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A Duke Energy Company

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March 8, 2000

U. S. Nuclear Regulatory Commission
Attention: Document Control Desk
Washington, DC 20555

Subject: Oconee Nuclear Station
Docket 50-269, -270, -287
Selected Licensee Commitments Manual (SLC)

Gentlemen:

Pursuant to 10CFR 50.4 and 50.71, please find attached 7 copies of the latest revisions to the Oconee Selected Licensee Commitments Manual (SLC). The SLC Manual is Chapter 16.0 of the Oconee Updated Final Safety Analysis Report (UFSAR). This manual is intended to contain commitments and other station issues that warrant higher control, but are not appropriate for inclusion into the Technical Specifications (TS). Instead of being updated with the annual UFSAR Update, the SLC Manual will be updated as necessary throughout the year.

Very truly yours,

W. R. McCollum, Jr.
Vice President
Oconee Nuclear Station

CMB/cmb
Attachment

xc: Luis A. Reyes
Regional Administrator, Region II

D. E. LaBarge, ONRR

M. C. Shannon
Oconee Senior Resident Inspector

A001

March 8, 2000

To: Manual Holders

Subject: Oconee Selected Licensee Commitments Manual (SLC) Revision

On February 10, 2000, Station Management approved changes to SLC 16.6.2, Containment Tendon Surveillance Program. This change was implemented on 2/17/00. The change revises the program to conform to ASME XI, IWL as required by 10 CFR 50.55a. This SLC is part of Tech Spec Amendment 310.

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Any questions concerning this revision may be directed to Bob Douglas at 864-885-3073.

Regulatory Compliance
By: Conice Breazeale
Regulatory Compliance

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Selected Licensee Commitments
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2/17/00

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SURVEILLANCE REQUIREMENTS

SURVEILLANCE		FREQUENCY
SR 16.6.2.1	----------NOTE---------- This SR may be conducted during MODE 1 provided design conditions regarding loss of adjacent tendons are satisfied at all times. ---------- Perform inservice examinations of concrete containment post-tensioning systems in accordance with Section XI, Subsection IWL of the ASME Boiler and Pressure Vessel Code and applicable addenda as required by 10 CFR 50.55a, as amended by relief granted in accordance with 10 CFR 50.55a(a)(3).	As specified by IWL and 10 CFR 50.55a, as amended by relief granted in accordance with 10 CFR 50.55a(a)(3).

Figure 16.6.2-1
Dome Tendon PLLs and MRVs

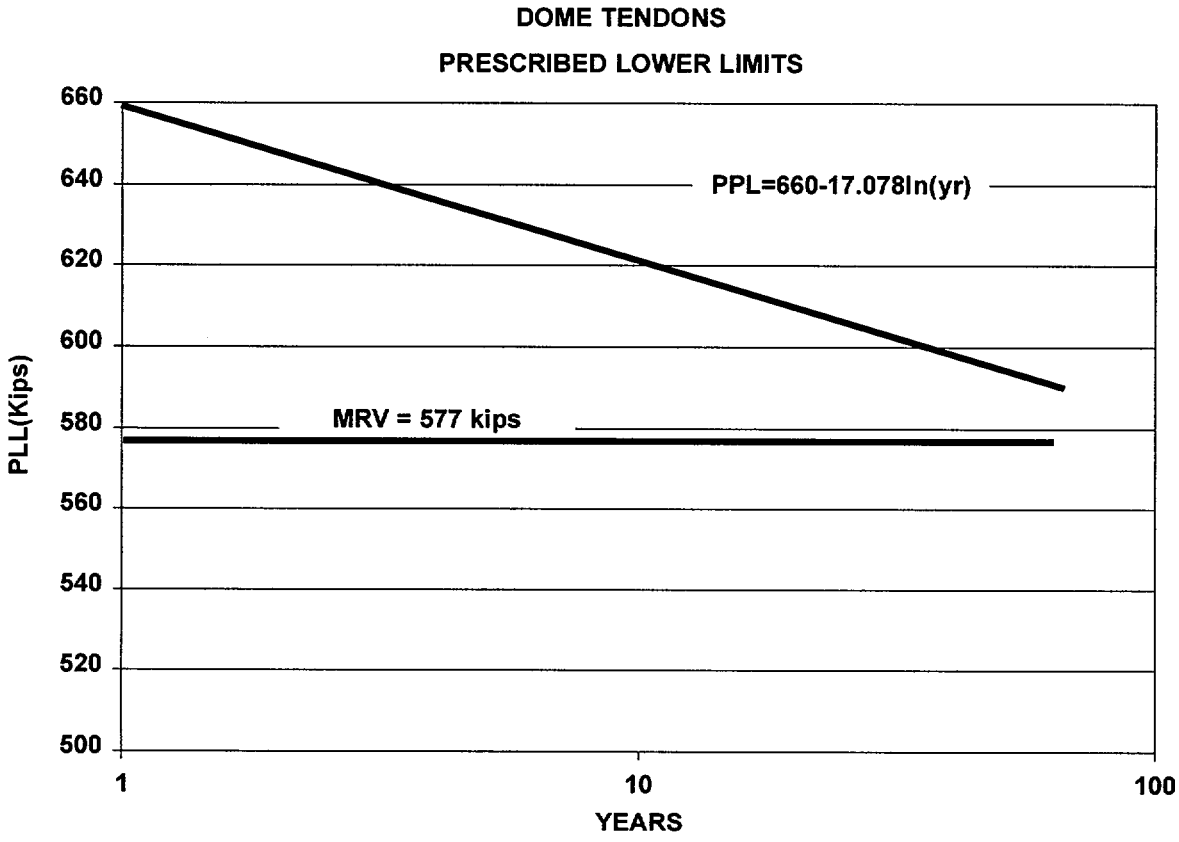


Figure 16.6.2-2
Hoop Tendon PLLs and MRVs

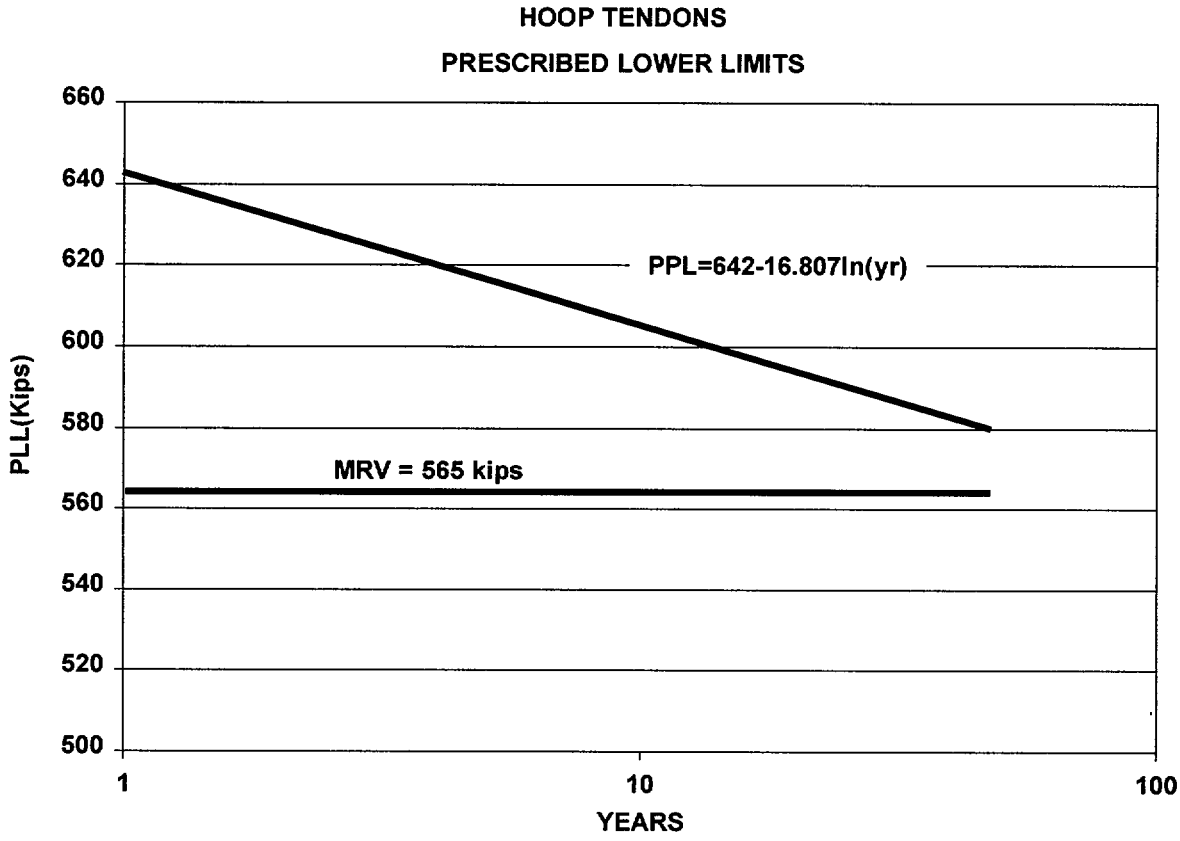
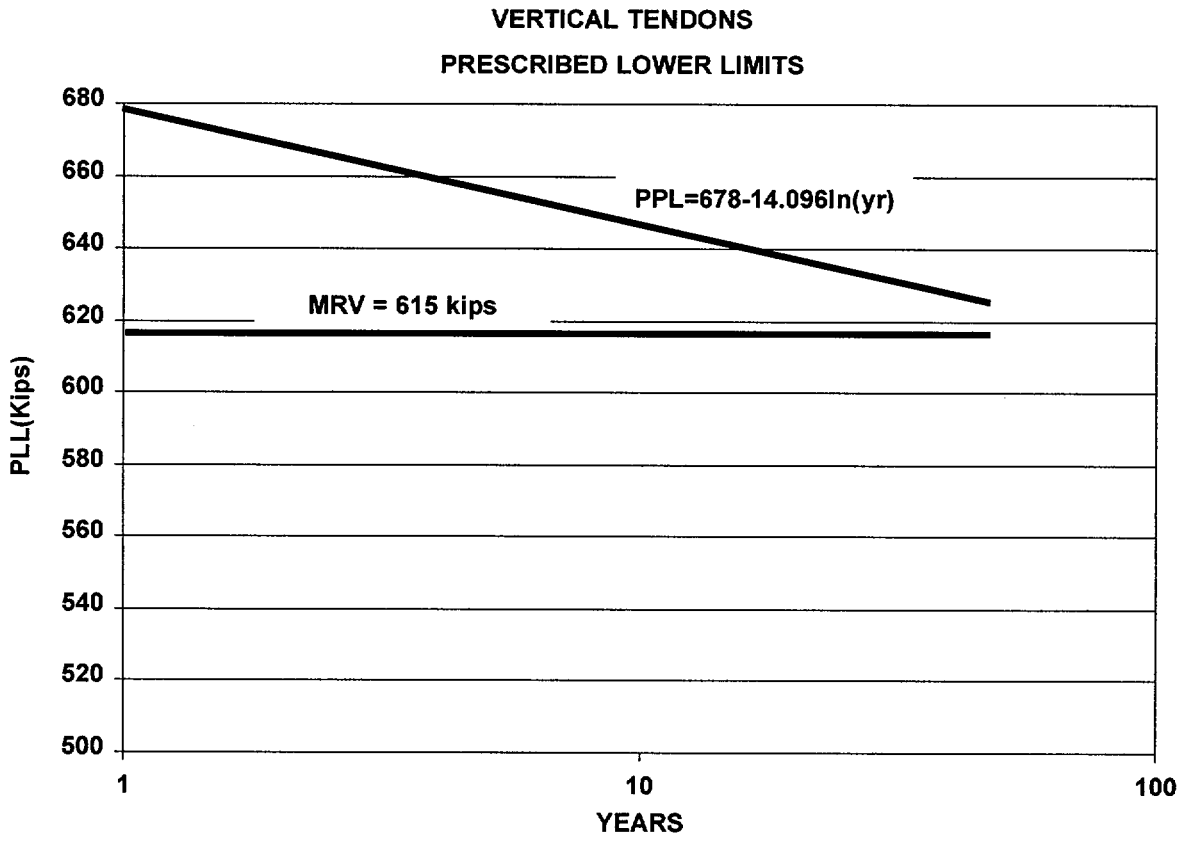


Figure 16.6.2-3
Vertical Tendon PLLs and MRVs



BASES

The NRC amended 10 CFR 50.55a on August 8, 1996 to include examination of Class MC and CC components. This rule became effective September 9, 1996, and incorporates by reference Subsections IWE and IWL of the ASME Code, Section XI. Subsections IWE and IWL provide requirements for inservice inspection, repair/replacement, and testing of both steel and concrete containments, including post-tensioning systems. Oconee has implemented a Containment Inservice Inspection Plan (Reference 1) to comply with the requirements of 10 CFR 50.55a. Prior to the implementation of the Containment Inservice Inspection Plan, containment structural integrity was verified in accordance with Regulatory Guide 1.35, Inservice Inspection of Ungrouted Tendons in Prestressed Concrete Containments, and the details were included in this SLC. Because the Oconee Containment Inservice Inspection Plan now contains the necessary details for inservice inspection of the containment, the details previously included in this SLC have been removed. The required actions and completion times upon detection of conditions of abnormal degradation have not been changed. Additionally, the Minimum Required Values (MRVs) and Prescribed Lower Limits (PLLs) for the dome, hoop, and vertical tendons have not been revised, and will continue to be maintained in this SLC. The surveillance requirements have been condensed to the following:

SR 16.6.2.1

This SR performs inservice examinations of post-tensioning systems of concrete containments in accordance with Section XI, Subsection IWL of the ASME Boiler and Pressure Vessel Code and applicable addenda as required by 10 CFR 50, Section 55a. The details of this SR are outlined in the Oconee Containment Inservice Inspection Plan.

Any condition which fails to meet the acceptance standards of IWL-3000 shall be considered as an indication of abnormal degradation of the reactor building. Additionally, any unacceptable condition listed in 10 CFR 50.55a(b)(2)(ix) shall also be considered as an indication of abnormal degradation of the reactor building.

References

1. Duke Document No. O-62-CISI-0001, "First Interval Containment Inservice Inspection Plan, Oconee Nuclear Station Units 1, 2, and 3."