	1.000E+20	.0000
34829.	31990.	2839.	1543.	1.000	1595.	1.160E+02	1.000E+20	.0000
34825.	31990.	2835.	1541.	1.000	1593.	1.000E+01	1.000E+20	.0000
34825.	31990.	2835.	1541.	1.000	1593.	6.000E+01	1.000E+20	.0000
34825.	31990.	2835.	1541.	1.000	1593.	1.000E+00	1.000E+20	.0000
34825.	31990.	2835.	1541.	1.000	1593.	1.000E+00	1.000E+20	.0000
34825.	31990.	2835.	1541.	1.000	1593.	4.000E+01	1.000E+20	.0000
34825.	32634.	2191.	2355.	1.000	1231.	6.000E+01	1.000E+20	.0000
34825.	32634.	2191.	2355.	1.000	1231.	1.000E+00	1.000E+20	.0000
34825.	32634.	2191.	2355.	1.000	1231.	1.270E+02	1.000E+20	.0000
34646.	32634.	2012.	2695.	1.000	1130.	1.010E+02	1.000E+20	.0000
34646.	33386.	1260.	1578.	1.000	708.	1.000E+00	1.000E+20	.0000
34646.	33581.	1065.	1120.	1.000	598.	1.000E+01	1.000E+20	.0000
34646.	33709.	937.	1137.	1.000	526.	1.000E+01	1.000E+20	.0000
34646.	33822.	824.	1455.	1.000	463.	1.000E+00	1.000E+20	.0000
34646.	33924.	722.	1228.	1.000	406.	1.000E+00	1.000E+20	.0000
34646.	33924.	722.	1310.	1.000	406.	1.000E+00	1.000E+20	.0000
34646.	34124.	522.	1067.	1.000	293.	1.000E+01	1.000E+20	.0000
34646.	34331.	315.	3398.	1.000	177.	1.000E+01	1.000E+20	.0000
34646.	34447.	199.	-603.	1.000	112.	1.200E+02	1.000E+20	.0000
34646.	34592.	54.	-130.	1.000	30.	3.500E+01	1.000E+20	.0000

=====
TOTAL USAGE FACTOR = .0108

5.0 ENVIRONMENTAL FATIGUE ANALYSIS

Environmental fatigue multipliers were computed for both normal water chemistry (NWC) and hydrogen water chemistry (HWC) conditions in Reference [18] for various regions of the VY RPV and attached piping.

The Recirculation Outlet nozzle has three materials: a Ni-Cr-Fe dissimilar metal weld (DMW), a low alloy steel forging, and a stainless steel safe end. To ensure the maximum CUF considering environmental effects was identified; locations in the safe end and nozzle forging were selected. This selection produces bounding environmental fatigue results for the entire nozzle assembly for the following reasons:

- The highest thermal stresses from the FEM analysis occur in the stainless steel safe end. Stainless steel F_{en} multipliers are significantly higher than Ni-Cr-Fe multipliers (F_{en} values are 2.55 or higher for stainless steel [18] vs. a constant value of 1.49 for Ni-Cr-Fe [19]). Therefore, evaluation of the safe end bounds the Ni-Cr-Fe weld material.
- The highest pressure stresses from the FEM analysis occur in the low alloy steel nozzle forging. Low alloy steel F_{en} multipliers are higher than Ni-Cr-Fe multipliers (F_{en} values are 2.45 or higher for low alloy steel [18] vs. a constant value of 1.49 for Ni-Cr-Fe [19]). Therefore, evaluation of the nozzle forging bounds the Ni-Cr-Fe weld material.

Based on VY-specific dates for plant startup and HWC implementation, as well as past and future predicted HWC system availability, it was determined that overall HWC availability is 47% over the sixty year operating period for VY. Therefore, for the purposes of the EAF assessment of the reactor recirculation outlet nozzle, it was assumed that HWC conditions exist for 47% of the time, and NWC conditions exist for 53% of the time over the 60-year operating life of the plant. RPV beltline region chemistry was assumed for both the reactor recirculation outlet nozzle safe end and blend radius locations, since both locations experience reactor conditions at all times.

For the safe end location, the environmental fatigue factors for NWC and HWC are 8.36 and 15.35, respectively, from Table 5 of Reference [18] for the RPV beltline region. This results in an EAF adjusted CUF as follows:

$$\text{60-Year CUF, } U_{60} = 0.0015$$

$$\text{Overall EAF multiplier, } F_{en} = (8.36 \times 53\% + 15.35 \times 47\%) = 11.65$$

$$\text{60-Year EAF CUF, } U_{60-env} = 0.0015 \times 11.65 = 0.0175$$

The EAF CUF value of 0.0175 for 60 years for the safe end is acceptable (i.e., less than the allowable value of 1.0).

The fatigue calculation documented in Section 4.0 for the blend radius location was performed for the nozzle base material since cladding is structurally neglected in modern-day fatigue analyses, per ASME Code, Section III, NB-3122.3 [17]. This is also consistent with Sections 5.7.1 and 5.7.4 of NUREG/CR-6260 [1]. Therefore, the cladding was neglected and EAF assessment of the nozzle base material was performed for the blend radius location.

For the blend radius location, the environmental fatigue factors for NWC and HWC are 12.43 and 2.45, respectively, from Table 5 of Reference [18] for the RPV beltline region. This results in an EAF adjusted CUF as follows:

$$\text{60-Year CUF, } U_{60} = 0.0108$$

$$\text{Overall EAF multiplier, } F_{en} = (12.43 \times 53\% + 2.45 \times 47\%) = 7.74$$

$$\text{60-Year EAF CUF, } U_{60-env} = 0.0108 \times 7.74 = 0.0836$$

The EAF CUF value of 0.0836 for 60 years for the blend radius is acceptable (i.e., less than the allowable value of 1.0).

6.0 CONCLUSIONS

This report documents a refined fatigue evaluation for the VY reactor recirculation outlet nozzle. The intent of this evaluation is to use refined transient definitions and the revised cyclic transient counts for 60 years for a computation of CUF, including EAF effects, that is more refined than previously performed fatigue analyses. The fatigue-limiting locations in the reactor recirculation outlet nozzle and safe end are included in the evaluation, to be consistent with NUREG/CR-6260 [1] needs for EAF evaluation for license renewal. The final fatigue results are considered to be a replacement to the values previously reported in the VY LRA.

The fatigue calculations for the VY reactor recirculation outlet nozzle were performed in accordance with ASME Code, Section III, Subsection NB-3200 methodology (1998 Edition, 2000 Addenda) [17]. The stress evaluation is summarized in Section 3.0, and the fatigue analysis is summarized in Section 4.0. The results in Section 4.0 reveal that the CUF for the limiting safe end location is 0.0015, and the CUF for the limiting blend radius location is 0.0108. Both of these values represent 60 years of plant operation, including all relevant EPU effects.

EAF calculations for the VY reactor recirculation outlet nozzle were also performed, as summarized in Section 5.0. The results in Section 5.0 reveal that the EAF CUF for the limiting safe end location is 0.0175, and the EAF CUF for the limiting blend radius location is 0.0836. Both of these values represent 60 years of plant operation, including all relevant EPU effects.

All fatigue allowables, both with and without EAF effects, are met, thus demonstrating acceptability for 60 years of operation.

7.0 REFERENCES

1. NUREG/CR-6260 (INEL-95/0045), "Application of NUREG/CR-5999 Interim Fatigue Curves to Selected Nuclear Power Plant Components," March 1995.
2. Vermont Yankee Drawing 5920-06623, Rev. 0, (Hitachi, Ltd. Drawing No IOR290-127), "Recirc. Outlet Safe End," SI File No. VY-16Q-204.
3. Vermont Yankee Drawing 5920-00238, Rev. 4, (Chicago Bridge & Iron Company, Contract No. 9-6201, Drawing No. 21) "36"x28" Nozzles Mk N1A/B," SI File No. VY-16QQ-204.
4. GE Design Specification No. 21A1115, Revision 4, "Vermont Yankee Reactor Pressure Vessel," October 21, 1969, SI File No. VY-05Q-210.
5. GE Design Specification No. 26A6019, Revision 1, "Reactor Vessel – Extended Power Uprate," June 2, 2003, SI File No. VY-05Q-236.
6. Kuo, A. Y., Tang, S. S., and Riccardella, P. C., "An On-Line Fatigue Monitoring System for Power Plants, Part I - Direct Calculation of Transient Peak Stress Through Transfer Matrices and Green's Functions," ASME PVP Conference, Chicago, 1986.
7. ANSYS, Release 8.1 (w/Service Pack 1), ANSYS, Inc., June 2004.
8. Structural Integrity Associates Calculation No. VY-16Q-304, Revision 0, "Recirculation Outlet Nozzle Finite Element Model."
9. American Society of Mechanical Engineers, Boiler and Pressure Vessel Code, Section II, Materials, Part D, "Properties (Customary)," 1998 Edition, 2000 Addenda.
10. Structural Integrity Associates Calculation No. VY-16Q-305, Revision 0, "Recirculation Outlet Stress History Development for Nozzle Green Function."
11. Structural Integrity Associates Calculation No. VY-16Q-306, Revision 0, "Fatigue Analysis of Recirculation Outlet Nozzle."
12. Reactor Thermal Cycles, Attachment 1, page 2 of Entergy Design Input Record (DIR) Revision 1, EC No. 1773, Rev. 0, "Environmental Fatigue Analysis for Vermont Yankee Nuclear Power Station," 7/26/07, SI File No. VY-16Q-209.

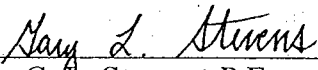
13. Nozzle Thermal Cycles (Recirculation Outlet), Attachment 1, page 4 of Entergy Design Input Record (DIR) EC No. 1773, Rev. 1, "Environmental Fatigue Analysis for Vermont Yankee Nuclear Power Station," 7/26/07, SI File No. VY-16Q-209.
14. Reactor Thermal Cycles for 60 Years of Operation, Attachment 1, page 1 of Entergy Design Input Record (DIR) EC No. 1773, Rev. 1, "Environmental Fatigue Analysis for Vermont Yankee Nuclear Power Station," 7/26/07, SI File No. VY-16Q-209.
15. Vermont Yankee Drawing 5920-0024, Rev. 11, (GE Drawing No. 919D294, Sheet No. 7), "Reactor Vessel," SI File No. VY-05Q-241.
16. Structural Integrity Associates Calculation No. SW-SPVF-01Q-301, Revision 0, "STRESS.EXE, P-V.EXE, and FATIGUE.EXE Software Verification."
17. American Society of Mechanical Engineers Boiler & Pressure Vessel Code, Section III, Rules for Construction of Nuclear Facility Components, 1998 Edition, 2000 Addenda.
18. Structural Integrity Associates Calculation No. VY-16Q-303, Revision 0, "Environmental Fatigue Evaluation of Reactor Recirculation Inlet Nozzle and Vessel Shell/Bottom Head."
19. EPRI Report No. TR-105759, "An Environmental Factor Approach to Account for Reactor Water Effects in Light Water Reactor Pressure Vessel and Piping Fatigue Evaluations," December 1995.
20. CB&I, RPV Stress Report Section: S9 "Stress Analysis Recirculation Outlet Nozzle Vermont Yankee Reactor Vessel." 9-6201, SI document, VY-16Q-204.

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File No.: VY-16Q-403
July 2007

**Environmental Fatigue Analysis
for the
Vermont Yankee
Reactor Pressure Vessel
Core Spray Nozzle**

Prepared for:
Entergy Nuclear Operations, Inc.
(Contract No. 10150394)

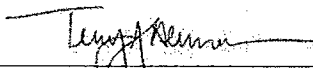
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1.0 INTRODUCTION

In Table 4.3-3 of the Vermont Yankee License Renewal Application (LRA), the 60-year cumulative usage factor (CUF) values for the reactor pressure vessel (RPV) core spray nozzle are reported as 0.625 (nozzle) and 0.182 (safe end). The safe end value was reported as a generic value, since no plant-specific value was determined. Application of environmentally assisted fatigue (EAF) multipliers, as required for the license renewal period, resulted in unacceptable EAF CUF values of 1.53 and 2.79 for the nozzle and safe end, respectively. Therefore, further refined analysis was necessitated to show acceptable EAF CUF results for this component.

This report documents a refined fatigue evaluation for the VY core spray nozzle. The intent of this evaluation is to use refined transient definitions and the revised cyclic transient counts for 60 years for a computation of CUF, including EAF effects, that is more refined than previously performed fatigue analyses. The fatigue-limiting locations in the core spray nozzle and safe end are included in the evaluation, to be consistent with NUREG/CR-6260 [1] needs for EAF evaluation for license renewal. The EAF effects for the core spray piping, which is also a NUREG/CR-6260 location, are considered to be covered by the nozzle and safe end calculations because the nozzle region bounds the piping¹. The resulting fatigue results will be used as a replacement to the values previously reported in the VY LRA.

The refined evaluation summarized in this report included the development of a detailed finite element model of the core spray nozzle, including relevant portions of the safe end, thermal sleeve, the RPV wall, and the weld overlay repair documented in Reference [2]. Thermal and pressure stress histories were developed for relevant transients affecting the core spray nozzle, including any effects of Extended Power Uprate (EPU), as specified by the VY RPV Design Specification [3], the VY EPU Design Specification [4] and other boiling water reactor (BWR) operating experience. The thermal and pressure stress histories were used to determine total stress and primary plus secondary stress for use in a subsequent fatigue evaluation. Stresses were also included due to loads from the attached piping for application in the stress/fatigue analysis, based on the bounding reaction loads obtained from the

¹ The nozzle stresses are more severe due to the nozzle discontinuity, and the nozzle thermal transients are more severe due to interaction with the hot RPV.

relevant design documents. The revised fatigue calculation was performed using Section III methodology from the 1998 Edition, 2000 Addenda of the ASME Code, and were performed using actual cycles from past plant operation projected out to 60 years of operation.

1.1 Green's Function Methodology

For the core spray nozzle evaluated as a part of this work, stress histories were computed by a time integration of the product of a pre-determined Green's Function and the transient data. This Green's Function integration scheme is similar in concept to the well-known Duhamel theory used in structural dynamics. A detailed derivation of this approach and examples of its application to specific plant locations is contained in Reference [5]. A general outline is provided in this section.

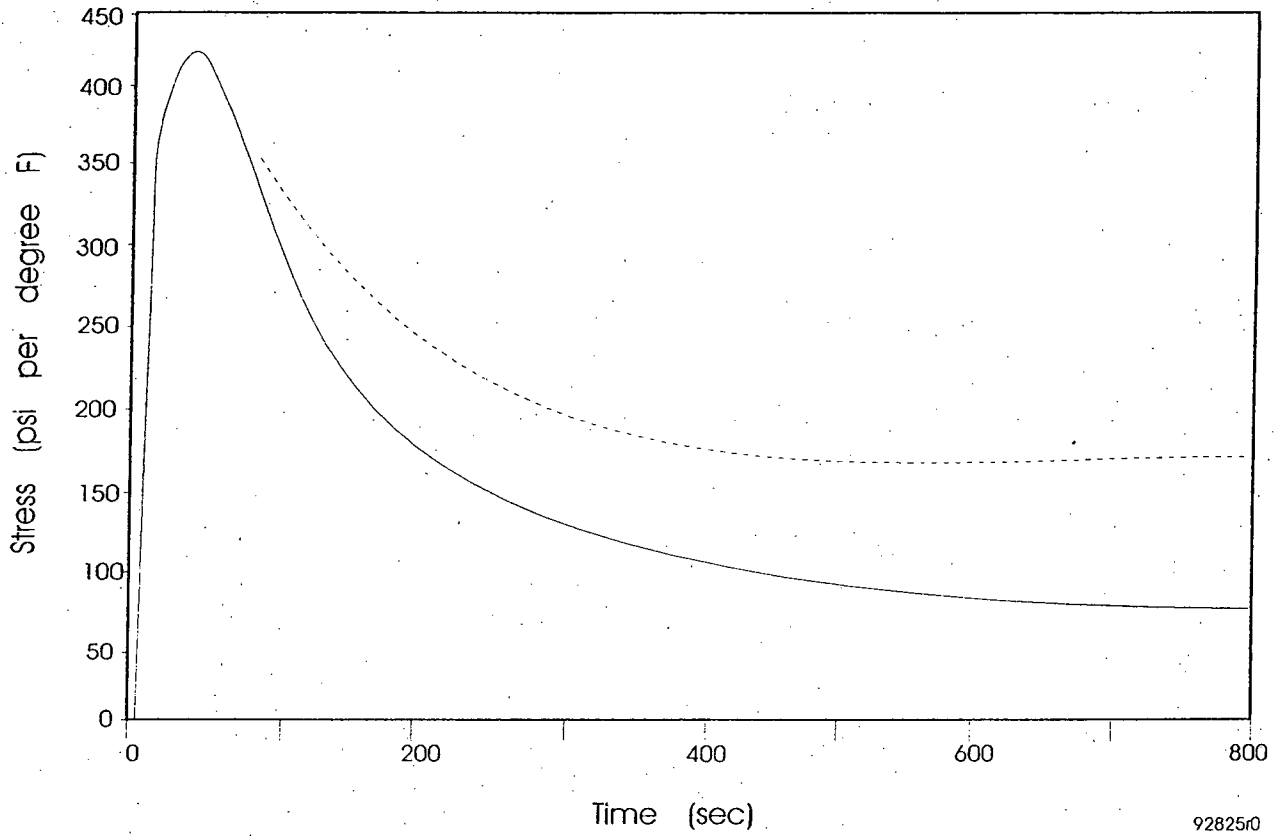
A Green's Function is derived by using finite-element methods to determine the transient stress response of the component to a step change in loading (usually a thermal shock). The critical location in the component is identified based on the maximum stress, and the thermal stress response over time is extracted for this location. This response to the input thermal step is the "Green's Function." Figure 1-1 shows a typical set of two Green's Functions, each for a different set of heat transfer coefficients (representing different flow rate conditions).

To compute the thermal stress response for an arbitrary transient, the loading parameter (usually local fluid temperature) is deconstructed into a series of step-loadings. By using the Green's Function, the response to each step can be quickly determined. By the principle of superposition, these can be added (algebraically) to determine the response to the original load history. The result is demonstrated in Figure 1-2. The input transient temperature history contains five step-changes of varying size, as shown in the upper plot in Figure 1-2. These five step changes produce the five successive stress responses in the second plot shown in Figure 1-2. By adding all five response curves, the real-time stress response for the input thermal transient is computed.

The Green's Function methodology produces identical results compared to running the input transient through the finite element model. The advantage of using Green's Functions is that many individual

transients can be run with a significant reduction of effort compared to running all transients through the finite element model. The trade-off in this process is that the Green's Functions are based on constant material properties and heat transfer coefficients. Therefore, these parameters are chosen to bound all transients that constitute the majority of fatigue usage, i.e., the heat transfer coefficients at 300°F bound the cold water injection transient. In addition, the instantaneous value for the coefficient of thermal expansion is used instead of the mean value for the coefficient of thermal expansion. This conservatism is more than offset by the benefit of not having to analyze every transient, which was done in the VY core spray nozzle evaluation.

Once the stress history is obtained for all transients using the Green's Function approach, the remainder of the fatigue analysis is carried out using traditional methodologies in accordance with ASME Code, Section III requirements.



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Note: A typical set of two Green's Functions is shown, each for a different set of heat transfer coefficients (representing different flow rate conditions).

Figure 1-1: Typical Green's Functions for Thermal Transient Stress

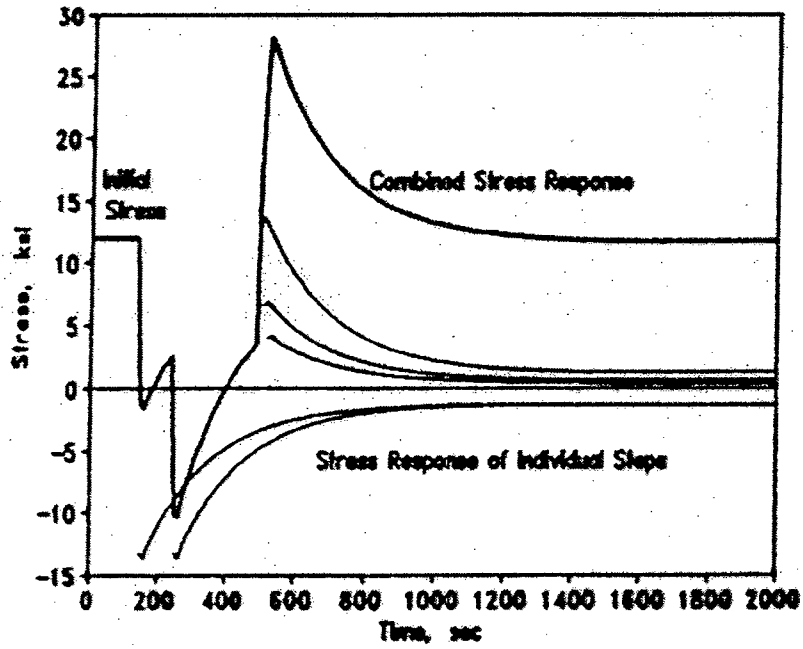
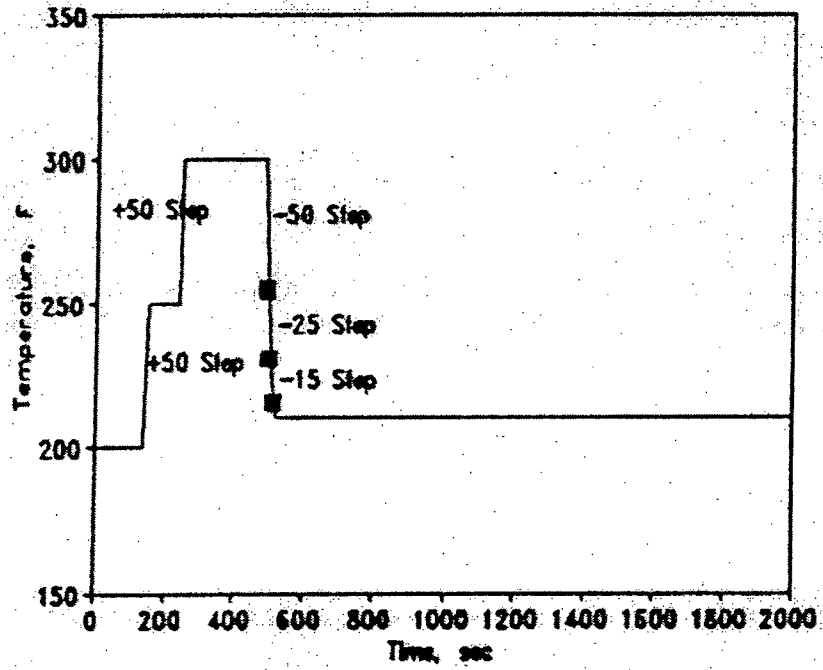


Figure 1-2: Typical Stress Response Using Green's Functions

2.0 FINITE ELEMENT MODEL

An ANSYS [6] finite element model (FEM) of the VY core spray nozzle and safe end was developed and used to perform the updated stress and fatigue analyses. The details of the model development are documented in the Reference [7] calculation.

The materials of the various components of the model are listed below:

- Safe End – SB 166 (72Ni-15Cr-8Fe, N06600)
- 8Ø x 10Ø Conc. Reduction – SA312 TP304 (18Cr-8Ni)
- Nozzle Forging – SA508 Class II (3/4Ni-1/2Mo-1/3Cr-V)
- Vessel – SA533 Grade B (Mn-1/2Mo-1/2Ni)
- Cladding – SA240 TP 304 (18Cr-8Ni)

In the FEM model, the radius of RPV was increased by a factor of two to account for the fact that the vessel portion of the finite element model is a sphere and the actual geometry is a cylinder.

Material properties were based upon the 1998 ASME Code, Section II, Part D, with 2000 Addenda [8], and are shown in Table 2-1. The properties were evaluated at an average temperature of 300°F. This average temperature is based on a thermal shock of 500°F to 100°F, which was applied to the FEM model for Green's Function development.

The finite element model, which includes the weld overlay, is shown in Figure 2-1.

Table 2-1: Material Properties @ 300°F (1)

Material ID	Part Description	Material		Modulus of Elasticity, e+6 psi [EX]	Coefficient of Thermal Expansion, e-6, in/in/°F [ALPX]	Thermal Conductivity, Btu/hr-ft-°F [KXX]	Thermal Diffusivity, ft ² /hr	Specific Heat, Btu/lb-°F [C] (2)
2	Safe End	SB 166	72Ni-15Cr-8Fe N06600	29.8	7.9	9.6	0.160	0.1157
2	Weld Overlay	INCONEL 82	72Ni-15Cr-8Fe N06600	29.8	7.9	9.6	0.160	0.1157
1	Nozzle	SA508 Class II	¾ Ni-1/2Mo-1/3 Cr-V	26.7	7.3	23.4	0.401	0.1193
3	Vessel	SA533 Grade B	Mn-1/2Mo-1/2Ni	28.0	7.7	23.4	0.401	0.1193
4	3/16 Clad	SA240 TP 304	18Cr-8Ni	27.0	9.8	9.8	0.160	0.1252
4	8Ø x 10Ø Conc. Reduction ⁽²⁾	SA312 ⁽²⁾ TP304	18Cr-8Ni	27.0	9.8	9.8	0.160	0.1252

- Note:
1. Material properties are evaluated at 300°F from the 1998 ASME Code, Section II, Part D, with 2000 Addenda [8], Poisson's ratio, which are assumed typical values and specific heat is calculated as $[k/pd]/12^3$.
 2. The 8Ø x 10Ø concentric reduction was modeled as a straight pipe with the material properties of the original design was replaced by a new material (SA403 T316L). These two stainless steels have the same modulus of elasticity and properties.

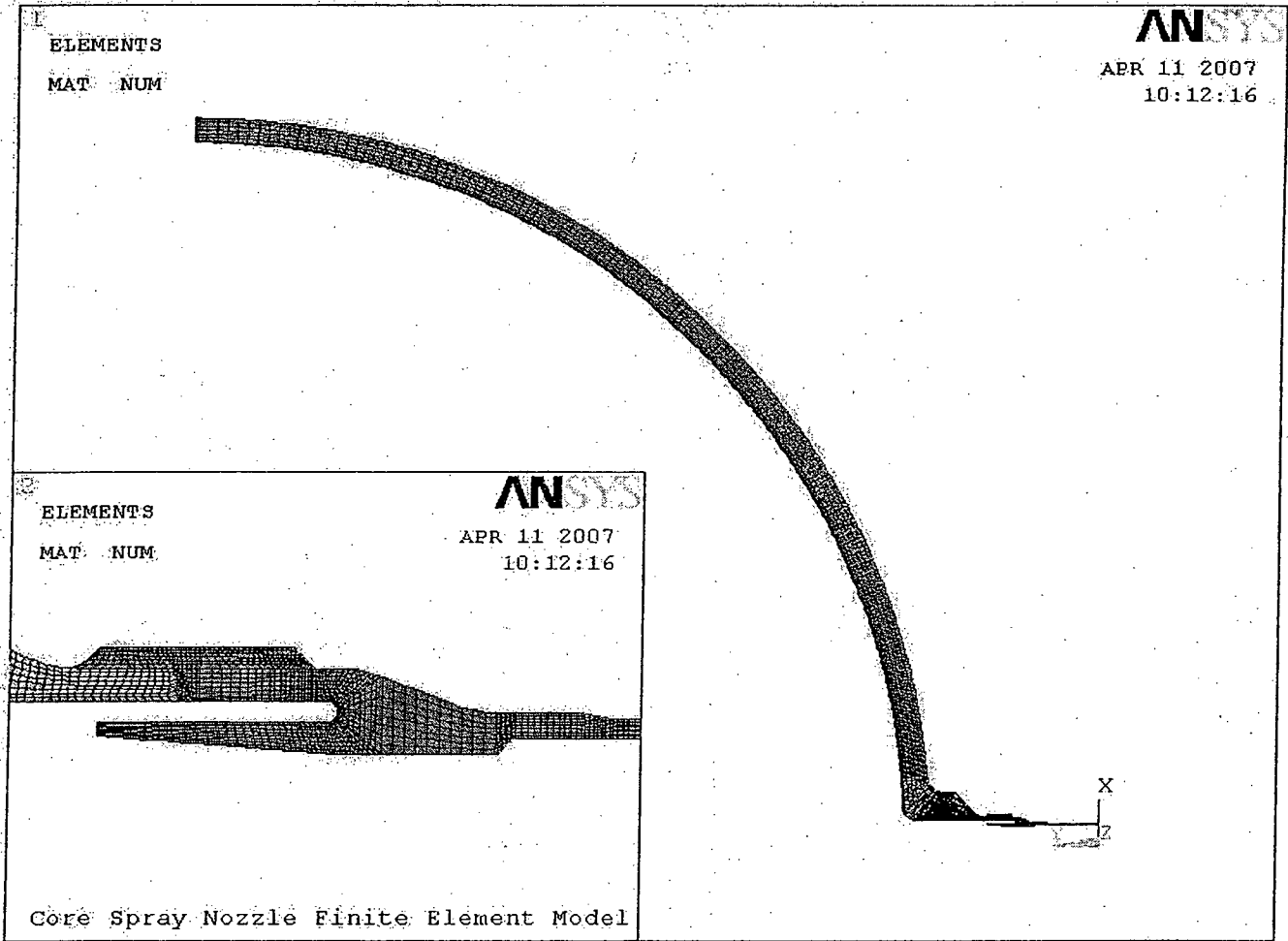


Figure 2-1: VY Core Spray Nozzle FEM

3.0 LOAD DEFINITIONS

The pressure and thermal stresses for the core spray nozzle for the revised fatigue evaluation were developed using the axisymmetric FEM model, described in Section 2.0 of this report. The details of the Green's function development and associated stress evaluation are documented in the Reference [9] and [10] calculations.

3.1 Thermal Loading

Thermal loads were applied to the core spray nozzle model to generate the Green's Function. As a first step in the Green's Function process, heat transfer coefficients were determined for various regions of the core spray FEM for two different flow cases: (1) 0% core spray flow, and (2) 100% core spray flow through the nozzle.

The 0% flow case simulates a stagnant condition of the core spray nozzle when not in operation and the entire core spray nozzle is at the same temperature as the RPV fluid. The heat transfer coefficients for the 0% flow case are for free convection (stagnant) conditions. The applied boundary fluid temperature is changed to simulate a thermal shock from 500°F to 100°F to develop the stress response on the core spray nozzle in the stagnant condition.

The 100% flow case simulates the operational condition of the core spray nozzle (i.e., the entire core spray nozzle experiences 100°F water due to injection). The heat transfer coefficients for the high flow case are for forced and free convection depending on the region of the FEM. The applied boundary fluid temperature is changed to simulate a thermal shock from 500°F to 100°F to develop the stress response on the core spray nozzle due to injection.

The temperature on the exterior of the reactor, nozzle, safe end and pipe was assumed to be 120°F (ambient). Figure 3-4 shows the heat transfer coefficient regions assumed for the core spray nozzle FEM. The applied heat transfer coefficients and the fluid temperatures are summarized in the sections that follow.

3.1.1 Heat Transfer Coefficients and Boundary Fluid Temperatures

Referring to Figure 3-4, heat transfer coefficients were applied as follows:

- The heat transfer coefficient for the outside surfaces of the FEM (Region 12) was a constant value of 0.2 BTU/hr-ft²-°F (3.858x10⁻⁷ BTU/sec-in²-°F).
- Table 3-1 shows the results of the heat transfer coefficient calculations for all of the thermal regions identified in Figure 3-4. The detailed heat transfer calculations for Regions 1, 3, 5, 7, 9, and 11 are contained in the Reference [9] calculation.
- In Regions 2, 4, 6, 8, and 10, the heat transfer coefficients are interpolated.

For both Green's Functions, a 500°F – 100°F thermal shock was run to determine the stress response. For the 0% flow case, the entire inside surface of the FEM was shocked. For the 100% flow case, only the nozzle flow path was shocked.

3.1.2 Green's Functions

The two flow-dependent thermal load cases outlined in previous section were run on the core spray nozzle FEM with the heat transfer coefficients and the fluid temperature conditions listed in Table 3-1. Two locations were selected for analysis (see Figures 3-5 and 3-6):

1. The critical safe end location was chosen as the node with the highest stress intensity due to thermal loading under nozzle flow conditions. The highest stress intensity due to thermal loading occurred at Node 3719 (see Figure 3-5), on the inside diameter of the nozzle safe end. Therefore, this node was selected for analysis.
2. The critical blend radius location was chosen based upon the highest pressure stress intensity. The critical location was selected as Node 2166, as shown in Figure 3-6.

Two stress intensity Green's Functions were developed for each location and each flow case: (1) total stress intensity, and (2) membrane plus bending stress intensity. The total stress intensity

Green's Functions for the safe end location are shown in Figures 3-7 and 3-8. The total stress intensity Green's Functions for the blend radius location are shown in Figures 3-9 and 3-10.

3.1.3 Thermal Transients

The transients analyzed for the core spray nozzle were developed based on the definitions in the original RPV Design Specification [3], as modified for EPU [4], as well as more recent definitions based on BWR operating experience. For BWR operating experience, the transients described in the thermal cycle diagrams for a BWR-4 plant similar in design and vintage to VY were obtained, and plant data from VY applied to each transient. The resulting thermal cycle diagrams are shown in References [11 and 12]. The final transients evaluated in the stress and fatigue analyses are shown in Figures 3-11 through 3-16.

The number of cycles projected for the 60-year operating life is used for each transient [13]. Tables 3-2 and 3-3 summarize the thermal transients for the safe end and blend radius locations, respectively.

3.2 Pressure Loading

A uniform pressure of 1,000 psi was applied along the inside surface of the core spray nozzle and the RPV wall. A pressure load of 1,000 psi was used because it is easily scaled up or down to account for different pressures that occur during transients. In addition, a cap load of 4,774 psi was applied to the piping at the end of the nozzle. This cap load was calculated as follows:

$$P_{\text{cap}} = \frac{PD_i^2}{(D_o^2 - D_i^2)}$$

where: P_{cap} = end cap pressure load (psi)
 P = unit pressure load = 1,000 psi
 D_i = inside diameter of end of FEM = 9.834"

$D_o =$ outside diameter of end of FEM = 10.815"

The calculated pressure was applied as a negative value so that it would exert tension on the end of the model. The nodes on the end of the FEM were coupled in the axial direction to ensure mutual displacement of the end of the nozzle due to attached piping. Figures 3-1, 3-2, and 3-3 show the internal pressure distribution, cap load, and symmetry condition applied to the vessel end of the model, respectively.

The internal pressure load case for Node 2166 (blend radius) resulted in a total stress intensity of 35,860 psi, and for Node 3719 (safe end) resulted in a total stress intensity of 12,030 psi. The membrane plus bending stress intensity at Node 2166 and Node 3719 are 34,970 psi and 12,020 psi, respectively.

3.3 Piping Loading

The piping stress intensities (stress caused by the attached piping) were determined for the two evaluated core spray nozzle locations. The design piping reactions that were used in the stress and fatigue evaluation are defined on the Reference [14] drawing. These loads represent shear and moment loadings on the nozzle resulting from thermal expansion of the attached piping and seismic loads. The loads are applicable at the piping end of the safe end, as shown in Figure 3-17. The stresses resulting from these loads were calculated by hand using classical structural mechanics formulas, as documented in Reference [10], and are shown in Tables 3-4 and 3-5 for the safe end and blend radius locations, respectively.

Table 3-1: Summary of Heat Transfer Coefficients

Regions	0% Flow		100% Flow	
	Initial Temperature °F	HTC Btu/hr-ft ² -°F	Initial Temperature °F	HTC Btu/hr-ft ² -°F
R1	500	143	500	2693
R2	500	Interpolated	500	Interpolated
R3 ⁽¹⁾	500	39	500	52
R4	500	Interpolated	500	Interpolated
R5 ⁽¹⁾	500	47	500	66
R6	500	Interpolated	500	Interpolated
R6B	500	97	500	97
R7A ⁽¹⁾	500	38	500	50
R7B ⁽¹⁾	500	20	500	23
R8	500	Interpolated	500	Interpolated
R9 ⁽¹⁾	500	33	500	41
R10	500	Interpolated	500	Interpolated
R11	500	500	500	500
R12	120	0.20	120	0.2

Table 3-2: Safe End Transients

Transient Number	Time (s)	Temp (°F)	Time Step (s)	Pressure (psig)	Flow Rate (GPM)
2. Design HYD Test 120 Cycles	---	100	---	0	
				1100	
				50	
3. Startup 300 Cycles	0	100		0	0
	16164	549	16164	1010	(0%)
	17164	549	1000	1010	
11. Loss of Feedwater Pumps 10 Cycles	0	526		1010	0
	3	526	3	1190	(0%)
	13	526	10	1135	
	233	300	220	1135	
	2213	500	1980	1135	
	2393	300	180	885	
	6893	500	4500	1135	
	7313	300	420	675	
	7613	300	300	675	
	11213	400	3600	240	
	16577	549	5364	1010	
	16637	549	60	1010	
	16638	542	1	1010	
16698	542	60	1010		
16699	526	1	1010		
17699	526	1000	1010		
14. SRV Blowdown 1 Cycle	0	526		1010	0
	600	375	600	400	(0%)
	11580	70	10980	50	
	12580	70	1000	50	
21-23. Shutdown 300 Cycles	0	549		1010	0
	6264	375	6264	50	(0%)
	6864	330	600	50	
	16224	100	9360	50	
17224	100	1000	50		
12. Hydrostatic Test 1 cycle	---	100	---	50	
				1563	
				50	
30. Emergency Shut Down 1 Cycle	0	549		1010	3200
	10	406	10	250	(100%)
	11	70	1	250	
	1011	70	1000	0	

Table 3-3: Blend Radius Transients

Transient Number	Time (s)	Temp (°F)	Time Step (s)	Pressure (psig)	Flow Rate (GPM)
2. Design HYD Test 120 Cycles	---	100	---	0	
				1100	
				50	
3. Startup 300 Cycles	0	100		0	0
	16164	549	16164	1010	(0%)
	24164	549	8000	1010	
11. Loss of Feedwater Pumps 10 Cycles	0	526		1010	0
	3	526	3	1190	(0%)
	13	526	10	1135	
	233	300	220	1135	
	2213	500	1980	1135	
	2393	300	180	885	
	6893	500	4500	1135	
	7313	300	420	675	
	7613	300	300	675	
	11213	400	3600	240	
	16577	549	5364	1010	
	16637	549	60	1010	
	16638	542	1	1010	
16698	542	60	1010		
16699	526	1	1010		
24699	526	8000	1010		
14. SRV Blowdown 1 Cycle	0	526		1010	0
	600	375	600	400	(0%)
	11580	70	10980	50	
	19580	70	8000	50	
21-23. Shutdown 300 Cycles	0	549		1010	0
	6264	375	6264	50	(0%)
	6864	330	600	50	
	16224	100	9360	50	
24224	100	8000	50		
24. Hydrostatic Test 1 Cycle	---	100	---	50	
				1563	
				50	
30. Emergency Shut Down 1 Cycle	0	549		1010	3200
	10	406	10	250	(100%)
	11	70	1	250	
	8011	70	8000	0	

Table 3-4: Stresses Due to Piping Loads for Safe End Location

Safe End External Piping Loads		
Parameters		
$F_x =$	2.50	kips
$F_y =$	4.60	kips
$F_z =$	1.70	kips
$M_x =$	264.00	in-kips
$M_y =$	85.20	in-kips
$M_z =$	105.60	in-kips
OD=	10.82	in
ID=	9.834	in
$R_N =$	5.16	in
L =	0.30	in
$t_N =$	0.49	in
$(M_x)_1 =$	262.60	in-kips
$(M_y)_1 =$	85.96	in-kips
$M_{xy} =$	276.31	in-kips
$F_{xy} =$	5.24	kips
$N_z =$	3.35	kips/in
$q_N =$	-0.31	kips/in
Primary Membrane Stress Intensity		
$PM_z =$	6.84	ksi
$\tau =$	-0.63	ksi
$SI_{max} =$	6.95	ksi
$SI_{max} =$	6949.94	psi

Table 3-5: Stresses Due to Piping Loads for Blend Radius Location

Blend Radius External Piping Loads		
Parameters		
$F_x =$	2.50	kips
$F_y =$	4.60	kips
$F_z =$	1.70	kips
$M_x =$	264.00	in-kips
$M_y =$	85.20	in-kips
$M_z =$	105.60	in-kips
OD=	13.87	in
ID=	11.750	in
$R_N =$	7.65	in
L =	30.82	in
$t_N =$	3.56	in
$(M_x)_2 =$	122.24	in-kips
$(M_y)_2 =$	162.24	in-kips
$M_{xy} =$	203.14	in-kips
$F_{xy} =$	5.24	kips
$N_z =$	1.14	kips/in
$q_N =$	-0.07	kips/in
Primary Membrane Stress Intensity		
$PM_z =$	0.32	ksi
$\tau =$	-0.02	ksi
$SI_{max} =$	0.32	ksi
$SI_{max} =$	322.52	psi

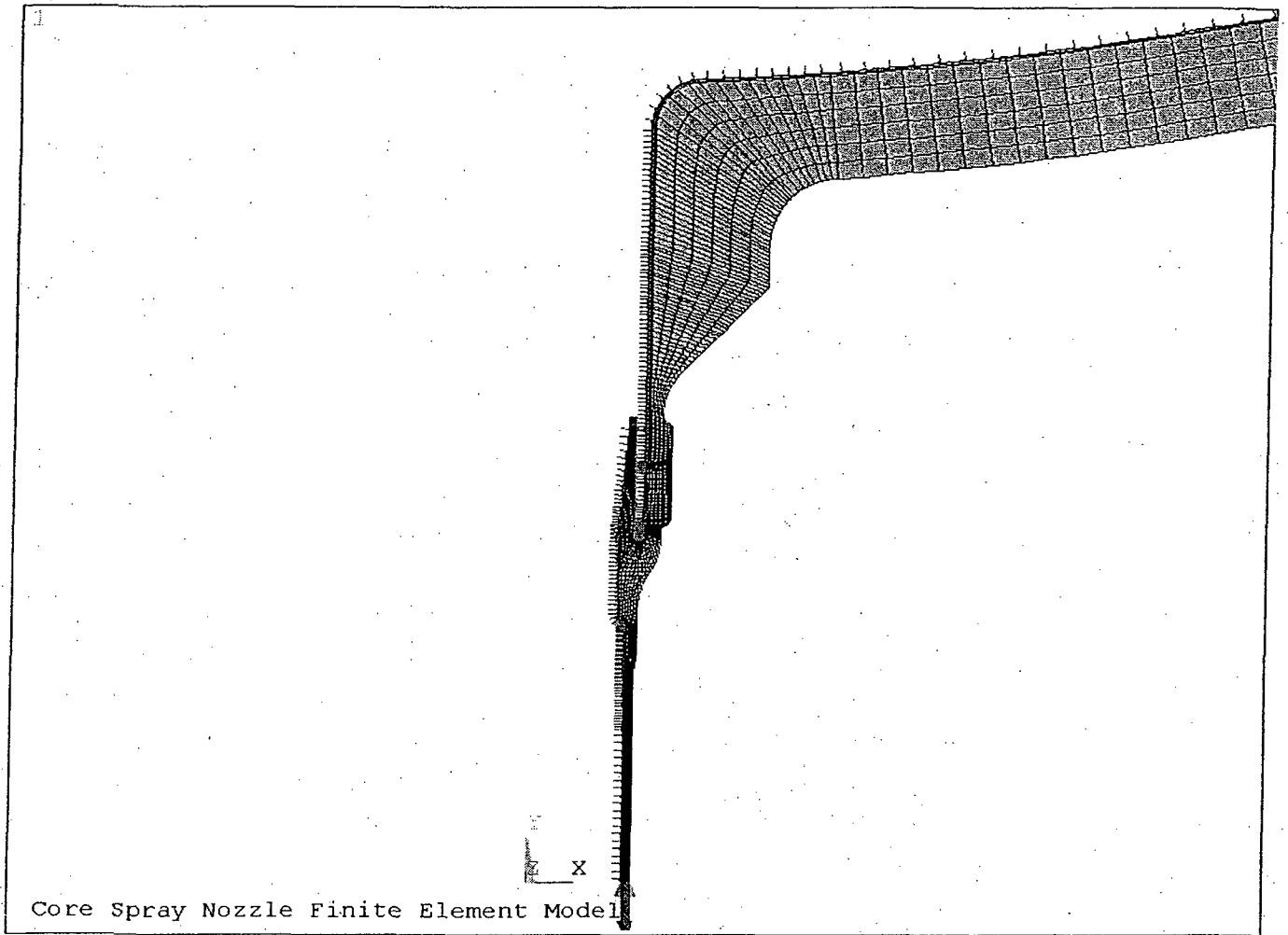


Figure 3-1: Core Spray Nozzle Internal Pressure Distribution

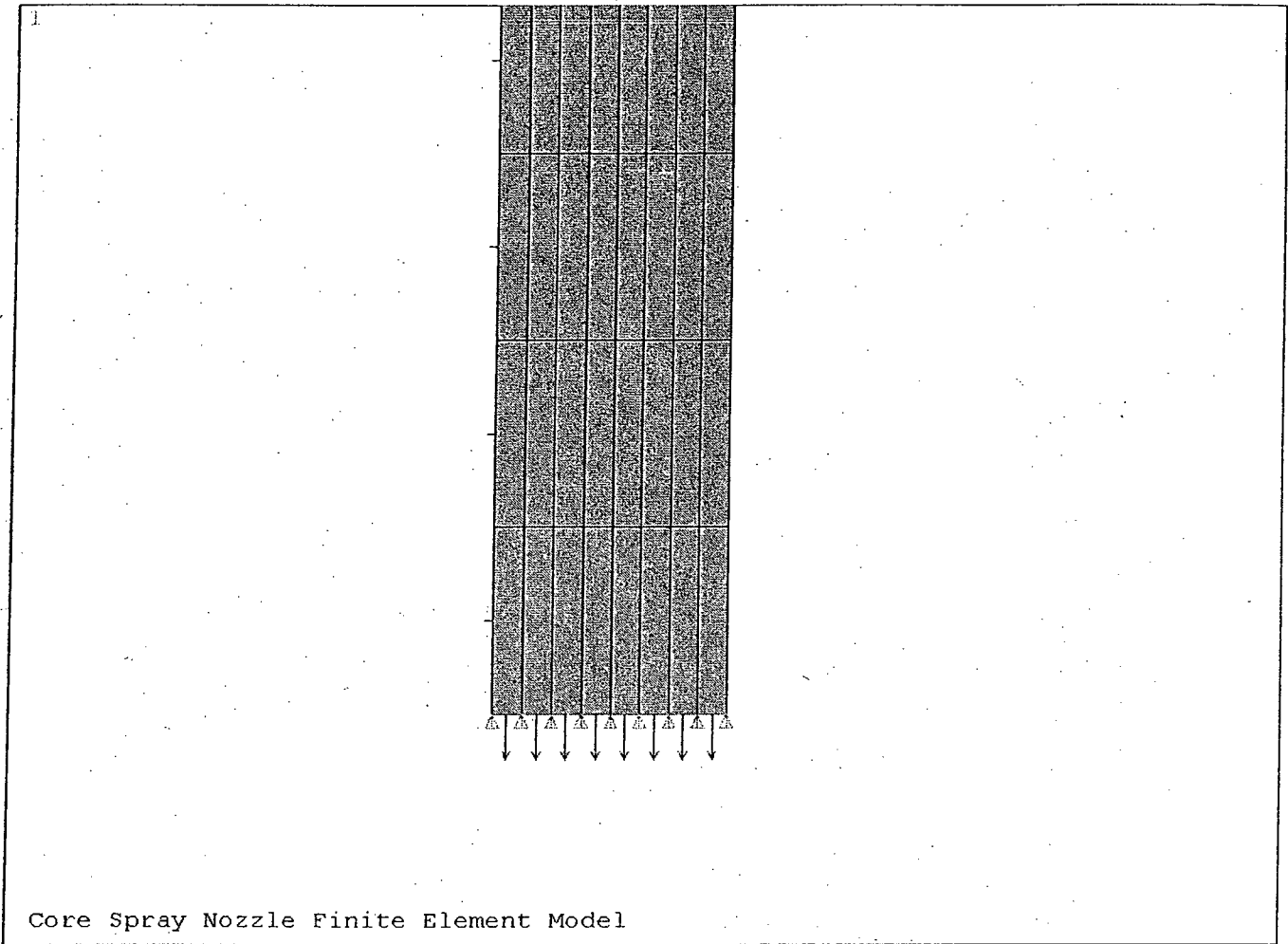


Figure 3-2: Core Spray Nozzle Pressure Cap Load

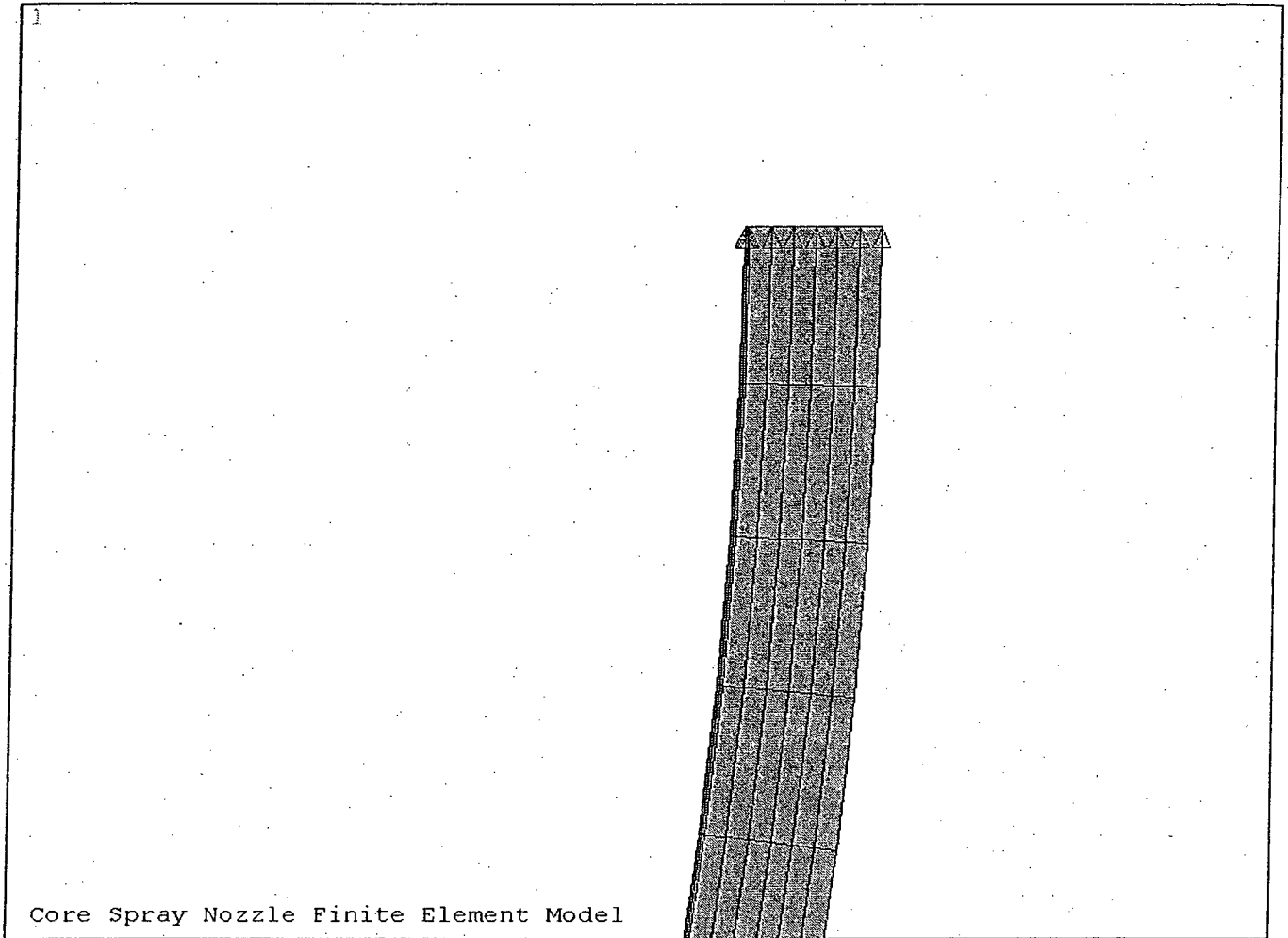


Figure 3-3: Core Spray Nozzle Vessel Boundary Condition

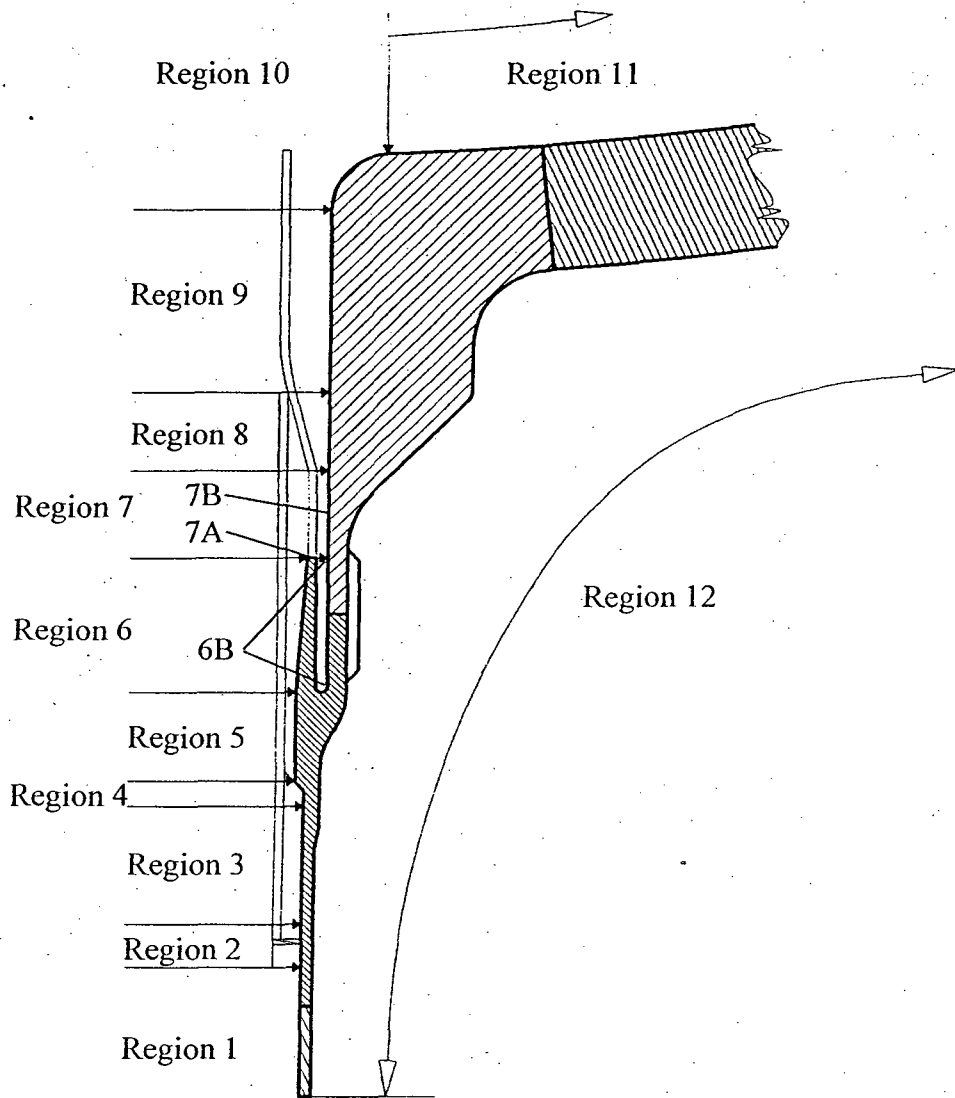


Figure 3-4: Thermal Regions

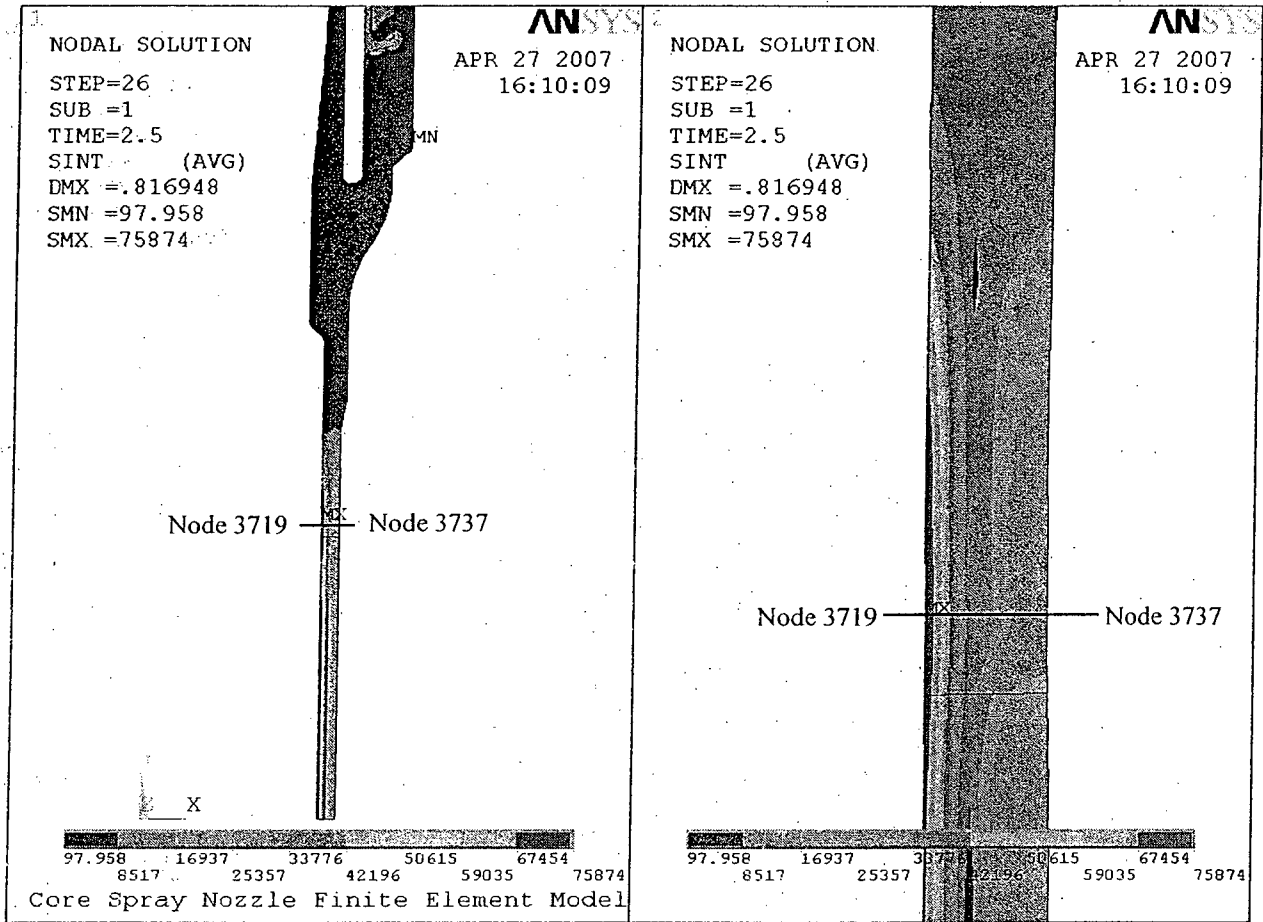


Figure 3-5: Safe End Critical Thermal Stress Location

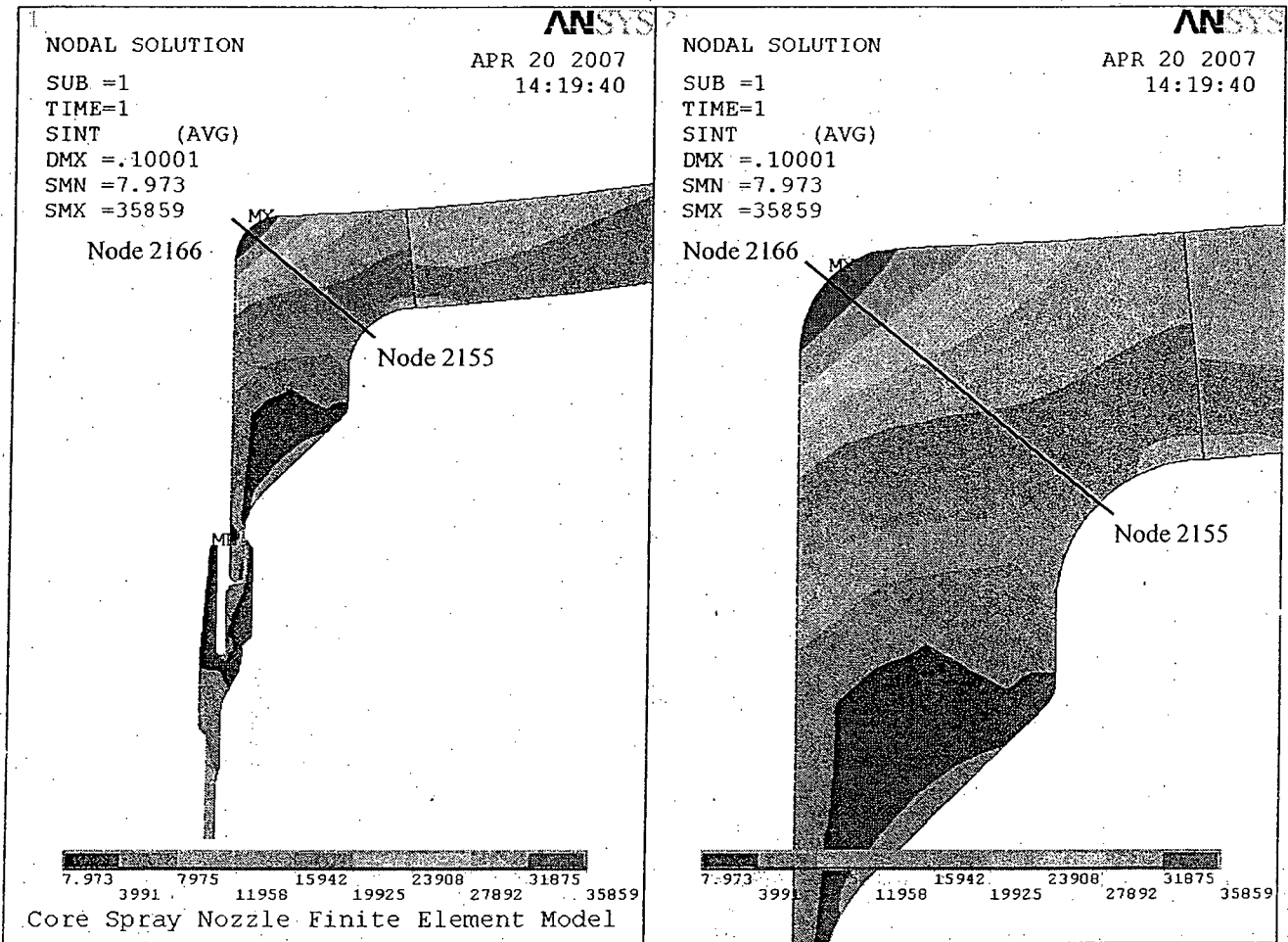


Figure 3-6: Blend Radius Critical Thermal Stress Location

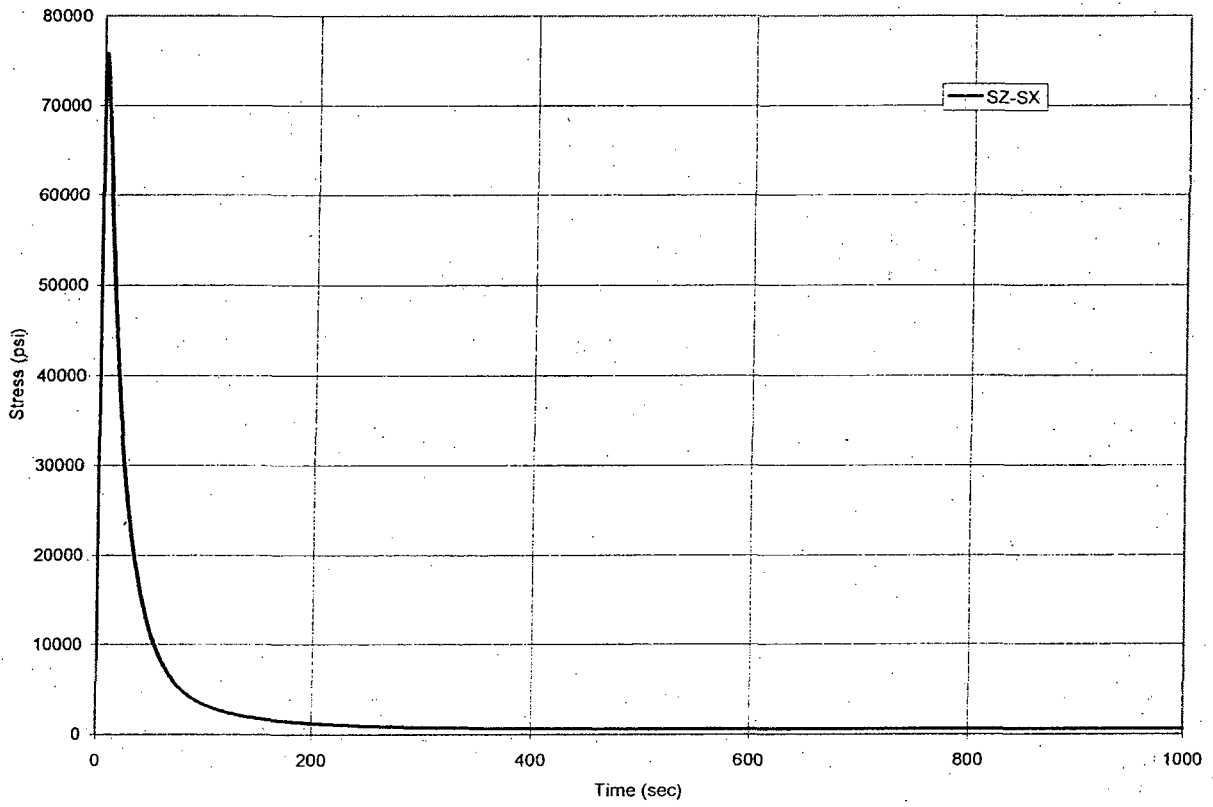


Figure 3-7: Safe End Green's Function for 100% Flow

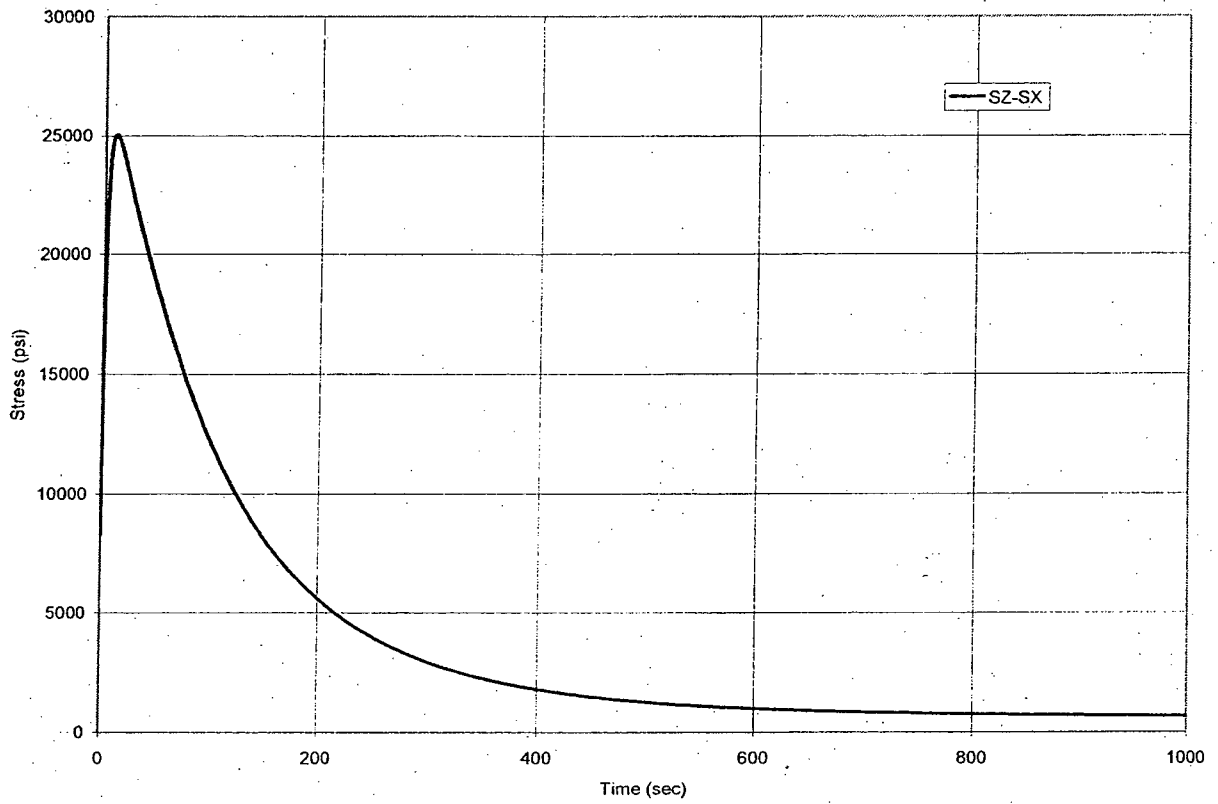


Figure 3-8: Safe End Green's Function for 0% Flow

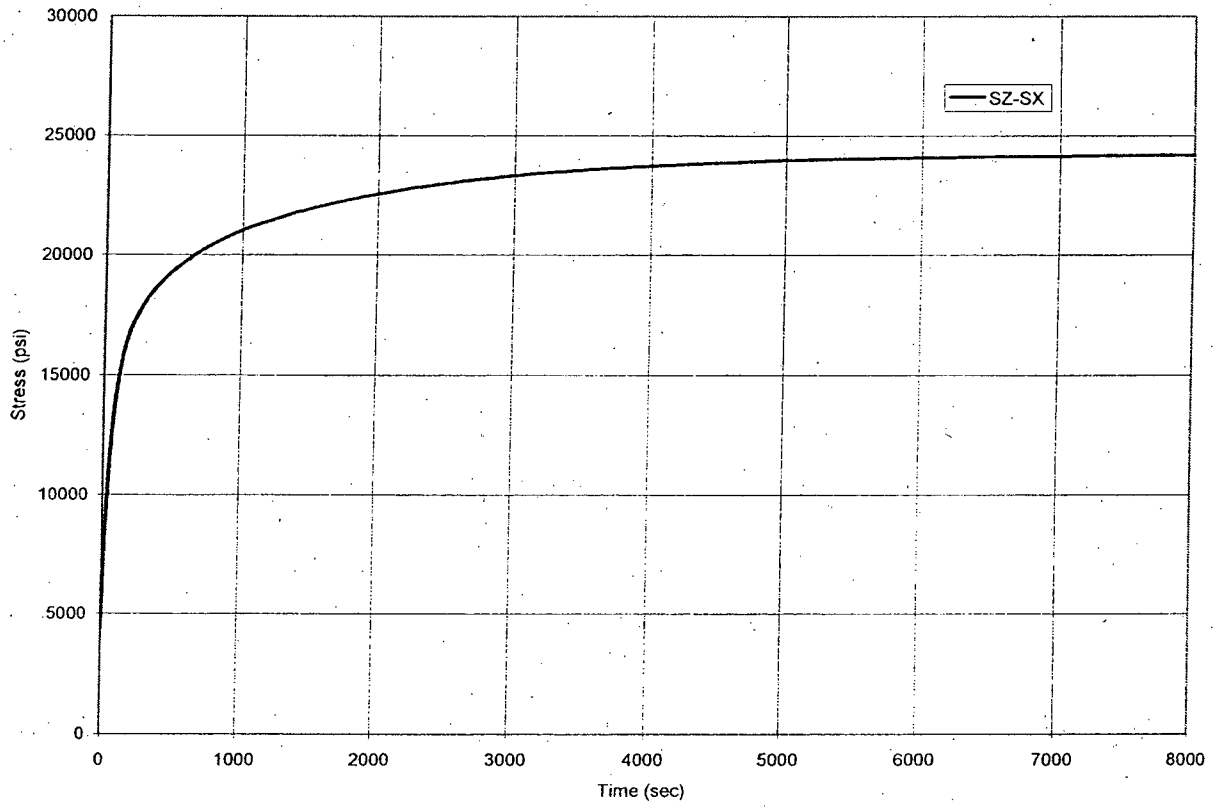


Figure 3-9: Blend Radius Green's Function for 100% Flow

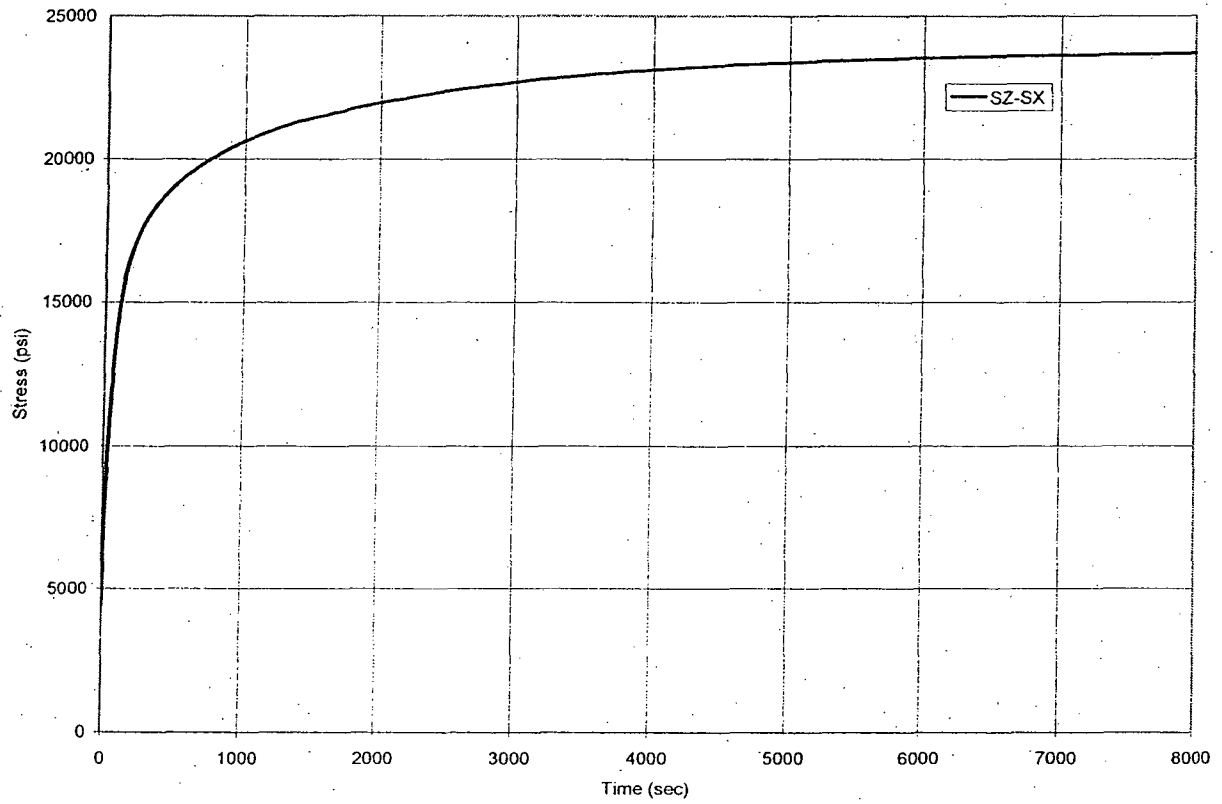


Figure 3-10: Blend Radius Green's Function for 0% Flow

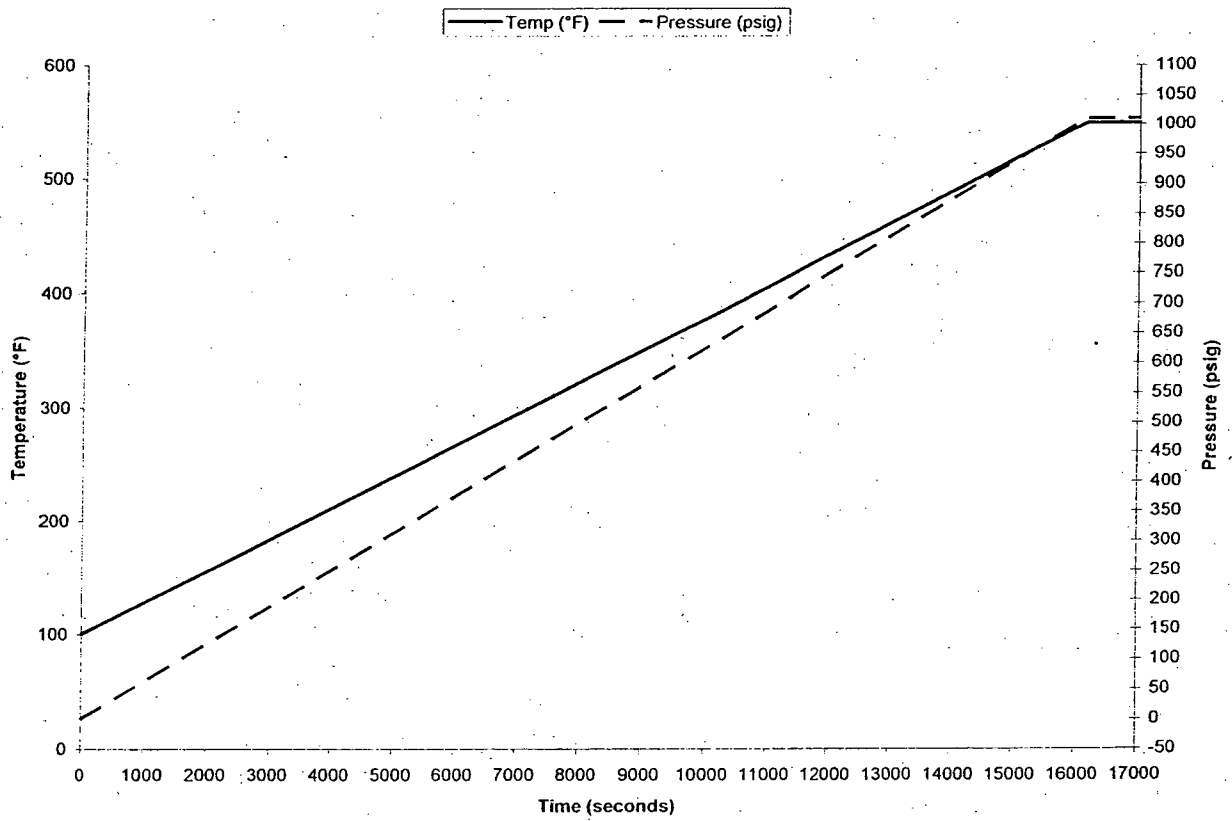


Figure 3-11: Transient 03: Startup

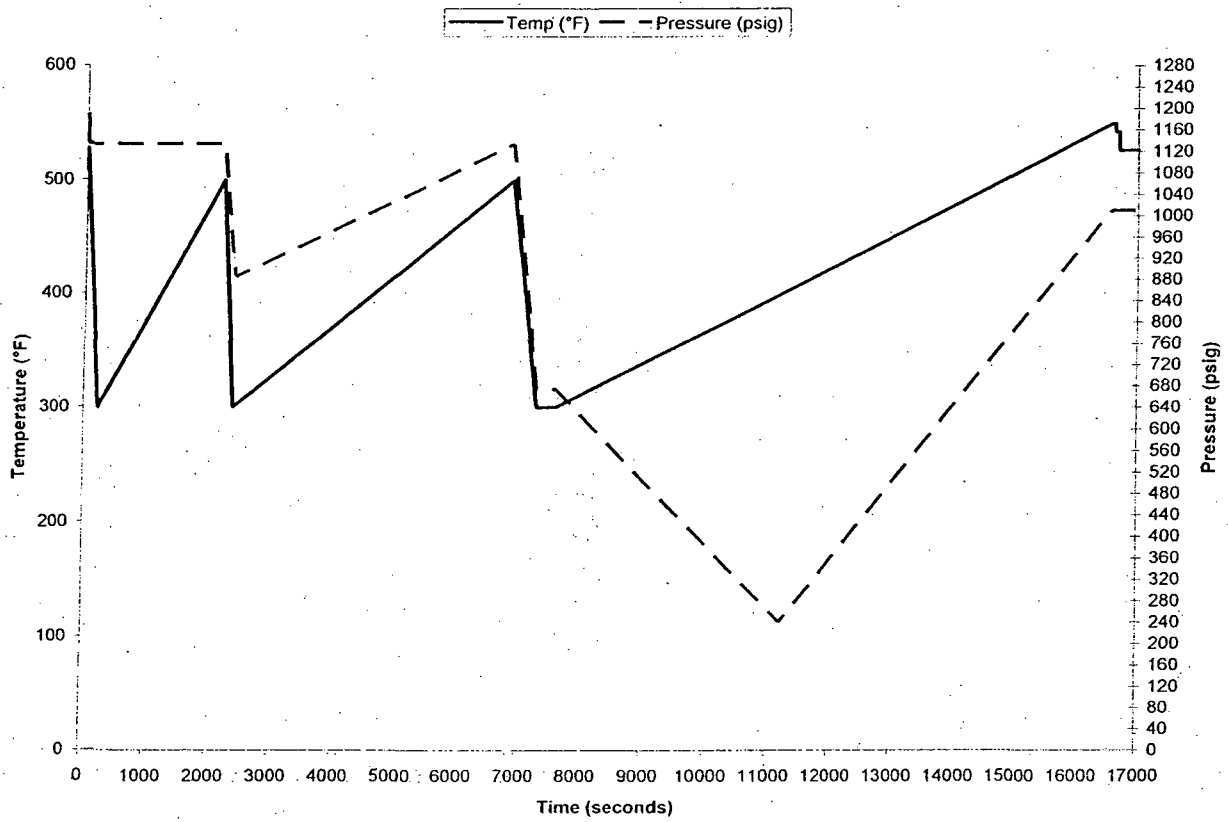


Figure 3-12: Transient 11: Loss of Feedwater Pumps, Isolation Valves Close

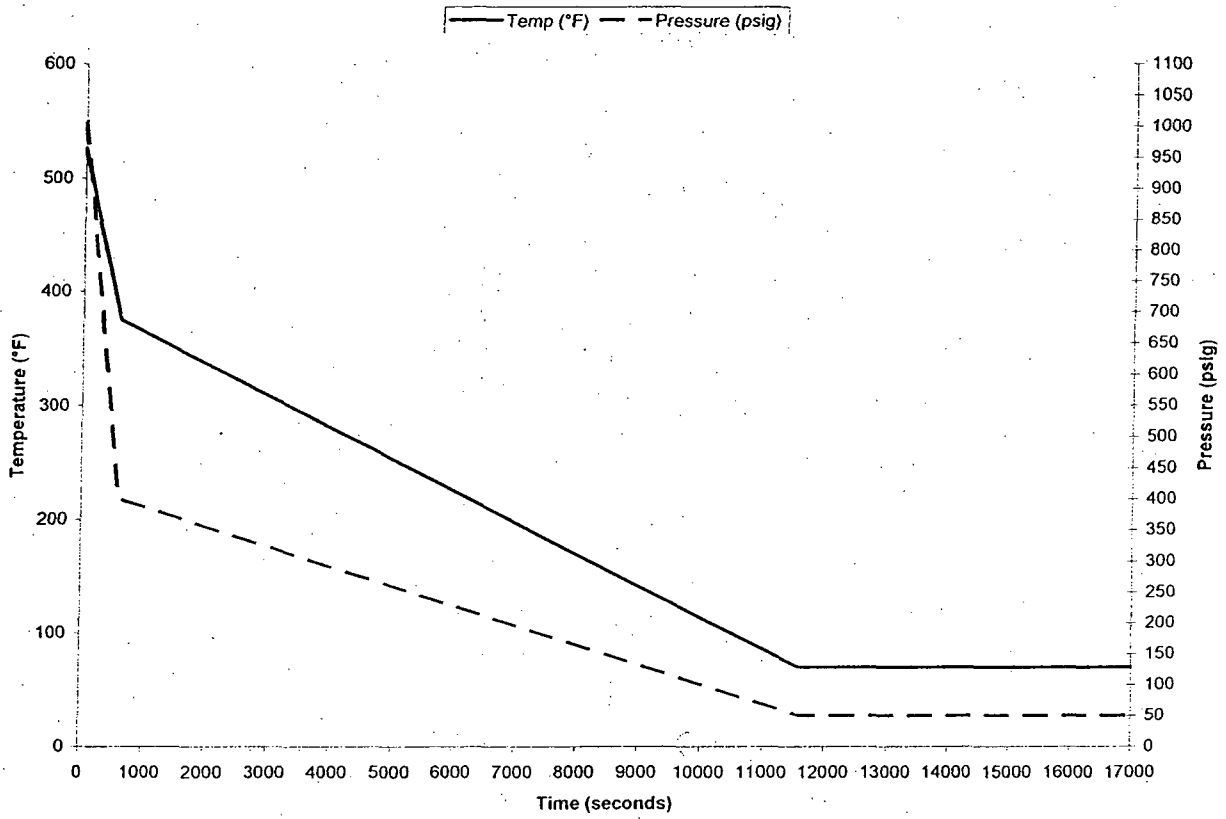


Figure 3-13: Transient 14: Single Relief of Safety Valve Blow Down

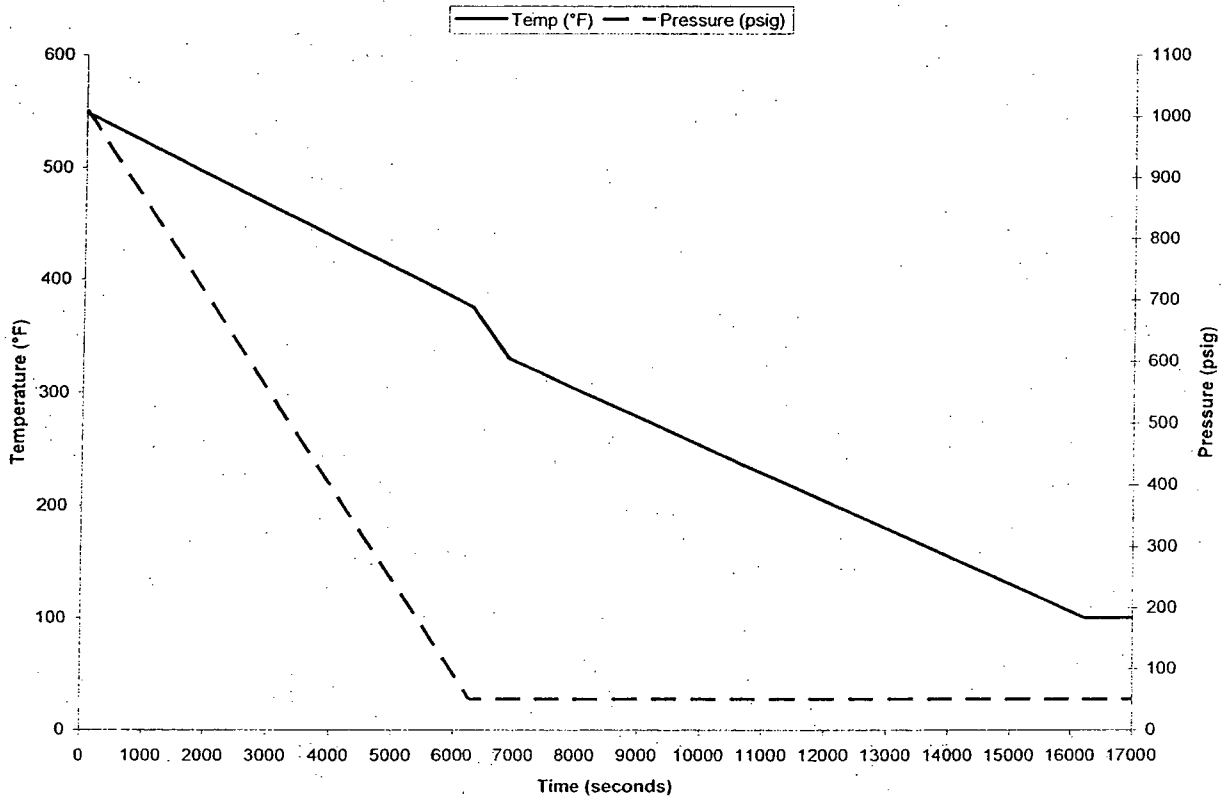


Figure 3-14: Transient 21-23: Shutdown Vessel Flooding

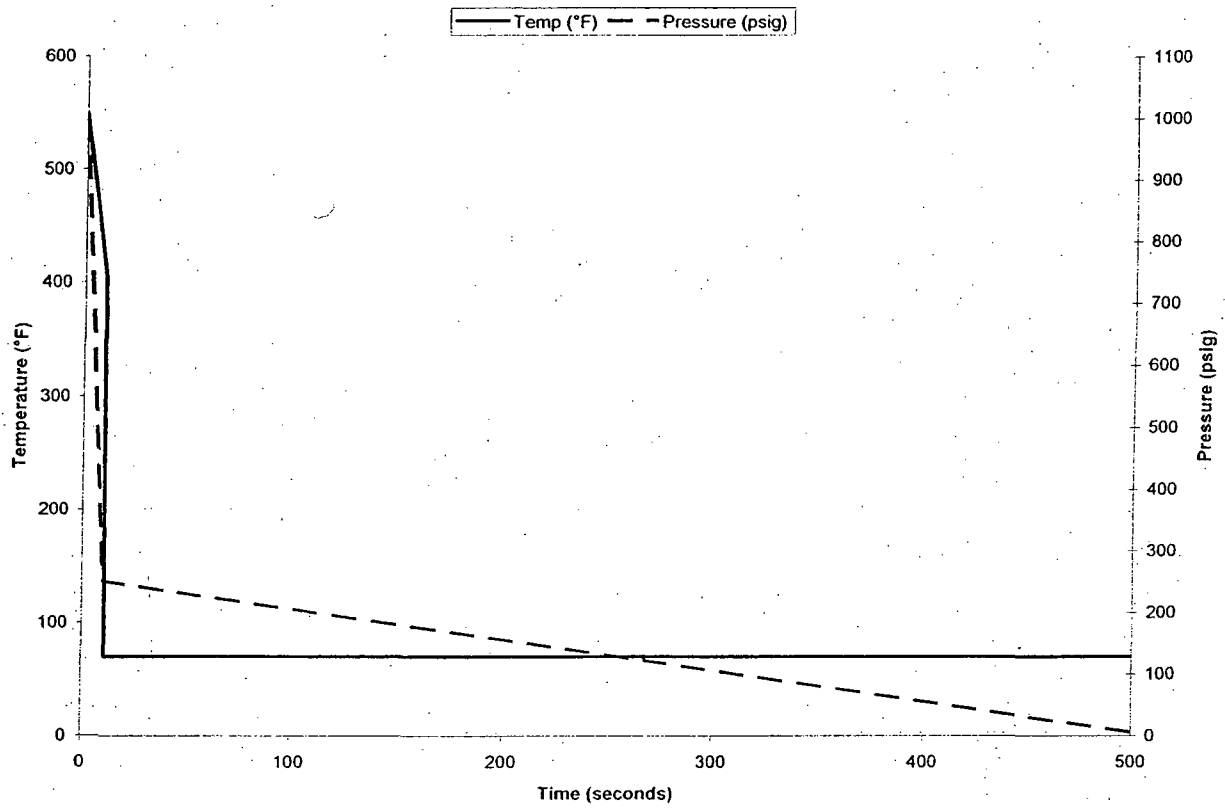


Figure 3-15: Transient 30: Emergency Shutdown 100% Flow (Safe End)

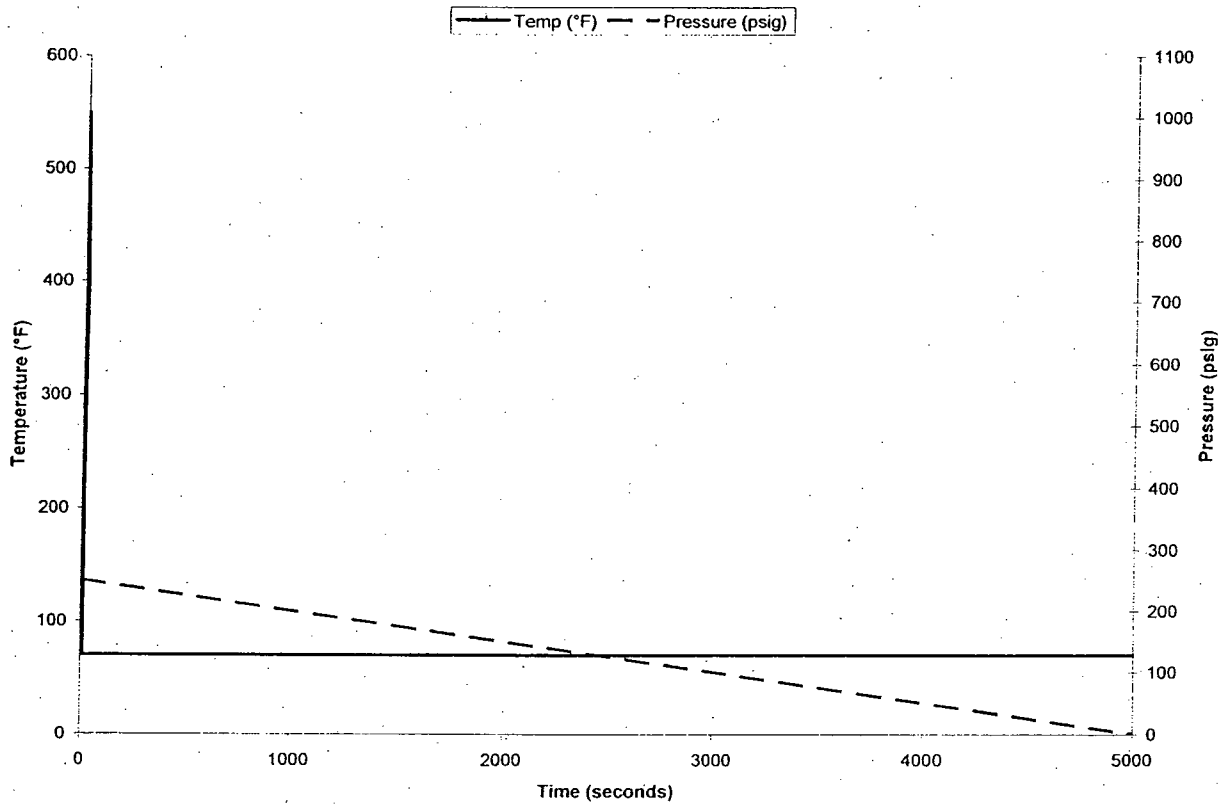


Figure 3-16: Transient 30: Emergency Shut Down 100% Flow (Blend Radius)

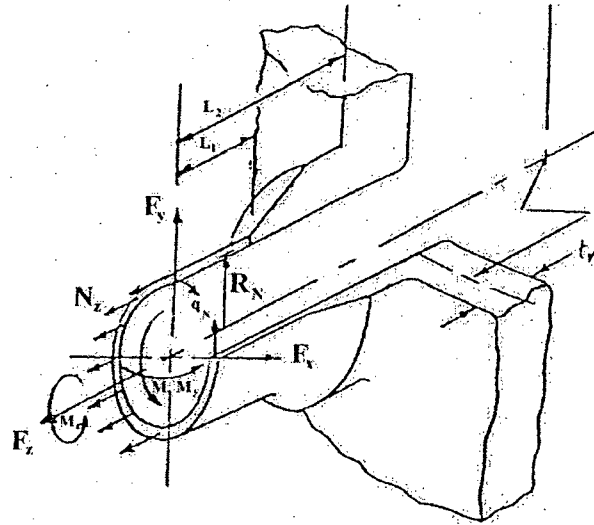


Figure 3-17: Pipe Reactions

4.0 STRESS AND FATIGUE ANALYSIS RESULTS

Fatigue calculations for the VY core spray nozzle were performed in accordance with ASME Code, Section III, Subsection NB-3200 methodology (1998 Edition, 2000 Addenda) [16].

Fatigue analysis was performed in the Reference [10] calculation for the two locations identified in Section 3.1.2, using the Green's Functions developed for these two locations and the 60-year projected cycle counts from Reference [13].

Three computer programs were used to facilitate the fatigue analysis process: STRESS.EXE, P-V.EXE, and FATIGUE.EXE. The first program, STRESS.EXE, calculates a stress history in response to a thermal transient using a Green's Function. The second program, P-V.EXE, reduces the stress history to peaks and valleys. The third program, FATIGUE.EXE, calculates fatigue from the reduced peak and valley history using ASME Code, Section III methodology. All three programs are explained in detail and were independently verified for use in the Reference [15] calculation.

In order to perform the fatigue analysis, input files with the necessary data were prepared and the three analysis programs were run. The program STRESS.EXE required the following three input files:

- **Green.dat:** This file contains the Green's Function. As discussed above, the core spray nozzle analyses utilize four Green's Functions: a membrane plus bending stress intensity Green's Function and a total stress intensity Green's Function for both the safe end and blend radius locations.
- **Green.cfg:** A configuration file containing parameters that describe the Green's Function.
- **Transnt.inp:** This file contains the input transient history defined in Tables 3-2 and 3-3.

Tables 4-1 and 4-2 show the stresses for each location that were used in the fatigue analysis. Columns 2 through 5 of Table 4-1 (for the safe end) and Table 4-2 (for the blend radius) show

the final peak and valley output. The pressure values for Column 6 in each table were determined from the transient pressures, specified in Tables 3-2 and 3-3. The pressure stress intensities from Section 3.2 were scaled appropriately for each transient case. The scaled piping stress values are shown in columns 9 and 10 of Tables 4-1 and 4-2. The piping stress intensities from Section 3.3 were scaled based on the transient case RPV fluid temperature and assuming no stress occurs at an ambient temperature of 70°F. Both of these stress intensities were then added to the thermal stress intensity peak and valley points to calculate the final stress values used for the fatigue analysis. In the case of the piping load stress intensities, the sign of the stress intensity was conservatively set to the same sign as the thermal stress intensity to ensure bounding fatigue usage results. Columns 11 and 12 of Tables 4-1 and 4-2 show the summation of all stresses for each thermal peak and valley stress point. The last column shows the number of cycles associated with each peak or valley based on the cycle counts shown in Tables 3-2 and 3-3.

The program FATIGUE.EXE performs the ASME Code peak event-pairing required to calculate a fatigue usage value. The input data for the configuration input file for FATIGUE.EXE, which is named FATIGUE.CFG, is shown in Table 4-3.

The core spray piping adjacent to the safe end was also analyzed because of its proximity to the maximum safe end thermal stress location. For this fatigue analysis, the stress results of the safe end were used with stainless steel material properties and a value of 1.8 was selected for K_t at the weld location, based on the maximum value given in ASME Code, Section III, Table NB-3681(a)-1 [16].

The results of the fatigue analysis are presented in Tables 4-4, 4-5 and 4-6 for the safe end, blend radius, and piping for 60 years, respectively.

Table 4-1: Core Spray Nozzle Safe End Stress Summary

1	2	3	4	5	6	7	8	9	10	11	12	13
Transient Number	Time (s)	Total Stress (psi)	M+B Stress (psi)	Temperature F	Pressure (psig)	Total Pressure Stress (psi)	M+B Pressure Stress (psi)	Total Piping Stress (psi)	M+B Piping Stress (psi)	Total Total Stress (psi)	Total M+B Stress (psi)	Number of Cycles (60 years)
2				100	0	0	0	413	413	413	413	120
				100	1100	13233	13222	413	413	13646	13635	120
				100	50	602	601	413	413	1014	1014	120
3	0	661	759	100	0	0	0	413	413	1074	1172	300
	17164	9240	10700	549	1010	12150	12140	6592	6592	27982	29432	300
11	0	8802	10236	526	1010	12150	12140	6276	6276	27228	28652	10
	3	8802	10236	526	1190	14316	14304	6276	6276	29393	30815	10
	13	8802	10236	526	1135	13654	13643	6276	6276	28732	30154	10
	164	11645	11598	408	1135	13654	13643	4657	4657	29956	29898	10
	672	4808	5791	344	1135	13654	13643	3775	3775	22237	23209	10
	2374	11140	10841	361	912	10971	10962	4005	4005	26116	25808	10
	2955	4722	5577	325	916	11019	11010	3509	3509	19250	20096	10
	7054	9518	10162	441	959	11537	11527	5100	5100	26155	26789	10
	7930	4491	5276	309	637	7663	7657	3287	3287	15441	16219	10
	16709	9960	11116	526	1010	12150	12140	6276	6276	28386	29532	10
14	17699	8802	10236	526	1010	12150	12140	6276	6276	27228	28652	10
	0	8802	10236	526	1010	12150	12140	6276	6276	27228	28652	1
	152	9499	10570	497	855	10286	10277	5880	5880	25664	26727	1
21-23	12580	91	95	70	50	602	601	0	0	693	696	1
	0	9242	0	549	1010	12150	12140	6592	6592	27984	18732	300
24	17224	664	0	100	50	602	601	413	413	1678	1014	300
				100	50	602	601	413	413	1014	1014	1
				100	1563	18803	18787	413	413	19216	19200	1
30				100	50	602	601	413	413	1014	1014	1
	0	9280	10800	549	1010	12150	12140	6592	6592	28022	29532	1
	13	85600	44694	162	250	3002	3000	1260	1260	89862	48953	1
	1011	12	10	70	0	0	0	0	0	12	10	1

- NOTES:
- Column 1: Transient number identification.
 - Column 2: Time during transient where a maximum or minimum stress intensity occurs from P-V.OUT output file.
 - Column 3: Maxima or minima total stress intensity from P-V.OUT output file.
 - Column 4: Maxima or minima membrane plus bending stress intensity from P-V.OUT output file.
 - Column 5: Temperature per total stress intensity.
 - Column 6: Pressure per Table 3-4.
 - Column 7: Total pressure stress intensity from the quantity (Column 6 x 12,030)/1000.
 - Column 8: Membrane plus bending pressure stress intensity from the quantity (Column 6 x 12,020)/1000.
 - Column 9: Calculated using the total external stress from Table 3-4 as 6949.94 psi*(Column 5 - 70°F)/(575°F - 70°F).
 - Column 10: Same as Column 9, but for M+B stress.
 - Column 11: Sum of total stresses (Columns 3, 7, and 9).
 - Column 12: Sum of membrane plus bending stresses (Columns 4, 8, and 10).
 - Column 13: Number of cycles for the transient (60 years).

Table 4-2: Core Spray Nozzle Blend Radius Stress Summary

1	2	3	4	5	6	7	8	9	10	11	12	13
Transient Number	Time (s)	Total Stress (psi)	M+B Stress (psi)	Temperature F	Pressure (psig)	Total Pressure Stress (psi)	M+B Pressure Stress (psi)	Total Piping Stress (psi)	M+B Piping Stress (psi)	Total Total Stress (psi)	Total M+B Stress (psi)	Number of Cycles (60 years)
2				100	0	0	0	19	19	19	19	120
				100	1100	39446	38467	19	19	39465	38486	120
				100	50	1793	1749	19	19	1812	1768	120
3	0	23700	12600	100	0	0	0	19	19	23719	12619	300
	24164	2100	3180	549	1010	36219	35320	306	306	38625	38806	300
11	0	3209	3644	526	1010	36219	35320	291	291	39719	39255	10
	3	3209	3644	526	1190	42673	41614	291	291	46174	45550	10
	526	10458	5374	330	1135	40701	39691	166	166	51325	45231	10
	2222	5488	1664	490	1122	40235	39236	268	268	45991	41168	10
	2860	11776	7444	321	911	32668	31858	160	160	44605	39462	10
	6903	5435	3621	495	1124	40307	39306	272	272	46013	43199	10
	8012	12577	6791	390	627	22484	21926	204	204	35265	28921	10
	16640	2772	4370	542	1010	36219	35320	301	301	39292	39991	10
	16991	3389	4115	526	1010	36219	35320	291	291	39899	39726	10
	24699	3209	3644	526	1010	36219	35320	291	291	39719	39255	10
14	0	3209	3644	526	1010	36219	35320	291	291	39719	39255	1
	19580	25122	13197	70	50	1793	1749	0	0	26915	14946	1
21-23	0	2103	3161	549	1010	36219	35320	306	306	38628	38787	300
	24224	23680	12568	100	50	1793	1749	19	19	25492	14336	300
24				100	50	1793	1749	19	19	1812	1768	1
				100	1563	56049	54658	19	19	56068	54677	1
				100	50	1793	1749	19	19	1812	1768	1
30	0	2040	2950	549	1010	36219	35320	306	306	38565	38576	1
	8011	25700	14900	70	0	0	0	0	0	25700	14900	1

- NOTES:
- Column 1: Transient number identification.
 - Column 2: Time during transient where a maximum or minimum stress intensity occurs from P-V.OUT output file.
 - Column 3: Maxima or minima total stress intensity from P-V.OUT output file.
 - Column 4: Maxima or minima membrane plus bending stress intensity from P-V.OUT output file.
 - Column 5: Temperature per total stress intensity.
 - Column 6: Pressure per Table 3-5.
 - Column 7: Total pressure stress intensity from the quantity (Column 6 x 35,860)/1000.
 - Column 8: Membrane plus bending pressure stress intensity from the quantity (Column 6 x 34,970)/1000.
 - Column 9: Calculated using the total external stress from Table 3-5 as 322.52 psi*(Column 5-70°F)/(575°F -70°F).
 - Column 10: Same as Column 9, but for M+B stress.
 - Column 11: Sum of total stresses (Columns 3, 7, and 9).
 - Column 12: Sum of membrane plus bending stresses (Columns 4, 8, and 10).
 - Column 13: Number of cycles for the transient (60 years).

Table 4-3: Fatigue Parameters Used in the Core Spray Nozzle Fatigue Analysis

	Blend Radius (SA508 Class II)	Safe End (N06600)	Piping (Stainless Steel)
Parameters m and n for Computing K_e	2.0 & 0.2 (low alloy steel)	1.7 & 0.3	1.7 & 0.3
Design Stress Intensity Values, S_m	26,700 psi @ 600°F	23,300 psi @ 600°F	17,000 psi @ 600°F
Elastic Modulus from Applicable Fatigue Curve	30.0×10^6 psi	28.3×10^6 psi	28.3×10^6 psi
Elastic Modulus Used in Finite Element Model (300°F)	26.7×10^6 psi	29.8×10^6 psi	27.0×10^6 psi
The Geometric Stress Concentration Factor K_t	1.0	4.0 ^{See Note 1}	1.8

Note: 1. Conservative bounding value per ASME Code, Subsection NB-3600 to conservatively cover adjacent thread and weld regions.

Table 4-4: Fatigue Results for Core Spray Nozzle Safe End

LOCATION = LOCATION NO. 1 -- SAFE END
 FATIGUE CURVE = 2 (1 = CARBON/LOW ALLOY, 2 = STAINLESS STEEL)
 m = 1.7
 n = .3
 Sm = 23300. psi
 Ecurve = 2.830E+07 psi
 Eanalysis = 2.980E+07 psi
 Kt = 4.00

MAX	MIN	RANGE	MEM+BEND	Ke	Salt	Napplied	Nallowed	U
89862.	-12.	89874.	48963.	1.000	112423.	1.000E+00	1.213E+03	.0008
29956.	413.	29543.	29485.	1.000	56029.	1.000E+01	1.910E+04	.0005
29393.	413.	28980.	30402.	1.000	57068.	1.000E+01	1.746E+04	.0006
28732.	413.	28319.	29741.	1.000	55813.	1.000E+01	1.946E+04	.0005
28386.	413.	27973.	29119.	1.000	54762.	1.000E+01	2.140E+04	.0005
28022.	413.	27609.	29119.	1.000	54590.	1.000E+00	2.174E+04	.0000
27984.	413.	27571.	18319.	1.000	39187.	7.900E+01	1.244E+05	.0006
27984.	693.	27291.	18036.	1.000	38651.	1.000E+00	1.341E+05	.0000
27984.	1014.	26970.	17718.	1.000	38045.	1.200E+02	1.460E+05	.0008
27984.	1014.	26970.	17718.	1.000	38045.	1.000E+00	1.460E+05	.0000
27984.	1014.	26970.	17718.	1.000	38045.	1.000E+00	1.460E+05	.0000
27984.	1074.	26910.	17560.	1.000	37792.	9.800E+01	1.514E+05	.0006
27982.	1074.	26908.	28260.	1.000	53033.	2.020E+02	2.517E+04	.0080
27982.	1678.	26304.	28418.	1.000	52971.	9.800E+01	2.532E+04	.0039
27228.	1678.	25550.	27638.	1.000	51502.	1.000E+01	2.919E+04	.0003
27228.	1678.	25550.	27638.	1.000	51502.	1.000E+01	2.919E+04	.0003
27228.	1678.	25550.	27638.	1.000	51502.	1.000E+00	2.919E+04	.0000
26155.	1678.	24477.	25775.	1.000	48339.	1.000E+01	4.021E+04	.0002
26116.	1678.	24438.	24794.	1.000	46923.	1.000E+01	4.673E+04	.0002
25664.	1678.	23986.	25713.	1.000	48017.	1.000E+00	4.159E+04	.0000
22237.	1678.	20559.	22195.	1.000	41379.	1.000E+01	9.257E+04	.0001
19250.	1678.	17572.	19082.	1.000	35526.	1.000E+01	2.135E+05	.0000
19216.	1678.	17538.	18186.	1.000	34234.	1.000E+00	2.691E+05	.0000
15441.	1678.	13763.	15205.	1.000	28195.	1.000E+01	1.001E+06	.0000
13646.	1678.	11968.	12621.	1.000	23661.	1.200E+02	1.772E+06	.0001

TOTAL USAGE FACTOR = .0184

Table 4-5: Fatigue Results for the Core Spray Nozzle Blend Radius

LOCATION = LOCATION NO. 2 -- BLEND RADIUS
 FATIGUE CURVE = 1 (1 = CARBON/LOW ALLOY, 2 = STAINLESS STEEL)
 m = 2.0
 n = .2
 Sm = 26700. psi
 Ecurve = 3.000E+07 psi
 Eanalysis = 2.670E+07 psi
 Kt = 1.00

MAX	MIN	RANGE	MEM+BEND	Ke	Salt	Napplied	Nallowed	U
56068.	19.	56049.	54658.	1.000	31488.	1.000E+00	1.896E+04	.0001
51325.	19.	51306.	45212.	1.000	28824.	1.000E+01	2.501E+04	.0004
46174.	19.	46155.	45531.	1.000	25930.	1.000E+01	3.460E+04	.0003
46013.	19.	45994.	43180.	1.000	25839.	1.000E+01	3.498E+04	.0003
45991.	19.	45972.	41149.	1.000	25827.	1.000E+01	3.503E+04	.0003
44605.	19.	44586.	39443.	1.000	25048.	1.000E+01	3.848E+04	.0003
39899.	19.	39880.	39707.	1.000	22404.	1.000E+01	5.695E+04	.0002
39719.	19.	39700.	39236.	1.000	22303.	1.000E+01	5.824E+04	.0002
39719.	19.	39700.	39236.	1.000	22303.	1.000E+00	5.824E+04	.0000
39465.	19.	39446.	38467.	1.000	22161.	3.800E+01	6.012E+04	.0006
39465.	1812.	37653.	36718.	1.000	21153.	8.200E+01	7.572E+04	.0011
39292.	1812.	37480.	38223.	1.000	21056.	1.000E+01	7.747E+04	.0001
38628.	1812.	36816.	37019.	1.000	20683.	2.800E+01	8.466E+04	.0003
38628.	1812.	36816.	37019.	1.000	20683.	1.000E+00	8.466E+04	.0000
38628.	1812.	36816.	37019.	1.000	20683.	1.000E+00	8.466E+04	.0000
38628.	23719.	14909.	26168.	1.000	8376.	2.700E+02	5.366E+07	.0000
38625.	23719.	14906.	26187.	1.000	8374.	3.000E+01	5.375E+07	.0000
38625.	25492.	13133.	24470.	1.000	7378.	2.700E+02	3.042E+08	.0000
38565.	25492.	13073.	24240.	1.000	7344.	1.000E+00	3.374E+08	.0000
35265.	25492.	9773.	14585.	1.000	5490.	1.000E+01	1.000E+20	.0000
26915.	25492.	1423.	610.	1.000	799.	1.000E+00	1.000E+20	.0000
25700.	25492.	208.	564.	1.000	117.	1.000E+00	1.000E+20	.0000
								=====
TOTAL USAGE FACTOR =								.0043

Table 4-6: Fatigue Results for the Core Spray Stainless Steel Piping

LOCATION = LOCATION NO. 1 -- SS Piping
 FATIGUE CURVE = 2 (1 = CARBON/LOW ALLOY, 2 = STAINLESS STEEL)
 m = 1.7
 n = .3
 Sm = 17000. psi
 Ecurve = 2.830E+07 psi
 Eanalysis = 2.700E+07 psi
 Kt = 1.80

MAX	MIN	RANGE	MEM+BEND	Ke	Salt	Napplied	Nallowed	U
89862.	-12.	89874.	48963.	1.000	67629.	1.000E+00	8.006E+03	.0001
29956.	413.	29543.	29485.	1.000	27845.	1.000E+01	1.042E+06	.0000
29393.	413.	28980.	30402.	1.000	27934.	1.000E+01	1.031E+06	.0000
28732.	413.	28319.	29741.	1.000	27310.	1.000E+01	1.110E+06	.0000
28386.	413.	27973.	29119.	1.000	26868.	1.000E+01	1.171E+06	.0000
28022.	413.	27609.	29119.	1.000	26678.	1.000E+00	1.198E+06	.0000
27984.	413.	27571.	18319.	1.000	22130.	7.900E+01	2.272E+06	.0000
27984.	693.	27291.	18036.	1.000	21864.	1.000E+00	2.392E+06	.0000
27984.	1014.	26970.	17718.	1.000	21563.	1.200E+02	2.539E+06	.0000
27984.	1014.	26970.	17718.	1.000	21563.	1.000E+00	2.539E+06	.0000
27984.	1014.	26970.	17718.	1.000	21563.	1.000E+00	2.539E+06	.0000
27984.	1074.	26910.	17560.	1.000	21465.	9.800E+01	2.588E+06	.0000
27982.	1074.	26908.	28260.	1.000	25950.	2.020E+02	1.311E+06	.0002
27982.	1678.	26304.	28418.	1.000	25700.	9.800E+01	1.354E+06	.0001
27228.	1678.	25550.	27638.	1.000	24978.	1.000E+01	1.485E+06	.0000
27228.	1678.	25550.	27638.	1.000	24978.	1.000E+01	1.485E+06	.0000
27228.	1678.	25550.	27638.	1.000	24978.	1.000E+00	1.485E+06	.0000
26155.	1678.	24477.	25775.	1.000	23634.	1.000E+01	1.779E+06	.0000
26116.	1678.	24438.	24794.	1.000	23202.	1.000E+01	1.889E+06	.0000
25664.	1678.	23986.	25713.	1.000	23351.	1.000E+00	1.850E+06	.0000
22237.	1678.	20559.	22195.	1.000	20080.	1.000E+01	3.442E+06	.0000
19250.	1678.	17572.	19082.	1.000	17209.	1.000E+01	7.481E+06	.0000
19216.	1678.	17538.	18186.	1.000	16816.	1.000E+00	8.600E+06	.0000
15441.	1678.	13763.	15205.	1.000	13588.	1.000E+01	1.000E+20	.0000
13646.	1678.	11968.	12621.	1.000	11564.	1.200E+02	1.000E+20	.0000

TOTAL USAGE FACTOR = .0005

5.0 ENVIRONMENTAL FATIGUE ANALYSIS

Environmental fatigue multipliers were computed for both normal water chemistry (NWC) and hydrogen water chemistry (HWC) conditions in Reference [17] for various regions of the VY RPV and attached piping. Based on VY-specific dates for plant startup and HWC implementation, as well as past and future predicted HWC system availability, it was determined that overall HWC availability is 47% over the sixty year operating period for VY. Therefore, for the purposes of the EAF assessment of the core spray nozzle, it was assumed that HWC conditions exist for 47% of the time, and NWC conditions exist for 53% of the time over the 60-year operating life of the plant. RPV upper region chemistry was assumed for both the core spray nozzle safe end and blend radius locations, since both locations experience reactor conditions for all times except during core spray injections, (which are rare occurrences).

For the safe end location, the environmental fatigue factors for pre-HWC and post-HWC are both 1.49 from Reference [18]. This results in an EAF adjusted CUF as follows:

$$\text{60-Year CUF, } U_{60} = 0.0184 \text{ (from Table 4-4)}$$

$$\text{Overall EAF multiplier, } F_{en} = 1.49$$

$$\text{60-Year EAF CUF, } U_{60-evn} = 0.0184 \times 1.49 = 0.0274$$

The EAF CUF value of 0.0274 for 60 years for the safe end is acceptable (i.e., less than the allowable value of 1.0).

For the stainless steel piping, the environmental fatigue factors for pre-HWC and post-HWC are both 8.36 from Table 4 of Reference [17] for the RPV upper region. This results in an EAF adjusted CUF as follows:

$$\text{60-Year CUF, } U_{60} = 0.0005 \text{ (from Table 4-6)}$$

$$\text{Overall EAF multiplier, } F_{en} = 8.36$$

$$\text{60-Year EAF CUF, } U_{60-evn} = 0.0005 \times 8.36 = 0.0042$$

The EAF CUF value of 0.0042 for 60 years for the blend radius is acceptable (i.e., less than the allowable value of 1.0).

The fatigue calculation documented in Section 4.0 for the blend radius location was performed for the nozzle base material, since cladding is structurally neglected in modern-day fatigue analyses, per ASME Code, Section III, NB-3122.3 [16]. This is also consistent with Sections 5.7.1 and 5.7.4 of NUREG/CR-6260 [1]. Therefore, the cladding was neglected and EAF assessment of the nozzle base material was performed for the blend radius location.

For the blend radius location, the environmental fatigue factors for pre-HWC and post-HWC are 11.14 and 8.82, respectively, from Table 4 of Reference [17] for the RPV upper region. This results in an EAF adjusted CUF as follows:

60-Year CUF, $U_{60} = 0.0043$ (from Table 4-5)

Overall EAF multiplier, $F_{en} = (11.14 \times 53\% + 8.82 \times 47\%) = 10.05$

60-Year EAF CUF, $U_{60-evn} = 0.0043 \times 10.05 = 0.0432$

The EAF CUF value of 0.0432 for 60 years for the blend radius is acceptable (i.e., less than the allowable value of 1.0).

6.0 CONCLUSIONS

This report documents a refined fatigue evaluation for the VY core spray nozzle. The intent of this evaluation is to use refined transient definitions and the revised cyclic transient counts for 60 years for a computation of CUF, including EAF effects, that is more refined than previously performed fatigue analyses. The fatigue-limiting locations in the core spray nozzle, safe end, and piping are included in the evaluation, to be consistent with NUREG/CR-6260 [1] needs for EAF evaluation for license renewal. The final fatigue results are considered to be a replacement to the values previously reported in the VY LRA.

The fatigue calculations for the VY core spray nozzle were performed in accordance with ASME Code, Section III, Subsection NB-3200 methodology (1998 Edition, 2000 Addenda) [16]. The stress evaluation is summarized in Section 3.0, and the fatigue analysis is summarized in Section 4.0. The results in Section 4.0 reveal that the CUF for the limiting safe end location is 0.0184, the CUF for the limiting blend radius location is 0.0043, and the CUF for the stainless steel piping is 0.0005. All of these values represent 60 years of plant operation, including all relevant EPU effects.

EAF calculations for the VY core spray nozzle were also performed, as summarized in Section 5.0. The results in Section 5.0 reveal that the EAF CUF for the limiting safe end location is 0.0274, the EAF CUF for the limiting blend radius location is 0.0432, and the EAF CUF for the stainless steel piping is 0.0042. All of these values represent 60 years of plant operation, including all relevant EPU effects.

All fatigue allowables, both with and without EAF effects, are met, thus demonstrating acceptability for 60 years of operation.

7.0 REFERENCES

1. NUREG/CR-6260 (INEL-95/0045), "Application of NUREG/CR-5999 Interim Fatigue Curves to Selected Nuclear Power Plant Components," March 1995.
2. Structural Integrity Associates Report No. SIR-86-037, Revision 0, "Evaluation of Modifications to Vermont Yankee Core Spray Nozzle and Safe End," January 1987, SI Project No. YAEC-08.
3. GE Design Specification No. 21A1115, Revision 4, "Vermont Yankee Reactor Pressure Vessel," October 21, 1969, SI File No. VY-05Q-210.
4. GE Design Specification No. 26A6019, Revision 1, "Reactor Vessel – Extended Power Uprate," August 29, 2003, SI File No. VY-05Q-236.
5. Kuo, A. Y., Tang, S. S., and Riccardella, P. C., "An On-Line Fatigue Monitoring System for Power Plants, Part I - Direct Calculation of Transient Peak Stress Through Transfer Matrices and Green's Functions," ASME PVP Conference, Chicago, 1986.
6. ANSYS, Release 8.1A1 (w/Service Pack 1), ANSYS, Inc., June 2004.
7. Structural Integrity Associates Calculation No. VY-16Q-308, Revision 0, "Core Spray Nozzle Finite Element Model."
8. American Society of Mechanical Engineers, Boiler and Pressure Vessel Code, Section II, Materials, Part D, "Properties (Customary)," 1998 Edition, 2000 Addenda.
9. Structural Integrity Associates Calculation No. VY-16Q-309, Revision 0, "Core Spray Nozzle Green's Functions."

10. Structural Integrity Associates Calculation No. VY-16Q-310, Revision 0, "Fatigue Analysis of Core Spray Nozzle."
11. Reactor Thermal Cycles, Attachment I of Entergy Design Input Record (DIR) Revision 1, EC No. 1773, Rev. 0, "Environmental Fatigue Analysis for Vermont Yankee Nuclear Power Station," 7/26/07, SI File No. VY-16Q-209.
12. Nozzle Thermal Cycles (Drain, Core Spray & Head Spray), Attachment I of Entergy Design Input Record (DIR) Revision 1, EC No. 1773, Rev. 0, "Environmental Fatigue Analysis for Vermont Yankee Nuclear Power Station," 7/26/07, SI File No. VY-16Q-209.
13. Reactor Thermal Cycles for 60 Years of Operation, Attachment I of Entergy Design Input Record (DIR) Revision 1, EC No. 1773, Rev. 0, "Environmental Fatigue Analysis for Vermont Yankee Nuclear Power Station," 7/26/07, SI File No. VY-16Q-209.
14. Vermont Yankee Drawing 5920-00024, Rev. 11, (GE Drawing No. 919D294, Sheet No. 7), "Reactor Vessel," SI File No. VY-05Q-241.
15. Structural Integrity Associates Calculation No. SW-SPVF-01Q-301, Revision 0, "STRESS.EXE, P-V.EXE, and FATIGUE.EXE Software Verification."
16. American Society of Mechanical Engineers Boiler & Pressure Vessel Code, Section III, Rules for Construction of Nuclear Facility Components, 1998 Edition, 2000 Addenda.
17. Structural Integrity Associates Calculation No. VY-16Q-303, Revision 0, "Environmental Fatigue Evaluation of Reactor Recirculation Inlet Nozzle and Vessel Shell/Bottom Head."

18. EPRI Report No. TR-105759, "An Environmental Factor Approach to Account for Reactor Water Effects in Light Water Reactor Pressure Vessel and Piping Fatigue Evaluations," December 1995.

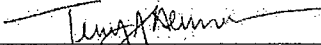
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December 2007

**Summary Report of Plant-Specific
Environmental Fatigue Analyses
for the
Vermont Yankee Nuclear Power Station**

Prepared for:
Entergy Nuclear Operations, Inc.
(Contract Order No. 10150394)

Prepared by:
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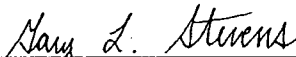
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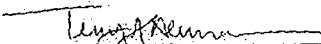
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1.0 INTRODUCTION

This report provides the results of plant-specific environmental fatigue calculations for the Vermont Yankee Nuclear Power Station (VYNPS). These calculations are performed to satisfy Nuclear Regulatory Commission (NRC) requirements for Entergy Nuclear Vermont Yankee's (ENVY's) License Renewal Application for VYNPS, submitted to the NRC in 2006.

Generic Safety Issue (GSI) 166 [1], later renumbered as GSI-190 [2], was identified by the NRC staff because of concerns about the effects of reactor water environments on fatigue life during the period of extended operation [3]. GSI-190 was closed in December 1999, based on a memorandum from NRC-RES to NRC-NRR [4]. Timing of issue closure required the first two license renewal applicants – Baltimore Gas & Electric Company for the Calvert Cliffs Nuclear Power Plant and Duke Energy for the Oconee Nuclear Station – to address GSI-190 in their applications prior to issue closure. Each of the applicants developed responses to the NRC staff without the benefit of information from GSI-190 closure. Subsequent license renewal applicants have had the benefit of this information that could be used to guide the resolution of the fatigue design basis and time limited aging analyses (TLAA) issues.

This report addresses VYNPS reactor water environmental effects on the fatigue life of selected fatigue-sensitive reactor coolant system (RCS) components, in accordance with the resolution of GSI-190, as required by Chapter X, "Time Limited Aging Analyses Evaluation of Aging Management Programs Under 10CFR54.21(c)(1)(iii), Section X.M1 "Metal Fatigue of Reactor Coolant Pressure Boundary", of the Generic Aging Lessons Learned (GALL) Report [5]. Consistent with the requirements of the GALL report, the method chosen for this environmentally-assisted fatigue (EAF) evaluation is based on evaluation of the locations identified in NUREG/CR-6260 [6] and the NRC-accepted EAF relationships generated from laboratory data, as documented in References [7] and [8].

2.0 BACKGROUND

As a part of the NRC's Fatigue Action Plan [3], incorporation of environmental fatigue effects originally involved a reduced set of fatigue design curves, such as those proposed by Argonne National Laboratory (ANL) in NUREG/CR-5999 [9]. As a part of the effort to close GSI-166 (later GSI-190) for operating nuclear power plants during the current 40-year licensing term, Idaho National Engineering Laboratory (INEL) evaluated fatigue-sensitive component locations at plants designed by all four U. S. nuclear steam supply system (NSSS) vendors. The ANL fatigue curves were used by INEL to recalculate the cumulative usage factors (CUFs) for fatigue-sensitive component locations in early and late vintage Combustion Engineering (CE) pressurized water reactors (PWRs), early and late vintage Westinghouse PWRs, early and late vintage General Electric (GE) boiling water reactors (BWRs), and Babcock & Wilcox Company (B&W) PWRs. The results of the INEL calculations were published in NUREG/CR-6260 [6]. The INEL calculations took advantage of conservatisms present in governing ASME Code fatigue calculations, including the numbers of actual plant transients relative to the numbers of design-basis transients, but did not recalculate stress ranges based on actual plant transient profiles. The BWR calculations, especially the early-vintage GE BWR calculations, are directly relevant to VYNPS.

The fatigue-sensitive component locations chosen for the older-vintage GE BWR plant were: (1) the reactor vessel shell and lower head, (2) the reactor vessel feedwater nozzle, (3) the reactor recirculation piping (including the reactor inlet and outlet nozzles), (4) the core spray line reactor vessel nozzle and associated Class 1 piping, (5) the residual heat removal (RHR) return line Class 1 piping, and (6) the feedwater line Class 1 piping. For the recirculation, RHR, and feedwater piping locations, INEL performed representative design-basis fatigue calculations. This is because no CUF calculations had originally been performed since the piping systems for the selected BWR plant were initially designed and analyzed in accordance with the criteria of USAS B31.1-1967 [10].

The six RCS component locations described above are evaluated for EAF effects for VYNPS in this report through separate plant-specific analyses of nine VY component locations (with report section numbers indicated): the reactor pressure vessel (RPV) shell and lower head (3.1); the RPV shell at the shroud support junction (3.1); the feedwater nozzle (3.2); the recirculation / residual heat removal Class 1 piping (3.3.1 and 3.5); the recirculation inlet nozzle forging (3.3.2); the recirculation inlet nozzle safe end (3.3.2); the recirculation outlet nozzle forging (3.3.3); the core spray nozzle, safe end, and Class 1 piping (3.4); and the feedwater Class 1 piping (3.6).

The calculations reported in NUREG/CR-6260 were based on the interim reduced fatigue design curves given in NUREG/CR-5999 [9]. Such an approach penalizes the component location fatigue analysis unnecessarily, because research has shown that a combination of environmental conditions is required before reactor water environmental effects become pronounced. The strain rate must be sufficiently low and the strain range must be sufficiently high to cause continuing rupture of the passivation layer that protects the exposed surface area. Temperature, dissolved oxygen content, metal sulfur content, and water flow rate are additional variables to be considered. In order to take these parameters into consideration, EPRI and GE jointly developed a method, called the F_{en} approach [11], which permits reactor water environmental effects to be applied selectively, as justified by parameter combinations.

In 1999, the NRC staff raised a number of issues relative to the use of the EPRI/GE methodology in various industry applications. Those issues, coupled with more recent laboratory fatigue data in simulated LWR reactor water environments generated by ANL for carbon and low-alloy steels and stainless steels, resulted in a revised F_{en} methodology, as published in NUREG/CR-6583 [7] for carbon and low alloy steels, and NUREG/CR-5704 [8] for stainless steels. The methodology documented in these reports was used to evaluate environmental effects for VYNPS components, as described in Section 3.0 of this report.

3.0 ENVIRONMENTAL FATIGUE CALCULATIONS

Section 2.0 identifies the locations evaluated in NUREG/CR-6260 for the older vintage GE plant, which corresponds to VYNPS. NUREG/CR-6260 provided an assessment of these six selected component locations with respect to environmental fatigue using the older reduced environmental fatigue curves. Potential reactor water environmental effects are evaluated using the updated F_{en} methodology on a plant-specific basis in this subsection, in order to address the associated effects on fatigue as required by the GALL Report [5].

For each of the components identified in Section 2.0, environmental fatigue calculations were performed. The details of these calculations are documented in the Reference [12, 17, 18, 21, 22 and 24] calculations. The calculations were carried out using the appropriate methodology contained in NUREG/CR-6583 for carbon/low alloy steel material, and in NUREG/CR-5704 for stainless steel material. This methodology is as follows:

$$\begin{aligned} \text{For Carbon Steel [7]:} \quad F_{en} &= \exp(0.585 - 0.00124T' - 0.101 S^* T^* O^* \epsilon^*) \\ &= \exp(0.554 - 0.101 S^* T^* O^* \epsilon^*) \end{aligned}$$

$$\begin{aligned} \text{For Low Alloy Steel [7]:} \quad F_{en} &= \exp(0.929 - 0.00124T' - 0.101 S^* T^* O^* \epsilon^*) \\ &= \exp(0.898 - 0.101 S^* T^* O^* \epsilon^*) \end{aligned}$$

Note that the above expressions have been corrected as summarized in Reference [23].

where:	F_{en}	=	fatigue life correction factor
	T'	=	25°C (NUREG/CR-6583, Section 6, F_{en} relative to air)
	S^*	=	S for $0 < \text{sulfur content}, S \leq 0.015 \text{ wt. } \%$
		=	0.015 for $S > 0.015 \text{ wt. } \%$
	T^*	=	0 for $T < 150^\circ\text{C}$
		=	$(T - 150)$ for $150 \leq T \leq 350^\circ\text{C}$
	T	=	fluid service temperature ($^\circ\text{C}$)
	O^*	=	0 for dissolved oxygen, $\text{DO} < 0.05 \text{ parts per million (ppm)}$
		=	$\ln(\text{DO}/0.04)$ for $0.05 \text{ ppm} \leq \text{DO} \leq 0.5 \text{ ppm}$
		=	$\ln(12.5)$ for $\text{DO} > 0.5 \text{ ppm}$

$$\begin{aligned}\dot{\epsilon}^* &= 0 \text{ for strain rate, } \dot{\epsilon} > 1\%/sec \\ &= \ln(\dot{\epsilon}) \text{ for } 0.001 \leq \dot{\epsilon} \leq 1\%/sec \\ &= \ln(0.001) \text{ for } \dot{\epsilon} < 0.001\%/sec\end{aligned}$$

For Types 304 and 316 Stainless Steel [8]: $F_{en} = \exp(0.935 - T^* \dot{\epsilon}^* O^*)$

where:

$$\begin{aligned}F_{en} &= \text{fatigue life correction factor} \\ T &= \text{fluid service temperature (}^\circ\text{C)} \\ T^* &= 0 \text{ for } T < 200^\circ\text{C} \\ &= 1 \text{ for } T \geq 200^\circ\text{C} \\ \dot{\epsilon}^* &= 0 \text{ for strain rate, } \dot{\epsilon} > 0.4\%/sec \\ &= \ln(\dot{\epsilon}/0.4) \text{ for } 0.0004 \leq \dot{\epsilon} \leq 0.4\%/sec \\ &= \ln(0.0004/0.4) \text{ for } \dot{\epsilon} < 0.0004\%/sec \\ O^* &= 0.260 \text{ for dissolved oxygen, DO } < 0.05 \text{ parts per million (ppm)} \\ &= 0.172 \text{ for DO } \geq 0.05 \text{ ppm}\end{aligned}$$

Bounding F_{en} values are determined or, where necessary, computed for each load pair in a detailed fatigue calculation. The environmental fatigue is then determined as $U_{env} = (U) (F_{en})$, where U is the original fatigue usage, and U_{env} is the EAF usage factor.

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Since implementation of HWC in 2003, VYNPS's availability has exceeded 98.5% and the objective for future HWC system availability is a minimum of 99% [12]. With these considerations, the overall availability for HWC since implementation at VYNPS until the end of the 60-year operating period was estimated at 98.5%.

Some nozzles, (e.g., recirculation outlet nozzle) have three materials: a Ni-Cr-Fe dissimilar metal weld (DMW), a low alloy steel forging, and a stainless steel safe end. To ensure the maximum CUF considering environmental effects was identified, locations in both the safe end and nozzle forging were selected. This selection produces bounding environmental fatigue results for the entire nozzle assembly for the following reasons:

- The highest thermal stresses from the finite-element model (FEM) analysis occur in the stainless steel safe end. Stainless steel F_{en} multipliers at VYNPS are significantly higher than Ni-Cr-Fe multipliers (F_{en} values are 2.55 or higher for stainless steel [12] vs. a constant value of 1.49 for Ni-Cr-Fe [11]). Therefore, evaluation of the safe end bounds the Ni-Cr-Fe weld material.
- The highest pressure stresses from the FEM analysis occur in the low alloy steel nozzle forging. Low alloy steel F_{en} multipliers at VYNPS are higher than Ni-Cr-Fe multipliers (F_{en} values are 2.45 or higher for low alloy steel [12] vs. a constant value of 1.49 for Ni-Cr-Fe [11]). Therefore, evaluation of the nozzle forging bounds the Ni-Cr-Fe weld material.

The number of cycles for forty years was adjusted based on the number of cycles actually experienced by the plant, projected out to 60 years of operation [14]. In addition, VYNPS has implemented extended power uprate (EPU). These effects have been incorporated into the evaluations documented in this report. With the use of this information, the CUF values documented in this report are applicable for 60 years of operation.

The environmental fatigue calculations are shown in Tables 3-1 through 3-9 and summarized in Table 3-10. Component-specific details are provided in the subsections that follow.

3.1 Reactor Vessel Shell and Lower Head

The environmental fatigue calculations for the reactor vessel shell and lower head location are shown in Table 3-1. The limiting CUF value reported in the VY LRA for the RPV shell/bottom

head location corresponds to a point located on the outside surface of the RPV bottom head at the junction with the support skirt. Therefore, this location is not exposed to the reactor coolant, and EAF effects do not apply. Based on this, evaluation of the limiting location along the inside surface of the RPV bottom head was performed.

The calculations shown in Table 3-1 are for the RPV lower head at the area with the highest alternating stress, which represents the limiting RPV bottom head location [12]. Reference [15] is the governing stress report for this low alloy steel location. The design fatigue calculation for the limiting RPV lower head location is reproduced in Table 3-1. The effects of EPU as well as conservative cycle counts for 60 years of plant operation are incorporated in this table. The final results in Table 3-1 show an EAF adjusted CUF of 0.0809 for 60 years, which is acceptable (i.e., less than the allowable value of 1.0).

The calculations shown in Table 3-2 are for the RPV shell at the RPV shell junction to the shroud support plate, which represents the limiting RPV shell location exposed to the reactor coolant [12]. Reference [16] is the governing stress report for this low alloy steel location. The design fatigue calculation for the limiting RPV shell location is reproduced in Table 3-2, which considers the effects of EPU and conservative cycle counts were used for 60 years of plant operation. The final results in Table 3-2 show an EAF adjusted CUF of 0.7364 for 60 years, which is acceptable (i.e., less than the allowable value of 1.0).

3.2 Reactor Vessel Feedwater Nozzle

The environmental fatigue calculations for the reactor vessel feedwater nozzle location are summarized in Table 3-3. The calculations summarized in Table 3-3 show both the blend radius, which represents the limiting feedwater nozzle location, and the safe end. Reference [17] contains the governing fatigue calculation for this location. Upper RPV region chemistry was assumed for the feedwater nozzle blend radius location, since this location is exposed to the reactor water chemistry in this region, whereas feedwater line chemistry was assumed for the safe end location.

The governing fatigue calculation for the limiting feedwater nozzle locations includes the effects of EPU and cycle counts for 60 years of operation obtained from Attachment 1 of Reference [14]. The blend radius cumulative usage factor (CUF) from system cycling is 0.0636 for 60 years. The safe end CUF is 0.1471 for 60 years. Although the carbon steel safe end has a higher CUF prior to considering environmental effects, the environmental multiplier from Table 3-3 results in a higher CUF at the low alloy steel blend radius. For the safe end location, the EAF adjusted CUF is 0.2560 for 60 years. For the blend radius location, EAF adjusted CUF is 0.6392 for 60 years, which is acceptable (i.e., less than the allowable value of 1.0).

3.3 Reactor Recirculation Piping (Including the Reactor Inlet and Outlet Nozzles)

Three locations were identified for the reactor recirculation piping in NUREG/CR-6260: the reactor vessel nozzle (includes both the inlet and outlet nozzles), and the recirculation piping. The evaluations for each of these components are described in the following subsections.

3.3.1 Reactor Recirculation Piping

Two locations (both stainless steel) were identified for VY for the reactor recirculation/RHR piping that correspond to the equivalent locations to those identified in NUREG/CR-6260: the RHR return tee connection to the recirculation piping, and the valve to pipe weld at the RHR isolation valve. Reference [18] contains the governing fatigue calculations for these locations. These analyses determined the limiting location to be at the RHR return tee.

The environmental fatigue calculations for the limiting recirculation/RHR piping location is summarized in Table 3-4, which includes the effects of EPU and cycle counts for 60 years of plant operation.

A review of the shutdown cooling mode of operation since the time of recirculation piping replacement in 1986 was performed by VYNPS, and the number of cycles per loop was conservatively estimated to be 150 through Year 60 [14]. Based on this, the cycle counts for the SIR-07-132-NPS, Rev. 1



Recirculation piping were reduced by a factor of 150/300 (50%) for all transients with the exception of transients that have fewer than 10 transient cycles. To ensure this cycle reduction adequately considered the potential impact on the RHR piping, the full number of transient cycles listed in Attachment 1 of Reference [14] was initially applied to the PIPESTRESS model and the highest CUF for the RHR piping was lower than the value obtained for the recirculation piping with reduced cycles.

Due to replacement of the recirculation piping, HWC conditions exist for 39% of the time, and NWC conditions exist for 61% of the time. This is based on 17.5 years of operation with NWC between March 1986 when the piping was replaced and November 2003 when HWC was implemented, and 46 years of operation from March 1986 to the end of the period of extended operation in March 2032. Using the bounding EAF multipliers (8.36 for HWC and 15.35 for NWC) [12], the overall multiplier is 12.62. Applying this to the 60-Year CUF of 0.0590 results in a total environmentally assisted CUF of 0.7446.

3.3.2 Reactor Recirculation Inlet Nozzle

References [15, 19 and 20] are the applicable stress reports for this location. An evaluation was performed for both the inlet nozzle forging (low alloy steel) and the safe end (stainless steel).

The environmental fatigue calculations for the recirculation inlet nozzle forging location are shown in Table 3-5. The governing fatigue calculation for the recirculation inlet nozzle location is reproduced in Table 3-5 [12], which includes the effects of EPU and cycle counts for 60 years of plant operation from Attachment 1 of Reference [14]. The final results show an EAF adjusted CUF of 0.5034 for 60 years, which is acceptable (i.e., less than the allowable value of 1.0).

The environmental fatigue calculations for the recirculation inlet nozzle safe end location are shown in Table 3-6. The governing fatigue calculation for the recirculation inlet nozzle location is reproduced in Table 3-6 [12], which includes the effects of EPU and cycle counts for 60 years

of plant operation from Attachment 1 of Reference [14]. The final results show an EAF adjusted CUF of 0.0199 for 60 years, which is acceptable (i.e., less than the allowable value of 1.0).

3.3.3 Reactor Recirculation Outlet Nozzle

The recirculation outlet nozzle was evaluated for environmental fatigue effects. Reference [24] is the fatigue calculation for this location. An evaluation was performed for both the outlet nozzle safe end (stainless steel) and the nozzle inner corner blend radius (low alloy steel). The results for the limiting nozzle forging location are reported here.

The environmental fatigue calculations for the limiting recirculation outlet nozzle forging blend radius location are shown in Table 3-7 [24], which includes the effects of EPU and cycle counts for 60 years of plant operation from Attachment 1 of Reference [14]. The final results in Table 3-7 show an EAF adjusted CUF of 0.0836 for 60 years, which is acceptable (i.e., less than the allowable value of 1.0).

3.4 Core Spray Line Reactor Vessel Nozzle and Associated Class 1 Piping

Locations that were evaluated in NUREG/CR-6260 included the reactor vessel nozzle blend radius (low alloy steel), the reactor vessel nozzle safe end (Alloy 600) and the core spray piping (stainless steel).

Reference [21] is the applicable fatigue calculation for these locations, which shows the nozzle limiting location to be the blend radius. The design fatigue calculations for the limiting location at the core spray nozzle, safe end, and piping are summarized in Table 3-8 [21], which include the effects of EPU and cycle counts for 60 years of plant operation from Attachment 1 of Reference [14]. The cumulative fatigue usage, prior to considering environmental effects for the blend radius, is 0.0166. Factoring in the environmental multiplier from Table 3-8 [12], the EAF adjusted CUF is 0.1668 for 60 years, which is acceptable (i.e., less than the allowable value of 1.0).



3.5 RHR Return Line Class 1 Piping

The environmental fatigue calculations for the RHR return line Class 1 piping are covered by the calculations in Subsection 3.3.1 above.

3.6 Feedwater Line Class 1 Piping

The environmental fatigue calculation for the limiting feedwater Class 1 piping location (carbon steel) is summarized in Table 3-9. The calculations shown in Table 3-9 are for the limiting feedwater Class 1 piping location. Per Reference [22], the limiting total fatigue usage for the analyzed feedwater/high pressure coolant injection (HPCI) piping system occurs on the riser to the RPV feedwater nozzle N4B. The limiting fatigue usage value for the feedwater Class 1 piping location is 0.1661, which includes the effects of EPU and cycle counts for 60 years of plant operation from Attachment 1 of Reference [14]. The final results in Table 3-9 show the EAF adjusted CUF of 0.2890 for 60 years, which is acceptable (i.e., less than the allowable value of 1.0).

3.7 Summary of Results

The results of the calculations contained in Tables 3-1 through 3-9 are summarized in Table 3-10.

It is noteworthy that the CUF results presented in this section include uniformly applied environmental effects without consideration of threshold criteria that might indicate an absence of conditions that would lead to environmental fatigue effects. Furthermore, conservative values were applied for temperature, strain rate and metal sulfur content in calculating environmental multipliers. Therefore, the environmental adjustments to the CUF results are considered to be conservative.

Table 3-1. Environmental Fatigue Evaluation for the Reactor Vessel Shell

Component: RPV Shell/Bottom Head
 NUREG/CR-6260 CUF: 0.032 (for reference only)
 Reference: NUREG/CR-6260, p. 5-102
 Stress Report CUF: 0.0057 (for Point 14, see below)
 Material: Low Alloy Steel (Material = A-533 Gr. B)

Design Basis CUF Calculation for 40 years:

$E_{\text{fatigue curve}}/E_{\text{analysis}} = 1.149$ Conservatively used minimum E of 26.1 from Section S2 Appendix of RPV Stress Report.
 Power Uprate = 1.0067 $= (549 - 100) / (546 - 100)$ per 4.4.1.b of 26A6019, Rev. 1
 $K_1 = 1.000$ stress concentration factor
 $m = 2.0$ NB-3228.5 of ASME Code, Section III
 $n = 0.2$ NB-3228.5 of ASME Code, Section III
 $S_m = 26,700$ psi (ASME Code, Section II, Part D)

$P_L + P_B + Q$ (see Note 1)	K_e (see Note 2)	S_{alt} (see Note 3)	n (see Note 4)	N (see Note 5)	U
44,526	1.00	25,762	200	35,300	0.0057
Total, $U_{40} =$					0.0057

- Notes: 1. $P_L + P_B + Q$ is obtained for Point 14 from p. A52 of VYC-378, Rev. 0.
 2. K_e computed in accordance with NB-3228.5 of ASME Code, Section III.
 3. $S_{alt} = 0.5 * K_e * K_1 * E_{\text{fatigue curve}}/E_{\text{analysis}} * \text{Power Uprate} * (P_L + P_B + Q)$.
 4. n for 40 years is the number of Heatup-Cooldown cycles, per p. B8 of VYC-378, Rev. 0.
 5. N obtained from Figure I-9, 1 of Appendix I of ASME Code, Section III.
 6. n for 60 years is the projected number of Heatup-Cooldown cycles.

Revised CUF Calculation for 60 Years:

$P_L + P_B + Q$ (see Note 1)	K_e (see Note 2)	S_{alt} (see Note 3)	n (see Note 6)	N (see Note 4)	U
44,526	1.00	25,762	300	35,300	0.0085
Total, $U_{60} =$					0.0085

Environmental CUF Calculation for 60 Years:

Maximum $F_{\text{en-HWC}}$ Multiplier for HWC Conditions = 5.39
 Maximum $F_{\text{en-NWC}}$ Multiplier for NWC Conditions = 13.17
 $U_{\text{env-60}} = U_{60} * F_{\text{en-NWC}} * 0.53 + U_{60} * F_{\text{en-HWC}} * 0.47 = 0.0809$
 Overall Multiplier = $U_{\text{env-60}}/U_{60} = 9.51$

Table 3-2. Environmental Fatigue Evaluation for the Reactor Vessel Shell at Shroud Support

Component: RPV Shell at Shroud Support
 NUREG/CR-6260 CUF: 0.032 (for reference only)
 Reference: NUREG/CR-6260, p. 5-102
 Stress Report CUF: 0.0549 (for Point 9, see below)
 Material: Low Alloy Steel (Material = A-533 Gr. B)

Design Basis CUF Calculation for 40 years:

Hydrotest H_1 = 26,240 psi (p. S3-97 of RPV Stress Report)
 Hydrotest H_2 = -1,250 psi (p. S3-97 of RPV Stress Report)
 Stress Concentration Factor, K_1 = 2.40 (p. S3-99d of RPV Stress Report)
 Hydrotest $K_1 H_1$ = 62,976 psi (p. S3-97 of RPV Stress Report)
 Improper Startup H_1 = 28,060 psi (p. S3-98 of RPV Stress Report)
 Improper Startup H_2 = -1,025 psi (p. S3-98 of RPV Stress Report)
 Improper Startup Skin Stress = 156,099 psi (p. S3-98 of RPV Stress Report)
 Improper Startup $K_1 H_1$ + Skin Stress = 223,443 psi (p. S3-98 of RPV Stress Report)
 Warmup H_1 = -5,707 psi (p. S3-99a of RPV Stress Report)
 Warmup H_2 = -102 psi (p. S3-99a of RPV Stress Report)
 Warmup $K_1 H_1$ = -13,696 psi (p. S3-99a of RPV Stress Report)
 $E_{fatigue\ curve} / E_{analysis}$ = 1.0417 30,0 / 28,8 per S3-99f of RPV Stress Report and ASME Code fatigue curve
 Power Uprate = 1.0067 = (549 - 100) / (546 - 100) per 4.4.1.b of 26A6019, Rev. 1
 m = 2.0 NB-3228.5 of ASME Code, Section III
 n = 0.2 NB-3228.5 of ASME Code, Section III
 S_m = 26,700 psi (ASME Code, Section II, Part D)

$P_L + P_B + Q$ (see Note 1)	Events	K_e (see Note 2)	S_{alt} (see Note 3)	n (see Note 4)	N (see Note 5)	U
34,690	Improper Startup - Warmup	1.00	124,825	5	332	0.0151
33,095	Hydrotest - Warmup	1.00	40,804	322	8,095	0.0398
Total, U_{40} =						0.0549

- Notes: 1. $P_L + P_B + Q$ is computed for Point 9 based on the $[(H_H - H_L)_{Event 1} - (H_H - H_L)_{Event 2}]$ stress intensity.
 2. K_e computed in accordance with NB-3228.5 of ASME Code, Section III.
 3. $S_{alt} = 0.5 * K_e * E_{fatigue\ curve} / E_{analysis} * Power\ Uprate * [(K_1 H_H - H_L)_{Event 1} - (K_1 H_H - H_L)_{Event 2}]$.
 4. n for 40 years is the number of cycles as follows per p. S3-99e and S3-99f of the RPV Stress Report:

Improper Startup =	5	cycles
Hydrotest =	2	cycles
Isothermal at 70°F and 1,000 psi =	120	cycles (same as number of Startup events)
Warmup-Cooldown =	199	cycles
Warmup-Blowdown =	1	cycle
TOTAL =	327	cycles

5. N obtained from Figure I-9.1 of Appendix I of ASME Code, Section III.

6. n for 60 years is the projected number of cycles as follows:

Improper Startup =	1	cycles
Hydrotest =	1	cycles
Isothermal at 70°F and 1,000 psi =	300	cycles (same as number of Startup events)
Warmup-Cooldown =	300	cycles
Warmup-Blowdown =	1	cycle
TOTAL =	603	cycles

Revised CUF Calculation for 60 Years:

$P_L + P_B + Q$ (see Note 1)	Events	K_e (see Note 2)	S_{alt} (see Note 3)	n (see Note 6)	N (see Note 4)	U
34,690	Improper Startup - Warmup	1.00	124,825	1	332	0.0030
33,095	Hydrotest - Warmup	1.00	40,804	602	8,095	0.0744
Total, U_{60} =						0.0774

Environmental CUF Calculation for 60 Years:

Maximum F_{en-HWC} Multiplier for HWC Conditions = 5.39
 Maximum F_{en-NWC} Multiplier for NWC Conditions = 13.17
 $U_{env-60} = U_{60} * F_{en-NWC} * 0.53 + U_{60} * F_{en-HWC} * 0.47 = 0.7364$
 Overall Multiplier = $U_{env-60} / U_{60} = 9.51$

Table 3-3. Environmental Fatigue Evaluation for the Reactor Vessel Feedwater Nozzle Forging Blend Radius

<u>Low Alloy Steel:</u>			$F_{en} = \exp(0.898 - 0.101S^*T^*O^*\square)$		
Assume $S^* = 0.015$ (maximum) Assume $\square = \ln(0.001) = -6.908$ (minimum)			Assume $S^* = 0.015$ (maximum) Assume $\square = \ln(0.001) = -6.908$ (minimum)		
For a BWR with HWC environment (post-HWC implementation): DO = 97 ppb = 0.097 ppm, so $O^* = \ln(0.097/0.04) = 0.886$			For a BWR with NWC environment (pre-HWC implementation): DO = 114 ppb = 0.114 ppm, so $O^* = \ln(0.114/0.04) = 1.047$		
Thus:			Thus:		
T (°C)	T (°F)	F_{en}	T (°C)	T (°F)	F_{en}
0	32	2.45	0	32	2.45
50	122	2.45	50	122	2.45
100	212	2.45	100	212	2.45
150	302	2.45	150	302	2.45
200	392	3.90	200	392	4.25
250	482	6.20	250	482	7.35
288	550	8.82	288	550	11.14
Thus, maximum $F_{en} = 8.82$			Thus, maximum $F_{en} = 11.14$		
$[T^* = (T-150) \text{ for } T > 150^\circ\text{C}]$			$[T^* = (T-150) \text{ for } T > 150^\circ\text{C}]$		
<u>Carbon Steel:</u>			$F_{en} = \exp(0.554 - 0.101S^*T^*O^*\square)$		
Assume $S^* = 0.015$ (maximum) Assume $\square = \ln(0.001) = -6.908$ (minimum)			Assume $S^* = 0.015$ (maximum) Assume $\square = \ln(0.001) = -6.908$ (minimum)		
For a BWR with HWC environment (post-HWC implementation): DO = 40 ppb = 0.040 ppm < 0.050 ppm so $O^* = 0$			For a BWR with NWC environment (pre-HWC implementation): DO = 40 ppb = 0.040 ppm < 0.050 ppm so $O^* = 0$		
Thus:			Thus:		
T (°C)	T (°F)	F_{en}	T (°C)	T (°F)	F_{en}
0	32	1.74	0	32	1.74
50	122	1.74	50	122	1.74
100	212	1.74	100	212	1.74
150	302	1.74	150	302	1.74
200	392	1.74	200	392	1.74
250	482	1.74	250	482	1.74
288	550	1.74	288	550	1.74
Thus, maximum $F_{en} = 1.74$			Thus, maximum $F_{en} = 1.74$		
$[T^* = (T-150) \text{ for } T > 150^\circ\text{C}]$			$[T^* = (T-150) \text{ for } T > 150^\circ\text{C}]$		

No.	Component	Material	60-Year CUF	Overall Environmental Multiplier	60-Year Environmental CUF (1,2)
1	Feedwater Nozzle Forging Blend Radius	Low Alloy Steel	0.0636	10.05	0.6392
2	Feedwater Nozzle Forging Safe End	Carbon Steel	0.1471	1.74	0.2560

- Notes:
1. An F_{en} Multiplier was used for each respective component with the following conditions:
+ 47% HWC conditions and 53% NWC conditions
 2. Results using updated ASME Code fatigue calculations and actual cycles accumulated to-date and projected to 60 years.

Table 3-4. Environmental Fatigue Evaluation for the Recirculation/RHR Piping Tee

<u>Stainless Steel:</u>		$F_{en} = \exp(0.935 \cdot T \cdot \epsilon \cdot O^*)$	
For a BWR with HWC environment (post-HWC implementation): DO = 46 ppb = 0.046 ppm < 0.050 ppm, so $O^* = 0.260$ Conservatively use $T^* = 1$ for $T > 200^\circ\text{C}$ Thus:		For a BWR with NWC environment (pre-HWC implementation): DO = 123 ppb = 0.123 ppm > 0.05 ppm, so $O^* = 0.172$ Conservatively use $T^* = 1$ for $T > 200^\circ\text{C}$ Thus:	
$\epsilon^* = 0$ for $\epsilon > 0.4\%/sec$	so $F_{en} =$	2.55	so $F_{en} =$
$\epsilon^* = \ln(\epsilon/0.4)$ for $0.0004 \leq \epsilon \leq 0.4\%/sec$	so F_{en} ranges from	2.55	so F_{en} ranges from
	to	15.35	to
$\epsilon^* = \ln(0.0004/0.4)$ for $\epsilon < 0.0004\%/sec$	so $F_{en} =$	15.35	so $F_{en} =$
	Thus, maximum $F_{en} =$	15.35	Thus, maximum $F_{en} =$
			8.36

No.	Component	Material	60-Year CUF	Overall Environmental Multiplier	60-Year Environmental CUF (1,2)
1	Recirculation /RHR Piping Return Tee	Stainless Steel	0.0590	12.62	0.7446

- Notes: 1. An F_{en} multiplier was used for each respective component with the following conditions:
+ 39% HWC conditions and 61% NWC conditions
2. Results using updated ASME Code fatigue calculations and actual cycles accumulated to-date and projected to 60 years.

Table 3-5. Environmental Fatigue Evaluation for the Reactor Recirculation Inlet Nozzle Forging

Component: Recirculation Inlet Nozzle Forging
 NUREG/CR-6260 CUF: 0.310 (for reference only)
 Reference: NUREG/CR-6260, p. 5-105
 Stress Report CUF: 0.0433 (updated for Point 12, see below)
 Material: Low Alloy Steel (Material = A-508 Cl. II per p. I-S8-4 of CBIN Stress Report Section S8)

Design Basis CUF Calculation for 40 years:

$E_{\text{fatigue curve}}/E_{\text{analysis}} = 1.1278$ = 30.0 / 26.6 (per p. I-S8-24 of CBIN Stress Report Section S8 and ASME Code fatigue curve)
 Power Uprate = 1.0067 = (549 - 100) / (546 - 100) per 4.4.1.b of 26A6019, Rev. 1
 $K_t = 1.660$ stress concentration factor (p. A270 of VYC-378, Rev. 0)
 $m = 2.0$ NB-3228.5 of ASME Code, Section III
 $n = 0.2$ NB-3228.5 of ASME Code, Section III
 $S_m = 26,700$ psi (ASME Code, Section II, Part D)

$P_L + P_B + Q$ (see Note 1)	Skin Stress (see Note 2)	K_a (see Note 3)	S_{all} (see Note 4)	n (see Note 5)	N (see Note 6)	U
43,110	15,145	1.00	49,224	200	4,614	0.0433
Total, $U_{40} =$						0.0433

- Notes: 1. $P_L + P_B + Q$ is obtained for Point 12 from p. A270 of VYC-378, Rev. 0.
 2. Skin Stress is obtained for Point 12 from p. A270 of VYC-378, Rev. 0.
 3. K_a computed in accordance with NB-3228.5 of ASME Code, Section III.
 4. $S_{all} = 0.5 * K_a * E_{\text{fatigue curve}}/E_{\text{analysis}} * \text{Power Uprate} * [(P_L + P_B + Q) K_t + \text{Skin Stress}]$.
 5. n for 40 years is the number of Heatup-Cooldown cycles, per p. B28 of VYC-378, Rev. 0.
 6. N obtained from Figure I-9, 1 of Appendix I of ASME Code, Section III.
 7. n for 60 years is the projected number of Heatup-Cooldown cycles.

Revised CUF Calculation for 60 Years:

$P_L + P_B + Q$ (see Note 1)	Skin Stress (see Note 2)	K_a (see Note 3)	S_{all} (see Note 4)	n (see Note 5)	N (see Note 7)	U
43,110	15,145	1.00	49,224	300	4,614	0.0650
Total, $U_{60} =$						0.0650

Environmental CUF Calculation for 60 Years:

Maximum F_{en-HWC} Multiplier for HWC Conditions = 2.45
 Maximum F_{en-NWC} Multiplier for NWC Conditions = 12.43
 $U_{env-60} = U_{60} * F_{en-NWC} * 0.53 + U_{60} * F_{en-HWC} * 0.47 = 0.5034$
 Overall Multiplier = $U_{env-60}/U_{60} = 7.74$

Table 3-6. Environmental Fatigue Evaluation for Reactor Recirculation Inlet Nozzle Safe End

Component: Recirculation Inlet Nozzle Safe End
 NUREG/CR-6260 CUF: 0.310 (for reference only)
 Reference: NUREG/CR-6260, p. 5-105
 Stress Report CUF: 0.0017 (updated for Location 6-1, see below)
 Material: Stainless Steel (316L per p. 8 of 23A4292, Rev. 4)

Design Basis CUF Calculation for 40 years:

$E_{fatigue\ curve}/E_{analysis} = 1.1076$ = 28.3 / 25.55 (per p. 62 of Reference [18] and ASME Code fatigue curve)
 $Power\ Uprate = 1.0067$ = (549 - 100) / (546 - 100) per 4.4.1.b of 26A6019, Rev. 1
 $K_t = 1.280$ stress concentration factor (p. B27 of VYC-378, Rev. 0)
 $m = 1.7$ NB-3228.5 of ASME Code, Section III
 $n = 0.3$ NB-3228.5 of ASME Code, Section III
 $S_m = 16,600$ psi (ASME Code, Section II, Part D)

P_L+P_B+Q (see Note 1)	$P+Q+F$ (see Note 2)	K_e (see Note 3)	S_{alt} (see Note 4)	n (see Note 5)	N (see Note 6)	U
47,183	36,972	1.00	26,385	2,076	1,242,266	0.0017
Total, $U_{40} =$						0.0017

- Notes: 1. P_L+P_B+Q is obtained for Surface I (after weld overlay) from p. 117 of Reference [18].
 2. $P+Q+F$ is obtained for Point 6-1 from p. 118 of Reference [18] (BEFORE weld overlay).
 3. K_e computed in accordance with NB-3228.5 of ASME Code, Section III.
 4. $S_{alt} = 0.5 * K_e * E_{fatigue\ curve}/E_{analysis} * Power\ Uprate * [(P+Q+F) K_t]$.
 5. n for 40 years is the number of cycles as follows per p. B26 of VYC-378, Rev. 0:

Design Hydrotest = 130	
<u>Loss of Feedpumps Composite:</u>	
Startup/Shutdown =	290
SRV Blowdown =	8
Loss of Feedwater Pumps	30
SCRAM =	270
10 events x 3 up/down cycles per event	
Normal +/- Seismic =	11
10 cycles of upset seismic, plus 1 Level C seismic event	
Normal =	739
= Sum of all of above events	
Zeroload =	598
= Startup/Shutdown + SRV Blowdown + Scram + LOFP	
Total number of cycles = 2,076	

6. N obtained from Figure I-9.2 of Appendix I of ASME Code, Section III.
 7. n for 60 years is the projected number of cycles as follows:

Design Hydrotest = 120	
<u>Loss of Feedpumps Composite:</u>	
Startup/Shutdown =	300
SRV Blowdown =	1
Loss of Feedwater Pumps	30
SCRAM =	289
10 events x 3 up/down cycles per event	
All remaining scrams	
Normal +/- Seismic =	11
Assume the same	
Normal =	751
= Sum of all of above events	
Zeroload =	620
= Startup/Shutdown + SRV Blowdown + Scram + LOFP	
Total number of cycles = 2,122	

Revised CUF Calculation for 60 Years:

P_L+P_B+Q (see Note 1)	$P+Q+F$ (see Note 2)	K_e (see Note 3)	S_{alt} (see Note 4)	n (see Note 5)	N (see Note 7)	U
47,183	36,972	1.00	26,385	2,122	1,242,266	0.0017
Total, $U_{60} =$						0.0017

Environmental CUF Calculation for 60 Years:

Maximum F_{en-HWC} Multiplier for HWC Conditions = 15.35
 Maximum F_{en-NWC} Multiplier for NWC Conditions = 8.36
 $U_{env-60} = U_{60} * F_{en-NWC} * 0.53 + U_{60} * F_{en-HWC} * 0.47 = 0.0199$
 Overall Multiplier = $U_{env-60}/U_{60} = 11.64$

Table 3-7. Environmental Fatigue Evaluation for Recirculation Outlet Nozzle Forging

<i>Low Alloy Steel:</i>			$F_{en} = \exp(0.898 - 0.101S^*T^*O^*e^*)$		
			Assume $S^* = 0.015$ (maximum)		
			Assume $e^* = \ln(0.001) = -6.908$ (minimum)		
For a BWR with HWC environment (post-HWC implementation): DO = 46 ppb = 0.046 ppm DO < 0.050 ppm, so $O^* = 0$ Thus:			For a BWR with NWC environment (pre-HWC implementation): DO = 123 ppb = 0.123 ppm, so $O^* = \ln(0.123/0.04) = 1.123$ Thus:		
T (°C)	T (°F)	F_{en}	T (°C)	T (°F)	F_{en}
0	32	2.45	0	32	2.45
50	122	2.45	50	122	2.45
100	212	2.45	100	212	2.45
150	302	2.45	150	302	2.45
200	392	2.45	200	392	4.42
269.45	517.01	2.45	269.45	517.01	10.00
288	550	2.45	288	550	12.43
Thus, maximum $F_{en} =$			Thus, maximum $F_{en} =$		
2.45			12.43		
			[T* = (T-150) for T > 150°C]		

No.	Component	Material	60-Year CUF	Overall Environmental Multiplier	60-Year Environmental CUF (1,2)
1	Recirculation Outlet Nozzle Forging Blend Radius	Low Alloy Steel	0.0108	7.74	0.0836

- Notes:
1. An F_{en} multiplier was used for each respective component with the following conditions:
+ 47% HWC conditions and 53% NWC conditions
 2. Results using updated ASME Code fatigue calculations and actual cycles accumulated to-date and projected to 60 years.

Table 3-8. Environmental Fatigue Evaluation for Core Spray Reactor Vessel
Nozzle Forging Blend Radius, Safe End, and Piping

<u>Low Alloy Steel:</u>			$F_{en} = \exp(0.898 - 0.101S^*T^*O^*)$		
			Assume $S^* = 0.015$ (maximum) Assume $O^* = \ln(0.001) = -6.908$ (minimum)		
For a BWR with HWC environment (post-HWC implementation): DO = 97 ppb = 0.097 ppm, so $O^* = \ln(0.097/0.04) = 0.886$			For a BWR with NWC environment (pre-HWC implementation): DO = 114 ppb = 0.114 ppm, so $O^* = \ln(0.114/0.04) = 1.047$		
Thus:			Thus:		
T (°C)	T (°F)	F_{en}	T (°C)	T (°F)	F_{en}
0	32	2.45	0	32	2.45
50	122	2.45	50	122	2.45
100	212	2.45	100	212	2.45
150	302	2.45	150	302	2.45
200	392	3.90	200	392	4.25
250	482	6.20	250	482	7.35
288	550	8.82	288	550	11.14
Thus, maximum $F_{en} = 8.82$			Thus, maximum $F_{en} = 11.14$		
			$[T^* = (T-150) \text{ for } T > 150^\circ\text{C}]$		
<u>Stainless Steel:</u>			$F_{en} = \exp(0.935 - T^*O^*)$		
For a BWR with HWC environment (post-HWC implementation): DO = 97 ppb = 0.097 ppm > 0.050 ppm, so $O^* = 0.172$ Conservatively use $T^* = 1$ for $T > 200^\circ\text{C}$ Thus:			For a BWR with NWC environment (pre-HWC implementation): DO = 114 ppb = 0.114 ppm > 0.05 ppm, so $O^* = 0.172$ Conservatively use $T^* = 1$ for $T > 200^\circ\text{C}$ Thus:		
$O^* = 0$ for $\sigma > 0.4\%/sec$ so $F_{en} = 2.55$			$O^* = 0$ for $\sigma > 0.4\%/sec$ so $F_{en} = 2.55$		
$O^* = \ln(\sigma/0.4)$ for $0.0004 \leq \sigma \leq 0.4\%/sec$ so F_{en} ranges from 2.55 to 8.36			$O^* = \ln(\sigma/0.4)$ for $0.0004 \leq \sigma \leq 0.4\%/sec$ so F_{en} ranges from 2.55 to 8.36		
$O^* = \ln(0.0004/0.4)$ for $\sigma < 0.0004\%/sec$ so $F_{en} = 8.36$			$O^* = \ln(0.0004/0.4)$ for $\sigma < 0.0004\%/sec$ so $F_{en} = 8.36$		
Thus, maximum $F_{en} = 8.36$			Thus, maximum $F_{en} = 8.36$		

No.	Component	Material	60-Year CUF	Overall Environmental Multiplier	60-Year Environmental CUF (1,2)
1	Core Spray Nozzle Forging Blend Radius	Low Alloy Steel	0.0166	10.05	0.1668
2	Core Spray Nozzle Safe End	Ni-Cr-Fe	0.0398	1.49	0.0593
3	Core Spray Piping	Stainless Steel	0.0011	8.36	0.0092

- Notes:
1. An F_{en} Multiplier was used for each respective component with the following conditions:
+ 47% HWC conditions and 53% NWC conditions
 2. Results using updated ASME Code fatigue calculations and actual cycles accumulated to-date and projected to 60 years.

Table 3-9. Environmental Fatigue Evaluation for the Feedwater Line Class 1 Piping

<i>Carbon Steel:</i>			$F_{en} = \exp(0.554 - 0.101S^*T^*O^*\epsilon^*)$		
			Assume $S^* = 0.015$ (maximum)		
			Assume $\epsilon^* = \ln(0.001) = -6.908$ (minimum)		
For a BWR with HWC environment (post-HWC implementation): DO = 40 ppb = 0.040 ppm < 0.050 ppm so $O^* = 0$ Thus:			For a BWR with NWC environment (pre-HWC implementation): DO = 40 ppb = 0.040 ppm < 0.050 ppm so $O^* = 0$ Thus:		
T (°C)	T (°F)	F_{en}	T (°C)	T (°F)	F_{en}
0	32	1.74	0	32	1.74
50	122	1.74	50	122	1.74
100	212	1.74	100	212	1.74
150	302	1.74	150	302	1.74
200	392	1.74	200	392	1.74
250	482	1.74	250	482	1.74
288	550	1.74	288	550	1.74
Thus, maximum $F_{en} =$			Thus, maximum $F_{en} =$		
1.74			1.74		
			<small>($T^* = (T-150)$ for $T > 150^\circ\text{C}$)</small>		

No.	Component	Material	60-Year CUF	Overall Environmental Multiplier	60-Year Environmental CUF (1,2)
1	Feedwater Piping Riser to RPV Nozzle N4B	Carbon Steel	0.1661	1.74	0.2890

- Notes:
1. An F_{en} multiplier was used for each respective component with the following conditions:
 - + 47% HWC conditions and 53% NWC conditions
 2. Results using updated ASME Code fatigue calculations and actual cycles accumulated to-date and projected to 60 years.

Table 3-10. Summary of Environmental Fatigue Calculations for VYNPS

No.	Component	Material	40-Year Design CUF ⁽¹⁾	60-Year CUF ⁽²⁾	Overall Environmental Multiplier ⁽³⁾	60-Year Environmental CUF
1	RPV Shell/Bottom Head	Low Alloy Steel	0.0057	0.0085	9.51	0.0809
2	RPV Shell at Shroud Support	Low Alloy Steel	0.0549	0.0774	9.51	0.7364
3	Feedwater Nozzle Blend Radius	Low Alloy Steel	(4)	0.0636	10.05	0.6392
4	Recirculation/RHR Class 1 Piping (Return Tee)	Stainless Steel	(4)	0.0590	12.62	0.7446
5	Recirculation Inlet Nozzle Forging	Low Alloy Steel	0.0433	0.0650	7.74	0.5034
6	Recirculation Inlet Nozzle Safe End	Stainless Steel	0.0017	0.0017	11.64	0.0199
7	Recirculation Outlet Nozzle Forging	Low Alloy Steel	(4)	0.0108	7.74	0.0836
8	Core Spray Nozzle Forging Blend Radius ⁽⁵⁾	Low Alloy Steel	(4)	0.0166	10.05	0.1668
9	Feedwater Class 1 Piping	Carbon Steel	(4)	0.1661	1.74	0.2890

- Notes:
1. Updated 40-year CUF calculation based on recent ASME Code methodology and design basis cycles.
 2. CUF results using updated ASME Code methodology and actual cycles accumulated to-date and projected to 60 years.
 3. An F_{en} multiplier was used for each respective component with the following conditions:
 - + 47% HWC conditions and 53% NWC conditions
 4. 40 year values were not calculated for these locations
 5. Only the highest CUF from Table 3-8 is shown

4.0 SUMMARY AND CONCLUSIONS

The results of Tables 3-1 through 3-9, as summarized in Table 3-10, demonstrate that the fatigue usage factor, including environmental effects, remains within the allowable value of 1.0 for 60 years of operation for the following component locations:

- ✓ Reactor vessel shell, bottom head and shroud support
- ✓ Reactor vessel feedwater nozzle
- ✓ Reactor recirculation piping (including the reactor inlet and outlet nozzles)
- ✓ Core spray line reactor vessel nozzle and associated Class 1 piping
- ✓ Feedwater line Class 1 piping

Therefore, the environmental fatigue assessment results for all of the NUREG/CR-6260 locations associated with the older vintage BWR plant are acceptable for 60 years of operation for VYNPS.

5.0 REFERENCES

1. U. S. Nuclear Regulatory Commission, Generic Safety Issue 166, "Adequacy of Fatigue Life of Metal Components."
2. U. S. Nuclear Regulatory Commission, Generic Safety Issue 190, "Fatigue Evaluation of Metal Components for 60-Year Plant Life."
3. SECY-95-245, "Completion of the Fatigue Action Plan," James M. Taylor, Executive Director for Operations, U. S. Nuclear Regulatory Commission, Washington, DC, September 25, 1995.
4. Memorandum, Ashok C. Thadani, Director, Office of Nuclear Regulatory Research, to William D. Travers, Executive Director for Operations, Closeout of Generic Safety Issue 190, "Fatigue Evaluation of Metal Components for 60 Year Plant Life," U. S. Nuclear Regulatory Commission, Washington, DC, December 26, 1999.
5. NUREG-1801, Revision 1, "Generic Aging Lessons Learned (GALL) Report," U.S. Nuclear Regulatory Commission, September 2005.
6. NUREG/CR-6260 (INEL-95/0045), "Application of NUREG/CR-5999 Interim Fatigue Curves to Selected Nuclear Power Plant Components," March 1995.
7. NUREG/CR-6583 (ANL-97/18), "Effects of LWR Coolant Environments on Fatigue Design Curves of Carbon and Low-Alloy Steels," March 1998.
8. NUREG/CR-5704 (ANL-98/31), "Effects of LWR Coolant Environments on Fatigue Design Curves of Austenitic Stainless Steels," April 1999.
9. NUREG/CR-5999 (ANL-93/3), "Interim Fatigue Design Curves for Carbon, Low-Alloy, and Austenitic Stainless Steels in LWR Environments," April 1993.
10. USAS B31.1 - 1967, USA Standard Code for Pressure Piping, "Power Piping," American Society of Mechanical Engineers, New York.
11. EPRI Report No. TR-105759, "An Environmental Factor Approach to Account for Reactor Water Effects in Light Water Reactor Pressure Vessel and Piping Fatigue Evaluations," December 1995.
12. Structural Integrity Associates Calculation No. VY-16Q-303, Revision 0, " Environmental Fatigue Evaluation of Reactor Recirculation Inlet Nozzle and Vessel Shell/Bottom Head."

13. INFORMATION REDACTED
14. Entergy Design Input Record (DIR) Rev. 1, EC No. 1773, Rev. 0, "Environmental Fatigue Analysis for Vermont Yankee Nuclear Power Station," 7/26/07, SI File No. VY-16Q-209.
15. VY Calculation No. VYC-378, Revision 0, "Vermont Yankee Reactor Cyclic Limits for Transient Events," 10/16/85, SI File No. VY-05Q-211.
16. Chicago Bridge & Iron RPV Stress Report, Section S3, Revision 4, "Stress Analysis, Shroud Support, Vermont Yankee Reactor Vessel, CB&I Contract 9-6201," 2/3/70, SI File No. VY-16Q-203.
17. Structural Integrity Associates Calculation No. VY-16Q-302, Revision 0, "Fatigue Analysis of Feedwater Nozzle."
18. Structural Integrity Associates Calculation No. VY-16Q-307, Revision 0, "Recirculation Class 1 Piping Fatigue and EAF Analysis."
19. CB&I RPV Stress Report, Section S8, Revision 4, "Stress Analysis, Recirculation Inlet Nozzle, Vermont Yankee Reactor Vessel, CB&I Contract 9-6201," 2/3/70, SI File No. VY-16Q-203.
20. GE Nuclear Energy Certified Stress Report No. 23A4292, Revision 0, "Reactor Vessel – Recirculation Inlet Safe End Nozzle," January 21, 1985, SI File No. VY-16Q-203.
21. Structural Integrity Associates Calculation No. VY-16Q-310, Revision 1, "Fatigue Analysis of Core Spray Nozzle."
22. Structural Integrity Associates Calculation No. VY-16Q-311, Revision 0, "Feedwater Class 1 Piping Fatigue Analysis."
23. EPRI/BWRVIP Memo. No. 2005-271, "Potential Error in Existing Fatigue Reactor Water Environmental Effects Analyses," July 1, 2005
24. Structural Integrity Associates Calculation No. VY-16Q-306, Revision 0, "Fatigue Analysis of Recirculation Outlet Nozzle."



Structural Integrity Associates, Inc.

File No.: VY-19Q-301

CALCULATION PACKAGE

Project No.: VY-19Q

PROJECT NAME:

Provide VY Support for Questions Related to Environmental Fatigue Analyses

CONTRACT NO.:

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CLIENT:

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PLANT:

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CALCULATION TITLE:

Design Inputs and Methodology for ASME Code Confirmatory Fatigue Usage Analysis of Reactor Feedwater Nozzle

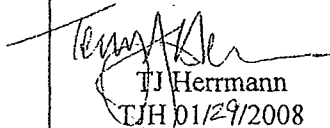

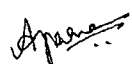
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1.0 OBJECTIVE

The objective of this calculation package is to establish the design inputs and methodology to be used for an ASME Code, Section III fatigue usage calculation of the reactor pressure vessel (RPV) feedwater (FW) nozzle at Vermont Yankee Nuclear Power Station (VYNPS).

2.0 METHODOLOGY

A detailed fatigue usage analysis of the FW nozzle will be performed using the methodology of Subarticle NB-3200 of Section III of the ASME Code [1]. The analysis will be used as a confirmatory analysis for comparison with a previous fatigue usage analysis that was done using simplified methods. Therefore, only the fatigue portion of the ASME Code methodology will be used, and the analysis will be a fatigue assessment only, and not a complete ASME Code analysis.

Finite element analysis will be performed using a previously-developed axisymmetric finite element model (FEM) of the FW nozzle. Thermal transient analysis will be performed using the FEM for each defined transient. Concurrent with the thermal transients are pressure and piping interface loads; for these loads, unit load analyses (finite element analysis for pressure, and manual calculations for piping loads) will be performed. The stresses from these analyses will be scaled appropriately based on the magnitude of the pressure and piping loads during each thermal transient, and combined with stresses from the thermal transients. Additional scaling of pressure stresses will be performed to account for nozzle corner contour effects (i.e., the effects of approximating the nozzle-to-RPV intersection of two cylinders with an axisymmetric model). Other stress concentration factors (SCFs) will be applied as appropriate.

All six components of the stress tensor will be used for stress calculations. The stress components for the non-axisymmetric loads (shear and moment piping loads) can have opposite signs depending upon which side of the nozzle is being examined. Therefore, when combining stress components from these loads with stress components from thermal transients and other loads, the signs of the stress components will be adjusted to maximize the magnitude of the stress component ranges.

The fatigue analysis will be performed at previously-examined locations for direct comparison of results. Stresses will be linearized at these locations. The linearized primary plus secondary membrane plus bending stress will be used to determine the value of K_e to be used in the simplified elastic-plastic analysis in accordance with ASME Code NB-3200 methodology. Environmental fatigue multipliers will be applied in accordance with NUREG/CR-6583 [15].



3.0 ASSUMPTIONS/DESIGN INPUTS

3.1 Assumptions

- 3.1.1 *Power uprate effects are considered as being applied to the entire period of operation. The higher pressures, flows, and temperatures at uprate conditions are used in determining and applying heat transfer coefficients [3, Section 3.2] [2, Section 3.1].*
- 3.1.2 *The Boltup transient [2, Tables 1 and 2] analysis does not affect the FW nozzle and is therefore excluded from the transients analyzed.*
- 3.1.3 *Where the flow rates in the thermal cycle diagram are at a value not calculated in Table 2, the next highest flow rate heat transfer coefficient will be used. This results in a higher heat transfer coefficient and is therefore conservative.*
- 3.1.4 *The effect of non-uniform geometries is judged to be insignificant for flow inside the safe end, because of the smooth transition and small geometry changes as shown in Figure 6. The smaller inner diameter (9.669") at the safe end was used to calculate heat transfer coefficients, resulting in a higher flow velocity and therefore conservative values.*
- 3.1.5 *The annulus leakage flow rate used is 31 GPM for EPU conditions [3, Section 3.2].*
- 3.1.6 *Density and Poisson's ratio used in the FEM are assumed typical values of $\rho = 0.283 \text{ lb/in}^3$ and 0.3, respectively.*
- 3.1.7 *For purposes of linearizing stress at the nozzle corner, the effect of the cladding is conservatively neglected.*
- 3.1.8 *Stress components due to piping loads are scaled assuming no stress occurs at an ambient temperature of 70°F and the full values are reached at reactor design temperature, 575°F, as was done in the previous analysis [2, Section 3.4].*

3.2 ASME Code Edition

The analysis will be performed in a manner consistent with the fatigue usage rules in NB-3200 of Section III of the ASME Code; the 1998 Edition with Addenda through 2000 [1] will be used, for consistency with the previous analysis [2].

3.3 Transients

Previously developed thermal and pressure transients [2, Section 3.1 and Tables 1 and 2] are used for this analysis. The transients to be evaluated are shown in Table 1. For each transient, the time, nozzle fluid temperature (T_{noz}), RPV pressure, percent FW flow rate, and number of cycles are included. In some cases, flow rates and T_{noz} values from the nozzle thermal cycle diagram [10, Attachment 1, p. 3] are used to reduce excess conservatism. Note that the only difference between the nozzle corner and the safe end transients in the referenced document is the length of the steady state time increment used at the end of the transients. These steady state periods are not included in Table 1; the analyst will use a value greater than or equal to the largest steady time increment from the referenced document.

At the inside surface of the RPV, the Region A temperature from the reactor thermal cycle diagram [10, Attachment 1, p. 2] shall be applied. Table 1 also includes these values as T_{RPV} .



Table 1: Transients

Transient	Time,	T _{noz} , °F	T _{RPV} , °F	P, psig	FW	Cycles
	sec				Flow, %	
1. Boltup	0	70	70	0	0%	123
2. Design Hydrotest	0	70	70	0	0%	120
	1080	100	100	0	0%	
	1680	100	100	1100	0%	
	5280	100	100	1100	0%	
3. Startup	5880	100	100	50	0%	300
	0	100	100	50	0%	
	16164	549	549	1010	0%	
	0	549	549	1010	0%	
4. Turbine Roll and Increased to Rated Power	1	100	549	1010	25%*	300
	1801	100	549	1010	25%*	
	1802	260	549	1010	25%*	
	3602	392	549	1010	100%	
	0	392	549	1010	100%	
5. Daily Reduction 75% Power	900	310	549	1010	100%*	10,000
	2700	310	549	1010	100%*	
	3600	392	549	1010	100%	
	0	392	549	1010	100%	
6. Weekly Reduction 50% Power	1800	280	549	1010	100%*	2,000
	3600	280	549	1010	100%*	
	5400	392	549	1010	100%	
	0	392	549	1010	100%	
9. Turbine Trip at 25% Power	1800	265	549	1010	100%	10
	1980	265	549	1010	25%*	
	2340	90	549	1010	25%*	
	2520	90	549	1010	25%*	
	3420	265	549	1010	25%*	
	3600	265	549	1010	100%	
	5400	392	549	1010	100%	
	0	392	549	1010	100%	
10. FW Heater Bypass	90	265	549	1010	100%	70
	1890	265	549	1010	100%	
	2070	392	549	1010	100%	
	0	392	549	1010	100%	



Transient	Time, sec	T _{noz} , °F	T _{RPV} , °F	P, psig	FW Flow, %	Cycles
11. Loss of FW Pumps	0	392	549	1010	100%	10
	1	565	565	1010	0%	
	3.5	565	565	1190	0%	
	4.5	50	565	1184.5	40%	
	13.5	50	565	1135	40%	
	184.5	50	565	1135	40%	
	1564.5	440	565	1135	0%	
	1565.5	565	565	1135	0%	
	2165.5	565	565	1135	0%	
	2166.5	50	565	1135	40%*	
	2346.5	50	532	885	40%*	
	5406.5	440	549	1055	0%	
	5407.5	549	549	1055	0%	
	6727.5	565	565	1135	0%	
	6728.5	50	565	1135	25%*	
	7148.5	50	502	675	25%*	
	7448.5	300	502	675	0%	
	11048.5	400	400	232	0%	
	16411.5	549	549	885	0%	
	16412.5	549	549	1010	0%	
	18212.5	549	549	1010	0%	
	18213.5	100	549	1010	25%*	
	20013.5	100	549	1010	25%*	
	20014.5	260	549	1010	25%*	
	21814.5	392	549	1010	100%	
12/13/15. Turbine	0	392	549	1010	100%	289
Generator Trip,	10	392	565/600**	1135/1375**	100%	
Reactor Overpressure,	15	392	565/600**	1135/1375**	100%	
Other SCRAMs	30	392	539	940	100%	
	90	275	539	940	25%*	
	990	100	539	940	25%*	
	2790	100	539	940	25%*	
	2791	260	539	940	100%	
	3210	291	549	1010	100%	
	4591	392	549	1010	100%	
14. SRV Blowdown	0	392	549	1010	100%	1
	60	275	531.6	885	100%	
	960	100	365	50	25%*	
19. Reduction to 0% Power	0	392	549	1010	100%	300
	1800	265	549	1010	25%*	
20. Hot Standby (Heatup Portion)	0	265	549	1010	25%*	300
	1	440	549	1010	0%	
	3925	549	549	1010	0%	
20A. Hot Standby (FW Injection Portion)	0	549	549	1010	0%	300
	1	100	549	1010	25%	
	181	100	549	1010	25%	
	241	290	549	1010	0%	
	451	549	549	1010	25%	
21-23. Shutdown	0	549	549	1010	25%*	300
	6264	375	375	50	25%*	
	6864	330	330	50	25%*	
	15144	100	100	50	0%	

Transient	Time,	T _{noz} , °F	T _{RPV} , °F	P, psig	FW	Cycles
	sec				Flow, %	
24. Hydrostatic Test	0	100	100	50	0%	1
	600	100	100	1563	0%	
	1200	100	100	1563	0%	
	1800	100	100	50	0%	
25. Unbolt	0	100	100	0	0%	123
	1080	70	70	0	0%	

* Flow rate is conservatively rounded up to one of the three flow rates considered (25%, 40%, 100%).

** The second value applies for one cycle; the first value applies for the rest of the cycles.

3.4 Heat Transfer Coefficients, Condensation

When steam floods a relatively cold component, the steam condenses on the component surface. Holman [5, p. 413] gives the following equation for average heat transfer coefficient:

$$\bar{h} = 0.555 \{ \rho(\rho - \rho_v) g k^3 h'_{fg} / [\mu D (T_g - T_w)] \}^{1/4}, \text{ where}$$

ρ = mass density of liquid,

ρ_v = mass density of vapor,

g = acceleration of gravity,

k = conductivity of liquid at average temperature,

$h'_{fg} = h_{fg} + 0.68c(T_g - T_w)$,

h_{fg} = heat of condensation at vapor temperature,

c = specific heat of liquid at average temperature,

T_g = saturated vapor temperature = T_{final} ,

T_w = pipe inner wall temperature = $T_{initial}$,

μ = viscosity of liquid at average temperature

D = inner diameter of pipe

The portion of the equation inside the brackets, $\rho(\rho - \rho_v) g k^3 h'_{fg} / [\mu D (T_g - T_w)]$, has the following units:

$$\frac{(\text{lbm})^2}{(\text{ft}^3)^2} \frac{(\text{ft})}{(\text{sec}^2)} \frac{(\text{Btu})^3}{(\text{hr-ft-}^\circ\text{F})^3} \frac{(\text{Btu})}{(\text{lbm})} \frac{(\text{ft-hr})}{(\text{lbm})} \frac{(\text{ft})}{(\text{ft})} \frac{(\text{ft}^6)}{(\text{ft}^6)} \frac{(\text{sec}^2)}{(\text{sec}^2)} \frac{(\text{Btu}^4)}{(\text{hr}^3\text{-ft}^3\text{-}^\circ\text{F}^3)} \frac{(\text{ft-hr})}{(^\circ\text{F})} \frac{(3600^2 \text{ sec}^2)}{(\text{hr}^2)}$$

$$\frac{12960000 \text{ Btu}^4}{\text{hr}^4\text{-ft}^8\text{-}^\circ\text{F}^4}$$

After taking the fourth root, this becomes 60 Btu/hr-ft²-°F. Steam properties are interpolated at T_g , and water properties are interpolated at T_f , which is taken as the average of T_g and T_w . Then, h'_{fg} and heat transfer coefficient h are calculated for each set of steam properties, water properties, T_g and T_w . Tables 2 and 3 list selected properties of liquid water [12, Table 1-8] and saturated steam [13], respectively.

Table 2: Properties of Liquid Water

T, °F	ρ , lbm/ft ³	c, Btu/lbm- °F	μ , lbm/ft- hr	v, ft ² /sec	k, Btu/hr- ft-°F
300	57.3	1.03	0.468	2.27E-06	0.395
400	53.6	1.08	0.335	1.74E-06	0.382
500	49.0	1.19	0.252	1.43E-06	0.349
600	42.4	1.51	0.208	1.37E-06	0.293

Table 3: Properties of Saturated Steam

T _g , °F	v _g , ft ³ /lbm	h _{fg} , Btu/lbm
545	0.4449	649.6
550	0.4249	641.6
565	0.3703	616.4

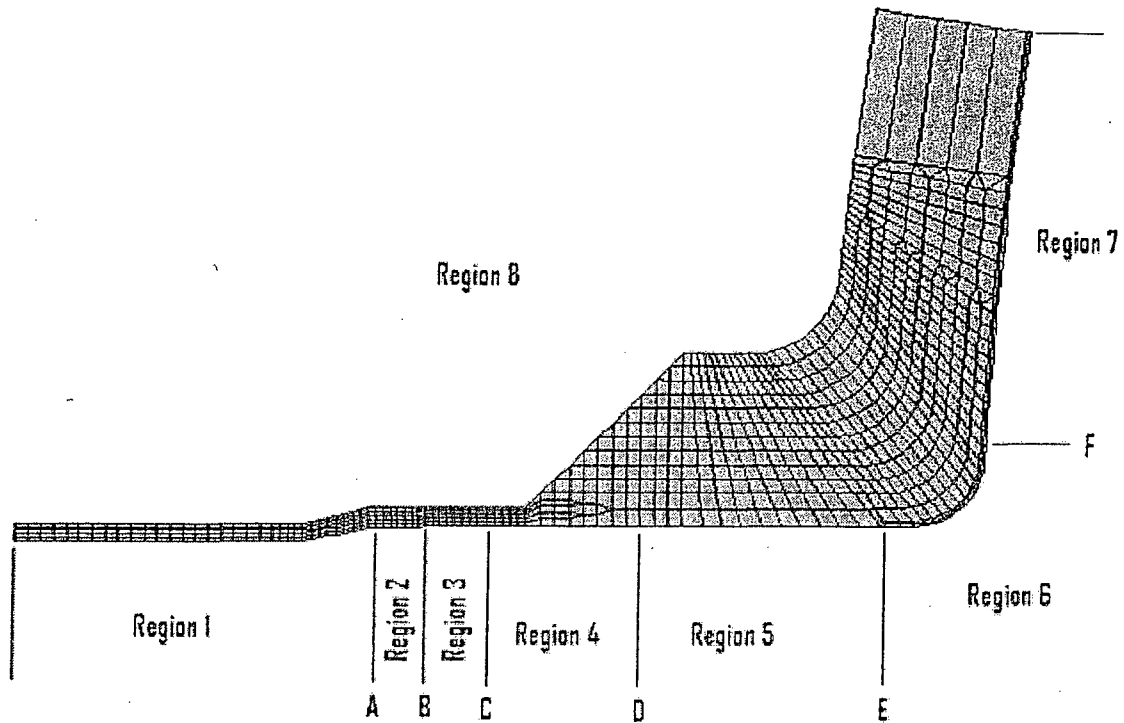
3.5 Heat Transfer Coefficients, Forced Flow and Natural Circulation

Table 4 summarizes the force flow and natural circulation heat transfer coefficients to be used in the analysis [3, Section 3.2.1]. For each flow rate, values are taken at 300°F as in the previous analysis. These values are within 11% of the maximum values for a given flow rate, and are more than 30% greater than the minimum values for a given flow rate [3, Table 4] [4, Tables 4 and 5]. Therefore, the use of heat transfer coefficients at 300°F is bounding for the most severe transients, which occur at a wide range of temperatures. Figure 1 illustrates the heat transfer coefficient regions [4, Figure 6].

Table 4: Forced Flow and Natural Circulation Heat Transfer Coefficients, Btu/hr-ft²-°F

Region	100% flow	40% flow	25% flow	0% flow, water
1	3705	1780	1222	144
2	*	*	*	*
3	1489	743	504	109
4	*	*	*	*
5	177	89	60	12
6	*	*	*	*
7	864	864	864	864
8	0.2	0.2	0.2	0.2

* Linearly transition between the values for the adjacent regions.



- Notes: Point A: End of thermal sleeve = Node 204 = 0.25" from feedwater inlet side of thermal sleeve flat.
 Point B: Beginning of annulus = Node 252.
 Point C: Beginning of thermal sleeve transition = approximately 4.0" from Point A = Node 294.
 Point D: End of thermal sleeve transition = approximately 9.5" from Point A = Node 387.
 Point E: End of inner nozzle corner (nozzle side) = Node 553.
 Point F: End of inner nozzle corner (vessel wall side) = Node 779.

Figure 1: Nozzle and Vessel Wall Thermal and Heat Transfer Boundaries

3.6 Finite Element Model

The ANSYS program [6] will be used to perform the finite element analysis. A previously-developed axisymmetric model will be used [4, file FW.INP], except that temperature-dependent material properties will be used. Table 5 shows the applicable material properties [14].



Table 5: Temperature-Dependent Material Properties

Material No.	Description	Temperature, °F	Young's Modulus, $E \times 10^6$ (psi)	Mean Coefficient of Thermal Expansion, $\alpha \times 10^{-6}$ (in/in-°F)	Conductivity, k (BTU/hr-ft-°F) (see Note 1)	Diffusivity, d (ft ² /hr)	Specific Heat, c_p (BTU/lbm-°F) (see Note 5)
1	SA533 Grade B, A508 Class II (see Note 2)	70	27.8	6.4	23.5	0.458	0.105
		200	27.1	6.7	23.6	0.425	0.114
		300	26.7	6.9	23.4	0.401	0.119
		400	26.1	7.1	23.1	0.378	0.125
		500	25.7	7.3	22.7	0.356	0.130
		600	25.2	7.4	22.2	0.336	0.135
2	SS Clad (see Note 3)	70	28.3	8.5	8.6	0.151	0.116
		200	27.6	8.9	9.3	0.156	0.122
		300	27.0	9.2	9.8	0.160	0.125
		400	26.5	9.5	10.4	0.165	0.129
		500	25.8	9.7	10.9	0.170	0.131
		600	25.3	9.8	11.3	0.174	0.133
3	A508 Class I (see Note 4)	70	29.3	6.4	35.1	0.695	0.103
		200	28.6	6.7	33.6	0.613	0.112
		300	28.1	6.9	32.3	0.561	0.118
		400	27.5	7.1	30.9	0.512	0.123
		500	27.1	7.3	29.5	0.472	0.128
		600	26.5	7.4	28.0	0.433	0.132
4	A106 Grade B (see Note 4)	70	29.3	6.4	35.1	0.695	0.103
		200	28.6	6.7	33.6	0.613	0.112
		300	28.1	6.9	32.3	0.561	0.118
		400	27.5	7.1	30.9	0.512	0.123
		500	27.1	7.3	29.5	0.472	0.128
		600	26.5	7.4	28.0	0.433	0.132

- Notes:
1. Convert to BTU/sec-in-°F for input to ANSYS.
 2. Properties of A508 Class II are used (3/4Ni-1/2Mo-1/3Cr-V).
 3. Properties of 18Cr - 8Ni austenitic stainless steel are used.
 4. Composition = C-Si; k and d for plain carbon steel are used [11].
 5. Calculated as $[k/(pd)]/12^3$.

Stresses will be extracted and linearized at two locations, both on the inside surface. The critical safe end location is Node 192, which has the highest stress intensity due to thermal loading under high flow conditions [3, Section 4.0 and Figures 6 and 7]. The corresponding linearization path is from Node 192 to Node 187 (Figure 2 [3, Figure 7]).

The critical nozzle corner location is Node 657 at the base metal of the nozzle, chosen based upon the highest pressure stress [3, Section 4.0 and Figures 8 and 9]. The corresponding linearization path is from Node 657 to Node 645 (Figure 3 [3, Figure 9]).

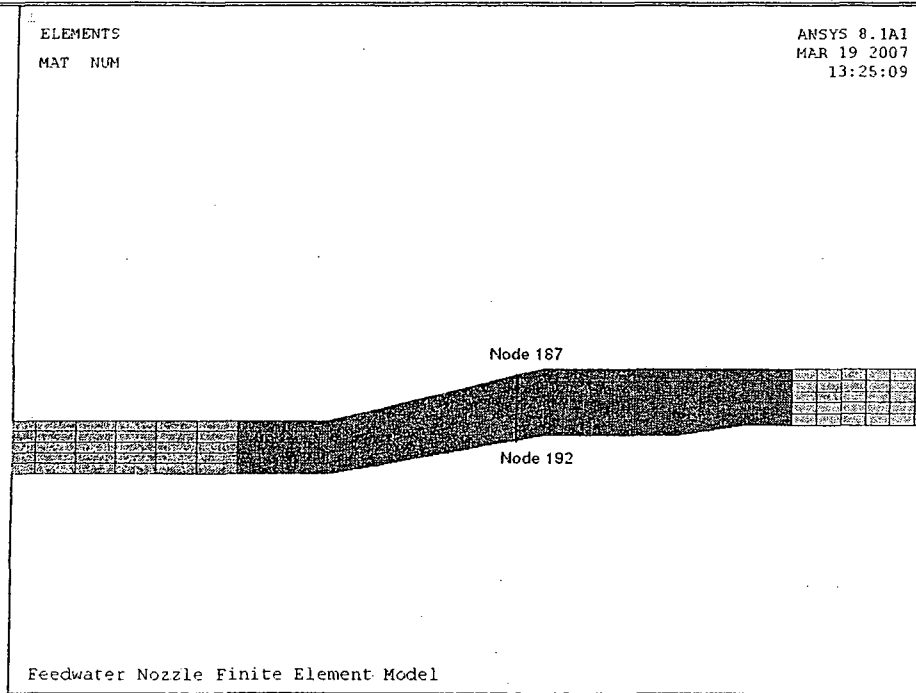


Figure 2: Safe End Linearization Path

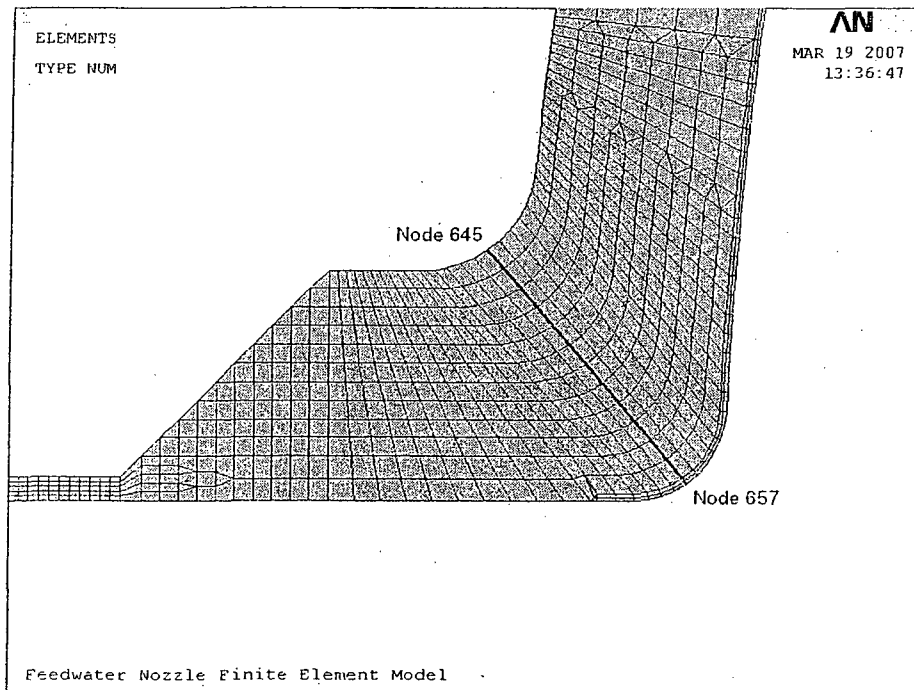


Figure 3: Nozzle Corner Linearization Path



3.7 Nozzle Corner Effects

The axisymmetric model has the effect of modeling the cylindrical RPV as spherical. To partially counter the resulting reduction in stress in the RPV wall, the radius in the model was increased by a factor of 1.5 [3, p. 8]. This yields a general membrane stress that equals the average of the hoop and axial stress for the cylinder.

Stresses from the axisymmetric analysis will need to be increased to account for the three-dimensional (3-D) geometry. A factor of 1.333 has been established in a previous calculation package that modeled the nozzle [3, p. 9], to achieve an overall pressure multiplication of 2.0. This is consistent with the maximum value used in prior VYNPS analyses [7, Appendix A, p. 4-10].

No other SCF is required at the nozzle corner inside surface, since this location has no stress riser.

3.8 Piping Interface Loads

The previous analysis of the FW nozzle calculated membrane axial and shear stresses due to the piping interface loads by closed form solution, then combined them into stress intensities for the two locations of interest [2, Section 3.4]. All shear stresses were treated as existing in the same plane.

In this analysis, the stress components are recalculated in Section 4.3 taking into account through-wall distribution. Forces and moments are taken from the same reference as before [8, Table 3].

3.9 SCFs, Safe End

In the previous analysis, an SCF of 1.34 was used for the safe end location for all load conditions [2]. That value was obtained from the original design basis evaluation for the FW nozzle. For the current analysis, the SCF is updated to reflect modern-day ASME Code fatigue usage analysis methodology for consistency with the rest of the evaluation.

At the safe end inside surface, guidance is taken from the piping analysis rules in Subarticle NB-3600 of Section III of the ASME Code [1]. These rules specify stress indices C_1 , C_2 , and C_3 , which are applied to nominal stress to yield primary plus secondary membrane plus bending stress (P+Q); and K_1 , K_2 , and K_3 , which are applied to nominal stress along with the C factors to yield total stress (P+Q+F). The subscripts indicate the type of loading: 1 for pressure, 2 for moments, and 3 for thermal transients. Stress indices for a reducer are used.

Section 4.3 contains calculations of the safe end SCFs. For stresses due to piping loads, the moment stress indices C_2 and K_2 are applied to the nominal stress components at the safe end. For pressure stresses, the ANSYS model is sufficient to account for the effects of gross structural discontinuity such that C_1 is not needed. To account for the effects of local structural discontinuity, K_1 is applied to the linearized P+Q stress to yield P+Q+F. These factors are conservatively applied to all six components of the stress tensor.

For thermal stresses, C_3 and K_3 are given as 1.0 [1, Table NB-3681(a)-1]; therefore, no SCF is required.

3.10 Environmental Fatigue Multipliers

The environmental fatigue multipliers for the safe end and nozzle corner will be calculated in accordance with NUREG/CR-6583 methodology [15].

4.0 CALCULATIONS

4.1 Heat Transfer Coefficients, Condensation

Condensation heat transfer coefficients are calculated with the formula shown in Section 3.4 for times during which the nozzle is filled with steam at Region A temperature [10, Attachment 1, p. 3]. This is done in the sheet labeled "Condensation" in Excel workbook *VY-19Q-301.xls* in the project computer files. The highest heat transfer coefficient values for the transient temperature range are used. These are provided in Table 6.

Table 6: Condensation Heat Transfer Coefficients, Btu/hr-ft²-°F

Region	0% flow, steam
1	598
2	*
3	1515
4	*
5	874
6	*
7	**
8	**

* Linearly transition between the values for the adjacent regions.

** Use values from Table 4, since these are bounding and there is no change in temperature.

4.2 Piping Interface Loads

From general structural mechanics, the membrane plus bending stresses at the inside surface of a thick-walled cylinder are:

$$\sigma_{z1} = \text{axial stress due to axial force} = F_z/A$$

$$\sigma_{z2} = \text{axial stress due to bending moment} = M_{xy}(ID/2)/I$$

$$\sigma_z = \sigma_{z1} + \sigma_{z2}$$

$$\tau_{t\theta} = \text{shear stress due to torsion} = M_z(ID/2)/J$$

$$\tau_{rz} = \text{shear stress due to shear force} = 2F_{xy}/A, \text{ where}$$

$F_x, F_y, F_z, M_x, M_y,$ and M_z are forces and moments at the pipe-to-safe end weld

$$M_{xL} = \text{moment about x axis translated by length } z = -L = M_x - F_y L$$

$$M_{yL} = \text{moment about y axis translated by length } z = -L = M_y + F_x L$$

$$M_{xy} = \text{resultant bending moment} = (M_{xL}^2 + M_{yL}^2)^{0.5}$$

$$F_{xy} = \text{resultant shear force} = (F_x^2 + F_y^2)^{0.5}$$

ID, OD = inside and outside diameters

$$A = \text{area of cross section} = (\pi/4)(OD^2 - ID^2)$$

$$I = \text{moment of inertia} = (\pi/64)(OD^4 - ID^4)$$

$$J = \text{polar moment of inertia} = (\pi/32)(OD^4 - ID^4)$$

Figure 4 shows the coordinate system for the forces and moments [8, Figure 1]. The shear stresses are expressed in a local coordinate system with r radial (X in ANSYS coordinates), θ circumferential (Z in ANSYS coordinates), and Z axial (Y in ANSYS coordinates). Table 7 shows the calculation of stresses; ID, OD, and L are taken from the previous piping load stress calculations [2, Section 3.4]. Forces and moments are taken from the same reference as before, except that signs are chosen to maximize stress [8, Table 3].

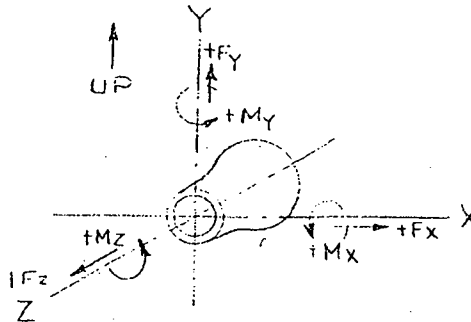


Figure 4: Coordinate System for Forces and Moments

Table 7: Membrane Plus Bending Stresses Due to Piping Loads

	Safe End	Nozzle Corner
F_x , kip	3.00	3.00
F_y , kip	-15.00	-15.00
F_z , kip	3.20	3.20
M_x , kip-in	336.00	336.00
M_y , kip-in	156.00	156.00
M_z , kip-in	480.00	480.00
L, in	12.09	27.57
M_{xL} , kip-in	517.31	749.58
M_{yL} , kip-in	192.26	238.72
M_{xy} , kip-in	551.88	786.67
F_{xy} , kip-in	15.30	15.30
OD, in	11.86	22.67
ID, in	10.409	10.750
A, in ²	25.28	312.73
I, in ⁴	393.28	12300.41
J, in ⁴	786.55	24600.82
σ_{z1} , ksi	0.127	0.010
σ_{z2} , ksi	7.304	0.344
σ_z , ksi	7.430	0.354
τ_{θ} , ksi	3.176	0.105
τ_{rz} , ksi	1.210	0.098



4.3 SCFs, Safe End

Figure 5 shows the geometry parameters used in calculating stress indices for reducers [1, Figure NB-3683.6-1], and Figure 6 shows the feedwater nozzle safe end geometry [9]. Comparing the two figures gives the following values:

$$L_1 = 0", r_1 = 0.75", D_1 = 12.000", t_1 = (12 - 10.515)/2 = 0.7425"$$

$$L_2 = 0", r_2 = 0.75", D_2 = 10.840", t_2 = (10.840 - 9.669)/2 = 0.5855"$$

$$\alpha = 10^\circ$$

(L_1 and L_2 are taken as zero because the location of interest is on the radius of curvature.)

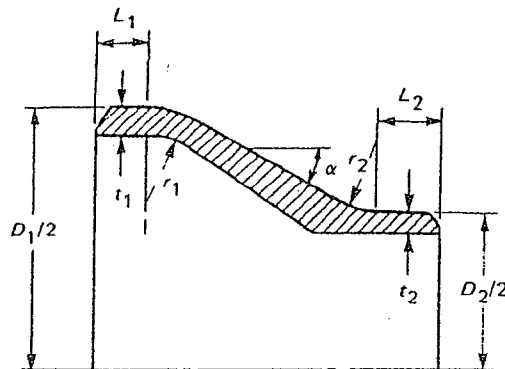


Figure 5: Reducer Geometry Parameters



Equations for stress indices are taken from the ASME Code [1, NB-3683.6]. For K_1 and K_2 , since the location of interest is not on a weld, the equation for flush welds is used:

$$K_1 = K_2 = 1.1 - 0.1 L_m / (D_m t_m)^{0.5}, \text{ where}$$

$$L_m / (D_m t_m)^{0.5} = \text{the lesser of } L_1 / (D_1 t_1)^{0.5} \text{ and } L_2 / (D_2 t_2)^{0.5}$$

Since $L_1 = L_2 = 0$, one finds:

$$K_1 = K_2 = 1.1 - 0.1 (0) = 1.1$$

Since r_1 and r_2 are less than $0.1D_1$, C_2 is given as:

$$C_2 = 1.0 + 0.0185 \alpha (D_n / t_n)^{0.5}, \text{ where}$$

$$D_n / t_n = \text{the larger of } D_1 / t_1 \text{ and } D_2 / t_2$$

The bounding D/t value is $D_2/t_2 = 10.840/0.5855 = 18.514$, so that:

$$C_2 = 1.0 + 0.0185 (10) (18.514)^{0.5} = 1.796$$

$$C_2 K_2 = 1.796 (1.1) = 1.976$$

5.0 RESULTS OF ANALYSIS

This calculation package specifies the ASME Code edition, finite element model, thermal and pressure transients (Table 1), and heat transfer coefficients (Tables 4 and 6) to be used in a fatigue usage analysis of the FW nozzle at VYNPS. Thermal transient and pressure stress components will be calculated using ANSYS, and piping load stress components are calculated herein using closed form solutions (Table 7).

Linearized stress components at Nodes 192 (safe end inside surface) and 657 (nozzle corner inside surface) will be used for the fatigue usage analysis. At the nozzle corner, P+Q and P+Q+F pressure stress components will be increased by a factor of 1.333. For the nozzle corner location, the stresses used in the evaluation shall be for the base metal only; that is, the cladding material should be unselected prior to stress extraction. At the safe end, linearized P+Q pressure stress components will be multiplied by 1.1 to yield P+Q+F pressure stress components, and nominal stress components due to piping loads are multiplied by 1.796 to yield P+Q stress components and 1.976 to yield P+Q+F stress components.

The fatigue usage analysis will consider all six stress components, and will be performed using the NB-3200 rules of Section III of the ASME Code [1]. Calculated fatigue usage factors will be multiplied by the overall F_{en} of 1.74 for the safe end [2, Section 5.0] and values to be developed in a subsequent calculation package, to be assigned file number VY-19Q-303, for the nozzle corner.



6.0 REFERENCES

1. American Society of Mechanical Engineers, Boiler and Pressure Vessel Code, Section III, Subsection NB, 1998 Edition with Addenda through year 2000.
2. SI Calculation Package, *Fatigue Analysis of Feedwater Nozzle*, Revision 0, SI File No. VY-16Q-302.
3. SI Calculation Package, *Feedwater Nozzle Stress History Development for Greens Functions*, Revision 0, SI File No. VY-16Q-301.
4. SI Calculation Package, *Feedwater Nozzle Finite Element Model and Heat Transfer Coefficients*, Revision 0, SI File No. VY-10Q-301.
5. Holman, J.P., *Heat Transfer*, Fifth Edition, McGraw-Hill, 1981.
6. ANSYS, Release 8.1 (w/Service Pack 1), ANSYS, Inc., June 2004. (Listed for reference only; this program is not used in this calculation package.)
7. Entergy Document VYC-378, Revision 0, *Vermont Yankee Reactor Cyclic Limits for Transient Events*, SI File No. VY-05Q-211.
8. GE Drawing No. 919D294, Revision 11, Sheet 7, *Reactor Vessel, Spec. Control*, SI File No. VY-05Q-241.
9. Ebasco Drawing 5920-234R1, 08/03/67, *Safe End Detail for Nozzles MK. N4A THRU N4D*, (CB&I Contract 9-6201, Drawing #M14, Revision 0, 05/02/67), SI File No. VY-05Q-215.
10. Entergy Document EC No. 1773, Revision 0 (Design Input Revision 1), *Environmental Fatigue Analysis for Vermont Yankee Nuclear Power Station*, SI File No. VY-16Q-209.
11. Letter MLH-08-001 from M.L. Herrera (SI) to N. Lobo (ASME), *ASME Code, Section II, Part D, 1998 Edition and Later, Subpart 2, Table TCD*, January 10, 2008.
12. Cheremisinoff, N., *Heat Transfer Pocket Handbook*, Gulf Publishing Co., Houston, 1984.
13. Keenan, J.H., Keyes, F.G., Hill, P.G., Moore, J.G., *Steam Tables, Thermodynamic Properties of Water Including Vapor, Liquid, and Solid Phases (English Units)*, John Wiley & Sons, 1969.
14. American Society of Mechanical Engineers, Boiler and Pressure Vessel Code, Section II, Part D, 1998 Edition with Addenda through year 2000.
15. NUREG/CR-6583 (ANL-97/18), *Effects of LWR Coolant Environments on Fatigue Design Curves of Carbon and Low-Alloy Steels*, March 1998.



Structural Integrity Associates, Inc.

File No.: VY-19Q-302

CALCULATION PACKAGE

Project No.: VY-19Q

PROJECT NAME:

Provide VY Support for Questions Related to Environmental Fatigue Analyses

CONTRACT NO.:

10163217

CLIENT:

Entergy Nuclear Operations, Inc.

PLANT:

Vermont Yankee Nuclear Power Station

CALCULATION TITLE:

ASME Code Confirmatory Fatigue Evaluation of Reactor Feedwater Nozzle

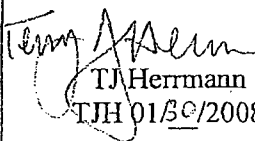
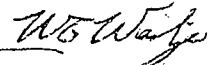
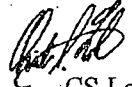
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1.0 OBJECTIVE

The objective of this calculation package is to perform an ASME Code, Section III fatigue usage calculation for the reactor pressure vessel (RPV) feedwater (FW) nozzle at Vermont Yankee Nuclear Power Station (VYNPS).

2.0 METHODOLOGY

The methodology to be used for this evaluation was established in a previous calculation package [1], and is summarized herein. A previously-developed finite element model (FEM) is analyzed using the ANSYS program [2]. Thermal transient analysis is performed for each defined transient, and the thermal stresses are added to stresses due to pressure and piping loads, which are scaled based on the magnitudes of the pressure and piping loads. Stress concentration factors (SCFs) are applied as appropriate. All six components of the stress tensor are used for stress calculations.

The fatigue calculation is performed at previously-examined locations, and uses the methodology of Subarticle NB-3200 of Section III of the ASME Code [3]. Environmental fatigue usage analysis will be performed in a separate calculation package.

3.0 DESIGN INPUTS

3.1 Finite Element Analysis

A previous calculation package specifies all design input [1]. The FEM input file is taken from the previous analysis of the FW nozzle [4, file *FW.INP*], and modified to include temperature-dependent properties [1, Table 5]. The modified file is named *FW-GEOM.INP*, and is used as input to the files in which the thermal transient and stress analyses are performed. Figure 1 shows the FEM [4, Figure 4].

For the thermal transient ANSYS analysis, previously defined thermal transients [1, Table 1] are analyzed, applying heat transfer coefficients [1, Tables 4 and 6] as appropriate based on flow rate. Bounding reactor temperature is used for Transients 12/13/15 [1, Table 1], called Transient 13 herein. (In VESLFAT, Transient 13 is run separately since it has a higher reactor pressure.) For ramps during which the flow rate undergoes a ramp change [5, Attachment 1, p. 3], the set of heat transfer coefficients with the largest values is used. This is done because ANSYS always applies changes to the heat transfer coefficients as step changes, even if the temperature undergoes a ramp change.

Note that, for three time periods during Transient 11 [1, Table 1], the nozzle is filled with steam at Region A temperature [5, Attachment 1, p. 3, Note 1], such that heat transfer coefficients for condensation apply [1, Table 6]. Since it takes a finite amount of time for the water to drain and condensation to begin, the condensation heat transfer coefficients are not applied until the load step after the Region A temperature is reached.

Stress analysis is performed using the temperature distributions calculated in the thermal transient ANSYS analysis as input. At the vessel wall, Y displacement is set to zero, and X displacement is

unconstrained, as was previously done [6, Figure 4]. At the FW pipe, Y displacement is coupled to account for the adjacent piping, as was previously done [6, files *FWS_VY_25.INP*, *FWS_VY_40.INP*, and *FWS_VY_100.INP*]. Figure 1 shows the locations of these boundary conditions.

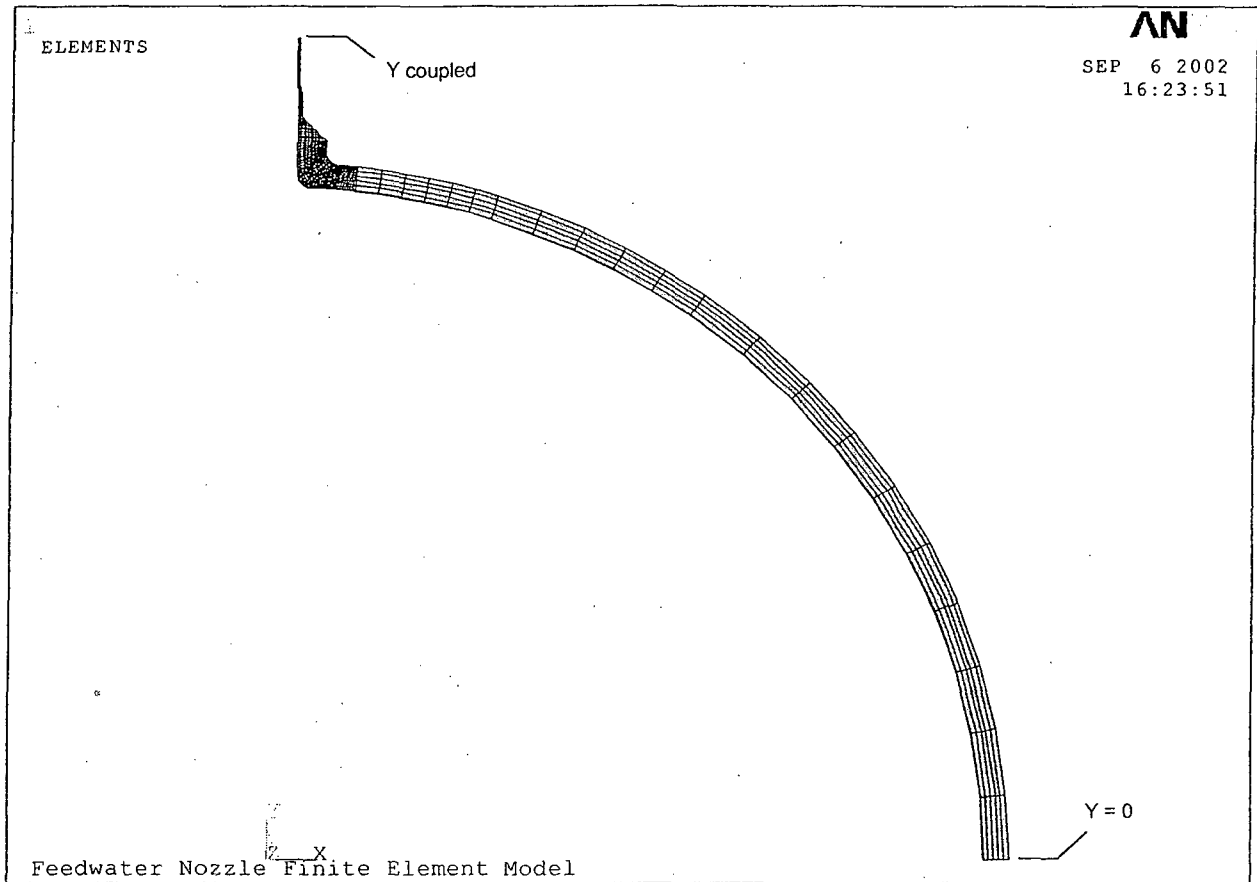


Figure 1: ANSYS Finite Element Model

All ANSYS input files, listed below, are saved in the project computer files:

- FW-GEOM.INP*: Geometry and material properties
- FW-HTBC.INP*: Set heat transfer boundary conditions
- TRAN02-T.INP*, *TRAN02-S.INP*: Transient 2, thermal and stress analysis
- TRAN03-T.INP*, *TRAN03-S.INP*: Transient 3, thermal and stress analysis
- TRAN04-T.INP*, *TRAN04-S.INP*: Transient 4, thermal and stress analysis
- TRAN05-T.INP*, *TRAN05-S.INP*: Transient 5, thermal and stress analysis
- TRAN06-T.INP*, *TRAN06-S.INP*: Transient 6, thermal and stress analysis
- TRAN09-T.INP*, *TRAN09-S.INP*: Transient 9, thermal and stress analysis
- TRAN10-T.INP*, *TRAN10-S.INP*: Transient 10, thermal and stress analysis
- TRAN11-T.INP*, *TRAN11-S.INP*: Transient 11, thermal and stress analysis
- TRAN13-T.INP*, *TRAN13-S.INP*: Transient 12/13/15, thermal and stress analysis

TRAN14-T.INP, TRAN14-S.INP: Transient 14, thermal and stress analysis
TRAN19-T.INP, TRAN19-S.INP: Transient 19, thermal and stress analysis
TRAN20-T.INP, TRAN20-S.INP: Transient 20, thermal and stress analysis
TRAN20AT.INP, TRAN20AS.INP: Transient 20A, thermal and stress analysis
TRAN21-T.INP, TRAN21-S.INP: Transient 21, thermal and stress analysis
TRAN25-T.INP, TRAN25-S.INP: Transient 25, thermal and stress analysis

3.2 Stress Calculation

Linearized stress components at Nodes 192 (safe end inside surface) and 657 (nozzle corner inside surface) are used for the fatigue usage analysis [1, Section 3.6], as shown in Figure 2 [6, Figures 7 and 9]. For the nozzle corner location, the stresses used in the evaluation are for the base metal only; that is, the cladding material is unselected prior to stress extraction. The stress components from the thermal stress analyses are combined with stress components due to pressure and piping loads. A unit pressure stress analysis was performed using ANSYS in a previous calculation package [6], and stress component results are taken from that analysis [6, files *PSE.OUT* and *PBLEND.OUT*]. Piping load stress components are taken from previous calculations using closed form solutions [1, Table 7].

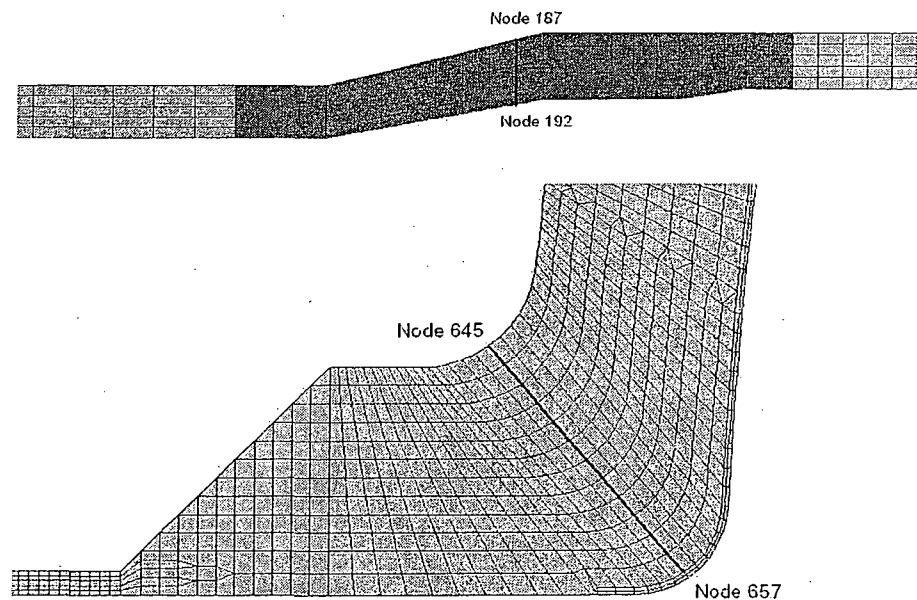


Figure 2: Linearization Paths

SCFs are applied to the pressure and piping load stress components to yield primary plus secondary membrane plus bending stress components (P+Q) and the total (primary plus secondary plus peak) stress components (P+Q+F) as specified in the methodology calculation package [1].



3.3 Fatigue Usage Analysis, General

The VESLFAT program [7] is used to perform the fatigue usage analysis in accordance with the fatigue usage portion of NB-3200 [3]. VESLFAT performs the analysis required by NB-3222.4(e) [3] for Service Levels A and B conditions defined by the user. The VESLFAT program computes the primary plus secondary and total stress ranges for all events and performs a correction for elastic-plastic analysis, if appropriate.

The program computes the stress intensity range based on the stress component ranges for all event pairs [3, NB-3216.2]. The program evaluates the stress ranges for primary plus secondary and primary plus secondary plus peak stress based upon six components of stress (3 direct and 3 shear stresses). If the primary plus secondary stress intensity range is greater than $3S_m$, then the total stress range is increased by the factor K_e , as described in NB-3228.5 [3]. The value of S_m is specified as a function of temperature. The input maximum temperature for both states of a load set pair is used to determine the temperature upon which S_m is determined from the user-defined values.

When more than one load set is defined for either of the event pair loadings, the stress differences are determined for all of the potential loadings, saving the maximum for the event pair, based on the pair producing the largest alternating total stress intensity (S_{alt}), including the effects of K_e . The principal stresses for the stress ranges are determined by solving for the roots of the cubic equation:

$$S^3 - (\sigma_x + \sigma_y + \sigma_z)S^2 + (\sigma_x \sigma_y + \sigma_y \sigma_z + \sigma_z \sigma_x - \tau_{xy}^2 - \tau_{xz}^2 - \tau_{yz}^2)S - (\sigma_x \sigma_y \sigma_z + 2 \tau_{xy} \tau_{xz} \tau_{yz} - \sigma_z \tau_{xy}^2 - \sigma_y \tau_{xz}^2 - \sigma_x \tau_{yz}^2) = 0$$

The stress intensities for the event pairs are reordered in decreasing order of S_{alt} , including a correction for the ratio of modulus of elasticity (E) from the fatigue curve divided by E from the analysis. This allows a fatigue table to be created to eliminate the number of cycles available for each of the events of an event pair, allowing determination of fatigue usage per NB-3222.4(e) [3]. For each load set pair in the fatigue table, the allowable number of cycles is determined based on S_{alt} .

For the VYNPS FW nozzle analysis, transients that consist of both upward and downward temperature and pressure ramps are split so that each successive ramp is treated separately. Table 1 shows the transients as input to VESLFAT [1, Table 1]. The numbers of cycles in Table 1 are entered in VESLFAT input files *VFAT-II.CYC* (safe end) and *VFAT-2I.CYC* (nozzle corner).

Table 1: Transients as Input to VESLFAT

VESLFAT Load Set	Transient	Start Time, sec**	Temp. Change	Pressure Change	Cycles
1	1_Boltup	0	None	None	*
2	2_DesHydro1	0	Upward	Upward	120
3	2_DesHydro2	5280	None	Downward	120
4	3_Startup	0	Upward	Upward	300
5	4_TurbRoll1	0	Downward	None	300
6	4_TurbRoll2	1801	Upward	None	300
7	5_DailyRed1	0	Downward	None	10,000
8	5_DailyRed2	2700	Upward	None	10,000
9	6_WklyRed1	0	Downward	None	2,000
10	6_WklyRed2	3600	Upward	None	2,000
11	9_TurbTrip1	0	Downward	None	10
12	9_TurbTrip2	2520	Upward	None	10
13	10_FWHByp1	0	Downward	None	70
14	10_FWHByp2	1890	Upward	None	70
15	11_LoFP1	0	Upward	Upward	10
16	11_LoFP2	3.5	Downward	Downward	10
17	11_LoFP3	184.5	Upward	None	10
18	11_LoFP4	2165.5	Downward	Downward	10
19	11_LoFP5	2346.5	Upward	Upward	10
20	11_LoFP6	6727.5	Downward	Downward	10
21	11_LoFP7	7148.5	Upward	Downward	10
22	11_LoFP8	11048.5	Upward	Upward	10
23	11_LoFP9	18212.5	Downward	None	10
24	11_LoFP10	20013.5	Upward	None	10
25	12_TGTripp1	0	None	Upward	288
26	12_TGTripp2	15	Downward	Downward	288
27	12_TGTripp3	2790	Upward	Upward	288
28	13_Overpr1	0	None	Upward	1
29	13_Overpr2	15	Downward	Downward	1
30	13_Overpr3	2790	Upward	Upward	1
31	14_SRVBlwdn	0	Downward	Downward	1
32	19_RedTo0pct	0	Downward	None	300
33	20_HSHeatup	0	Upward	None	300
34	20A_HSFWinj1	0	Downward	None	300
35	20A_HSFWinj2	181	Upward	None	300
36	21_Shutdown	0	Downward	Downward	300
37	24_HydroTest1	0	None	Upward	1
38	24_HydroTest2	1200	None	Downward	1
39	25_Unbolt	0	Downward	None	123

* Since this transient does not affect the FW nozzle, it is not considered in the cyclic evaluation.

** Note that stress peaks may occur after the start of the subsequent ramp.

3.4 Material Properties, VESLFAT

Material properties are entered in VESLFAT input files *VFAT-11.FDT* (safe end) and *VFAT-21.FDT* (nozzle corner). Table 2 lists the temperature-dependent material properties used in the analysis [1, Table 5] [8], and Table 3 lists the fatigue curve for the nozzle and safe end materials [3, Appendix I, Table I-9.1 and Figure I-9.1]. VESLFAT automatically scales the stresses by the ratio of E on the fatigue curve to E in the analysis, for purposes of determining allowable numbers of cycles, as required by the ASME Code.

Other material properties are input as follows:

$m = 3.0$, $n = 0.2$, parameters used to calculate factor K_e , safe end [9]

$m = 2.0$, $n = 0.2$, parameters used to calculate factor K_e , nozzle corner [9]

E from fatigue curve = 30,000 ksi [3, Appendix I, Table I-9.1 and Figure I-9.1] [9]

Table 2: Temperature-Dependent Material Properties, VESLFAT

Material	T, °F	E, psi	S_m , ksi	S_y , ksi
A508 Class I (safe end)	70	$29.3(10)^6$	23.3	36.0
	200	$28.6(10)^6$	21.9	33.0
	300	$28.1(10)^6$	21.3	31.8
	400	$27.5(10)^6$	20.6	30.8
	500	$27.1(10)^6$	19.4	29.3
	600	$26.5(10)^6$	17.8	27.6
A508 Class II (nozzle)	70	$27.8(10)^6$	26.7	50.0
	200	$27.1(10)^6$	26.7	47.0
	300	$26.7(10)^6$	26.7	45.5
	400	$26.1(10)^6$	26.7	44.2
	500	$25.7(10)^6$	26.7	43.2
	600	$25.2(10)^6$	26.7	42.1

Table 3: Carbon/Low Alloy Steel Fatigue Curve

Number of Cycles	S_a , ksi
10	580
20	410
50	275
100	205
200	155
500	105
1,000	83
2,000	64
5,000	48
10,000	38
20,000	31
50,000	23
100,000	20
200,000	16.5
500,000	13.5
1,000,000	12.5



4.0 CALCULATIONS

Table 4 contains the stress components at the locations of interest for the 1,000 psi pressure case [6, files *PSE.OUT* and *PBLEND.OUT*] and for the piping loads [1, Table 7], corresponding to a reactor temperature of 575°F [1, Section 3.1.8].

Table 4: Stress Components Before SCF, psi

Loading	Type	Node	S _x	S _y	S _z	S _{xy}	S _{xz}	S _{yz}
Unit	Membrane	192	-810.7	6116	7853	-450.1	0	0
Pressure,	plus Bending	657	-705.2	1198	24020	-3590	0	0
1,000 psi	Total	657	-705.2	985.5	27590	-121.1	0	0
Piping Loads	Nominal	192	0	7430	0	1210	3176	0
at 575°F		657	0	354	0	98	105	0

SCFs are applied to the pressure and piping load stress components to yield P+Q and P+Q+F stress components as follows [1]:

Pressure:

Safe end (Node 192):

Membrane plus bending from ANSYS equals P+Q

Membrane plus bending from ANSYS is multiplied by 1.1 to yield P+Q+F

Nozzle corner (Node 657):

Membrane plus bending from ANSYS is multiplied by 1.333 to yield P+Q

Total stress from ANSYS is multiplied by 1.333 to yield P+Q+F

Piping Loads:

Safe end (Node 192):

Nominal stresses are multiplied by 1.796 to yield P+Q

Nominal stresses are multiplied by 1.976 to yield P+Q+F

Nozzle corner (Node 657):

Nominal stresses are used as is for P+Q and P+Q+F

Table 5 shows the stress components with SCFs. The piping load stress components are applied as having negative signs, to yield the largest stress component ranges.

Table 5: Stress Components With SCF, psi

Load	Node	Membrane plus Bending						Total					
		S _x	S _y	S _z	S _{xy}	S _{xz}	S _{yz}	S _x	S _y	S _z	S _{xy}	S _{xz}	S _{yz}
Pressure	192	-811	6116	7853	-450	0	0	-892	6728	8638	-495	0	0
	657	-940	1597	32019	-4785	0	0	-940	1314	36777	-161	0	0
Piping	192	0	13344	0	2173	5704	0	0	14682	0	2391	6276	0
	657	0	354	0	98	105	0	0	354	0	98	105	0

The calculations of VESLFAT stress input are automated in Excel workbooks *VFAT-1I.XLS* (safe end) and *VFAT-2I.XLS* (nozzle). These files are organized with sheets labeled as follows:

- Overview: Contains general information.

- Other Stresses: Contains calculation of pressure and piping load as shown in Tables 4 and 5.
- Rearranger: There are 16 Rearranger sheets, one for each transient as analyzed by ANSYS. In these sheets, thermal stresses are copied from Excel workbook *StressResults.xls*, which contains the results of the ANSYS stress linearization for each transient, and rearranged to conform to VESLFAT input format (including switching the shear stress components S_{xz} and S_{yz} as required by VESLFAT). Time-varying scale factors for the piping loads (based on FW nozzle fluid temperature) and pressure are determined, and used to scale the unit load stresses, which are then added to the thermal stresses. Time-varying pressure is also included in the VESLFAT stress input. The VESLFAT stress input also includes time-varying metal temperature, from the ANSYS output, which is used to determine temperature-dependent properties from the values in Table 2.
- VESLFAT: Contains the VESLFAT stress input, obtained from sheets named Rearranger. Load set numbers are entered on this sheet, as defined in Table 1. These sheets are saved to VESLFAT input files *VFAT-11.STR* (safe end) and *VFAT-21.STR* (nozzle corner). To avoid double counting of stress states, the initial time steps of each load set before the first stress peak are not included.

The files with extension STR are edited if necessary to remove some intermediate stress points, since VESLFAT has a limit of 3,000 total stress states.

5.0 RESULTS OF ANALYSIS

Tables 6 and 7 give the detailed fatigue usage results for the safe end and the nozzle corner, respectively, from VESLFAT output files *VFAT-11.FAT* (safe end) and *VFAT-21.FAT* (nozzle corner). All VESLFAT input and output files are saved in the project computer files.

Table 6: Fatigue Usage Results for Safe End

Load Set A	Load Set B	n	S_{11} , psi	K_e	S_{alt} , psi	N	Usage
15 11_LoFP1	18 11_LoFP4	10	61435	1.115	57352	2836.19	0.0035
20 11_LoFP6	27 12_TGTrip3	10	49698	1	40800	8098.01	0.0012
27 12_TGTrip3	34 20A_HSFWinj1	278	42194	1	37182	10769	0.0258
30 13_Overpr3	34 20A_HSFWinj1	1	42194	1	37182	10769	0.0001
33 20_HSHeatup	34 20A_HSFWinj1	21	42563	1	35966	12060	0.0017
5 4_TurbRoll1	33 20_HSHeatup	279	43986	1	35597	12491	0.0223
6 4_TurbRoll2	23 11_LoFP9	10	39882	1	32197	17579	0.0006
5 4_TurbRoll1	6 4_TurbRoll2	21	39842	1	32178	17615	0.0012
16 11_LoFP2	35 20A_HSFWinj2	10	40708	1	31762	18413	0.0005
35 20A_HSFWinj2	37 24_HydroTest1	1	20956	1	13081	664055	0.0000
17 11_LoFP3	35 20A_HSFWinj2	10	20399	1	12667	887275	0.0000
19 11_LoFP5	35 20A_HSFWinj2	10	19602	1	12135	infinite	0.0000
						TOTAL =	0.0571



Table 7: Fatigue Usage Results for Nozzle Corner

Load Set A	Load Set B	n	S _n , psi	K _c	S _{alt} , psi	N	Usage
2 2_DesHydro1	16 11_LoFP2	10	65109	1	46047	5655.78	0.0018
2 2_DesHydro1	20 11_LoFP6	10	50344	1	43990	6477.19	0.0015
2 2_DesHydro1	18 11_LoFP4	10	50150	1	43205	6832.83	0.0015
2 2_DesHydro1	11 9_TurbTrip1	10	65712	1	43011	6924.58	0.0014
2 2_DesHydro1	5 4_TurbRoll1	80	64296	1	43008	6925.97	0.0116
5 4_TurbRoll1	39 25_Unbolt	123	63308	1	41430	7738.36	0.0159
3 2_DesHydro2	5 4_TurbRoll1	97	61437	1	40391	8343.98	0.0116
3 2_DesHydro2	23 11_LoFP9	10	63138	1	40101	8524.36	0.0012
3 2_DesHydro2	34 20A_HSFWinj1	13	49069	1	39657	8810.73	0.0015
34 20A_HSFWinj1	38 24_HydroTest2	1	49097	1	39622	8833.38	0.0001
34 20A_HSFWinj1	36 21_Shutdown	286	49111	1	39616	8837.88	0.0324
26 12_TGTripp2	36 21_Shutdown	14	60379	1	38556	9578.4	0.0015
21 11_LoFP7	26 12_TGTripp2	10	49395	1	33091	16015	0.0006
26 12_TGTripp2	31 14_SRVBlwdn	1	42902	1	27518	28831	0.0000
22 11_LoFP8	26 12_TGTripp2	10	32212	1	24687	40237	0.0002
4 3_Startup	26 12_TGTripp2	253	30212	1	23513	46728	0.0054
4 3_Startup	29 13_Overpr2	1	30212	1	23513	46728	0.0000
4 3_Startup	28 13_Overpr1	1	28966	1	19423	111118	0.0000
4 3_Startup	32 19_RedTo0pct	45	24083	1	17665	156402	0.0003
32 19_RedTo0pct	33 20_HSHeatup	255	18765	1	12883	761835	0.0003
13 10_FWHByp1	33 20_HSHeatup	45	19637	1	12679	879615	0.0001
13 10_FWHByp1	35 20A_HSFWinj2	25	20388	1	12624	914934	0.0000
35 20A_HSFWinj2	37 24_HydroTest1	1	19850	1	12359	infinite	0.0000
9 6_WklyRed1	35 20A_HSFWinj2	274	19341	1	11952	infinite	0.0000
TOTAL =							0.0889

6.0 CONCLUSIONS AND DISCUSSIONS

A previously-developed FEM was analyzed using the ANSYS program. Thermal transient analysis was performed for each defined transient, and the thermal stresses were added to stresses due to pressure and piping loads, which were scaled based on the magnitudes of the pressure and piping loads. SCFs were applied as appropriate. All six components of the stress tensor were used for stress calculations. The fatigue calculation was performed at previously-examined locations, and used the methodology of Subarticle NB-3200 of Section III of the ASME Code.

The 60-year CUF for the safe end location was determined to be 0.0571, and the CUF for the nozzle corner location was determined to be 0.0889. Both values are less than the ASME Code allowable value of 1.0.

Environmental fatigue usage analysis will be performed in a separate calculation package.



7.0 REFERENCES

1. SI Calculation Package, *Design Inputs and Methodology for ASME Code Confirmatory Fatigue Usage Analysis of Reactor Feedwater Nozzle*, Revision 0, SI File No. VY-19Q-301.
2. ANSYS, Release 8.1 (w/Service Pack 1), ANSYS, Inc., June 2004.
3. American Society of Mechanical Engineers, Boiler and Pressure Vessel Code, Section III, Subsection NB, 1998 Edition with Addenda through year 2000.
4. SI Calculation Package, *Feedwater Nozzle Finite Element Model and Heat Transfer Coefficients*, Revision 0, SI File No. VY-10Q-301.
5. Entergy Document EC No. 1773, Revision 0 (Design Input Revision 1), *Environmental Fatigue Analysis for Vermont Yankee Nuclear Power Station*, SI File No. VY-16Q-209.
6. SI Calculation Package, *Feedwater Nozzle Stress History Development for Green Functions*, Revision 0, SI File No. VY-16Q-301.
7. VESLFAT, Version 1.42, 02/06/07, Structural Integrity Associates.
8. American Society of Mechanical Engineers, Boiler and Pressure Vessel Code, Section II, Part D, 1998 Edition with Addenda through year 2000.
9. SI Calculation Package, *Fatigue Analysis of Feedwater Nozzle*, Revision 0, SI File No. VY-16Q-302.



Structural Integrity Associates, Inc.

File No.: VY-19Q-303

CALCULATION PACKAGE

Project No.: VY-19Q

PROJECT NAME:

Provide VY Support for Questions Related to Environmental Fatigue Analyses

CONTRACT NO.:

10163217

CLIENT:

Entergy Nuclear Operations, Inc.

PLANT:

Vermont Yankee Nuclear Power Station

CALCULATION TITLE:

Feedwater Nozzle Environmental Fatigue Evaluation

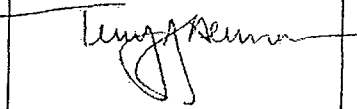
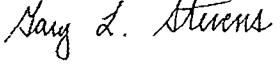

Document Revision	Affected Pages	Revision Description	Project Manager Approval Signature & Date	Preparer(s) & Checker(s) Signatures & Date
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1.0 INTRODUCTION

The purpose of this calculation is to perform a plant-specific evaluation of reactor water environmental effects for the reactor pressure vessel (RPV) feedwater nozzle identified in NUREG/CR-6260 [1] for the older vintage General Electric (GE) plant for the Vermont Yankee Nuclear Power Station (VY).

2.0 APPROACH

Per Chapter X, "Time-Limited Aging Analyses Evaluation of Aging Management Programs Under 10 CFR 54.21(c)(1)(iii)," Section X.M1, "Metal Fatigue of Reactor Coolant Pressure Boundary," of the Generic Aging Lessons Learned (GALL) Report [2], detailed, vintage-specific, fatigue calculations are required for plants applying for license renewal for the locations identified for the appropriate vintage plant in NUREG/CR-6260.

In this calculation, detailed environmentally assisted fatigue (EAF) calculations are performed for VY for one of the locations associated with the older vintage GE plant in NUREG/CR-6260. The older-vintage GE plant is the appropriate comparison to VY since the original piping design at VY was in accordance with USAS B31.1 [3], as well as the fact that the older-vintage boiling water reactor (BWR) in NUREG/CR-6260 was a BWR-4 plant, which is the same as VY.

Entergy performed an initial assessment of EAF effects for VY in their License Renewal Application (LRA) that was submitted to the NRC in January 2006. Table 4.3-3 of the VY LRA provides the results of those evaluations. All but two of the VY locations evaluated for EAF in the LRA did not yield acceptable results for 60 years of operation, as they were based on generic analysis results from NUREG/CR-6260 that were not VY-specific. Plant-specific analyses have been recently completed to address those components for VY. Relevant chemistry input for this calculation is contained in Reference [5]. This calculation documents the EAF evaluation for the feedwater nozzle locations.



3.0 METHODOLOGY

Per Section X.M1 of the GALL Report [2], the EAF evaluation must use the appropriate F_{en} relationships from NUREG/CR-6583 [4] (for carbon/low alloy steels), which are the materials under consideration for the feedwater nozzle. Per Figure 2 and Table 2 of Reference [6], the two locations being evaluated are the feedwater nozzle safe end (carbon steel) and the feedwater nozzle forging corner (low alloy steel). Based on the materials of these locations, the appropriate expressions are:

For Carbon Steel [4, p. 69]:
$$F_{en} = \exp(0.585 - 0.00124T' - 0.101S^*T^*O^*\dot{\epsilon}^*) \quad (1)$$

Substituting $T' = 25^\circ\text{C}$ in the above expression, as required by NUREG/CR-6583 to relate room temperature air data to service temperature data in water [7], the following is obtained:

$$F_{en} = \exp(0.585 - 0.00124(25^\circ\text{C}) - 0.101 S^* T^* O^* \dot{\epsilon}^*) \quad (2)$$

$$= \exp(0.554 - 0.101 S^* T^* O^* \dot{\epsilon}^*) \quad (3)$$

For Low Alloy Steel [4, p. 69]:
$$F_{en} = \exp(0.929 - 0.00124T' - 0.101S^*T^*O^*\dot{\epsilon}^*) \quad (4)$$

Substituting $T' = 25^\circ\text{C}$ in the above expression, as required by NUREG/CR-6583 to relate room temperature air data to service temperature data in water [7], the following is obtained:

$$F_{en} = \exp(0.929 - 0.00124(25^\circ\text{C}) - 0.101 S^* T^* O^* \dot{\epsilon}^*) \quad (5)$$

$$= \exp(0.898 - 0.101 S^* T^* O^* \dot{\epsilon}^*) \quad (6)$$

- where [4, pp. 60 and 65]:
- F_{en} = fatigue life correction factor
 - S^* = S for $0 < \text{sulfur content}, S \leq 0.015 \text{ wt. } \%$
 - = 0.015 for $S > 0.015 \text{ wt. } \%$
 - T^* = 0 for $T < 150^\circ\text{C}$
 - = $(T - 150)$ for $150 \leq T \leq 350^\circ\text{C}$
 - T = fluid service temperature ($^\circ\text{C}$)
 - O^* = 0 for dissolved oxygen, $\text{DO} < 0.05 \text{ parts per million (ppm)}$
 - = $\ln(\text{DO}/0.04)$ for $0.05 \text{ ppm} \leq \text{DO} \leq 0.5 \text{ ppm}$
 - = $\ln(12.5)$ for $\text{DO} > 0.5 \text{ ppm}$
 - $\dot{\epsilon}^*$ = 0 for strain rate, $\dot{\epsilon}^* > 1\%/ \text{sec}$
 - = $\ln(\dot{\epsilon}^*)$ for $0.001 \leq \dot{\epsilon}^* \leq 1\%/ \text{sec}$
 - = $\ln(0.001)$ for $\dot{\epsilon}^* < 0.001\%/ \text{sec}$

Bounding F_{en} values were determined in Reference [5]. The values determined in Table 3 of Reference [5] will be used for the carbon steel feedwater nozzle safe end location, where feedwater DO levels are low and the F_{en} value is a constant value of 1.74 for all temperatures for both hydrogen water chemistry (HWC) and normal water chemistry (NWC) conditions. For the low alloy steel



nozzle corner location, the applicable F_{en} values are shown in Table 4 of Reference [5]. Since there is a significant variation in values with temperature, F_{en} values will be computed for each load pair in the detailed fatigue calculation for this location.

The environmental fatigue is determined as $U_{env} = (U) (F_{en})$, where U is the original fatigue usage and U_{env} is the environmentally assisted fatigue (EAF) usage factor. All calculations can be found in Excel spreadsheet "VY-19Q-303 (Env. Fat. Calcs).xls" associated with this calculation.

From Table 1 of Reference [5], the following water chemistry input applies for the low alloy steel nozzle corner location:

- Over the 60-year operating life of the plant, HWC conditions exist for 47% of the time, and NWC conditions exist for 53% of the time.
- For the RPV Upper Region, which is applicable to the nozzle corner location, DO is 114 ppb pre-HWC and 97 ppb post-HWC.

With these assumptions, the cumulative usage factor (CUF) values documented in this calculation are considered applicable for sixty years of operation including all relevant EAF and EPU effects.

4.0 CALCULATIONS

From Table 6 of Reference [6], the CUF for the safe end for 60 years of operation is 0.0571. Thus, the EAF CUF for 60 years is $0.0571 \times 1.74 = 0.0994$, which is less than the allowable value of 1.0 and is therefore acceptable.

The CUF for the nozzle corner for 60 years of operation is shown in Table 7 of Reference [6], and has a value of 0.0889. This calculation is reproduced in Table 1, along with EAF calculations on a load pair basis using the F_{en} expression in Equation (6) above for low alloy steel. The final EAF CUF for 60 years is 0.3531, which is less than the allowable value of 1.0 and is therefore acceptable. The overall F_{en} multiplier for this location is 3.97.

5.0 CONCLUSIONS

In this calculation, EAF calculations were performed in accordance with the GALL Report [2] for the feedwater nozzle safe end (carbon steel) and nozzle corner (low alloy steel) locations. These locations were selected based on the locations identified in NUREG/CR-6260 for the older vintage GE plant and plant-specific fatigue calculations that determined the limiting locations for VY. Calculations for the remaining NUREG/CR-6260 locations are documented in other calculations.

The EAF results for the locations identified above indicate that the fatigue usage factors, including environmental effects, are within the allowable value for 60 years of operation. The calculations for both locations make use of the 60-year projected cycles for VY and incorporate EPU effects



(conservatively assumed to apply for all 60 years of operation). Therefore, no additional evaluation is required for these components, and the GALL requirements are satisfied.

6.0 REFERENCES

1. NUREG/CR-6260 (INEL-95/0045), "Application of NUREG/CR-5999 Interim Fatigue Curves to Selected Nuclear Power Plant Components," March 1995.
2. NUREG-1801, Revision 1, "Generic Aging Lessons Learned (GALL) Report," U. S. Nuclear Regulatory Commission, September 2005.
3. USAS B31.1.0 – 1967, USA Standard Code for Pressure Piping, "Power Piping," American Society of Mechanical Engineers, New York.
4. NUREG/CR-6583 (ANL-97/18), "Effects of LWR Coolant Environments on Fatigue Design Curves of Carbon and Low-Alloy Steels," March 1998.
5. SI Calculation Package, *Environmental Fatigue Evaluation of Reactor Recirculation Inlet Nozzle and Vessel Shell/Bottom Head*, Revision 0, SI File No. VY-16Q-303.
6. SI Calculation Package, *ASME Code Confirmatory Fatigue Evaluation of Reactor Feedwater Nozzle*, Revision 0, SI File No. VY-19Q-302.
7. EPRI/BWRVIP Memo No. 2005-271, "Potential Error in Existing Fatigue Reactor Water Environmental Effects Analyses," July 1, 2005.

Table 1: EAF Calculations for the Feedwater Nozzle Corner

CUF Calculation from Table 7 of Reference [6]:

 EAF Calculations: **HWC DO** **NWC DO**
 % HWC = 97 **114** **ppb**
 = 47% = 53% = % NWC

Index	Load #1	Description #1	n ₁ (cycles)	Load #2	Description #2	n ₂ (cycles)	n (cycles)	S _n (psi)	K _t	S ₁₀ (psi)	t _{allow}	U	T _{Max} (°F) ^[1]	T _{Max} (°C)	HWC F _{en} ^[2]	NWC F _{en} ^[2]	U _{eqv} ^[3]
1	2	2_DesHydro1	120	16	11_LoFP2	10	10	65,109	1,000	46,047	5,655.78	0.00177	356.0	150.0	3.242	3.410	0.00589
2	2	2_DesHydro1	110	20	11_LoFP6	10	10	50,344	1,000	43,890	6,477.19	0.00164	381.0	153.9	3.687	2.971	0.00592
3	2	2_DesHydro1	100	18	11_LoFP4	10	10	50,150	1,000	43,204	6,632.83	0.00146	389.0	158.3	3.842	4.159	0.00588
4	2	2_DesHydro1	90	11	9_TurbTrip1	10	10	55,712	1,000	43,911	6,924.58	0.00144	351.0	177.2	3.159	3.308	0.00468
5	2	2_DesHydro1	80	5	4_TurbRoll1	300	80	64,296	1,000	43,008	6,525.97	0.01155	330.0	182.2	3.309	3.454	0.03936
6	5	4_TurbRoll1	220	39	25_Unbolt	123	123	63,308	1,000	41,430	7,738.36	0.01589	360.0	182.2	3.309	3.494	0.05418
7	3	2_DesHydro2	120	5	4_TurbRoll1	97	97	61,437	1,000	40,391	6,343.98	0.01193	360.0	182.2	3.309	3.494	0.03961
8	3	2_DesHydro2	23	23	11_LoFP9	10	10	63,139	1,000	40,101	6,524.36	0.00117	353.0	178.3	3.192	3.349	0.06384
9	3	2_DesHydro2	13	34	20A_HSFWinj1	300	13	49,069	1,000	39,557	6,810.73	0.00148	368.0	197.8	3.823	4.144	0.00589
10	34	20A_HSFWinj1	287	38	24_HydroTest2	1	1	49,097	1,000	39,522	6,833.38	0.00011	368.0	197.8	3.823	4.144	0.00045
11	34	20A_HSFWinj1	286	36	21_Shutdown	300	286	49,111	1,000	39,616	6,837.88	0.00236	368.0	197.8	3.823	4.144	0.12921
12	26	12_TGTripp2	298	26	21_Shutdown	14	14	60,379	1,000	38,596	9,578.40	0.00146	349.0	176.1	3.127	3.268	0.00468
13	21	11_LoFP7	10	26	12_TGTripp2	274	10	49,395	1,000	33,091	16,615.00	0.00052	424.0	217.8	4.601	5.160	0.00305
14	26	12_TGTripp2	264	31	14_SRVBlwdn	1	1	42,902	1,000	27,518	28,831.00	0.00003	349.0	176.1	3.127	3.268	0.00011
15	22	11_LoFP8	10	26	12_TGTripp2	263	10	32,212	1,000	24,687	40,237.00	0.00026	538.0	281.1	8.277	10.330	0.00233
16	4	3_Startup	300	26	12_TGTripp2	253	253	30,212	1,000	23,513	46,728.00	0.00541	503.0	261.7	6.912	8.347	0.00454
17	4	3_Startup	47	29	13_Overpr2	1	1	30,212	1,000	23,513	46,728.00	0.00002	503.0	261.7	6.912	8.347	0.00016
18	4	3_Startup	46	28	13_Overpr1	1	1	28,966	1,000	19,423	111,118.00	0.00001	503.0	261.7	6.912	8.347	0.00007
19	4	3_Startup	45	32	19_RedToOpct	300	45	24,083	1,000	17,665	156,402.00	0.00029	503.0	261.7	6.912	8.347	0.00021
20	32	19_RedToOpct	255	33	20_HSHHeatup	300	255	18,765	1,000	12,883	761,835.00	0.00033	543.0	263.9	8.493	10.649	0.00323
21	13	10_FWHByp1	70	33	20_HSHHeatup	45	45	15,637	1,000	12,679	679,615.00	0.00005	548.0	266.7	8.714	10.978	0.00051
22	13	10_FWHByp1	25	35	20A_HSFWinj2	300	25	20,388	1,000	12,624	914,934.00	0.00003	549.0	287.2	8.759	11.045	0.00027
23	35	20A_HSFWinj2	275	37	24_HydroTest1	1	1	19,850	1,000	12,359	infinite	0.00000	546.0	286.7	8.714	10.978	0.00000
24	9	6_VklyRed1	2000	35	20A_HSFWinj2	274	274	19,341	1,000	11,352	infinite	0.00000	548.0	286.7	8.714	10.978	0.00000
Total, U ₁₀ = 0.08893													Total, U _{10-NWC} = 0.35306				
													Overall F _{en} = 3.970				

Transient Maximum Temperatures:

From "Fat-2i A1"

Index	Load #1	Description #1	n ₁ (cycles)	Load #2	Description #2	Index	T1 ^[1]	s1 ^[2]	T2 ^[1]	s2 ^[2]	S _n (psi)	T (°F) ^[1]
1	2	2_DesHydro1	120	16	11_LoFP2	1	2	30	16	14	65,109	356
2	2	2_DesHydro1	110	20	11_LoFP6	2	2	30	20	3	50,344	381
3	2	2_DesHydro1	100	18	11_LoFP4	3	2	30	18	7	50,150	389
4	2	2_DesHydro1	90	11	9_TurbTrip1	4	2	30	11	6	55,712	351
5	2	2_DesHydro1	80	5	4_TurbRoll1	5	2	30	5	12	64,296	360
6	5	4_TurbRoll1	220	39	25_Unbolt	6	5	12	39	23	63,308	360
7	3	2_DesHydro2	120	5	4_TurbRoll1	7	3	1	5	12	61,437	360
8	3	2_DesHydro2	23	23	11_LoFP9	8	3	1	23	7	63,139	363
9	3	2_DesHydro2	13	34	20A_HSFWinj1	9	3	1	34	21	49,069	388
10	34	20A_HSFWinj1	287	38	24_HydroTest2	10	34	21	38	1	49,097	388
11	34	20A_HSFWinj1	286	36	21_Shutdown	11	34	21	36	156	49,111	388
12	26	12_TGTripp2	298	26	21_Shutdown	12	26	6	36	156	60,379	349
13	21	11_LoFP7	10	26	12_TGTripp2	13	21	68	26	7	49,395	424
14	26	12_TGTripp2	264	31	14_SRVBlwdn	14	26	6	31	41	42,902	349
15	22	11_LoFP8	10	26	12_TGTripp2	15	22	1	26	7	32,212	538
16	4	3_Startup	300	26	12_TGTripp2	16	4	1	26	7	30,212	503
17	4	3_Startup	47	29	13_Overpr2	17	4	1	29	7	30,212	503
18	4	3_Startup	46	28	13_Overpr1	18	4	1	28	1	28,966	503
19	4	3_Startup	45	32	19_RedToOpct	19	4	1	32	45	24,083	503
20	32	19_RedToOpct	255	33	20_HSHHeatup	20	32	45	33	26	18,765	543
21	13	10_FWHByp1	70	33	20_HSHHeatup	21	13	23	33	26	15,637	548
22	13	10_FWHByp1	25	35	20A_HSFWinj2	22	13	23	35	14	20,388	549
23	35	20A_HSFWinj2	275	37	24_HydroTest1	23	35	14	37	1	19,850	548
24	9	6_VklyRed1	2000	35	20A_HSFWinj2	24	9	10	35	14	19,341	548

- Notes: 1. T_{Max} is the maximum temperature of the two paired load states, and represents the metal (node) temperature at the location being analyzed. This is determined from the VESL FAT output from Reference [6], which is included as "T" in the "Transient Maximum Temperatures" table above.
2. F_{en} values computed using Equation (6) with S* conservatively set to a maximum value of 0.615, and the transformed s_{min} rate conservatively set to a minimum value of ln(0.001) = -6.906 for all load pairs.
3. U_{eqv} = [U x HWC F_{en} x % HWC] + [U x NWC F_{en} x % NWC].
4. T1 and T2 represent the load number for Load #1 and Load #2, respectively, and s1 and s2 represent the state number for each of these loads.

LICENSEE: Entergy Nuclear Operations, Inc.

FACILITY: Vermont Yankee Nuclear Power Station

SUBJECT: SUMMARY OF MEETING HELD ON JANUARY 8, 2008, BETWEEN THE U.S. NUCLEAR REGULATORY COMMISSION STAFF AND ENTERGY NUCLEAR OPERATIONS, INC. REPRESENTATIVES TO DISCUSS THE RESPONSE TO A REQUEST FOR ADDITIONAL INFORMATION PERTAINING TO THE VERMONT YANKEE NUCLEAR POWER STATION LICENSE RENEWAL APPLICATION

On January 8, 2008, the Nuclear Regulatory Commission staff (the staff) met with members of Entergy Nuclear Operations, Inc. (the applicant) in a public meeting to discuss the response to a request for additional information (RAI) made by the staff pertaining to the Vermont Yankee Nuclear Power Station (VYNPS) license renewal application. The applicant had an opportunity to comment on this summary.

A list of attendees is provided in Enclosure 1. The meeting agenda is provided in Enclosure 2. Comments made by the public during the meeting are provided in Enclosure 3. A copy of the slides presented by the applicant is provided as Enclosure 4. A summary of the discussion follows:

Background

In a letter dated November 27, 2007, the staff issued RAI 4.3.3-2 to the applicant. The purpose of the request was to gather additional information on the calculations used at VYNPS to reanalyze their time-limited aging analysis (TLAA) that addresses environmentally-assisted fatigue. In a letter dated December 11, 2007, the applicant provided its response to RAI 4.3.3-2 to the staff. The staff reviewed the response and further questioned the methodology described in the submittal and statements made that shear stresses are negligible during a conference call with the applicant on December 18, 2007. During the call, the applicant requested a face-to-face meeting to ensure that its position pertaining to this highly technical issue was properly and effectively communicated.

Discussion

During the meeting, the applicant made a slide presentation of the reactor vessel nozzle environmental fatigue analyses for license renewal at VYNPS. The applicant reviewed the terminology used in the response to RAI 4.3.3-2. The applicant reviewed the analyzed vessel nozzle configurations to identify where the shear stresses in the nozzles were negligible, where they were not, and the effects on the fatigue analysis at the locations where shear stresses were not negligible.

The applicant explained that nozzle corner, blend radius, and inner radius are interchangeable terms for locations with geometrical discontinuities; that is, locations where stresses are maximum. The applicant explained that the methodology employed incorporates the use of axisymmetric modeling rather than an exact configuration 3-Dimensional (3-D) or 2-Dimensional (2-D) modeling. They explained that axisymmetric modeling is a 3-D model except that it models the nozzle/vessel interface as a sphere with a multiplier to account for pressure stress effects, rather than a pipe joined to a cylinder. With the aid of colored graphs, the applicant demonstrated specific nozzles where shear stresses are negligible. The applicant also discussed the various conservatisms used in their analysis.

Conclusion

1. Based on the analyses performed, the applicant presented its conclusion that the effects of shear stresses were negligible or were separately addressed for all nozzles analyzed.
2. Based on its interaction with the NRR staff during the meeting, the applicant agreed to perform additional confirmatory work and submit results to the staff for review and acceptance. This work effort will include:
 - 1) Performing benchmarking calculations on the feedwater nozzle, which is the most limiting component, using the axisymmetric finite element model, taking fully into account all stress components on the nozzle and using the ANSYS FEM computer code to model all defined transients;
 - 2) Demonstrating that the Vermont Yankee specific benchmarking calculations bound the results for the Core Spray and Recirculation outlet nozzles.
 - 3) Calculating fatigue usage factors (CUFs) using NRC approved ASME Section III NB-3200 methods; and
 - 4) Comparing the resulting CUFs to the previous environmental assisted fatigue calculations to establish whether the previous calculations are adequate.

Jonathan G. Rowley, Project Manager
Projects Branch 2
Division of License Renewal
Office of Nuclear Reactor Regulation

Docket No. 50-271

NEC072278

Enclosures:

1. Attendance List
2. Agenda
3. Public Comments
4. Presentation slides

cc w/encls: See next page

Jonathan G. Rowley, Project Manager
Projects Branch 2
Division of License Renewal
Office of Nuclear Reactor Regulation

Docket No. 50-271

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DATE	/ /	/ /	/ /	/ /

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MEETING BETWEEN THE NRC STAFF AND ENTERGY NUCLEAR OPERATIONS, INC.
VERMONT YANKEE NUCLEAR POWER STATION
LICENSE RENEWAL APPLICATION

6003 EXECUTIVE BOULEVARD
ROOM EBB1B15
ROCKVILLE, MARYLAND

MEETING ATTENDANCE LIST
JANUARY 8, 2008

PARTICIPANTS

AFFILIATIONS

Jonathan Rowley	U.S. Nuclear Regulatory Commission (NRC)
Samson Lee	NRC
John Fair	NRC
Kenneth Chang	NRC
PT Kuo	NRC
Rani Franovich	NRC
Robert Sun	NRC
Qi Gan	NRC
Kaihwa Hsu	NRC
Ricardo Rodriguez	NRC
Mary Baty	NRC
Perry Buckberg	NRC
Evelyn Gettys	NRC
Yeon-Ki Chung	NRC
Peter Wen	NRC
On Yee	NRC
Gary Hammer	NRC
David Mannai	Entergy Nuclear Operations, Inc. (Entergy)
John McCann	Entergy
Jay Thayer	Entergy
Matias Travieso-Diaz	Entergy
Michael Metell	Entergy
Garry Young	Entergy
Norm Rademacher	Entergy

Enclosure 1

Alan Cox

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New England Coalition (NEC)

David Lochbaum

Union of concerned Scientist

THE FOLLOWING PARTICAPATED

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Sarah Hoffman

Vermont Department of Public Service

John Sipos

New York Office of the Attorney General

Paul Eddy

New York Office of the Attorney General

Joan Leary Matthews

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Strategic Teaming and Resource Sharing

Chalmer Myer

Southern Nuclear Operating Company

Bob Audette

The Brattleboro Reformer

Susan Smallheer

The Rutland Herald

Sally Shaw

Fred Mogolesko

Entergy

MEETING BETWEEN THE NRC STAFF AND ENTERGY NUCLEAR OPERATIONS, INC.
VERMONT YANKEE NUCLEAR POWER STATION
LICENSE RENEWAL APPLICATION

6003 EXECUTIVE BOULEVARD
ROOM EBB1B15
ROCKVILLE, MARYLAND

AGENDA

JANUARY 8, 2008

- | | | |
|------|---|------------|
| I. | Introduction and opening remarks | 10 minutes |
| II. | Discussion of Response to Request for Additional Information
(Response to RAI 4.3.3-2) | 80 minutes |
| III. | Public Comments | 30 minutes |
| IV. | Adjourn | |

Enclosure 2

~7~

NEC072283

MEETING BETWEEN THE NRC STAFF AND ENTERGY NUCLEAR OPERATIONS, INC.
VERMONT YANKEE NUCLEAR POWER STATION
LICENSE RENEWAL APPLICATION
ROCKVILLE, MARYLAND
ROOM EBB1B15

MEETING MINUTES
JANUARY 8, 2008

In a letter dated November 27, 2007, the staff issued RAI 4.3.3-2 to the applicant. The purpose of the request was to gather additional information on the calculations used at VYNPS to reanalyze their time-limited aging analysis (TLAA) that addresses environmentally-assisted fatigue. In a letter dated December 11, 2007, the applicant provided its response to RAI 4.3.3-2 to the staff. The staff reviewed the response and further questioned the methodology described in the submittal and statements made that shear stresses are negligible during a conference call with the applicant on December 18, 2007. During the call, the applicant requested a face-to-face meeting to ensure that its position pertaining to this highly technical issue was properly and effectively communicated.

Discussion

During the meeting, the applicant made a slide presentation of the reactor vessel nozzle environmental fatigue analyses for license renewal at VYNPS. The applicant reviewed the terminology used in the response to RAI 4.3.3-2. The applicant reviewed the analyzed vessel nozzle configurations to identify where the shear stresses in the nozzles were negligible, where they were not, and the effects on the fatigue analysis at the locations where shear stresses were not negligible.

The applicant explained that nozzle corner, blend radius, and inner radius are interchangeable terms for locations with geometrical discontinuities; that is, locations where stresses are a maximum. The applicant explained that their methodology incorporates the use of axisymmetric modeling rather than an exact configuration 3-Dimensional (3-D) or 2-Dimensional (2-D) modeling. They explained that axisymmetric modeling is a 3-D model except that it models the nozzle/vessel interface as a sphere with a multiplier to account for pressure stress effects, rather than a pipe joined to a cylinder. With the aid of colored graphs, the applicant demonstrated specific nozzles where shear stresses are negligible. The applicant also discussed the various conservatisms used in their analysis.

Conclusion

1. Based on the analyses performed, the applicant presented its conclusion that the effects of shear stresses were negligible or were separately addressed for all nozzles analyzed.

Enclosure 3

2. Based on its interaction with the NRR staff during the meeting, the applicant agreed to perform additional confirmatory work and submit results to the staff for review and acceptance. This work effort will include:

- 1) Performing benchmarking calculations on the feedwater nozzle, which is the most limiting component, using the axisymmetric finite element model, taking fully into account all stress components on the nozzle and using the ANSYS FEM computer code to model all defined transients;
- 2) Demonstrating that the Vermont Yankee specific benchmarking calculations bound the results for the core spray and recirculation outlet nozzles.
- 3) Calculating fatigue usage factors (CUFs) using NRC approved ASME Section III NB-3200 methods; and
- 4) Comparing the resulting CUFs to the previous environmental assisted fatigue calculations to establish whether the previous calculations are adequate

Enclosure 3

UNITED STATES
NUCLEAR REGULATORY COMMISSION
OFFICE OF NUCLEAR REACTOR REGULATION
WASHINGTON, DC 20555-0001

April 11, 2008

**NRC REGULATORY ISSUE SUMMARY 2008-10
FATIGUE ANALYSIS OF NUCLEAR POWER PLANT COMPONENTS**

ADDRESSEES

All holders of operating licenses for nuclear power reactors, except those who have permanently ceased operations and have certified that fuel has been permanently removed from the reactor vessel.

INTENT

The U.S. Nuclear Regulatory Commission (NRC) is issuing this regulatory issue summary (RIS) to inform licensees of an analysis methodology used to demonstrate compliance with the American Society of Mechanical Engineers Boiler and Pressure Vessel Code (ASME Code) fatigue acceptance criteria that could be nonconservative if not correctly applied.

BACKGROUND INFORMATION

Title 10 of the *Code of Federal Regulations* (10 CFR) Part 54, "Requirements for Renewal of Operating Licenses for Nuclear Power Plants," requires that applicants for license renewal perform an evaluation of time-limited aging analyses relevant to structures, systems, and components within the scope of license renewal. The fatigue analysis of the reactor coolant pressure boundary components is an issue that involves time-limited assumptions. In addition, the staff has provided guidance in NUREG-1800, Rev. 1, "Standard Review Plan for Review of License Renewal Applications for Nuclear Power Plants," issued September 2005. NUREG-1800, Rev. 1, specifies that the effects of the reactor water environment on fatigue life be evaluated for a sample of components to provide assurance that cracking because of fatigue will not occur during the period of extended operation. Since the reactor water environment has a significant impact on the fatigue life of components, many license renewal applicants have performed supplemental detailed analyses to demonstrate acceptable fatigue life for these components.

10 CFR 50.55a, "Codes and Standards," specifies the ASME Code requirements for operating reactors. Some operating facilities may have performed supplemental detailed analysis of components because of new loading conditions identified after the plant began operation.

ML080950235

SUMMARY OF ISSUE

The staff identified a concern regarding the methodology used by some license renewal applicants to demonstrate the ability of nuclear power plant components to withstand the cyclic loads associated with plant transient operations for the period of extended operation. This particular analysis methodology involves the use of the Green's function to calculate the fatigue usage during plant transient operations such as startups and shutdowns.

The Green's function approach involves performing a detailed stress analysis of a component to calculate its response to a step change in temperature. This detailed analysis is used to establish an influence function, which is subsequently used to calculate the stresses caused by the actual plant temperature transients. This methodology has been used to perform fatigue calculations and as input for on-line fatigue monitoring programs. The Green's function methodology is not in question. The concern involves a simplified input for applying the Green's function in which only one value of stress is used for the evaluation of the actual plant transients. The detailed stress analysis requires consideration of six stress components, as discussed in ASME Code, Section III, Subsection NB, Subarticle NB-3200. Simplification of the analysis to consider only one value of the stress may provide acceptable results for some applications; however, it also requires a great deal of judgment by the analyst to ensure that the simplification still provides a conservative result.

The staff has requested that recent license renewal applicants that have used this simplified Green's function methodology perform confirmatory analyses to demonstrate that the simplified Green's function analyses provide acceptable results. The confirmatory analyses retain all six stress components. To date, the confirmatory analysis of one component, a boiling-water reactor feedwater nozzle, indicated that the simplified input for the Green's function did not produce conservative results in the nozzle bore area when compared to the detailed analysis. However, the confirmatory analysis still demonstrated that the nozzle had acceptable fatigue usage.

Licensees may have also used the simplified Green's function methodology in operating plant fatigue evaluations for the current license term. For plants with renewed licenses, the staff is considering additional regulatory actions if the simplified Green's function methodology was used.

BACKFIT DISCUSSION

This RIS informs addressees of a potential nonconservative calculation methodology and reminds them that the ASME Code fatigue analysis should be performed properly. For license renewal, metal fatigue is evaluated as a time-limited aging analysis in accordance with 10 CFR 54.21(c). The associated staff review guidance appears in Section 4.3, "Metal Fatigue Analysis," of NUREG-1800, Rev. 1. For operating reactors, the ASME Code requirements appear in 10 CFR 50.55a. This RIS does not impose a new or different regulatory staff position. It requires no action or written response and, therefore, is not a backfit under 10 CFR 50.109, "Backfitting." Consequently, the NRC staff did not perform a backfit analysis.

FEDERAL REGISTER NOTIFICATION

A notice of opportunity for public comment on this RIS was not published in the *Federal Register* because the RIS is informational.

CONGRESSIONAL REVIEW ACT

The NRC has determined that this RIS is not a rule as designated by the Congressional Review Act (5 U.S.C. §§801–808) and; therefore, is not subject to the Act.

PAPERWORK REDUCTION ACT STATEMENT

This RIS does not contain information collection requirements that are subject to the requirements of the Paperwork Reduction Act of 1995 (44 U.S.C. 3501 et seq.).

Public Protection Notification

The NRC may not conduct or sponsor, and a person is not required to respond to, a request for information or an information collection requirement unless the requesting document displays a currently valid Office of Management and Budget control number.

CONTACT

Please direct any questions about this matter to the technical contacts listed below.

/RA/

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Note: The NRC's generic communications may be found on the NRC public Web site,
<http://www.nrc.gov>, under Electronic Reading Room/Document Collections.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

OFFICE OF THE
GENERAL COUNSEL

April 3, 2008

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In the Matter of
AMERGEN ENERGY COMPANY, LLC
(License Renewal for Oyster Creek Nuclear Generating Station)
Docket No. 50-219-LR

Dear Administrative Judges:

Enclosed for your information is a copy of the April 3, 2008 Notification of Information in the Matter of Oyster Creek Nuclear Generating Station License Renewal Application, which the Staff has provided to the Commission.

Sincerely,

A handwritten signature in black ink, appearing to read "James E. Adler".

James E. Adler
Counsel for the NRC Staff

Enclosure: As Stated

cc w/enclosure: Service List

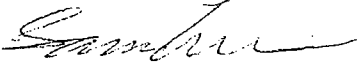


UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

April 3, 2008

Board Notification 2008-01

MEMORANDUM TO: Chairman Klein
Commissioner Jaczko
Commissioner Lyons
Commissioner Svinicki
Atomic Safety and Licensing Board
All Parties

FROM: Samson S. Lee, Acting Director 
Division of License Renewal
Office of Nuclear Reactor Regulation

SUBJECT: NOTIFICATION OF INFORMATION IN THE MATTER OF OYSTER
CREEK NUCLEAR GENERATING STATION LICENSE RENEWAL
APPLICATION

In conformance with the Commission's policy on notification to the Commission and the Atomic Safety Licensing Board (ASLB) regarding significant new information, this memorandum provides the following information.

The staff is reviewing the use of a simplified method to calculate cumulative usage factors (CUF) that may not be conservative. Oyster Creek Nuclear Generating Station (Oyster Creek) used this simplified fatigue calculation method for one type of nozzle, the recirculation nozzle at the plant. This type of calculation was not applicable to the drywell shell analysis, which is the subject of the appealed contention pending before the Commission. Although, this simplified calculation is not relevant to the contention in the proceeding that was before the ASLB, we are providing this information, because this may be an issue of public interest.

The staff plans to ask Oyster Creek to perform a confirmatory analysis consistent with the methodology in Section III of the ASME Code. However, the staff believes that the safety significance of using the simplified analysis method is low based on the risk assessments performed by the staff in resolving generic safety issues (GSI)-166 and GSI-190.

Docket No. 50-219

cc: See next page

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Accession Number: ML080930335

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Oyster Creek Nuclear Generating Station - 2 -

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