Template NRR-058

Mr. James A. Hutton
Director-Licensing, MC 62A-1
PECO Energy Company
Nuclear Group Headquarters
Correspondence Control Desk
P.O. Box No. 195
Wayne, PA 19087-0195

March 10, 2000

SUBJECT: LIMERICK GENERATING STATION, UNIT 1, CORRECTION OF AMENDMENT NO. 137 REGARDING CHANGES TO THE ALLOWABLE AS-FOUND MAIN STEAM SAFETY-RELIEF VALVE CODE SAFETY FUNCTION LIFT SETPOINT TOLERANCE (TAC NO. MA8160)

Dear Mr. Hutton:

On November 10, 1999, the U.S. Nuclear Regulatory Commission issued Amendment No. 137 to Facility Operating License No. NPF-39 for Limerick Generating Station, Unit 1 (LGS Unit 1), in response to your application dated January 12, 1999, as supplemented January 29, March 10, and September 20, 1999. In the above cited amendment, we inadvertently issued LGS Unit 2 Technical Specifications (TSs) Bases page B 3/4 4-2 instead of LGS Unit 1 TS Bases page B 3/4 4-2. The correct TS Bases page B 3/4 4-2 is enclosed.

We regret any inconvenience this may have caused. If you have any questions regarding this matter, please call me at 301-415-1483.

Sincerely,

/RA/

Bartholomew C. Buckley, Sr. Project Manager, Section 2
Project Directorate I
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket No. 50-352

Enclosure:

cc w/encl: See next page

DISTRIBUTION:

File Center	MO'Brien	WBeckner	Elmbro
PUBLIC	BBuckley	PDI-2 r/f	GHammer
OGC	ACRS	GThomas	MFerdas
GHill (2)	JClifford	JHannon	EAdensam
JWermiel			

ACRS FILE CENTER COPY

DOCUMENT NAME: G:\PDI-2\Limerick\LI18160.LTR

To receive a copy of this document, indicate in the box: "**C**" = Copy without attachment/enclosure "**E**" = Copy with attachment/enclosure "**N**" = No copy

OFFICE	PM/PDI-2 BCB	E	LA/PDI-2		OGC		SC/PDI-2	
NAME	BBuckley/vw		MO'Brien				JClifford	
DATE	3 / 9 /00		3/9 /00		/ /00		3 / 10 /00	

Official Record Copy

DF01

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

March 10, 2000

Mr. James A. Hutton
Director-Licensing, MC 62A-1
PECO Energy Company
Nuclear Group Headquarters
Correspondence Control Desk
P.O. Box No. 195
Wayne, PA 19087-0195

SUBJECT: LIMERICK GENERATING STATION, UNIT 1, CORRECTION OF AMENDMENT NO. 137 REGARDING CHANGES TO THE ALLOWABLE AS-FOUND MAIN STEAM SAFETY-RELIEF VALVE CODE SAFETY FUNCTION LIFT SETPOINT TOLERANCE (TAC NO. MA8160)

Dear Mr. Hutton:

On November 10, 1999, the U.S. Nuclear Regulatory Commission issued Amendment No. 137 to Facility Operating License No. NPF-39 for Limerick Generating Station, Unit 1 (LGS Unit 1), in response to your application dated January 12, 1999, as supplemented January 29, March 10, and September 20, 1999. In the above cited amendment, we inadvertently issued LGS Unit 2 Technical Specifications (TSs) Bases page B 3/4 4-2 instead of LGS Unit 1 TS Bases page B 3/4 4-2. The correct TS Bases page B 3/4 4-2 is enclosed.

We regret any inconvenience this may have caused. If you have any questions regarding this matter, please call me at 301-415-1483.

Sincerely,

Bartholomew C. Buckley

Bartholomew C. Buckley, Sr. Project Manager, Section 2
Project Directorate I
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket No. 50-352

Enclosure: TS Bases page B 3/4 4-2, LGS Unit 1

cc w/encl: See next page

Limerick Generating Station, Unit 1

cc:

J. W. Durham, Sr., Esquire
Sr. V.P. & General Counsel
PECO Energy Company
2301 Market Street
Philadelphia, PA 19101

Manager-Limerick Licensing, 62A-1
PECO Energy Company
965 Chesterbrook Boulevard
Wayne, PA 19087-5691

Mr. James D. von Suskil, Vice President
Limerick Generating Station
Post Office Box 2300
Sanatoga, PA 19464

Plant Manager
Limerick Generating Station
P.O. Box 2300
Sanatoga, PA 19464

Regional Administrator, Region I
U.S. Nuclear Regulatory Commission
475 Allendale Road
King of Prussia, PA 19406

Senior Resident Inspector
U.S. Nuclear Regulatory Commission
Limerick Generating Station
P.O. Box 596
Pottstown, PA 19464

Chairman
Board of Supervisors
of Limerick Township
646 West Ridge Pike
Linfield, PA 19468

Chief-Division of Nuclear Safety
PA Dept. of Environmental Resources
P.O. Box 8469
Harrisburg, PA 17105-8469

Director-Site Engineering
Limerick Generating Station
P.O. Box 2300
Sanatoga, PA 19464

Manager-Experience Assessment
Limerick Generating Station
P.O. Box 2300
Sanatoga, PA 19464

Library
U.S. Nuclear Regulatory Commission
Region I
475 Allendale Road
King of Prussia, PA 19406

Senior Manager-Operations
Limerick Generating Station
P.O. Box 2300
Sanatoga, PA 19464

Dr. Judith Johnsrud
National Energy Committee
Sierra Club
433 Orlando Avenue
State College, PA 16803

RECIRCULATION SYSTEM (Continued)

Plant specific calculations can be performed to determine an applicable region for monitoring neutron flux noise levels. In this case the degree of conservatism can be reduced since plant to plant variability would be eliminated. In this case, adequate margin will be assured by monitoring the region which has a decay ratio greater than or equal to 0.8.

Neutron flux noise limits are also established to ensure early detection of limit cycle neutron flux oscillations. BWR cores typically operate with neutron flux noise caused by random boiling and flow noise. Typical neutron flux noise levels of 1-12% of rated power (peak-to-peak) have been reported for the range of low to high recirculation loop flow during both single and dual recirculation loop operation. Neutron flux noise levels which significantly bound these values are considered in the thermal/mechanical design of GE BWR fuel and are found to be of negligible consequence. In addition, stability tests at operating BWRs have demonstrated that when stability related neutron flux limit cycle oscillations occur they result in peak-to-peak neutron flux limit cycles of 5-10 times the typical values. Therefore, actions taken to reduce neutron flux noise levels exceeding three (3) times the typical value are sufficient to ensure early detection of limit cycle neutron flux oscillations.

Typically, neutron flux noise levels show a gradual increase in absolute magnitude as core flow is increased (constant control rod pattern) with two reactor recirculation loops in operation. Therefore, the baseline neutron flux noise level obtained at a specific core flow can be applied over a range of core flows. To maintain a reasonable variation between the low flow and high flow end of the flow range, the range over which a specific baseline is applied should not exceed 20% of rated core flow with two recirculation loops in operation. Data from tests and operating plants indicate that a range of 20% of rated core flow will result in approximately a 50% increase in neutron flux noise level during operation with two recirculation loops. Baseline data should be taken near the maximum rod line at which the majority of operation will occur. However, baseline data taken at lower rod lines (i.e. lower power) will result in a conservative value since the neutron flux noise level is proportional to the power level at a given core flow.

3/4.4.2 SAFETY/RELIEF VALVES

The safety valve function of the safety/relief valves operates to prevent the reactor coolant system from being pressurized above the Safety Limit of 1325 psig in accordance with the ASME Code. A total of 12 OPERABLE safety/relief valves is required to limit reactor pressure to within ASME III allowable values for the worst case upset transient.

Demonstration of the safety/relief valve lift settings will occur only during shutdown. The safety/relief valves will be removed and either set pressure tested or replaced with spares which have been previously set pressure tested and stored in accordance with manufacturers recommendations in the specified frequency.