PSEG Nuclear LLC
P.O. Box 236, Hancocks Bridge, New Jersey 08038-0236

PSEG
Nuclear LLC

10CFR50.73

AUG 30 2007
LR-N07-0220

United States Nuclear Regulatory Commission
ATTN: Document Control Desk
Washington, DC 20555-001

Hope Creek Generating Station Unit 1
Facility Operating License No. NPF-57
Docket No. 50-354

Subject: Licensee Event Report 2007-003-00

In accordance with 10 CFR 50.73(a)(2)(i)(B), PSEG Nuclear LLC, is submitting Licensee Event Report Number 2007-003-00, Docket No. 50-354.

Should you have any questions concerning this letter, please contact Mr. Francis D. Possessky at (856) 339-1160.

Sincerely,

John F. Perry
Plant Manager
Hope Creek Generating Station

Attachment: Licensee Event Report

IE22
NRR

cc: Mr. S. Collins, Administrator - Region 1
U.S. Nuclear Regulatory Commission
475 Allendale Road
King of Prussia, PA 19406

Mr. R. Ennis, Licensing Project Manager - Hope Creek
U.S. Nuclear Regulatory Commission
Mail Stop 08B1
Washington, DC 20555-0001

USNRC Resident Inspector office - Hope Creek (X24)

Mr. P. Mulligan, Manager IV (Acting)
Bureau of Nuclear Engineering
PO Box 415
Trenton, New Jersey 08625

NRC FORM 366 (6-2004) U.S. NUCLEAR REGULATORY COMMISSION

APPROVED BY OMB: NO. 3150-0104 EXPIRES: 06/30/2007

Estimated burden per response to comply with this mandatory collection request: 50 hours. Reported lessons learned are incorporated into the licensing process and fed back to industry. Send comments regarding burden estimate to the Records and FOIA/Privacy Service Branch (T-5 F52), U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001, or by internet e-mail to infocollects@nrc.gov, and to the Desk Officer, Office of Information and Regulatory Affairs, NEOB-10202, (3150-0104), Office of Management and Budget, Washington, DC 20503. If a means used to impose an information collection does not display a currently valid OMB control number, the NRC may not conduct or sponsor, and a person is not required to respond to, the information collection.

LICENSEE EVENT REPORT (LER)

(See reverse for required number of digits/characters for each block)

1. FACILITY NAME	2. DOCKET NUMBER	3. PAGE
Hope Creek Generating Station	05000 354	1 OF 3

4. TITLE

Grab Samples Not Performed As Required By Technical Specification 3.3.7.1

5. EVENT DATE			6. LER NUMBER			7. REPORT DATE			8. OTHER FACILITIES INVOLVED	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REV NO.	MONTH	DAY	YEAR	FACILITY NAME	DOCKET NUMBER
									N/A	
07	01	2007	2007	- 003 -	00	08	30	2007	FACILITY NAME: N/A	DOCKET NUMBER

9. OPERATING MODE

1

10. POWER LEVEL

100

11. THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR§: *(Check all that apply)*

- ☐ 20.2201(b)
- ☐ 20.2201(d)
- ☐ 20.2203(a)(1)
- ☐ 20.2203(a)(2)(i)
- ☐ 20.2203(a)(2)(ii)
- ☐ 20.2203(a)(2)(iii)
- ☐ 20.2203(a)(2)(iv)
- ☐ 20.2203(a)(2)(v)
- ☐ 20.2203(a)(2)(vi)
- ☐ 20.2203(a)(3)(i)
- ☐ 20.2203(a)(3)(ii)
- ☐ 20.2203(a)(4)
- ☐ 50.36(c)(1)(i)(A)
- ☐ 50.36(c)(1)(ii)(A)
- ☐ 50.36(c)(2)
- ☐ 50.46(a)(3)(ii)
- ☐ 50.73(a)(2)(i)(A)
- ☒ 50.73(a)(2)(i)(B)
- ☐ 50.73(a)(2)(i)(C)
- ☐ 50.73(a)(2)(ii)(A)
- ☐ 50.73(a)(2)(ii)(B)
- ☐ 50.73(a)(2)(iii)
- ☐ 50.73(a)(2)(iv)(A)
- ☐ 50.73(a)(2)(v)(A)
- ☐ 50.73(a)(2)(v)(B)
- ☐ 50.73(a)(2)(v)(C)
- ☐ 50.73(a)(2)(v)(D)
- ☐ 50.73(a)(2)(vii)
- ☐ 50.73(a)(2)(viii)(A)
- ☐ 50.73(a)(2)(viii)(B)
- ☐ 50.73(a)(2)(ix)(A)
- ☐ 50.73(a)(2)(x)
- ☐ 73.71(a)(4)
- ☐ 73.71(a)(5)
- ☐ OTHER

Specify in Abstract below or in NRC Form 366A

12. LICENSEE CONTACT FOR THIS LER

FACILITY NAME	TELEPHONE NUMBER (Include Area Code)
Francis D. Possessky, Compliance Engineer	856-339-1160

13. COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT

CAUSE	SYSTEM	COMPONENT	MANU-FACTURER	REPORTABLE TO EPIX	CAUSE	SYSTEM	COMPONENT	MANU-FACTURER	REPORTABLE TO EPIX

14. SUPPLEMENTAL REPORT EXPECTED

☐ YES *(If yes, complete 15. EXPECTED SUBMISSION DATE)* ☒ NO

15. EXPECTED SUBMISSION DATE

MONTH	DAY	YEAR

ABSTRACT *(Limit to 1400 spaces, i.e., approximately 15 single-spaced typewritten lines)*

On July 01, 2007, offgas flow started rising. In anticipation of offgas flow approaching its upper limit of 75 standard cubic feet per minute (scfm) for instrument operability, Operations directed Radiation Protection (RP) to enter correction factor changes into the Offgas Pretreatment Radiation Monitoring System (RMS) as directed by Note 4 of the Main Condenser Vacuum abnormal operating procedure (AOP). Offgas Pretreatment RMS must be declared inoperable when offgas flow increases to 75scfm, and can be declared operable once the required correction factor changes are entered into the RMS. On July 04, 2007, when offgas flow decreased below 75 scfm, Operations requested RP to remove the correction factors that had been entered into the Offgas Pretreatment RMS on July 01, 2007. RP communicated to Operations that no correction factors were entered into the Offgas Pretreatment RMS, instead, alert setpoint changes had been made. Operations then declared the Offgas Pretreatment RMS inoperable, and had been inoperable since Offgas Flow went above 75 scfm on July 01, 2007. Tech Spec 3.3.7.1 required grab samples were not performed during the period of inoperability.

The root cause of this event was inadequate procedural direction to guide the RP technicians to perform the action as required by the Operations AOP.

The RP procedure, 'Control of Radiation Monitoring System Setpoints,' will be revised to include guidance to make temporary correction factor changes to the RMS when required.

LICENSEE EVENT REPORT (LER)

1. FACILITY NAME	2. DOCKET	6. LER NUMBER			3. PAGE
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	
Hope Creek Generating Station	05000354	2007	- 003 -	00	2 OF 3

17. TEXT (If more space is required, use additional copies of NRC Form 366A)

PLANT AND SYSTEM IDENTIFICATION

General Electric – Boiling Water Reactor (BWR/4)
Radiation Monitoring System – {IL}
Offgas System – {WF}

*Energy Industry Identification System {EIIS} codes and component function identifier codes appear as {SS/CCC}

IDENTIFICATION OF OCCURRENCE

Event Date/Time: July 01, 2007 - 06:02
Discovery Date/Time: July 04, 2007 - 03:00

CONDITIONS PRIOR TO OCCURRENCE

Hope Creek was in Operational Condition 1 with reactor power at 100%. No other structures, systems, or components were inoperable that contributed to the event.

DESCRIPTION OF OCCURRENCE

On June 30, 2007, offgas flow {WL/-} began rising from 34 scfm. The operating shift entered the Main Condenser Vacuum abnormal operating procedure (AOP), in anticipation of offgas flow rising to 75 scfm. The Shift Manager contacted the on-shift Radiation Protection (RP) technicians to inform them of the requirements in the AOP to change the correction factor of the Offgas Preteatment Radiation Monitoring System (RMS) {WF/-}. An RP supervisor was called in to support the RP technicians in performing the actions.

On July 01, at approximately 0330, offgas flow continued to rise to 70 scfm, and the Shift Manager (SM) directed RP to make the changes to Offgas Pretreatment RMS in accordance with the AOP. RP communicated to the SM that the RP procedure referenced in the AOP provides no guidance on changing RMS correction factors. Additionally, the RP procedure makes no reference to the calculation used to change the correction factor as mentioned in the AOP. The on shift Control Room Supervisor (CRS) developed the correction factor using calculation HA-0053 and provided this to RP. Through discussions with the RP Supervisor, RP determined that Form 3 of Control of Radiation Monitoring Setpoints should be used to incorporate the correction factor into alert setpoint changes. RP then informed the on shift CRS that they were prepared to make the required changes to the Offgas Pretreatment RMS. The CRS provided this information to the Shift Manager. At 0602 on July 1, offgas flow rose above 75 scfm and Operations declared the RMS inoperable in accordance with Technical Specification (TS) 3.3.7.1. At 0655, RP reported that the changes required to the Offgas Pretreatment RMS were completed. At 0700, Operations declared the Offgas Pretreatment RMS operable.

On July 04, 2007, offgas flow lowered below 75 scfm and Operations directed RP to remove the correction factor. RP reported to the shift that no correction factor was ever entered and that the changes made to the Offgas Pretreatment RMS were changes to the alert setpoints. As a result, the Offgas Pretreatment RMS was inoperable from July 01 to July 04, 2007. With the Offgas Pretreatment RMS inoperable, TS 3.3.7.1 requires the performance of periodic grab samples that were not performed. Therefore the event is reportable under 10CFR50.73 (a)(2)(i)(B) as a TS prohibited condition.

1. FACILITY NAME	2. DOCKET	6. LER NUMBER			3. PAGE
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	
Hope Creek Generating Station	05000354	2007	- 003 -	00	3 OF 3

17. TEXT (If more space is required, use additional copies of NRC Form 366A)

CAUSE OF OCCURRENCE

The root cause of this event was inadequate procedural direction to guide the RP technicians to perform the action as required by the Operations AOP.

PREVIOUS OCCURRENCES

A review of previous reportable events at Hope Creek was performed to determine if a similar event had occurred. No similar events were identified.

SAFETY CONSEQUENCES

The safety consequences of this event are minimal. This event resulted in no nuclear, radiological, or industrial safety consequences. Reactor analysis performed showed no increase in fission product activity during the period of the Offgas Pretreatment RMS inoperability, and the offgas samples obtained following the incident were as expected.

A review of this event determined that a Safety System Functional Failure (SSFF) has not occurred as defined in Nuclear Energy Institute (NEI) 99-02.

CORRECTIVE ACTIONS

The RP procedure, 'Control of Radiation Monitoring System Setpoints,' will be revised to include guidance to make temporary correction factor changes to RMS when required.

COMMITMENTS

This LER contains no commitments.