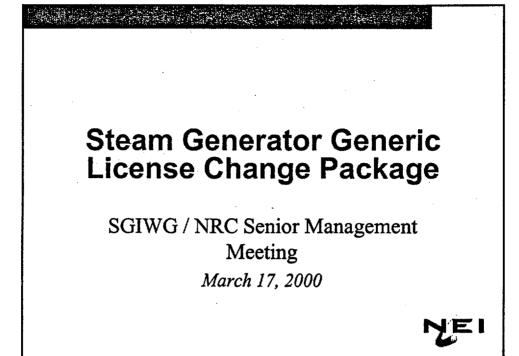
NRC FORM 658			U.S. NUCLEAR REGULATORY COMMISSION			
TRANSMITTAL OF MEETING HANDOUT MATERIALS FOR IMMEDIATE PLACEMENT IN THE PUBLIC DOMAIN						
person who iss materials, will t circumstances	ued the meeting notice). The c	omple I Desk	/ the person who announced the meeting (i.e., the ted form, and the attached copy of meeting handout on the same day of the meeting; under no g day after the meeting.			
DATE OF MEETING 03/17/2000	The attached document(s), which was/were handed out in this meeting, is/are to be placed					
	Docket Number(s)		N/A			
,	Plant/Facility Name	N/ 2	4			
TAC Number(s) (if available) N/A		4				
	Reference Meeting Notice	tice 2000-191 and 2000-192				
	Purpose of Meeting (copy from meeting notice)	Discuss the review of an industry initiative entitled				
		NE	NEI 97-06, Steam Generator Program Guidelines			
		(h	andorts discussed at both meetings)			
NAME OF PERSON WH	O ISSUED MEETING NOTICE		TITLE			
James Andersen			Project Manager			
OFFICE NRR						
DIVISION	······					
DE						
BRANCH EMCB						
Distribution of this Docket File/Centr PUBLIC	<u>s form and attachments:</u> al File		J 703			



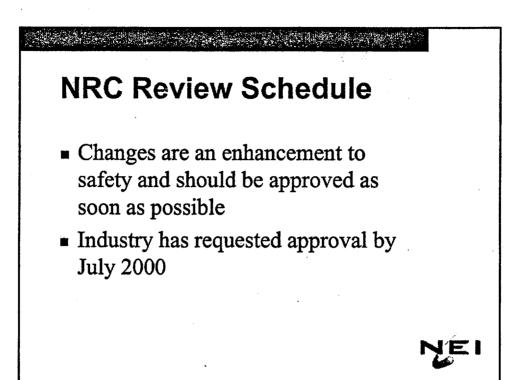
Agenda	
 Introduction Generic License Change Package 	All
Industry Commitment	SGIWG
NRC comments	NRC
NRC Review Process	NRC
NRC Review Schedule	All
 Tech Spec Amendments 	All
 Implementation 	All
 SG Program Support 	SGIWG
 Recent industry events 	SGIWG
 Conclusions 	All
	NEI

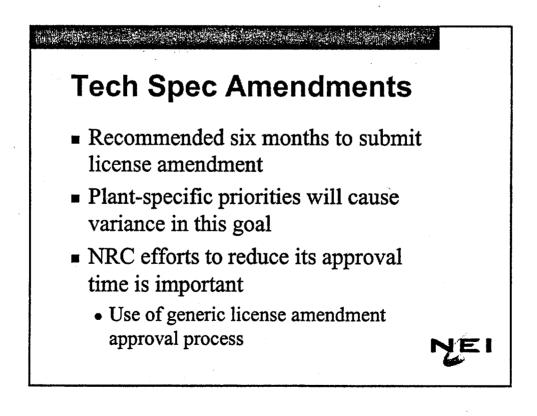
Introduction

- The proposed changes are an enhancement to plant safety
- The industry and NRC have worked closely in the development of the generic package
- The industry is behind the proposed changes
- It is time to resolve this issue

Industry Commitment

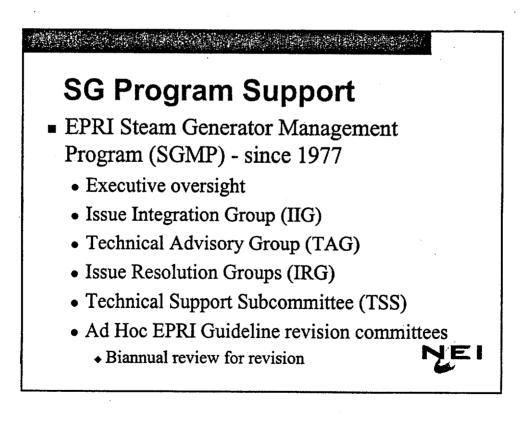
- 68 / 69 PWR Units have agreed to implement the license changes <u>as</u> <u>proposed</u>
- Changes to reflect plant-specific design and licensing basis will be necessary





Implementation

- Guideline workshops
- Benchmarking
- SG Program workshops
- Licensing workshops
- Confusion arising from dual guidance
 - DG-1074 and potential GL

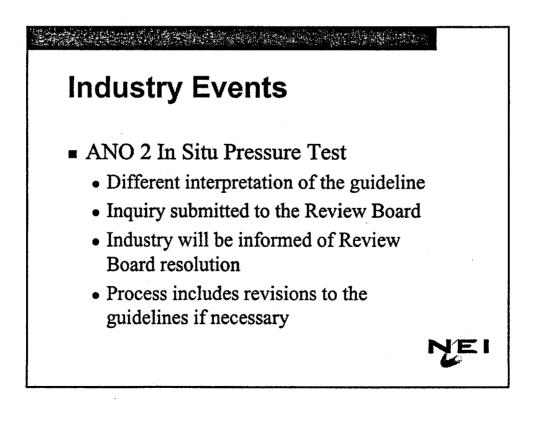


SG Program Support

INPO Steam Generator Program Reviews
 since 1995

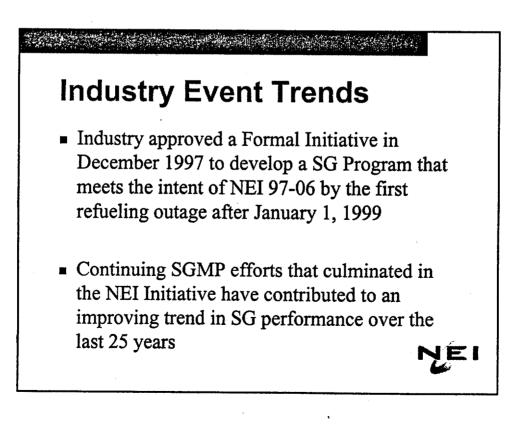
- Systematic review of all stations
- Use of industry peers

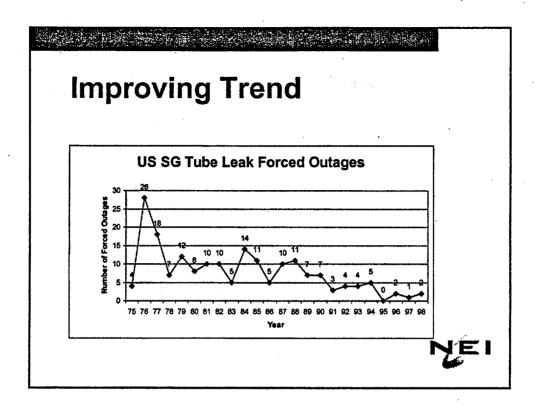
- Annual summary to industry
- Self assessments
- NEI/SGMP SG Review Board
- NEI, INPO, and EPRI SG Websites

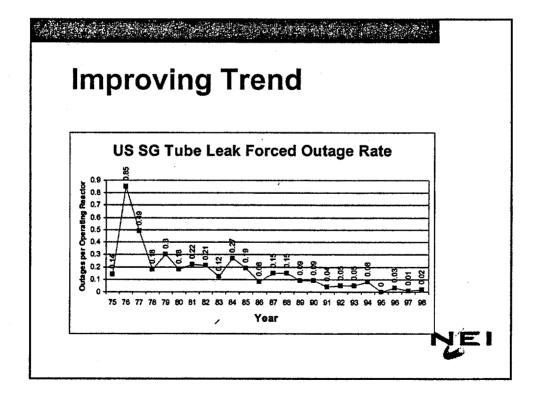


Industry Events

- Indian Point 2 Tube Leak
 - INPO assist team using industry peers
 - Too early to make conclusions on implications
 - Industry's SG Program Guidelines are based on self evaluation and evolution
 - Potential impact on guidelines is being evaluated







Conclusions

Improvement over existing requirements

- Promotes application of best practices
- Enhances safe operation
- Process based on self evaluation and evolution
- Encourages technical advances
- Industry is behind the improvements
- July 2000 approval is appropriate

Agenda, SG Generic License Change Package Meetings March 16 and 17, 2000

SGTF / NRC Staff Meeting March 17, 2000 10:00 AM – 12:00 PM 8B4 White Flint 1

•	 Generic License Change Package NRC technical comments on the package 	NRC	
	 Review process for the Generic License Change Package 	NRC	
	 Schedule for review of the Generic License Change Package 	NRC	
	 Implementation plan for the Tech Spec changes included in the Generic License Change Package Anticipated timeframe for adoption NRC review time for license amendment requests Need for licensing workshops to assist understanding of the Generic License Change Package and its implementation 	SGTF NRC All	
•	Internal industry efforts to facilitate SG Programs	SGTF	
 Generic SE on TIG Welded sleeves – comments on the document and approach 			
•	Implications of the IP2 event on NEI 97-06 SG Program	SGTF	

Comments on TIG Welded Sleeve Generic SER

- General We believe that this represents an acceptable approach.
- Items 3 and 4 seem to contradict item 1 Item 1 should also allow changes via the 50.59 process. What is meant by NRC approval? If item 1 can be interpreted similar to the ASME Code exception process, we can agree. Tech Spec changes should not be necessary.
- Sec 3.2, applicability to mill annealed alloy 600 The applicability should be extended to alloy 690 and thermally treated alloy 600. There is no difference in thermal expansion coefficient between mill annealed and thermally treated alloy 600. For 690 TT, there are no dissimilar materials.
- Report should refer to ABB C-E Nuclear Power, Inc. or ABB CENP.
- In #2 of the Conclusion, the NRC Staff raises the issue of axial forces due to differential thermal expansion due to locked tubes at the tube support plates. The Conclusion seems to be an odd place to raise this issue for the first time. This should be discussed in section 3.3.
- In #7 of the Conclusion, the NRC states "Prior to implementing the subject sleeve repair method, the licensee shall update its design basis documents to identify the approved sleeving method and to place conditions/clarifications on its use in accordance with this safety evaluation." We agree that we do need a TRM change. Do we need anything else? If not, what is the intent of this statement?
- We presume that a utility with an existing SER for TIG welded sleeves would be able to adopt the generic SER with only a 50.59 evaluation.