FINAL OUTLINES FOR THE DRESDEN INITIAL EXAMINATION - APRIL 2007

Facility: <u>Dresden</u>		Date of Examination: 4/23/07
Examination Level: RO 🛛 SF	RO 🗆	Operating Test Number: 2007-301
Administrative Topic (See Note)	Type Code*	Describe activity to be performed
Conduct of Operations	N, R	Initiate an Equipment Status Tag Generic.2.1.18
Conduct of Operations	D, R	Verify Acceptance Criteria met for the Acoustic Monitor Based on Test Results Generic.2.1.7
Equipment Control	N, R	Verify Reversal of Emergency Diesel Generator Cooling Water Flow Generic.2.2.12
Radiation Control	M, R	Select Personnel for Radiation Work Generic.2.3.4
Emergency Plan		
		SROs. RO applicants require only 4 items unless they are pics, when 5 are required.
* Type Codes & Criteria:	(D)irect (N)ew o	of room, (S)imulator, or Class(R)oom from bank (≤ 3 for ROs; ≤ 4 for SROs & RO retakes) or (M)odified from bank (≥ 1) ous 2 exams (≤ 1; randomly selected)

Facility: <u>Dresden</u>		Date of Examination: 4/23/07
Examination Level: RO S	10 🗵	Operating Test Number: 2007-301
Administrative Topic (See Note)	Type Code*	Describe activity to be performed
Conduct of Operations	P, R	Reactivation of an SRO License Generic.2.1.5
Conduct of Operations	D, R	Reportability Determination Generic.2.1.1
Equipment Control	D, R	Verify Semi-Annual HRSS AFU Operability Test Generic.2.2.12
Radiation Control	M, R	Select Personnel for Radiation Work Generic.2.3.4
Emergency Plan	N, R	Determine EP Classification and Fill out NARS form Generic.2.4.38
		SROs. RO applicants require only 4 items unless they are pics, when 5 are required.
Type Codes & Criteria:	(D)irect (N)ew o	ol room, (S)imulator, or Class(R)oom from bank (≤ 3 for ROs; ≤ 4 for SROs & RO retakes) or (M)odified from bank (≥ 1) ous 2 exams (≤ 1; randomly selected)

Facility: <u>Dresden</u>	Date of Exa	mination: <u>4/23/(</u>)7
Exam Level: RO 🛛 SRO-I 🗌 SRO-U 🗍	Operating T	est Number: 20	07-301
Control Room Systems® (8 for RO); (7 for SRC	0-I); (2 or 3 for SRO-U, includi	ng 1 ESF)	
System / JPM Title	,	Type Code*	Safety Function
a. Injection of Standby Liquid Control System (A, D, S	1	
b. Perform Core Spray Pump Test With Torus	A, P, S	2	
c. Unisolating One (1) Main Steam Line (23900	D, S	3	
d. Start the SDC System for Cooling Mode of C	D, S	4	
e. Vent the Torus with level less than 30 feet (2	295024.EA1.14)	A, D, L, S	5
f. Crosstie Busses 28 and 29 (262001.A4.04)		L, N, S	6
g. Place a Control Rod OOS on the RWM (201	006.A2.05)	L, N, S	7
h. SBGT Testing with receipt of an Auto Initiation	A, L, P, S	9	
In-Plant Systems® (3 for RO); (3 for SRO-I); (3	or 2 for SRO-U)		
i. Vent Scram Air Header for Alternate Insertio (295037.EA1.05)	on of Control Rods	D, E, R	1
j. Diesel Generator 2 Local Manual Start (2640	000.A4.04)	A, N, R	6
k. Transfer RPS to the Reserve Power Supply	(212000.K4.03)	D	7
@ All RO and SRO-I control room (and in-pla functions; all 5 SRO-U systems must serve overlap those tested in the control room.			
*Type Codes	Criteria for RO /	SRO-I / SRO-U	
(A)Iternate path	4-6 / 4-	6 / 2-3	
(C)ontrol room			
(D)irect from bank	<u><9/-</u>	_	
(E)mergency or abnormal in-plant	1 /≥1		
(L)ow-Power / Shutdown	≥1/≥	-	
(N)ew or (M)odified from bank including 1(A)	$\geq 2/\geq 3$	-	
(P)revious 2 exams (R)CA	≤3/≤3/≤2 (rand ≥1/ ≥	-	
(S)imulator	217 2	. ,	

Facility: <u>Dresden</u> Exam Level: RO ☐ SRO-I ☑ SRO-U ☐		mination: <u>4/23/0</u> est Number: <u>20</u>	
Control Room Systems® (8 for RO); (7 for SRC	0-I); (2 or 3 for SRO-U, includi	ng 1 ESF)	
System / JPM Title	}	Type Code*	Safety Function
a. Injection of Standby Liquid Control System (211000.A4.08)	A, D, S	1
b. Perform Core Spray Pump Test With Torus	A, P, S	2	
С.			
d. Start the SDC System for Cooling Mode of C	Operation (205000.A4.01)	D, S	4
e. Vent the Torus with level less than 30 feet (2	295024.EA1.14)	A, D, L, S	5
f. Crosstie Busses 28 and 29 (262001.A4.04)		L, N, S	6
g. Place a Control Rod OOS on the RWM (201	006.A2.05)	L, N, S	7
h. SBGT Testing with receipt of an Auto Initiation	on Signal (261000.A2.10)	A, L, P, S	9
In-Plant Systems® (3 for RO); (3 for SRO-I); (3			
i. Vent Scram Air Header for Alternate Insertio (295037.EA1.05)	on of Control Rods	D, E, R	1
j. Diesel Generator 2 Local Manual Start (264)	000.A4.04)	A, N, R	6
k. Transfer RPS to the Reserve Power Supply	(212000.K4.03)	D	7
@ All RO and SRO-I control room (and in-pla functions; all 5 SRO-U systems must serve overlap those tested in the control room.			
*Type Codes	Criteria for RO /	SRO-I / SRO-U	•
(A)Iternate path	4-6 / 4-	6 / 2-3	
(C)ontrol room (D)irect from bank (E)mergency or abnormal in-plant (L)ow-Power / Shutdown (N)ew or (M)odified from bank including 1(A) (P)revious 2 exams (R)CA	≤9 / ≤3 ≥1 / ≥ ≥1 / ≥ ≥2 / ≥ ≤3 / ≤2 (rand ≥1 / ≥	1 $/ \ge 1$ 1 $/ \ge 1$ 1 $/ \ge 1$ 2 $/ \ge 1$ comply selected)	
(S)imulator		· · <u>-</u> ·	

Facility: <u>Dresden</u> Exam Level: RO SRO-I SRO-U		mination: <u>4/23/0</u> est Number: <u>20</u>	
Control Room Systems® (8 for RO); (7 for SRC	0-I); (2 or 3 for SRO-U, includin	g 1 ESF)	
System / JPM Title	•	Type Code*	Safety Function
a.			
b. Perform Core Spray Pump Test With Torus (ESF)	Available (209001.A4.01)	A, P, S	2
C.			
d.			
e. Vent the Torus with level less than 30 feet (2	295024.EA1.14)	A, D, L, S	5
f.			
g. Place a Control Rod OOS on the RWM (201	1006.A2.05)	L, N, S	7
h.			
In-Plant Systems® (3 for RO); (3 for SRO-I); (3	or 2 for SRO-U)		
i. Vent Scram Air Header for Alternate Insertic (295037.EA1.05)	on of Control Rods	D, E, R	1
j. Diesel Generator 2 Local Manual Start (264	000.A4.04)	A, N, R	6
k.			
@ All RO and SRO-I control room (and in-plating functions; all 5 SRO-U systems must service overlap those tested in the control room.			
*Type Codes	Criteria for RO / S	SRO-1 / SRO-U	
(A)Iternate path	4-6 / 4-6	5 / 2-3	
(C)ontrol room (D)irect from bank	≤9/ ≤8	3 / < 4	
(E)mergency or abnormal in-plant	≥1/≥1		
(L)ow-Power / Shutdown	 ≥1/ ≥1	_ / <u>≥</u> 1	
(N)ew or (M)odified from bank including 1(A)	≥2/ ≥2	2 / ≥ 1	
(P)revious 2 exams	≤3/≤3/≤2 (rando	omly selected)	
(R)CA	≥1/≥	l / ≥1	
(S)imulator			

E\$-401	BWR Examination Outline	FORM ES-401-1

Facility Name: I	Dresden	Date of Exam: 4/23/07 RO K/A Category Points																
						RO	K/A	Cat	ego	ry P	oint		SRO-Only Points					
Tier	Group	K 1	K 2	К 3	K 4	K 5	К 6	A 1	A 2	A 3	A 4	G	Total	A	2	G	i*	Total
1. Emergency &	1	4	3	4				3	4			2	20	4	1	3	3	7
Abnormal	2	1	1	2		N/A		1	1	N.	/Α	1	7		2	•	1	3
Plant Evolutions	Tier Totals	5	4	6				4	5			3	27		6	4	1	10
2.	1	2	3	2	2	2	3	3	2	3	2	2	26	;	3	2	2	5
Plant	2	1	1	1	1	1	1	1	1	1	1	2	12	0	2		1	3
Systems	Tier Totals	3	4	3	3	3	4	4	3	4	3	4	38		5	3	3	8
3. Generic K	Generic Knowledge and Abilities					1		2	(3	4	4	10	1	2	3	4	7
(Categories					2]	3	7	2	3		10	2	2	2	1	,

- Note: 1. Ensure that at least two topics from every applicable K/A category are sampled within each tier of the RO and SRO-only outlines (i.e., except for one category in Tier 3 of the SRO-only outline, the "Tier Totals" in each K/A category shall not be less than two).
 - 2. The point total for each group and tier in the proposed outline must match that specified in the table. The final point total for each group and tier may deviate by ±1 from that specified in the table based on NRC revisions. The final RO exam must total 75 points and the SRO-only exam must total 25 points.
 - Systems/evolutions within each group are identified on the associated outline; systems or evolutions that do not apply
 at the facility should be deleted and justified; operationally important, site-specific systems that are not included
 on the outline should be added. Refer to ES-401, Attachment 2, for guidance regarding the elimination
 of inappropriate K/A statements.
 - Select topics from as many systems and evolutions as possible; sample every system or evolution in the group before selecting a second topic for any system or evolution.
 - Absent a plant-specific priority, only those K/As having an importance rating (IR) of 2.5 or higher shall be selected.
 Use the RO and SRO ratings for the RO and SRO-only portions, respectively.
 - Select SRO topics for Tiers 1 and 2 from the shaded systems and K/A categories.
 - 7.* The generic (G) K/As in Tiers 1 and 2 shall be selected from Section 2 of the K/A Catalog, but the topics must be relevant to the applicable evolution or system.
 - 6. On the following pages, enter the K/A numbers, a brief description of each topic, the topics' importance ratings (IRs) for the applicable license level, and the point totals (#) for each system and category. Enter the group and tier totals for each category in the table above; if fuel handling equipment is sampled in other than Category A2 or G* on the SRO-only exam, enter it on the left side of Column A2 for Tier 2, Group 2 (Note #1 does not apply). Use duplicate pages for RO and SRO-only exams.
 - For Tier 3, select topics from Section 2 of the K/A catalog, and enter the K/A numbers, descriptions, IRs, and point totals (#) on Form ES-401-3. Limit SRO selections to K/As that are linked to 10 CFR 55.43.

ES-401, Page 16 of 33

ES-401							ion Outline	Form E	S-401-1
Em	ergen	icy ar	id Ab	norm:	al Pla	ant Ev	volutions - Tier 1/Group 1 (RO)		
E/APE # / Name / Safety Function	1	2	3	1	2	G	K/A Topic(s)	IR .	#
295001 Partial or Complete Loss of Forced Core Flow Circulation / 1 & 4					0 3		Actual core flow; Ability to perform specific system and integrated plant procedures during different modes of plant operation.	3.3; 3.9	2
295003 Partial or Complete Loss of AC / 6	0						Effect of battery discharge rate on capacity	2.7	1
295004 Partial or Total Loss of DC Pwr / 6		0 2					Batteries	3	1
295005 Main Turbine Generator Trip / 3		0 8					A.C. electrical distribution.:	3.2	1
295006 SCRAM / 1	0 3						Reactivity control	3.7	1
295016 Control Room Abandonment / 7				0 6			Reactor water level	4	1
295018 Partial or Total Loss of CCW / 8		0			į	× .	System loads	3.3	1
295019 Partial or Total Loss of Inst. Air / 8				0 2	4		Instrument air system valves: Plant-Specific	3.3	1
295021 Loss of Shutdown Cooling / 4				0 3			Component cooling water systems: Plant-Specific	3.1	1
295023 Refueling Acc / 8	0 1						Radiation exposure hazards	3.6	1
295024 High Drywell Pressure / 5			0 6		1		Reactor SCRAM	4	1
295025 High Reactor Pressure / 3					4.4	04.0 4	Ability to recognize abnormal indications for system operating parameters which are entry-level conditions for emergency and abnormal operating procedures.	4	1
295026 Suppression Pool High Water Temp. / 5	0					122	Pump NPSH	3	1
295027 High Containment Temperature / 5							Not Applicable		0
295028 High Drywell Temperature / 5			0 4				Increased drywell cooling	3.6	1
295030 Low Suppression Pool Wtr Lvl / 5					6 1	1/,3	Suppression pool level	4.1	1
295031 Reactor Low Water Level / 2					0		Reactor water level	4.6	1
295037 SCRAM Condition Present and Power Above APRM Downscale or Unknown / 1			0 5				Cold shutdown boron weight: Plant-Specific	3.2	1
295038 High Off-site Release Rate / 9			0 4				Emergency depressurization	3.6	1
600000 Plant Fire On Site / 8					0		Need for pressurizing control room (recirculating mode)	2.5	1
K/A Category Totals:	4	3	4	3	4	2	Group Point Total:		20

ES-401							tion Outline	Form E	S-401-1
Eme	rgen	cy an K				ant E	volutions - Tier 1/Group 2 (RO)	Τ	
E/APE # / Name / Safety Function	1	5	К 3	1 1	A 2	G	K/A Topic(s)	IR	#
295002 Loss of Main Condenser Vac / 3					1				0
295007 High Reactor Pressure / 3									0
295008 High Reactor Water Level / 2			0 5				HPCI turbine trip: Plant-Specific	3.5	1
295009 Low Reactor Water Level / 2									0
295010 High Drywell Pressure / 5					0 3		Drywell radiation levels	3.3	1
295011 High Containment Temp / 5						*			0
295012 High Drywell Temperature / 5		0					Drywell ventilation	3.4	1
295013 High Suppression Pool Temp. / 5			0 1				Suppression pool cooling operation	3.6	1
295014 Inadvertent Reactivity Addition / 1						1000		_	0
295015 Incomplete SCRAM / 1									0
295017 High Off-site Release Rate / 9						01. 14	Knowledge of system status criteria which require the notification of plant personnel.	2.5	1
295020 Inadvertent Cont. Isolation / 5 & 7	0 4						Bottom head thermal stratification	2.5	1
295022 Loss of CRD Pumps / 1									0
295029 High Suppression Pool Wtr Lvl / 5									0
295032 High Secondary Containment Area Temperature / 5									0
295033 High Secondary Containment Area Radiation Levels / 9									0
295034 Secondary Containment Ventilation High Radiation / 9									0
295035 Secondary Containment High Differential Pressure / 5									0
295036 Secondary Containment High Sump/Area Water Level / 5									0
500000 High CTMT Hydrogen Conc. / 5				0 3			Containment Atmosphere Control System	3.4	1
K/A Category Totals:	1	1	2	1	1	1	Group Point Total:	<u>-</u>	7

ES-401						Р						tion Outline r 2/Group 1 (RO)	Form E	S-401-1
E/APE # / Name / Safety Function	K 1		К 3	K 4	K 5	K 6	A 1	A 2	А 3	A 4	G	K/A Topic(s)	IR	#
203000 RHR/LPCI: Injection Mode					0					6		Core cooling methods; System reset following automatic initiation: Plant-Specific	3.5; 3.9	2
205000 Shutdown Cooling	0											Recirculation loop temperature	3.4	1
206000 HPCI			0 2					200				Reactor pressure control: BWR-2, 3, 4	3.8	1
207000 Isolation (Emergency) Condenser			0 2									Reactor water level (EPG's address the isolation condenser as a water source): BWR-2, 3	3.8	1
209001 LPCS								0 2				Valve closures	3.2	1
209002 HPCS								180				Not Applicable		0
211000 SLC							0	10 M	0 1			Tank level ; Pump discharge pressure: Plant-Specific	3.6; 3.5	2
212000 RPS		0							0 7			RPS motor-generator sets; SCRAM air header pressure	3.2; 3.6	2
215003 IRM		0										IRM channels/detectors	2.5	1
215004 Source Range Monitor	T									0 4	6	SRM drive control switches	3.2	1
215005 APRM / LPRM		0 2										APRM channels	2.6	1
217000 RCIC												Not Applicable		0
218000 ADS					0						02. 22	ADS logic operation; Knowledge of limiting conditions for operations and safety limits.	3.8; 3.4	2
223002 PCIS/Nuclear Steam Supply Shutoff						0 5					01. 23	Containment instrumentation; Ability to perform specific system and integrated plant procedures during different modes of plant operation.	3; 3.9	2
239002 SRVs				0								Ensures even distribution of heat load to suppression pool and adequate steam condensing	3.4	1
259002 Reactor Water Level Control						0 4						Reactor feedwater flow input	3.1	1
261000 SGTS							0 2					Primary containment pressure	3.1	1
262001 AC Electrical Distribution								10				Exceeding current limitations	2.9	1
262002 UPS (AC/DC)				0								Transfer from preferred power to atternate power supplies	3.1	1
263000 DC Electrical Distribution	0											A.C. electrical distribution	3.3	1
264000 EDGs							0				0,450	Maintaining minimum load on emergency generator (to prevent reverse power)	3	1
300000 Instrument Air								e i	0 2		11	Air temperature	2.9	1
400000 Component Cooling Water						0						Valves	2.7	1
								í						0
K/A Category Totals:	2	3	2	2	2	3	3	2	3	2	2	Group Point Total:		26

ES-401					-	DI:			_			tion Outline r 2/Group 2 (RO)	Form ES	S-401-1
E/APE # / Name / Safety Function	K 1	K 2	К 3	K 4	K 5	K 6	_	A 2	A 3	A 4	G	K/A Topic(s)	IR	#
201001 CRD Hydraulic	-	0	3	4	5	0	_	4	3	۴		Backup SCRAM valve solenoids	3.5	1
201002 RMCS	┪	,												0
201003 Control Rod and Drive Mechanism								9				Stuck rod	3.4	1
201004 RSCS						Н	Н							0
201005 RCIS	_		Н						-	\vdash			1	0
201006 RWM					Н									0
202001 Recirculation						Н	0 3		_	H		Core flow	3.6	1
202002 Recirculation Flow Control		П	Η				Ť			0 B		Recirculation system flow	3.3	1
204000 RWCU								*						0
214000 RPIS		П			H				Γ)		0
215001 Traversing In-core Probe	٦	П	_	Н	Н		П		0	Г		Valve operation: Not-BWR1	2.5	1
215002 RBM			Г						Ť		04. 06	Knowledge symptom based EOP mitigation strategies.	3.1	1
216000 Nuclear Boiler Inst.		\vdash		Г										0
219000 RHR/LPCI: Torus/Pool Cooling Mode		一				\vdash			-					0
223001 Primary CTMT and Aux.		T							r				1	0
226001 RHR/LPCI: CTMT Spray Mode		Г			Г									0
230000 RHR/LPCI: Torus/Pool Spray Mode				Г		Γ								0
233000 Fuel Pool Cooling/Cleanup						0 7	Г			Γ		Component cooling water systems	2.7	1
234000 Fuel Handling Equipment														0
239001 Main and Reheat Steam								10.00						0
239003 MSIV Leakage Control		Γ												0
241000 Reactor/Turbine Pressure Regulator	Γ	Г		Γ	Γ		Γ						1	0
245000 Main Turbine Gen. / Aux.				0								Generator protection	2.7	1
256000 Reactor Condensate	0 9											Offgas condenser: Plant-Specific	2.9	1
259001 Reactor Feedwater				Γ				*						0
268000 Radwaste								3.4		Γ				0
271000 Offgas					0 6							Catalytic recombination	2.7	1
272000 Radiation Monitoring										Γ				0
286000 Fire Protection									3	Γ	*			0
288000 Plant Ventilation			Γ											0
290001 Secondary CTMT														0
290003 Control Room HVAC	Г										01 32	Ability to explain and apply system limits and precautions.	3.4	1
290002 Reactor Vessel Internals			0 5							Γ		Off-site radiation levels	2.9	1
K/A Category Totals:	1	1	1	1	1	1	1	1	1	1	2	Group Point Total:		12

ES-401							tion Outline	Form E	S-401-1
Emei	<u> </u>				T-STORY	nt Ev	rolutions - Tier 1/Group 1 (SRO)	1	
E/APE # / Name / Safety Function	1 1	К 2	К 3	A 1	A 2	G	K/A Topic(s)	IR	#
295001 Partial or Complete Loss of Forced Core Flow Circulation / 1 & 4									0
295003 Partial or Complete Loss of AC / 6						02. 25	Knowledge of bases in technical specifications for limiting conditions for operations and safety limits.	3.7	1
295004 Partial or Total Loss of DC Pwr / 6					0 3		Battery voltage	2.9	1
295005 Main Turbine Generator Trip / 3									0
295006 SCRAM / 1					- T	*			0
295016 Control Room Abandonment / 7									0
295018 Partial or Total Loss of CCW / 8					0 2		Cooling water temperature	3.2	1
295019 Partial or Total Loss of Inst. Air / 8							,		0
295021 Loss of Shutdown Cooling / 4						04. 31	Knowledge of annunciators alarms and indications, and use of the response instructions.	3.4	1
295023 Refueling Acc / 8									0
295024 High Drywell Pressure / 5									0
295025 High Reactor Pressure / 3	[0		Suppression pool level	3.9	1
295026 Suppression Pool High Water Temp. / 5						7			0
295027 High Containment Temperature / 5									0
295028 High Drywell Temperature / 5						02. 25	Knowledge of bases in technical specifications for limiting conditions for operations and safety limits.	3.7	1
295030 Low Suppression Pool Wtr Lvl / 5					78 - 12 - 12 - 13 - 14 - 15 - 15 - 15 - 15 - 15 - 15 - 15				0
295031 Reactor Low Water Level / 2					*	2			0
295037 SCRAM Condition Present and Power Above APRM Downscale or Unknown / 1					0 7		Containment conditions/isolations	4.2	1
295038 High Off-site Release Rate / 9					3				0
600000 Plant Fire On Site / 8									0
K/A Category Totals:	0	0	0	0	4	3	Group Point Total:		7

									S-401-1
Em	ergen	cy an	d Abr	orma	al Plar	nt Ev	olutions - Tier 1/Group 2 (SRO)		
E/APE # / Name / Safety Function	K 1	K 2	К 3	A 1	A 2	G	K/A Topic(s)	IR	#
295002 Loss of Main Condenser Vac / 3									0
295007 High Reactor Pressure / 3									0
295008 High Reactor Water Level / 2					7.0				0
295009 Low Reactor Water Level / 2					A. E. Sanda, S.				0
295010 High Drywell Pressure / 5									0
295011 High Containment Temp / 5									0
295012 High Drywell Temperature / 5									0
295013 High Suppression Pool Temp. / 5									0
295014 Inadvertent Reactivity Addition / 1						U P			0
295015 Incomplete SCRAM / 1									0
295017 High Off-site Release Rate / 9						100			0
295020 Inadvertent Cont. Isolation / 5 & 7					•				0
295022 Loss of CRD Pumps / 1						04.0 4	Ability to recognize abnormal indications for system operating parameters which are entry-level conditions for emergency and abnormal operating procedures.	4.3	1
295029 High Suppression Pool Wtr Lvl / 5					N. Tanada				0
295032 High Secondary Containment Area Temperature / 5					0 2	ď.	Equipment operability	3.5	1
295033 High Secondary Containment Area Radiation Levels / 9									0
295034 Secondary Containment Ventilation High Radiation / 9						a si			0
295035 Secondary Containment High Differential Pressure / 5						Penanda Penanda			0
295036 Secondary Containment High Sump/Area Water Level / 5					0		Cause of the high water level	3.8	1
500000 High CTMT Hydrogen Conc. / 5			 						0
K/A Category Totals:	0	0	0	0	2	Īī	Group Point Total:		3

S-401														orm ES	-401-1
·		T	_			_	_	100			_	Tier	2/Group 1 (SRO)	Т	
E/APE # / Name / Safety Function	1 1	К 2	К 3	K 4				1	2	A 3	A 4	G	K/A Topic(s)	IR	#
03000 RHR/LPCI: Injection			_	L											0
05000 Shutdown Cooling Mode	_				L										0
06000 HPCI															0
07000 Isolation (Emergency) condenser												04. 06	Knowledge symptom based EOP mitigation strategies.	4	1
09001 LPCS									0				Low suppression pool level	3.3	1
09002 HPCS															0
211000 SLC									9						0
212000 RPS															0
215003 IRM									0.000						0
215004 Source Range Monitor															0
215005 APRM / LPRM									0 7				Recirculation flow channels flow mismatch	3.4	1
217000 RCIC		T							H						0
218000 ADS	T	T	T	Ī	T										0
223002 PCIS/Nuclear Steam Supply Shutoff									24						0
239002 SRVs															0
259002 Reactor Water Level Control															0
261000 SGTS															0
262001 AC Electrical Distribution	T	T							0.0			300	Loss of off-site power	4.3	1
262002 UPS (AC/DC)															0
263000 DC Electrical Distribution															0
264000 EDGs									i.			**			0
300000 Instrument Air									•				Knowledge of which events related to system operations/status should be reported to outside agencies.	3.6	1
400000 Component Cooling Water			Ī									7 2 2		<u> </u>	0
	1														
K/A Category Totals:	Ť	٥	٥	न	٥	٥	0	0			، آ ر	,	Group Point Total:		5

ES-401 BWR Examination Outline Form ES-401										5-401-1					
Plant Systems - Tier 2/Group 2 (SRO)															
E/APE # / Name / Safety Function	К 1	К 2	К 3	К 4	K 5	К 6	A 1	A 2	А 3	4		3	K/A Topic(s)	IR	#
201001 CRD Hydraulic											Li				0
201002 RMCS															0
201003 Control Rod and Drive Mechanism															0
201004 RSCS								4							0
201005 RCIS										L					0
201006 RWM									L	L					0
202001 Recirculation															0
202002 Recirculation Flow Control						_					100				0
204000 RWCU							L	3		L			Flow control valve failure	2.9	1
214000 RPIS								**		L					0
215001 Traversing In-core Probe								71							0
215002 RBM								1							0
216000 Nuclear Boiler Inst.						l		8							0
219000 RHR/LPCI: Torus/Pool Cooling Mode						Γ									0
223001 Primary CTMT and Aux.															0
226001 RHR/LPCI: CTMT Spray Mode															0
230000 RHR/LPCI: Torus/Pool Spray Mode					L			1	-	L			Loss of coolant accident	4.1	1
233000 Fuel Pool Cooling/Cleanup							Ĺ.					**			0
234000 Fuel Handling Equipment	3,										1				0
239001 Main and Reheat Steam				L	L					L					0
239003 MSIV Leakage Control					L		L								0
241000 Reactor/Turbine Pressure Regulator														<u> </u>	0
245000 Main Turbine Gen. / Aux.				L	L										0
256000 Reactor Condensate			L	L	L										0
259001 Reactor Feedwater					L										0
268000 Radwaste					L			22		╵					0
271000 Offgas					L										0
272000 Radiation Monitoring															0
286000 Fire Protection										I					0
288000 Plant Ventilation												\$\$	Ability to perform without reference to procedures those actions that require immediate operation of system components and controls.	4	1
290001 Secondary CTMT	L				L			ļ							0
290003 Control Room HVAC															0
290002 Reactor Vessel Internals															0
					\prod	\prod									
K/A Category Totals:	0	0	0	0	0	0	0		0	T	0	1	Group Point Total:		3

Generic Knowle	dge and A	bilities O	utline (1	Tier 3)	Form ES-401-3

ES-401

Facility Name	e:Dresde	en Date of Exam:4/23/07				
Category	K/A #	Topic		0	SRO	
	2.1. 07	Ability to evaluate plant performance and make operational judgments based on operating characteristics, reactor behavior, and instrument interpretation.	1R 3.7	# 1	IR	#
	2.1. 01	Knowledge of conduct of operations requirements.	3.7	1		
1	2.1.					
Conduct of	2.1.			_		
Operations	2.1. 33	Ability to recognize indications for system operating parameters which are entry-level conditions for technical specifications.			4	1
	2.1. 34	Ability to maintain primary and secondary plant chemistry within allowable limits.			2.9	1
	Subtota			2		2
	2.2. 03	(multi-unit) Knowledge of the design, procedural, and operational differences between units.	3.1	1		
	2.2. 28	Knowledge of new and spent fuel movement procedures.	2.6	1		
2.	2.2. 04	(multi-unit) Ability to explain the variations in control board layouts, systems, instrumentation and procedural actions between units at a facility.	2.8	1		
Equipment Control	2.2.					
33711.37	2.2. 08	Knowledge of the process for determining if the proposed change, test, or experiment involves an unreviewed safety question.			3.3	1
	2.2. 23	Ability to track limiting conditions for operations.			3.8	1
<u> </u>	Subtota			3		2
	2.3. 09	Knowledge of the process for performing a containment purge.	2.5	1		
	2.3. 11	Ability to control radiation releases.	2.7	1	<u></u>	
3.	2.3.					
Radiation Control	2.3.					
	2.3. 03	Knowledge of SRO responsibilities for auxiliary systems that are outside the control room (e.g., waste disposal and handling systems).			2.9	1
		Knowledge of the process for performing a planned gaseous radioactive release.			3.2	1
	Subtota			2		2
	2.4. 27	Knowledge of fire in the plant procedure.	3	1		
	2.4. 24	Knowledge of loss of cooling water procedures.	3.3	1		
4. Emergency	2.4. 39	Knowledge of the RO's responsibilities in emergency plan implementation.	3.3	1		
Procedures	2.4.					
/ Plan	2.4.					
	2.4. 26	Knowledge of facility protection requirements including fire brigade and portable fire fighting equipment usage.			3.3	1
	Subtota	l		3		1
Tier 3 Point	Total			10		7

Facility:	<u>Dresden</u>		Scenar	rio No: <u>ILT-N-1</u> Op-Test No: <u>2007-301</u>
Examine	ers:			Operators:
	onditions: Reac	·		%, C RFP and IRM 11 OOS.
Event No.	Malf. No.	, -	Event 「ype*	Event Description
1	N/A	R	NSO SRO	Raise Reactor power by withdrawing control rods
2	RDFAILF5	1	NSO SRO	RPIS failure for rod F5 ^T
3	RDFCFLO	ı	NSO SRO	CRD Flow Controller Fails Low
4	ADS3ABF	С	ANSO SRO	Target Rock Bellows Failure ^T
5	HP6 HP7	С	ANSO SRO	Trip of 2A circ water pump and 2B fails to start
6	HP5	м	TEAM	Loss of Main Condenser Vacuum due to increased air leakage
7	ICSTMRB	М	TEAM	Iso cond steam inlet line leak into the Reactor Building

^{* (}N)ormal, (R)eactivity, (I)nstrument, (C)omponent, (M)ajor, (T)ech Spec call

Facility:	cility: <u>Dresden</u> Scenario No: <u>ILT-N-2</u> Op-Test No: <u>2007-301</u>										
Examine	Examiners: Operators:										
			<u>.</u>								
Initial Conditions: Reactor power ~15%, C RFP and IRM 11 OOS. Turnover: Continue with startup per DGP 1-1.											
Event No.	Malf. No.		Event Type*	Event Description							
1	NONE	R	NSO SRO	Raise power by withdrawing control rods							
2	RODG07ST	С	NSO SRO	Stuck Control Rod ^T							
3	NIA1POT	ı	NSO SRO	APRM 1 Fails Downscale with failure of half scram ^T							
4	Q22	С	ANSO SRO	Service Water Pump Trip							
5	HPINIT	1	ANSO SRO	Spurious HPCI Initiation ^T							
6	121	М	TEAM	Small steam leak inside the Drywell							
7	K23 K40	М	TEAM	Overcurrent on Busses 23-1 & 28 / Inability to Spray DW / Emergency Depressurization							

^{* (}N)ormal, (R)eactivity, (I)nstrument, (C)omponent, (M)ajor, (T)ech Spec call

A mm a maline D	On an artis Or Alice	Corm CC D
Appendix D	Scenario Outline	Form E2-D-

Facility:	cility: <u>Dresden</u> Scenario No: <u>ILT-N-3</u> Op-Test No: <u>2007-30</u>									
Examiners: Operators:										
Initial Conditions: Reactor power ~79%, C RFP and IRM 11 OOS.										
Turnover: Reduce power with Recirc, to remove 'B' FWRV from service for maintenance										
Event No.	Malf. No.		Event Type*	Event Description						
1	NONE	R	NSO SRO	Reduce power with Recirc flow						
2	NONE	N	NSO SRO	Remove 'A' FWRV from service						
3	SER1589 SER0710 T18	С	ANSO SRO	U2 Emergency Diesel Generator Inoperable due to cooling water pump failure T						
4	None	С	NSO	2B RFP develops an oil leak, requiring it to be secured.						
5	H33/H34	М	TEAM	Loss of Feedwater / Manual scram, with a partial ATWS resulting						
6	NVM100AP NVM100BP NVML29AP NVML29BP	М	TEAM	Loss of RPV water level indication, requires entry into DEOP 400-1 RPV FLOODING						

^{* (}N)ormal, (R)eactivity, (I)nstrument, (C)omponent, (M)ajor, (T)ech Spec call