

Commonwealth Edison Company
LaSalle Generating Station
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March 17, 2000

United States Nuclear Regulatory Commission
Attention: Document Control Desk
Washington, D.C. 20555

LaSalle County Station, Units 1 and 2
Facility Operating License Nos. NPF-11 and NPF-18
NRC Docket Nos. 50-373 and 50-374

Subject: Licensee Event Report No. 00-001-00

In accordance with 10 CFR 50.73(a)(2)(i), Commonwealth Edison (ComEd) Company is submitting Licensee Event Report No. 00-001-00, Docket No. 050-373.

Attachment A provides the commitment for this submittal.

Should you have any questions concerning this letter, please contact Mr. Frank A. Spangenberg, III, Regulatory Assurance Manager, at (815) 357-6761, extension 2383.

Respectfully,

A handwritten signature in black ink, appearing to read "Charles G. Pardee", with a long horizontal flourish extending to the right. Below the signature, the word "for" is written in a smaller, cursive script.

Charles G. Pardee
Site Vice President
LaSalle County Station

Attachments: Attachment A
Licensee Event Report

cc: Regional Administrator - NRC Region III
NRC Senior Resident Inspector - LaSalle County Station

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Attachment A
Regulatory Commitment(s)

ComEd is committing to the following actions. Any other actions discussed in this submittal represent intended or planned actions by ComEd. They are described for the NRC's information and are not regulatory commitments.

<i>Regulatory Commitment(s)</i>	<i>Tracking Number</i>
The instrument surveillance procedures LIS-RI-101/201, Unit 1/2 "RCIC Steam Line High Flow Isolation Calibration", will be revised to include quarterly calibration of the associated time delay relays.	ATM# 23924-14

LICENSEE EVENT REPORT (LER)

Estimated burden per response to comply with this mandatory information collection request: 50 hrs. Reported lessons learned are incorporated into the licensing process and fed back to industry. Forward comments regarding burden estimate to the Records Management Branch (T-6 F33), U. S. Nuclear Regulatory Commission, Washington, DC 20555-0001, and to the Paperwork Reduction Project (3150-0104), Office of Management and Budget, Washington, DC 20503. If an information collection does not display a currently valid OMB control number, the NRC may not conduct or sponsor, and a person is not required to respond to, the information collection.

FACILITY NAME (1): LaSalle County Station, Unit 1

DOCKET NUMBER (2) 05000373

PAGE (3)
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TITLE (4) Inadequate Implementation of Technical Specification Surveillance Requirements Results In Missed Surveillance on Reactor Core Isolation Cooling Time Delay Relays

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAME	DOCKET NUMBER
02	17	00	00	001	00	03	17	00	LaSalle County Station Unit 2	05000374
									FACILITY NAME	DOCKET NUMBER

OPERATING MODE (9)	1	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more) (11)
POWER LEVEL (10)	100	
		<input type="checkbox"/> 20.2201(b) <input type="checkbox"/> 20.2203(a)(2)(v) <input checked="" type="checkbox"/> 50.73(a)(2)(i) <input type="checkbox"/> 50.73(a)(2)(viii)
		<input type="checkbox"/> 20.2203(a)(1) <input type="checkbox"/> 20.2003(a)(3)(i) <input type="checkbox"/> 50.73(a)(2)(ii) <input type="checkbox"/> 50.73(a)(2)(x)
		<input type="checkbox"/> 20.2203(a)(2)(i) <input type="checkbox"/> 20.2003(a)(3)(ii) <input type="checkbox"/> 50.73(a)(2)(iii) <input type="checkbox"/> 73.71
		<input type="checkbox"/> 20.2203(a)(2)(ii) <input type="checkbox"/> 20.2003(a)(4) <input type="checkbox"/> 50.73(a)(2)(iv) <input type="checkbox"/> OTHER
		<input type="checkbox"/> 20.2203(a)(2)(iii) <input type="checkbox"/> 50.36(c)(1) <input type="checkbox"/> 50.73(a)(2)(v) Specify n Abstract below or in NRC Form 366A
		<input type="checkbox"/> 20.2203(a)(2)(iv) <input type="checkbox"/> 50.36(c)(2) <input type="checkbox"/> 50.73(a)(2)(vii)

LICENSEE CONTACT FOR THIS LER (12)

NAME: Danny Bost, Site Engineering Manager TELEPHONE NUMBER (Include Area Code): (815) 357-6761 Extension 2208

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO EPIX	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO EPIX

SUPPLEMENTAL REPORT EXPECTED (14)

<input type="checkbox"/> YES (If yes, complete EXPECTED SUBMISSION DATE)	<input checked="" type="checkbox"/> NO	EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines 16)

On February 17, 2000, Units 1 and 2 were in Operational Mode 1 at 100% power. At 1600 hours, based on a review of all safety-related time delay relays listed in procedure LTP-100-6, "Time Delay Relay (TDR) Calibration Program," it was concluded that the quarterly Technical Specification surveillance channel calibration for the Reactor Core Isolation Cooling (RCIC) steam line high flow isolation logic did not include time delay relays 1(2)E51A-K47 and 1(2)E51A-K48. The associated pressure switch is calibrated quarterly and the time delay relay is calibrated each refueling cycle. Since the time delay relay is part of the instrument channel, it should have been calibrated quarterly. Corrective actions were to calibrate the RCIC steam line high flow isolation logic time delay relays and to revise the instrument surveillance procedures.

The safety significance of the event was minimal. The operability of the Emergency Core Cooling Systems was not challenged. When tested, the time delay relays were found within calibration tolerances.

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TEXT CONTINUATION

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(If more space is required, use additional copies of NRC Form 366A)(17)

PLANT AND SYSTEM IDENTIFICATION

General Electric - Boiling Water Reactor, 3323 Megawatts Thermal Rated Core Power
Energy Industry Identification System (EIIS) codes are identified in the text as [XX].

A. CONDITION PRIOR TO EVENT

Unit(s): 1/2 Event Date: 02/17/00 Event Time: 1600 Hours
Reactor Mode(s): 1 Power Level(s): 100
Mode(s) Name: Run

B. DESCRIPTION OF EVENT

On February 17, 2000, while performing a review of parameters/functions that should be included in the Improved Technical Specification (ITS) submittal for Commonwealth Edison Boiling Water Reactors, Quad Cities and Dresden Stations identified that the required time delay on their low steam line pressure function was not specified in their current Technical Specifications. It was also identified that the time delay relays were calibrated at a different interval than the associated pressure switches. The time delay relays were being calibrated on a once per cycle frequency, while the pressure switches were calibrated on a quarterly frequency in accordance with the Technical Specifications.

In response to these findings, LaSalle County Station initiated Problem Identification Form (PIF) # L2000-00787 to determine if a similar deficiency existed. A review of safety-related time delay relays was performed. This review identified that only the Reactor Core Isolation Cooling (RCIC) [BN] steam line high flow isolation logic time delay relays 1(2)E51A-K47 and 1(2)E51A-K48 were not calibrated at the same interval as the associated instrument channel. In each of these cases, the associated pressure switch is calibrated quarterly and the time delay relay was calibrated each refueling cycle. Since the time delay relay is part of the instrument channel, it should have been calibrated quarterly.

This event is being reported pursuant to 10CFR50.73(a)(2)(i)(B), any event or condition prohibited by the plant's Technical Specification.

C. CAUSE OF EVENT

The root cause of the event was determined to be personnel error resulting from inadequate knowledge of industry standards when the time delay relay was added to the instrument channel in 1982, and when the subsequent system reviews were performed in 1991 and 1997.

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D. SAFETY ANALYSIS

The safety significance of this event was minimal. The operability of the redundant Emergency Core Cooling Systems was not challenged. When calibrated, the as-found settings for the time delay relays were found to be in calibration.

E. CORRECTIVE ACTIONS

Immediate Actions:

1. The RCIC steam line high flow isolation logic time delay relays for Unit 1 and 2 were calibrated to meet the quarterly surveillance requirements.
2. An extent of condition review of all safety-related time delay relays was performed. This review identified that only the Reactor Core Isolation Cooling (RCIC) [BN] steam line high flow isolation logic time delay relays 1(2)E51A-K47 and 1(2)E51A-K48 were not calibrated at the same interval as the associated instrument channel.

Corrective Actions to Prevent Recurrence

3. The instrument surveillance procedures LIS-RI-101/201, Unit 1/2 "RCIC Steam Line High Flow Isolation Calibration," will be revised to include quarterly calibration of the associated time delay relays (ATM #23924-14).

F. PREVIOUS OCCURRENCES

A review of Licensee Event Reports over the previous three years found no previous occurrences of a missed surveillance due to incorrect periodicity requirements.

G. COMPONENT FAILURE DATA

Since no component failure occurred, this section is not applicable.