

# INFORMATION

April 27, 2000

SECY-00-0095

For: The Commissioners  
From: James L. Blaha, Assistant for Operations, Office of the EDO  
Subject: SECY-00-0095 WEEKLY INFORMATION REPORT - WEEK  
ENDING APRIL 21, 2000

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\*No input this week.

*/RA/*

James L. Blaha  
Assistant for Operations, OEDO

Contact:  
K. Landis, OEDO

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***/RA/***

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## WEEKLY INFORMATION REPORT - WEEK ENDING APRIL 21, 2000

The Weekly Information Report is compiled by the Office of the EDO and includes highlights of Headquarters and Regional Office activities.

Contact: K. Landis, OEDO by E-mail: [kdl@nrc.gov](mailto:kdl@nrc.gov).

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\*No input this week

Office of Nuclear Reactor Regulation  
Items of Interest  
Week Ending April 21, 2000

LOCA Considerations on Steam Generator Tube Design

On April 6, 2000, the NRC held a public meeting with the B&W Owners Group (B&WOG) to discuss a forthcoming risk-informed topical report that would justify excluding the most limiting large break loss of coolant accident (LBLOCA), postulated in the high point of the hot leg of the reactor coolant system, from the SG tube design considerations. The B&WOG has proposed this approach to address their omission of differential thermal expansion loads that could result in the SG tubes as a result of the recovery from a LBLOCA. Analysis of these thermal loads was omitted in recent S/G tube repair amendments at several B&W facilities because of an incorrect interpretation of the staff's 1985 approval of the B&W Leak-Before-Break topical report, BAW-1847, Rev. 1. (These repair amendments involved a method of making a new mechanical joint to redefine the RCS pressure boundary near the ends of the S/G tubes. This is known as a re-roll repair). The staff identified the omission through questioning of a recent tube repair amendment.

The owners group stated that, based on preliminary risk-evaluations, the  $\Delta$ CDF and  $\Delta$ LERF values which result from not including the LBLOCA in the SG repair design are acceptably small (approximately  $10^{-9}$ ) relative to the guidelines in RG 1.174. The B&W licensees have followed the guidance in GL 91-18 and completed operability evaluations to justify continued operation of the roll-repaired tubes. The licensees are incorporating the latest information discussed at the meeting into updates of their operability evaluations. Based on the information discussed at the meeting, the staff believes that licensees have an acceptable basis for continued operation while the B&WOG prepares a risk-informed topical addressing this issue and it is reviewed by the staff.

The B&WOG expects to submit the risk-informed topical report in June 2000, and will need staff approval within 3 months to support the Fall outage at Oconee. The staff expects to be able to support a timely review of the risk-informed topical.

Prior to the April 6 meeting (March 9), the staff informed the B&WOG that, until the design issue is resolved: 1) LBLOCAs are still considered in the S/G design basis and, therefore, licensees need to perform operability evaluations in accordance with GL 91-18 to justify continued operation of the re-roll repairs that are in service, and 2) additional re-roll repairs should not be performed pending resolution of the issue.

During the April 6 meeting and an April 5, 2000 conference call, the B&WOG described its qualitative assessments which justify the operability of the repaired SG tubes, and their reasoning for concluding the tubes will continue to perform their design function. The re-roll repairs had not been designed for a LBLOCA in the high point of the RCS hot leg, which can cause a high temperature difference between the SG tubes and the shell. This temperature difference causes a thermally-induced strain and high tensile tube loads. However, the B&WOG stated that these SG tube conditions do not result in significant tube leakage, or in offsite dose consequences greater than what is currently analyzed. This results from the fact that, at the time of high temperature difference after the postulated LOCA, there is a very small

pressure difference (~45 psi) across the tubes, and the pressure difference remains small for the duration of the event. In addition, this initiating event sequence does not result in a consequential failure of the ability to isolate the secondary side of the SG, which further reduces the potential for offsite dose.

Since the vendor has already informed the staff and the licensees about the problem, the B&WOG does not consider this a Part 21 notification issue. As part of making the risk-informed submittal, the B&WOG licensees were requested to verify that this is only a design basis change addressable under RG 1.174 and that no exemptions from the regulations are required.

#### Issuance of Order Approving License Transfer for the Test Reactor at Waltz Mill, Pennsylvania

The U.S. Nuclear Regulatory Commission (NRC) issued an Order on April 13, 2000, approving the transfer of License No. TR-2 from the CBS Corporation to Viacom Inc. Also, a draft conforming amendment was issued in a letter to CBS and Viacom, which reflects the license as it will appear when the merger of CBS into Viacom becomes effective. When the NRC is notified of the effective date of the merger, the amendment will be issued final.

The Test Reactor (a 60 MW unit that has not operated since 1962) is presently being decommissioned. The biological shield has been cut away, and the reactor vessel has been lifted off its supports by a jacking tower. The vessel will be shipped as a unit to the ALARON Corporation in Wampum, Pennsylvania beginning the second week of May 2000. The licensee plans to submit a request to terminate the reactor license and to transfer the facility and residual radioactivity to an existing NRC License No. SNM 770.

#### Resolution of Trip Setpoint Differing Professional Opinion

A differing professional opinion (DPO) regarding technical specification (TS) Trip Setpoints and Allowable Values for instrumentation was submitted on May 21, 1998. A final decision regarding the DPO was issued February 16, 1999, which concluded that use of either the Trip Setpoints or Allowable Values satisfy the requirements of 10 CFR 50.36 that technical specifications contain Limiting Safety System Settings. However, the staff was directed to document the basis for the use of the Allowable Values to represent the Limiting Safety System Settings in the TS.

In order to implement the DPO resolution, the staff prepared proposed changes to the improved Standard Technical Specification (iSTS) bases to document the basis for using the Allowable Values as the Limiting Safety System Settings. These proposed changes were distributed among the NRC staff for review and comment and subsequently forwarded to the Nuclear Energy Institute Technical Specifications Task Force following established procedures for modifications to the iSTS. The staff formally approved these bases changes by letter from William Beckner to James Davis dated March 1, 2000. These bases are now considered part of the iSTS and represent the basis for the use of Allowable Values to represent the Limiting Safety System Settings. These bases will be included as part of a formal revision to the iSTS which is to be published later this year. In the interim, these bases have been placed on the NRC external web site and are available for adoption by licensees converting to the iSTS.

#### University of Missouri at Columbia - Unplanned High Radiation Field

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ENCLOSURE A

The University of Missouri at Columbia reported a potential to cause an overexposure on April 12, 2000. The licensee had been performing inspections of the pool liner to determine its condition. In its lower section, the reactor pool has two areas (the two areas merge into one pool in the upper section of the pool), one that contains the reactor and one that contains fuel storage racks (called the Z basket). A portion of the biological shield next to the Z basket area consists of blocks that can be removed to make a gamma irradiation facility by allowing the radiation from fuel in the Z basket storage racks to pass through the thin pool wall. The gamma irradiation facility has never been used and the blocks had never been removed until April 6, 2000, when they were removed to inspect the outer side of the pool wall on April 12, 2000. All fuel was removed from the Z basket area to the main part of the pool near the reactor prior to the block removal.

The reactor experienced an unscheduled shut down on April 12, 2000, that required the reactor to be refueled. All fuel storage locations near the reactor were filled with fuel. To allow the fuel movements necessary to replace the core, a fuel inspection rig was going to be put into the pool to hold one element. However, a change was made to the refueling procedure to put the fuel element in the rack in the Z basket. When the element was placed in the Z basket, the north beam floor area radiation monitor alarmed. A health physicist was sent to investigate, found elevated radiation levels as she approached the beam floor (an area that is administratively controlled as a high radiation area), realized that the fuel element was the cause of the problem, and alerted the operating crew who had already moved the fuel element within the Z basket. She then returned to the beam port floor and measured a radiation field of 200 rem/hr at the north edge of the area where the shielding was removed for the inspection. The operators then removed the fuel from the Z basket and terminated the event. The licensee estimates that the condition existed for about three minutes. No one was on the beam floor when the fuel element was placed in the Z basket area and no over exposures occurred.

A stand down on reactor operations was in effect from about 2:30 p.m. until 8:30 p.m. to evaluate and understand the event. The incident was discussed with reactor operations crews and others who were involved. The shielding was replaced and the Z basket area is currently tagged out to prevent fuel from being placed in the area. All non-routine work at the facility has been put on hold. The licensee assigned a group of managers to investigate root causes. Based on the reading of an area film badge that was in line with the beam, the licensee has performed a preliminary calculation that resulted in a beam dose rate of 312 rem/hr.

A team of three inspectors (a NRR Project Manager, a Non-Power Reactor Inspector, and a NRR Senior Radiation Specialist) conducted a reactive inspection at the site on April 14, 2000. The inspection team gathered additional facts concerning the event, and reviewed the licensee's response, root cause investigation and corrective actions. Preliminary inspection findings indicate that no over exposures of radiation to the facility staff had occurred and that the event had no potential to affect members of the public. The NRC inspectors also found that the licensee had identified weaknesses in their development of a procedure for conducting the pool liner inspection and in communications with the reactor staff on changes to the biological shielding that contributed to the event. The NRC will issue an inspection report in the near future.

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Office of Nuclear Material Safety and Safeguards  
Items of Interest  
Week Ending April 21, 2000

Final Rule Published Entitled, "Energy Compensation Sources for Well Logging and Other Regulatory Clarifications"

On April 17, 2000, the Nuclear Regulatory Commission published in the Federal Register (65 FR 20337) a final rule entitled, "Energy Compensation Sources for Well Logging and Other Regulatory Clarifications." This change to 10 CFR Part 39 improves, clarifies, updates, and reflects current practices in the well logging industry. The rule deals with: (1) low activity energy compensation sources; (2) tritium neutron generator target sources; (3) specific abandonment procedures in the event of an immediate threat; (4) changes to requirements for inadvertent intrusion on an abandoned source; (5) the codification of an existing generic exemption; and (6) updating this part to be consistent with the Commission's metrication policy.

Meeting with Agency for Health Care Research and Quality, Department of Health and Human Services

On April 20, 2000, representatives from the Division of Industrial and Medical Nuclear Safety, the Office of the General Counsel, and the Office of State and Tribal Programs met with Dr. John Eisenberg, Director of the Agency for Health Care Research and Quality (AHCRQ) and his staff. AHCRQ is the lead federal agency for the development of a new regulation that will require all hospitals participating in the Medicare program to have mandatory reporting requirements for deaths and serious injuries and voluntary reporting requirements of near-misses and close calls. The intent is to have the states run the programs and the data processed by AHCRQ. The meeting was to discuss the Nuclear Regulatory Commission's (NRC's) experience with its mandatory patient safety (i.e., misadministration) reporting requirements. The discussion focused on NRC's: (1) regulatory process in developing the misadministration/medical event rule; (2) relationship with Agreement States; (3) experience with the states' collecting and forwarding misadministration reporting data; (4) interactions with the medical community; (5) processing and analyzing the reporting information; and (6) processes for getting trend or important generic information back to the medical community.

Office of Nuclear Regulatory Research  
Items of Interest  
Week Ending April 21, 2000

Risk-Informed Inspection Notebooks

As part of the NRC's Revised Reactor Oversight Process, RES has provided inspectors with new inspection notebooks that facilitate risk-informed inspections. The initial risk-informed inspection notebooks have now been completed for all the reactor sites. This effort, part of the Significance Determination Process (SDP) – Phase 2, was an extension of the work at nine pilot plants that produced the notebook format that can be used readily by the inspectors in the field.

The main section of the inspection notebook contains the risk estimation worksheets. These worksheets link inspection findings to dominant sequences and contributors, a necessary step in the SDP used to risk-characterize inspection findings. In addition, the notebooks contain simplified event trees, dependency tables, and success criteria specific to each plant to facilitate the use of the worksheets. Taken together, this information will enable the inspector to focus attention on the most risk-significant inspection areas and to better understand inspection findings from a risk perspective. The inspection notebooks will be sent to each licensee for review and will be followed by a site visit. During the site visit, the notebook information will be validated or modified as appropriate.

There are two more efforts planned that will augment the current information and make the notebooks a more complete screening mechanism. The first is to add additional plant-specific sequences, and the second is to incorporate sequences caused by external events.

Welding of Irradiated Stainless Steel

In 1999, RES and the Electric Power Research Institute (EPRI) signed a Memorandum of Understanding (MOU) to jointly evaluate the feasibility of underwater welding of highly irradiated in-vessel stainless steel components of BWRs. The Boiling Water Reactor Vessels and Internals Project (BWR/VIP) is also partnered with EPRI on this effort. Welding of irradiated stainless steel is very problematic because of (1) cracking during the welding process as a result of helium entrapment in the material, and (2) cracking attributable to irradiation-damaged microstructure in the components. As part of this program, irradiated stainless steel samples were removed from jet pump riser brace pads in the Monticello reactor pressure vessel during January 2000. These samples were examined at the Pacific Northwest National Laboratory (PNNL) to determine the concentrations of boron and helium, as well as other radioisotopes that can affect the weldability of the stainless steel in the irradiated condition.

Preliminary results from this effort show that the concentration of helium in the samples removed from the jet pump riser brace pads ranged from approximately 0.008 to 0.0255 atomic parts per million (appm). Very limited welding repair experience in Japan suggests that routine welding techniques can be used when the concentration of helium is less than 0.05 appm. While these results are encouraging, sufficient experience does not yet exist to support a final conclusion. As part of the ongoing cooperative effort, RES will have additional samples obtained from vessel components at two other BWRs. These results will then be used to

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validate predictions regarding the range of helium expected for the BWR fleet. With this information and estimates of the thermal fluence, the feasibility of using standard welding techniques, as well as modified techniques, for structural repairs in these locations can be evaluated.

Qualified weld repair techniques for highly irradiated stainless steel components in BWRs offer licensees alternatives to costly replacements or labor intensive mechanical repairs. Component replacements or mechanical repairs (e.g., BWR core shroud tie rods) result in significant burden to licensees. However, weld repairs, if performed incorrectly, could exacerbate degradation and cracking of the components. Therefore, this work addresses both burden reduction and safety considerations.

#### Draft Report - Regulatory Effectiveness of the Station Blackout Rule

RES issued a draft report, "Regulatory Effectiveness of the Station Blackout Rule," for industry and public comment in April 2000. RES is reviewing selected regulations, starting with the station blackout (SBO) rule, to determine whether the requirements are achieving the desired outcomes. This initiative is part of an evolving program to make NRC activities and decisions more effective, efficient, and realistic. This report evaluates the effectiveness of the SBO rule by comparing regulatory expectations to outcomes. A set of baseline expectations was established from the SBO rule and related regulatory documents in the areas of coping capability, risk reduction, emergency diesel generator reliability, and value-impacts.

The report's conclusion is that the SBO rule is effective, and industry and NRC costs to implement the SBO rule were reasonable considering the outcomes. In implementing the SBO rule, some plants made hardware modifications (e.g., the addition of diesel generator or gas turbine generator power supplies). All plants generally maintained safety emergency diesel generator reliability at 0.95 or better and established SBO coping and recovery procedures. Consequently, the plants have gained SBO coping capability, reduced risk, increased the tolerance to a loss of ac offsite or onsite power, and many plants benefitted economically from the addition of power supplies. A comparison of the SBO rule expectations to the corresponding outcomes indicates that no additional generic actions are warranted and that no new generic safety issues have been identified.

However, consistent with the Principles of Good Regulation, opportunities to revise the regulatory documents have been identified. These revisions would provide for common use and interpretation of emergency diesel generator reliability terms, criteria, and measurements; delete the use of trigger values to assess compliance with emergency diesel generator reliability goals; and provide additional guidance in lieu of reactor shutdowns following limited losses of offsite or onsite power sources.

#### Workshop on PRA for Kalinin VVER 1000

RES has just concluded a 4-week workshop at BNL for the completion of a Level 1 PRA (including a fire and seismic analysis) for the Kalinin VVER 1000 nuclear plant in Russia. The project is a multi-year, cooperative effort between NRC and six Russian organizations, including the Russian regulatory agency, GAN, and its scientific/engineering counterpart, SEC-NRS. The

project has been sponsored by the U.S. Agency for International Development and is one of the first aimed at transferring PRA technology to the former Soviet Union. The results will be published jointly by NRC and GAN this summer once all translations and editing is complete. At that time, follow-on discussions will take place with GAN on possible future joint activities such as a Level 2 analysis.

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Incident Response Operations  
Items of Interest  
Week Ending April 21, 2000

RASCAL 3.0 Beta Test

A beta test version of RASCAL 3.0 was sent out to more than 80 reviewers. RASCAL 3.0 is a code used to determine recommendations for appropriate actions to protect the public from an accidental release of radioactive materials from a nuclear facility. The reviewers include staff from NRC, other Federal agencies, National laboratories, State emergency response agencies, commercial nuclear facilities, international agencies, and foreign facility operators and regulatory agencies. Comments on the beta test version are due by June 5, 2000.

Preliminary Notifications

1. PNO-I-00-009, Community Hospital Of Lancaster, TECHNETIUM-99M CONTAMINATION OF A PACKAGE IN EXCESS OF REGULATORY LIMITS.
2. PNO-III-00-015, Toledo Edison Co. (Davis Besse 1), PORTIONS OF PLANT EVACUATED WHILE NON-TOXIC FUMES CHECKED.
3. PNO-IV-00-009, International Radiography & Inspection Services, Inc., UNINTENTIONAL RADIOGRAPHY SOURCE DISCONNECT.
4. PNO-IV-00-010, St. Luke's Regional Medical Center, BRACHYTHERAPY MISADMINISTRATION.

Office of Administration  
Items of Interest  
Week Ending April 21, 2000

Energy Compensation Sources for Well Logging and Other Regulatory Clarifications (Part 39)

A final rule that amends the regulations governing licenses and radiation safety requirements for well logging was published in the Federal Register on April 17, 2000 (65 FR 20337). The final rule modifies the regulations applicable to low activity energy compensation sources; tritium neutron generator target sources, specific abandonment procedures in the event of an immediate threat, and inadvertent intrusion on an abandoned source. The final rule makes a number of additional changes that reflect developments made in well logging technology, current practices in the well logging industry, and current NRC procedures. The final rule becomes effective May 17, 2000.

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Chief Information Officer  
Items of Interest  
Week Ending April 21, 2000

Freedom of Information and Privacy Act Requests received during the 5-Day Period of  
April 14, 2000 through April 20, 2000:

Nine Mile Point, response to NRC's request for Additional Information (RAI) 460.12, including transmittal of the 10-24-83 analysis of design basis for hydrogen detonation pressure. (FOIA/PA-2000-0206)

Diablo Canyon, OI report and exhibits regarding a discrimination investigation. (FOIA/PA-2000-0207)

Sharon Harris plant, CP&L, spent fuel pool and head load calculations reference documents. (FOIA/PA-2000-0208)

List of licensees in the Island of Puerto Rico. (FOIA/PA-2000-0209)

Contract awarded for solicitation RS-HR-00-281. (FOIA/PA-2000-0210)

MLTS database. (FOIA/PA-2000-0211)

Point Beach plant, status summary of each allegation received since 1/1/98. (FOIA/PA-2000-0212)

Individual, transcripts and investigation reports 3-1999-012, 3-1999-033, and 3-1999-034 with exhibits. (FOIA/PA-2000-0213)

Office of Human Resources  
Items of Interest  
Week Ending April 21, 2000

<b>Arrivals</b>		
ANDRUSZKIEWICZ, Andrew	MATERIALS ENGINEER	NRR
MANON, Butt	OFFICE AUTOMATION ASSISTANT	NMSS
WERNER, Gregory	OPERATIONS ENGINEER	RIV

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Office of Public Affairs  
Items of Interest  
Week Ending April 21, 2000

Media Interest

Commissioner Jeffrey Merrifield was interviewed by Bloomberg Business News Service on the future of the nuclear power industry.

<b>Press Releases</b>	
<b>Headquarters:</b>	
00-66	NRC To Hold Two Public Meetings May 4 in Nevada
00-67	Note To Editors: Name Change For Office Of State Programs
00-68	NRC Revises Well Logging Regulations
00-69	NRC Advisory Joint Subcommittee To Meet In Rockville, Maryland On May 4
00-70	NRC Advisory Committee On Reactor Safeguards to Meet In Rockville, Maryland On May 10
00-71	NRC Advisory Committee On Reactor Safeguards to Meet In Rockville, Maryland On May 13
<b>Regions:</b>	
I-00-28	NRC To Discuss Performance At Three Mile Island Nuclear Plant
I-00-29	NRC, Consolidated Edison To Discuss Indian Point 2 Emergency Preparedness, Response Issues
III-00-17	NRC Staff To Meet With Public To Discuss Future Zion Decommissioning Activities

Office of International Programs  
Items of Interest  
Week Ending April 21, 2000

Brazilian Visit

On April 17, 2000, managers from several Brazilian government regulatory agencies visited the NRC. The visit was requested by the Institute of Brazilian Business and Public Management Issues at George Washington University as part of a comprehensive two-week information gathering tour. OGC, NMSS and IRO gave presentations on the statutory and regulatory background of the NRC.

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Office of the Secretary  
Items of Interest  
Week Ending April 21, 2000

Document Released to Public	Date	Subject
<b>Decision Documents</b>		
1. SECY-00-0077	3/30/2000	Modifications to the Reactor Safety Goal Policy Statement
<b>Negative Consent Documents</b>		
1. SECY-00-0066	3/15/2000	Proposed Response to State of Utah on Re-Examination of the Utah Land Ownership Exemption for the Envirocare of Utah, Inc., (Envirocare) Site
2. SRM on 00-0066	4/11/2000	Proposed Response to State of Utah on Re-Examination of the Utah Land Ownership Exemption for the Envirocare of Utah, Inc., (Envirocare) Site
<b>Information Papers</b>		
1. SECY-00-0087	4/13/2000	SECY-00-0087 - Weekly Information Report - Week Ending April 7, 2000
<b>Memoranda</b>		
1. M000331A	4/18/00	Briefing on Risk-Informed Regulation Implementation Plan (SECY-00-0062), Friday, March 31, 2000

Commission Correspondence

1. Letter to Congresswoman Edward J. Markey, dated April 17, 2000, concerns compliance with requirements of the Government in the Sunshine Act and the openness and transparency of the regulatory process.
2. Letter to James Riccio, Critical Mass Project, dated April 14, 2000, responds to views on the need for formal licensing hearings for a high-level nuclear waste repository.
3. Letter to Sandy Green, Board of County Commissioners, dated April 12, 2000, concerns request for enhanced role for the On-Site Representatives' office in Las Vegas, Nevada.
4. Letter to Katie Sweeney, National Mining Association, April 11, 2000, responds to request for release of legal analyses prepared by the Office of the General Counsel concerning NRC's jurisdiction over in-situ leach wellfields, and concurrent jurisdiction of non-

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agreement states and the NRC over nonradiological components of Section 11.e(2) byproduct material.

Federal Register Notices Issued

1. Advisory Committee on Reactor Safeguards; Meeting Notice.
2. Private Fuel Storage, L.L.C. (Independent Spent Fuel Storage Installation); Docket No. 72-22-ISFSI; Notice of Hearing and of Opportunity to Make Oral or Written Limited Appearance Statements.
3. [NUREG-1600] Revision of the NRC Enforcement Policy.

Region II  
Items of Interest  
Week Ending April 21, 2000

Cost Management Steering Committee Meeting

On April 18, the Regional Administrator attended the Cost Management Steering Committee meeting at the Price Waterhouse Coopers facility in Arlington, Virginia.

NRC/FBI Exercise Final Planning Meeting

On April 17, representatives from Region II, the Incident Response Operations, the Office of Nuclear Material Safety and Safeguards and the Office of Nuclear Reactor Regulation met in Headquarters to discuss the final planning for the FBI/NRC Emergency Exercise that is scheduled for May 24, 2000 in Lynchburg, Virginia.

Licensee Renewal Meeting

On April 19-20, representatives from Regions I, II, III and the License Renewal and Standardization Branch, in the Office of Nuclear Reactor Regulation, met in the Region II office to conduct a "Lessons Learned" seminar from their experiences with the license renewal inspections for Calvert Cliffs and Oconee. The outcome of the meeting was a proposed update revision to MC2516, Policy and Guidance for the License Renewal Inspection Programs and IP71002, License Renewal Inspection Procedure.

Region III  
Items of Interest  
Week Ending April 21, 2000

Public Meeting at D. C. Cook

On April 18, 2000, Region III and Headquarters staff members participated in a meeting with American Electric Power officials at the D.C. Cook Nuclear Power Plant near Bridgman, Michigan, to discuss the status of start-up preparations for D.C. Cook Unit 2. Three reporters attended the meeting.

Staff Meets with IDNS

On April 19, 2000, Region III staff members met with members of the Illinois Department of Nuclear Safety to discuss various memoranda of understanding the NRC has with IDNS concerning resident inspectors and the ASME code.

Region III Staff Addresses a Differing Professional View

Region III addressed a Differing Professional View regarding the appropriate severity level of an enforcement action involving byproduct material that had been found unattended during an NRC inspection in a NRC-licensed medical facility. Based on a review of the case's facts and the enforcement policy, Regional Administrator Jim Dyer agreed with the ad hoc DPV panel that, contrary to the filer's view, the violation was better characterized as a minor violation rather than as a severity level IV violations as originally issued.

Region IV  
Items of Interest  
Week Ending April 21, 2000

Predecisional Enforcement Conference with the Syncor International Corporation

The Regional Administrator along with Region IV staff members conducted a closed predecisional enforcement conference on April 18, 2000, with representatives of the Syncor International Corporation. The purpose of the conference was to discuss apparent violations associated with a transportation incident.

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Office of Congressional Affairs  
 Items of Interest  
 Week Ending April 21, 2000

CONGRESSIONAL HEARING SCHEDULE, NO. 13

OCA Contact	DATE & PLACE	TIME	WITNESS	SUBJECT	COMMITTEE
Combs	04/12/00 <b>POSTPONED</b>	TBA	Dr. Paperiello, Corps of Engineers, EPA, States	FUSRAP <b>POSTPONED</b>	Senators Smith/Baucus Environment & Public Works
Gerke	04/27/00 SH-216	9:30	TBA	Electricity Deregulation	Senators Murkowski/Bingaman Energy and Natural Resources
Gerke	05/04/00 SD-342	10:00	GAO, CRS, Paul Light	Has Reinventing Government Worked?	Senators Voinovich/Durbin Gov't Mgmnt, Restructuring & DC Governmental Affairs
Gerke	TBA 2154 RHOB	TBA	TBA	Effectiveness of GPRA	Reps. Horn/Turner Gov't Mgmnt, Info, & Technology Government Reform
Gerke	TBA 2154 RHOB	10:00	TBA	Financial Management Act	Reps. Horn/Turner Gov't Mgmnt, Info, & Technology Government Reform

The House is in recess until May 2.

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ENCLOSURE R