Indiana Michigan
Power Company
Cook Nuclear Plant
One Cook Place
Bridgman, MI 49106
616-465-5901

March 13, 2000 C0300-02

Docket Nos.: 50-315
50-316

U.S. Nuclear Regulatory Commission
ATTN: Document Control Desk
Mail Stop O-P1-17
Washington, DC 20555-0001

Donald C. Cook Nuclear Plant Units 1 and 2
1999 ANNUAL OPERATING REPORT

Technical Specification 6.9.1.5 of the Donald C. Cook Nuclear Plant requires that an annual report be submitted to address personnel exposure, steam generator in-service inspection results, challenges to power-operated relief and safety valves, and information regarding any instances when the I-131 specific activity limit was exceeded. Consistent with these requirements, a copy of the 1999 annual operating report is attached (Attachment 1).

The NRC staff has been notified that this transmittal was delayed due to an administrative issue regarding over reporting of RG 1.16 personnel exposure data. These conditions have been entered into our corrective action program.

Should you have any questions, please contact Robert C. Godley, Director of Regulatory Affairs, at (616) 466-2698.

Sincerely,

A. C. Bakken III
Site Vice President

/dms

Attachment

A001

c: J. E. Dyer
MDEQ - DW & RPD, w/o attachment
NRC Resident Inspector
R. Whale, w/o attachment

ATTACHMENT 1 TO C0300-02

1999 Annual Operating Report

1.0 INTRODUCTION

Plant Description

Indiana Michigan Power Company is the licensee of the Donald C. Cook Nuclear Plant. The plant is located north of Bridgman, Michigan. The plant consists of two nuclear units, each employing a Westinghouse pressurized water reactor nuclear steam supply system. Each reactor unit employs an ice condenser reactor containment system. The American Electric Power Service Corporation was the architect-engineer and constructor.

Units 1 and 2 reactor licensed power levels are 3250 Mwt and 3411 Mwt, respectively. The main condenser cooling method is open cycle using Lake Michigan water as the cooling source for each unit.

Both units remained shutdown the entire 1999 calendar year to resolve design basis concerns identified during a NRC Architect and Engineering inspection.

2.0 PERSONNEL RADIATION EXPOSURE SUMMARY

Page 2 of this attachment provides a summary of the number of station, utility, and contractor/other personnel receiving exposures greater than 100 millirem (mr) in 1999. This estimated dose is based on electronic dosimetry and reported in the format specified by Regulatory Guide 1.16.

The values shown in the individual categories (routine maintenance, etc.) represent the number of people who received greater than 100 mr in that particular category. The grand total figure represents the total number of people who received 100 mr, whether in one of the categories or multiple categories. A specific person could receive dose in two or more categories and still would be counted in the grand total. The summation of the totals in the individual categories would not necessarily equal the grand total.

Reg. Guide 1.16 Report
INDIANA MICHIGAN POWER / COOK NUCLEAR PLANT
Prepared for Year 1999

Number of Personnel and Person-Rem by Work and Job Function

	Number of Personnel > 100 mrem			Total Person-Rem		
	Station Employees	Utility Employees	Contractors and Others	Station Employees	Utility Employees	Contractors and Others
Reactor Operation & Surveillance						
-Maintenance	0	0	0	0.000	0.000	0.000
-Operations	9	0	0	1.237	0.000	0.000
-Health Physics	0	0	0	0.000	0.000	0.000
-Supervisory	0	0	0	0.000	0.000	0.000
-Engineering	0	0	0	0.000	0.000	0.000
Routine Maintenance						
-Maintenance	14	2	233	2.572	0.263	65.695
-Operations	0	0	1	0.000	0.000	0.247
-Health Physics	12	0	23	2.705	0.000	3.774
-Supervisory	0	0	0	0.000	0.000	0.000
-Engineering	0	0	8	0.000	0.000	1.197
Inservice Inspection						
-Maintenance	0	0	1	0.000	0.000	0.120
-Operations	0	0	0	0.000	0.000	0.000
-Health Physics	0	0	0	0.000	0.000	0.000
-Supervisory	0	0	0	0.000	0.000	0.000
-Engineering	0	0	0	0.000	0.000	0.000
Special Maintenance						
-Maintenance	0	0	10	0.000	0.000	1.452
-Operations	0	0	0	0.000	0.000	0.000
-Health Physics	0	0	0	0.000	0.000	0.000
-Supervisory	0	0	0	0.000	0.000	0.000
-Engineering	0	0	0	0.000	0.000	0.000
Waste Processing						
-Maintenance	0	0	24	0.000	0.000	11.066
-Operations	0	0	0	0.000	0.000	0.000
-Health Physics	4	0	2	0.780	0.000	0.439
-Supervisory	0	0	0	0.000	0.000	0.000
-Engineering	0	0	4	0.000	0.000	0.811
Refueling						
-Maintenance	0	0	7	0.000	0.000	1.086
-Operations	2	0	12	0.233	0.000	1.876
-Health Physics	3	0	2	0.440	0.000	0.293
-Supervisory	0	0	0	0.000	0.000	0.000
-Engineering	0	0	0	0.000	0.000	0.000
Totals						
-Maintenance	14	2	275	2.572	0.263	79.419
-Operations	11	0	13	1.470	0.000	2.123
-Health Physics	19	0	27	3.925	0.000	4.506
-Supervisory	0	0	0	0.000	0.000	0.000
-Engineering	0	0	12	0.000	0.000	2.008
Grand Totals	44	2	327	7.967	0.263	88.056

3.0 STEAM GENERATOR INSPECTIONS

During 1999, there were no Steam Generator tube in-service inspections performed on either Unit 1 or Unit 2.

4.0 CHALLENGES TO PRESSURIZER POWER OPERATED RELIEF VALVES (PORVs) AND SAFETY VALVES

During 1999, there were no challenges on either Unit 1 or Unit 2 to the pressurizer PORVs, or the pressurizer safety valves.

5.0 REACTOR COOLANT SPECIFIC ACTIVITY

During 1999, there were no instances on either Unit 1 or Unit 2 in which the reactor coolant dose equivalent I-131 specific activity exceeded the limits of Technical Specification 3.4.8 (greater than or equal to 1 μCi/g). Compliance was determined by routine gamma spectrometry analysis of reactor coolant.