

May 20, 2008

Mr. Charles G. Pardee
Chief Nuclear Officer (CNO) and Senior Vice President
Exelon Generation Company, LLC
Chief Nuclear Officer (CNO)
AmerGen Energy Company, LLC
200 Exelon Way
Kennett Square, PA 19348

SUBJECT: OYSTER CREEK NUCLEAR GENERATING STATION - NRC EXAMINATION
REPORT 05000219/2008301

Dear Mr. Pardee:

On April 22, 2008, the U.S. Nuclear Regulatory Commission (NRC) completed an examination at Oyster Creek Nuclear Generating Station. The enclosed report documents the examination findings, which were discussed on May 14, 2008, with Messrs. Gregg Ludlam, Peter Orphanos, and other members of your staff.

The examination included the evaluation of two applicants for reactor operator licenses and seven applicants for instant senior operator licenses. The written and operating examinations were developed using NUREG-1021, "Operator Licensing Examination Standards for Power Reactors," Revision 9. The license examiners determined that eight of the nine applicants satisfied the requirements of 10 CFR Part 55, and the appropriate licenses have been issued.

No findings of significance were identified during this examination.

In accordance with 10 CFR 2.390 of the NRC's "Rules of Practice," a copy of this letter and its enclosure will be available electronically for public inspection in the NRC Public Document Room or from the Publicly Available Records (PARS) component of NRC's document system (ADAMS). ADAMS is accessible from the NRC Web site at <http://www.nrc.gov/reading-rm/adams.html> (the Public Electronic Reading Room).

Sincerely,

/RA/

Samuel L. Hansell, Jr., Chief
Operations Branch
Division of Reactor Safety

Docket No. 50-219
License No. DPR-16

Mr. Charles G. Pardee
 Chief Nuclear Officer (CNO) and Senior Vice President
 Exelon Generation Company, LLC
 Chief Nuclear Officer (CNO)
 AmerGen Energy Company, LLC
 200 Exelon Way
 Kennett Square, PA 19348

SUBJECT: OYSTER CREEK NUCLEAR GENERATING STATION - NRC EXAMINATION
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DATE	05/20/08	05/20/08	05/20/08	05/20/08

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Enclosure: NRC Examination Report 05000219/2008301

cc w/encl:

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EXAMINATION REPORT

U.S. NUCLEAR REGULATORY COMMISSION

REGION I

Dockets: 50-219

Licenses: DPR-16

Report : 05000219/2008301

Licensee: AmerGen Energy Company LLC

Facility: Oyster Creek Nuclear Generating Station

Location: AmerGen Energy Company LLC
P O BOX 388
Forked River, NJ 08731

Dates: April 22, 2008 (Written Examination Administration)
April 14-18, 2008 (Operating Test Administration)
May 2, 2008 (NRC Examination Grading)
May 8, 2008 (Licensee Submitted Post Exam Package)

Inspectors: John G. Caruso, Chief Examiner, Senior Operations Engineer
Joseph M D'Antonio, Senior Operations Engineer
Brian C. Haagensen, Operations Engineer

Approved By: Samuel L. Hansell, Jr., Chief
Operations Branch
Division of Reactor Safety

Enclosure

SUMMARY OF FINDINGS

ER 05000219/2008301; April 14-22, 2008; Oyster Creek Initial Operator Licensing Examination Report.

NRC examiners evaluated the competency of two applicants for reactor operator licenses and seven applicants for instant senior operator licenses at Oyster Creek. The facility licensee developed the examinations using NUREG-1021, "Operator Licensing Examination Standards for Power Reactors," Revision 9. The written examination was administered by the facility on April 22, 2008. NRC examiners administered the operating tests on April 14-18, 2008. The license examiners determined that eight of the nine applicants satisfied the requirements of 10 CFR Part 55, and the appropriate licenses have been issued.

A. NRC-Identified and Self-Revealing Findings

No findings of significance were identified.

B. Licensee-Identified Violations

None.

REPORT DETAILS

4. OTHER ACTIVITIES (OA)

4OA5 Other Activities (Initial Operator License Examination).1 License Applicationsa. Scope

The examiners reviewed all nine license applications submitted by the licensee to ensure the applications reflected that each applicant satisfied relevant license eligibility requirements. The applications were submitted on NRC Form 398, "Personal Qualification Statement," and NRC Form 396, "Certification of Medical Examination by Facility Licensee." The examiners also audited five of the license applications in detail to confirm that they accurately reflected the subject applicant's qualifications. This audit focused on the applicant's experience and on-the-job training, including control manipulations that provided significant reactivity changes.

b. Findings

No findings of significance were identified.

.2 Operator Knowledge and Performancea. Examination Scope

On April 22, 2008, the licensee proctored the administration of the written examinations to all nine applicants. The licensee staff graded the written examinations, analyzed the results, and presented their analysis to the NRC on April 28, 2008.

The NRC examination team administered the various portions of the operating examination to all nine applicants on April 14-18, 2008. The two applicants for reactor operator licenses participated in three dynamic simulator scenarios, in a control room and facilities walkthrough test consisting of eleven system tasks, and an administrative test consisting of four administrative tasks. The seven applicants seeking an instant senior operator license participated in three dynamic simulator scenarios, a control room and facilities walkthrough test consisting of ten system tasks, and an administrative test consisting of five administrative tasks.

b. Findings

Eight of nine the applicants passed all parts of the operating test. One senior reactor operator applicant failed the dynamic simulator portion of the operating test. All nine applicants passed the written examination. For the written examinations, the reactor operator applicants' average score was 92.7 percent and ranged from 90.7 to 94.7 percent, the senior operator applicants' average score was 89.6 percent and ranged from 84.0 to 93.0 percent. The overall written examination average was 89.6 percent. The text of the examination questions, and the licensee's examination analysis may be accessed in the ADAMS system under the accession numbers noted in the attachment.

Chapter ES-403 and Form ES-403-1 of NUREG 1021 require the licensee to analyze the validity of any written examination questions that were missed by half or more of the applicants. The licensee conducted this performance analysis for three questions that met these criteria and submitted the analysis to the chief examiner. This analysis concluded that all of the questions were valid and did not propose and changes to the exam as administered.

.3 Initial Licensing Examination Development

a. Examination Scope

The facility licensee developed the examinations in accordance with NUREG-1021, Revision 9. All licensee facility training and operations staff involved in examination preparation and validation were on a security agreement. The facility licensee submitted both the written and operating examination outlines on January 17, 2008. The chief examiner reviewed the outlines against the requirements of NUREG-1021, Revision 9, and provided comments to the licensee. The facility licensee submitted the draft examination package on February 29, 2008. The chief examiner reviewed the draft examination package against the requirements of NUREG-1021, Revision 9, and provided comments to the licensee on the examination on March 16, 2008. The NRC conducted an onsite validation of the operating examinations and provided further comments during the week of March 10, 2008. The licensee satisfactorily completed the post exam submittal process on May 8, 2008.

b. Findings

The NRC approved the initial examination outline and advised the licensee to proceed with the operating examination development.

The examiners determined that the written and operating examinations initially submitted by the licensee were within the range of acceptability expected for a proposed examination. See Attachment 3 (Procedural Issues Identified) for additional details.

No findings of significance were identified.

.4 Simulation Facility Performance

a. Examination Scope

The examiners observed simulator performance with regard to plant fidelity during the examination validation and administration. See Attachment 2 (Simulator Fidelity Report) for additional details.

b. Findings

No findings of significance were identified.

.5 Examination Security

a. Examination Scope

The examiners reviewed examination security for examination development and during both the onsite preparation week and examination administration week for compliance with NUREG-1021 requirements. Plans for simulator security and applicant control were reviewed and discussed with licensee personnel.

b. Findings

No findings of significance were identified.

4OA6 Meetings, Including Exit

The chief examiner presented the examination results to Messrs. Gregg Ludlam, Director of Training, Peter Orphanos, Director of Operations, and other members of the licensee's management staff on May 14, 2008. The licensee acknowledged the findings presented.

The licensee did not identify any information or materials used during the examination as proprietary.

ATTACHMENT: SUPPLEMENTAL INFORMATION

ATTACHMENT
SUPPLEMENTAL INFORMATION
KEY POINTS OF CONTACT

Licensee Personnel

G. Ludlam, Director of Training
J. Costic, Operations Training Manager
N. Patrou, Nuclear Training Instructor
G. Young, Nuclear Training Instructor
J. Ruth, Corporate Nuclear Training
H. Tritt, Operations Shift Manager

NRC Personnel

M. Ferdas, Senior Resident Inspector

ITEMS OPENED, CLOSED, AND DISCUSSED

Opened

None

Opened and Closed

None

Closed

None

Discussed

None

ADAMS DOCUMENTS REFERENCED

Accession No. ML81330625 – FINAL-Written Exam
Accession No. ML081330603 – FINAL-Operating Exam, Section A
ML081330610 – FINAL-Operating Exam, Section B
ML081330618 – FINAL-Operating Exam, Section C

Attachments: Simulator Fidelity Report
Procedural Issues Identified

ES-501**Simulator Fidelity Report****Attachment 2**Facility Licensee: Oyster Creek Nuclear Generating StationFacility Docket No.: 50-219Operating Test Administered on: April 14-18, 2008

This form is to be used only to report observations. These observations do not constitute audit or inspection findings and, without further verification and review in accordance with IP 71111.11, are not indicative of noncompliance with 10 CFR 55.46. No licensee action is required in response to these observations.

While conducting the simulator portion of the operating tests, examiners observed the following items:

Item	Description
1	Main Steam Stop valve #2 indicator did not move from "closed" to "open" during the performance of SIM JPM 3 when it was "momentarily" depressed. The applicant had to hold the test pushbutton for approximately 5 seconds before the valve would move of the open seat. Minor Maintenance Item (MMI) 6086 was written to correct this problem.
2	During scenario #4 while crew B was being evaluated on 4/17/08 (1207-1536) core thermal power stopped updating on the plant computer. SWR 10810 was initiated to troubleshoot and repair the problem.
3	During scenario 3, the 1C reactor feedwater pump ammeter spiked high when the 1B RFP was started. MMI 6099 was written to review and correct the problem.
4	During scenario #4 while crew B was being evaluated on 4/17/08 (1207-1536) the crew was not able to vent to torus using Support Procedure 31. This simulator fidelity problem was known and had been documented prior to the exam.
5	Several labeling errors were noted prior to the exam: 1. APRM gain adjustment pots were not labeled. This corrected prior to the exam. 2. Control room ventilation fans were not labeled. This was corrected prior to the exam. 3. Two relays inside panel 6R and 7R were noted to be missing labels. MMI 6108 was prepared to correct this problem.
6	The simulator room lighting flickered a few times during one scenario. The simulator light switches were shifted from automatic to manual for the remainder of the scenarios. Simulator Work Request (SWR) 10811 was written to initiate troubleshooting and repairs.

Item	Procedural Issues Identified	Attachment 3
1	The field bus label for B recirc pump P-37-2 at the 4160 1B breaker did not match the unit designation in procedure ABN 30-1 step 2. The procedure lists the unit as power from bus "B2". The label on the breaker stated it was powered from bus "B4".	
2	The JPM task standard for approval of the 5 Rem emergency exposure limit was not clear. I was not clear if the 5 Rem limit was based upon the task, or included the applicant's prior exposure history. The procedure did not address this question. This problem will be investigated further and an IR will be prepared if required.	
3	While preparing procedures for the normal evolutions in the simulator scenarios, an error was noted in procedure 317, Feedwater. IR-00766967 was generated to address this issue.	
4	Procedure EP-AA-113 (Personnel Protective Actions) was found to be unclear. Applicants did not appear to understand the statement on the sea breeze effect. IR-00766723 was generated to address this issue.	
5	While performing the startup of a recirculation pump during a JPM, the recirc pump unexpectedly tripped when the hand switch was held in the "start" position for a prolonged period of time. A review of this action determined that the pump would actually trip if the switch was held for this period of time in the plant. The procedure directs the operator to hold the hand switch in the start position until amps have stabilized. The procedure does not provide further guidance to the operator regarding the maximum length of time to hold the start switch or how to determine when amps have stabilized. A procedure change will be made to address this issue.	