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UNITED STATES
NUCLEAR REGULATORY COMMISSION
ATOMIC SAFETY LICENSING BOARD

OFFICE OF SECRETARY
RULEMAKINGS AND
ADJUDICATIONS STAFF

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In re:	Docket Nos. 50-247-LR and 50-286-LR
License Renewal Application Submitted by	ASLBP No. 07-858-03-LR-BD01
Entergy Nuclear Indian Point 2, LLC, Entergy Nuclear Indian Point 3, LLC, and Entergy Nuclear Operations, Inc.	DPR-26, DPR-64 May 22, 2008

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**NEW YORK STATE'S SUPPLEMENTAL CITATION IN
SUPPORT OF ADMISSION OF CONTENTION 26A**

On May 14, 2008 – two weeks after the State of New York submitted its Reply to Entergy's Answer and NRC Staff's Response to New York's Supplemental Contention No.26-A (Metal Fatigue) – the NRC Staff posted on ADAMS a May 8, 2008 Summary of an April 3, 2008 telephone conference between Entergy and Staff regarding, *inter alia*, how much information NRC Staff would require Entergy to produce as part of its License Renewal Application.¹ The Summary is contained in Attachment 1 to this Supplement, and is also available at ML081190059. The May 8 Summary reveals that Entergy, with the acquiescence of Staff, does not intend to allow the details of how it will address metal fatigue issues to be made a part of this license renewal proceeding. Enclosure 1 to May 8, 2008 Summary of April 3, 2008 Telephone Conference, at pages 1 to 3.

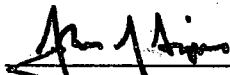
This newly-disclosed information supplements the statements made by New York State in its May 1, 2008 Reply at the end of the first full paragraph on page 10.

¹The New York State Office of the Attorney General received a copy of the May 8, 2008 Summary via U.S. Mail on May 19, 2008.

TEMPLATE=SECY-037

DS-03

Respectfully submitted,
May 22, 2008



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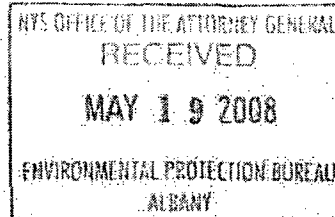
ATTACHMENT 1

**NRC May 8, 2008 Summary of an
April 3, 2008 telephone conference between Entergy and NRC Staff**

also available at ML081190059



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001



May 8, 2008

LICENSEE: Entergy Nuclear Operations, Inc.

FACILITY: Indian Point Nuclear Generating Unit Nos. 2 and 3

SUBJECT: SUMMARY OF TELEPHONE CONFERENCE CALL HELD ON APRIL 3, 2008, BETWEEN THE U.S. NUCLEAR REGULATORY COMMISSION AND ENTERGY NUCLEAR OPERATIONS, INC., CONCERNING RESPONSES TO REQUEST FOR ADDITIONAL INFORMATION RELATED TO THE INDIAN POINT NUCLEAR GENERATING UNIT NOS. 2 AND 3, LICENSE RENEWAL APPLICATION — METAL FATIGUE, BOLTED CONNECTIONS, AND BORAFLEX

The U.S. Nuclear Regulatory Commission (NRC or the staff) and representatives of Entergy Nuclear Operations, Inc., held a telephone conference call on April 3, 2008, to discuss and clarify the staff's draft request for additional information (D-RAI) concerning the Indian Point Nuclear Generating Unit Nos. 2 and 3, license renewal application. The telephone conference call was useful in clarifying the intent of the staff's D-RAI.

Enclosure 1 provides a listing of the participants and Enclosure 2 contains a listing of the D-RAI items discussed with the applicant, including a brief description on the status of the items.

The applicant had an opportunity to comment on this summary.

A handwritten signature in cursive script that reads "Kimberly Green".

Kimberly Green, Safety Project Manager
Projects Branch 2
Division of License Renewal
Office of Nuclear Reactor Regulation

Docket Nos. 50-247 and 50-286

Enclosures:

1. List of Participants
2. Summary of Discussion

cc w/encls: See next page

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**TELEPHONE CONFERENCE CALL
INDIAN POINT NUCLEAR GENERATING UNIT NOS. 2 AND 3
LICENSE RENEWAL APPLICATION**

**LIST OF PARTICIPANTS
APRIL 3, 2008**

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AFFILIATIONS

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Peter Wen	NRC
Jim Davis	NRC
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**DRAFT REQUEST FOR ADDITIONAL INFORMATION
INDIAN POINT NUCLEAR GENERATING UNIT NOS. 2 AND 3
LICENSE RENEWAL APPLICATION
METAL FATIGUE**

April 3, 2008

The U.S. Nuclear Regulatory Commission (NRC or the staff) and representatives of Entergy Nuclear Operations, Inc., held a telephone conference call on April 3, 2008, to discuss and clarify the following draft requests for additional information (D-RAIs) concerning the Indian Point Nuclear Generating Unit Nos. 2 and 3 license renewal application (LRA).

D-RAI 4.3.1.8-1

License Renewal Application Section 4.3.1 states "Current design basis fatigue evaluations calculate cumulative usage factors (CUFs) for components or sub-components based on design transient cycles." For CUF values listed in LRA Tables 4.3-13 and 4.3-14, please provide the methodology used with sufficient results of the fatigue analysis such that the staff can make a determination based on the guidance described in Standard Review Plan-License Renewal (SRP-LR) (NUREG-1800). Specifically, please describe the details of how environmentally assisted fatigue (EAF) is factored into the calculation of the CUF using F_{en} values.

Discussion: The applicant was uncertain as to whether the staff was requesting that they provide the evaluations or a description of evaluations. Based on the discussion with the applicant, the staff agreed to revise this question as follows. The revised question will be sent as a formal RAI.

License renewal application (LRA) Section 4.3.1 states "Current design basis fatigue evaluations calculate cumulative usage factors (CUFs) for components or sub-components based on design transient cycles." For CUF values listed in LRA Tables 4.3-13 and 4.3-14, please describe the details of how various environmental effects are factored into the calculation of the CUF using F_{en} values.

D-RAI 4.3.1.8-2

From the review of EAF analysis of other plants, it was found that the transfer function methodology used in the EAF analysis may not provide valid results, as it is dependent on the inputs. To assist the staff in its review, please provide the EAF analysis for all the NUREG/CR-6260 locations (components) at Indian Point unless it can be demonstrated that the CUF value is within the ASME Code limit of 1.0 by using the original 40-year analysis value adjusted for 60 years and multiplied by F_{en} , which is consistent with SRP-LR and ASME Code. This analysis should be completed by using NRC-approved fatigue software and the ASME Code, Section III, Subsection NB-3200 methodology (which defines the use of six stress components to determine the stress state and thereby calculates the principal stresses and stress intensities). Justify the analysis method, the load (stress) combination, and the results of the ASME Code analysis if 2-D axis-symmetric modeling is used. In addition, the analysis should apply ASME code rules such as elastic-plastic correction factor, K_e , and stress intensities correction factor for modulus of elasticity. This analysis should be performed without the use of the transfer function method.

Discussion: The applicant wanted clarification on the staff's request. The applicant pointed out that the request is a new staff position and that for previous plants, the staff has not requested the analyses to be provided and has accepted a commitment to perform the analyses two years prior to entering the period of extended operation as part of the Fatigue Monitoring Program in accordance with 10 CFR 54.21(c)(1)(iii). Subsequent to the telephone conference, the staff determined that no additional information is needed at this time. Therefore, a formal RAI will not be issued at this time.

D-RAI 4.3.1.8-3

SRP-LR Section 4.3.2.1.1.3 provides the basis for the staff acceptance of an aging management program to address environmental fatigue. It states, "[T]he staff has evaluated a program for monitoring and tracking the number of critical thermal and pressure transients for the selected reactor coolant system components. The staff has determined that this program is an acceptable aging management program to address metal fatigue of the reactor coolant system components according to 10 CFR 54.21(c)(1)(iii)." The staff is unable to determine if the Fatigue Monitoring Program of IP2 and IP3 contain sufficient details to satisfy this criterion, based on the NA items listed in LRA Tables 4.3-13 and 4.3-14. Please provide adequate details of the Fatigue Monitoring Program, specifically the fatigue analysis used in determining the CUF values for the NA locations and how IPEC plans to proceed in monitoring the locations of Tables 4.3-13 and 4.3-14 during the period of extended operation.

Discussion: The applicant wanted clarification on what the staff is requesting. Based on the discussion with the applicant, the staff agreed to revise this question as follows. The revised question will be sent as a formal RAI.

Standard Review Plan for Review of License Renewal Applications for Nuclear Power Plants (SRP-LR) Section 4.3.2.1.1.3 provides the basis for the staff acceptance of an aging management program to address environmental fatigue. It states, "[t]he staff has evaluated a program for monitoring and tracking the number of critical thermal and pressure transients for the selected reactor coolant system components. The staff has determined that this program is an acceptable aging management program to address metal fatigue of the reactor coolant system components according to 10 CFR 54.21(c)(1)(iii)." The staff is unable to determine if the Fatigue Monitoring Program for Indian Point 2 and Indian Point 3 contains sufficient details to satisfy this criterion. Please provide adequate details of the Fatigue Monitoring Program such that the staff can make a determination based on the criterion set forth in SRP-LR Section 4.3.2.1.1.3. Also, please explain in detail the corrective actions and the frequency that such actions will be taken so that the acceptance criteria will not be exceeded in the period of extended operation. (This RAI will be renumbered as RAI 4.3.1.8-2.)

D-RAI 4.3.1.8-4

Section B.1.12 of the LRA amendment, dated January 22, 2008, states, "If ongoing monitoring indicates the potential for a condition outside that analyzed above, IPEC may perform further reanalysis of the identified configuration using established configuration management processes as described above." Please explain in detail what is meant by the phrase "using established configuration management processes." Also, please explain in detail the corrective actions and the frequency that such actions will be taken so the acceptance criteria will not be exceeded in the period of extended operation.

Discussion: The applicant stated that it was unclear about the staff's request regarding "configuration management processes." In a subsequent call, the applicant explained that the configuration management processes referred to are those governed by its 10 CFR Part 50, Appendix B Quality Assurance program, and include design input verification and independent reviews which ensure that valid assumptions, transients, cycles, external loadings, analysis methods, and environmental fatigue life correction factors will be used in the fatigue analyses. Therefore, this portion of question is withdrawn and will not be sent as a formal RAI. The portion of the request that deals with corrective actions will be added to RAI 4.3.1.8-2 (as renumbered).

Non-EQ Bolted Cable Connection AMP

D-RAI 3.0.3.3.6-1

With regard to Indian Point Aging Management Program (AMP) B.1.22, "Non-EQ Bolted Cable Connection Program," the license renewal application states that inspection methods may include thermography, contact resistance testing, or other appropriate methods including visual, based on plant configuration and industry guidance. In Generic Aging Lessons Learned (GALL) AMP XI.E6, the staff recommends thermography, contact resistance testing, or other appropriate methods based on plant configuration and industry guidance for detecting loss of preload or bolt loosening. In the case where visual inspection will be the only method used, provide a technical basis of how this will be sufficient to detect loss of preload or loosening of bolted connections.

Discussion: The applicant stated that this question is similar to an audit question that has been answered and subsequently discussed during two telephone conferences. This issue is being reviewed by the Division of Engineering and, therefore, is withdrawn at this time. However, when the staff has reached a determination, a formal RAI may be issued at such time.

Boraflex AMP

D-RAI 3.0.3.2.3-1

Indian Point 2 Updated Final Safety Analysis Report, Revision 20, dated 2006, Section 14.2.1 on page 55 of 218, states in part that:

"Northeast Technology Corporation report NET-173-01 and NET-171-02 are based on conservative projections of amount of boraflex absorber panel degradation assumed in each sub-region. These projections are valid through the end of the year 2006."

Please confirm that the Boraflex neutron absorber panels in the Indian Point Unit 2 spent fuel pool have been re-evaluated for service through the end of the current licensing period. Also, please discuss the plans for updating the Boraflex analysis during the period of extended operation.

Discussion: The applicant indicated that the question is clear. This D-RAI will be sent as a formal RAI.

UNITED STATES OF AMERICA
NUCLEAR REGULATORY COMMISSION

ATOMIC SAFETY LICENSING BOARD

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In re:

License Renewal Application Submitted by

Entergy Nuclear Indian Point 2, LLC,
Entergy Nuclear Indian Point 3, LLC, and
Entergy Nuclear Operations, Inc.

Docket Nos. 50-247-LR and 50-286-LR

ASLBP No. 07-858-03-LR-BD01

DPR-26, DPR-64
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CERTIFICATE OF SERVICE

Pursuant to 28 U.S.C. § 1746 Teresa Fountain hereby declares:

I am over 18 years old and am an employee in the New York State Office of the Attorney General.

I hereby certify that on May 22, 2008, copies of "The State of New York's Supplemental Citation In Support of Contention 26A" were served upon the following persons via electronic mail and by deposit in the U.S. Postal Service with first class postage:

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I declare under penalty of perjury that the foregoing is true and correct.

Executed on:

this 22nd day of May 2008
Albany, New York



Teresa Fountain