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OCONEE NUCLEAR STATION
TECHNICAL SPECIFICATIONS
RESP TEAM: 561

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*****AMENDMENT NUMBERS 309, 309, 309*****

W R MCCOLLUM
VICE PRESIDENT
OCONEE NUCLEAR SITE

BY:
C M BREAZEALE CMB/AB ON03RC

ADD 1

February 21, 2000

RE: Oconee Nuclear Station
Technical Specifications

On January 18, 2000 the NRC issued Amendment No. 309/309/309 to Oconee Nuclear Station Facility Operating License DPR-38, DPR-47, and DPR-55. The amendments revise the Technical Specification to incorporate Topical Report DPC-NE-3005-P "Thermal Hydraulic Transient Analysis Methodology" and requirements for the Atmospheric Dump Valves.

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If you have any questions or problems, I may be contacted
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L. E. Nicholson
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By: Conice Breazeale

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SURVEILLANCE REQUIREMENTS

SURVEILLANCE		FREQUENCY
SR 3.4.1.1	<p>-----NOTE----- With three RCPs operating, the limits are applied to the loop with the highest pressure.</p> <p>-----</p> <p>Verify RCS loop pressure is within limits specified in the COLR.</p>	12 hours
SR 3.4.1.2	<p>-----NOTE----- With three RCPs operating, the limits are applied to the loop with the lowest loop average temperature for the condition where there is a 0°F ΔTc setpoint.</p> <p>-----</p> <p>Verify RCS loop average temperature is within limits specified in the COLR.</p>	12 hours
SR 3.4.1.3	Verify RCS total flow is within limits specified in the COLR.	12 hours
SR 3.4.1.4	<p>-----NOTE----- Not required to be performed until 7 days after stable thermal conditions are established in the higher power range of MODE 1.</p> <p>-----</p> <p>Verify by measurement RCS total flow rate is within limit specified in the COLR.</p>	18 months

3.4 REACTOR COOLANT SYSTEM (RCS)

3.4.10 Pressurizer Safety Valves

LCO 3.4.10 Two pressurizer safety valves shall be OPERABLE with lift settings ≥ 2425 psig and ≤ 2575 psig.

APPLICABILITY: MODES 1 and 2,
MODE 3 with all RCS cold leg temperatures $> 325^\circ\text{F}$.

-----NOTE-----

The lift settings are not required to be within the LCO limits for entry into the applicable portions of MODE 3 for the purpose of setting the pressurizer safety valves under ambient (hot) conditions. This exception is allowed for 36 hours following entry into the applicable portions of MODE 3 provided a preliminary cold setting was made prior to heatup.

ACTIONS

CONDITION	REQUIRED ACTION	COMPLETION TIME
A. One pressurizer safety valve inoperable.	A.1 Restore valve to OPERABLE status.	15 minutes
B. Required Action and associated Completion Time not met.	B.1 Be in MODE 3.	12 hours
<u>OR</u>	<u>AND</u>	
Two pressurizer safety valves inoperable.	B.2 Be in MODE 3 with any RCS cold leg temperature $\leq 325^\circ\text{F}$.	18 hours

SURVEILLANCE REQUIREMENTS

SURVEILLANCE		FREQUENCY
SR 3.4.10.1	Verify each pressurizer safety valve is OPERABLE in accordance with the Inservice Testing Program. Following testing, lift settings shall be within $\pm 1\%$.	In accordance with the Inservice Testing Program

3.7 PLANT SYSTEMS

3.7.4 Atmospheric Dump Valve (ADV) Flow Paths

LCO 3.7.4 The ADV flow path for each steam generator shall be OPERABLE.

APPLICABILITY: MODES 1, 2, and 3, and MODE 4 when steam generator is relied upon for heat removal.

ACTIONS

CONDITION	REQUIRED ACTION	COMPLETION TIME
A. One or both ADV flow path(s) inoperable.	A.1 Be in MODE 3.	12 hours
	<u>AND</u> A.2 Be in MODE 4 without reliance upon steam generator for heat removal.	24 hours

SURVEILLANCE REQUIREMENTS

SURVEILLANCE	FREQUENCY
SR 3.7.4.1 Cycle the valves that comprise the ADV flow paths.	18 months

3.9 REFUELING OPERATIONS

3.9.7 Unborated Water Source Isolation Valves

LCO 3.9.7 Each valve used to isolate unborated water sources shall be secured in the closed position.

APPLICABILITY: MODE 6.

ACTIONS

-----NOTE-----
Separate Condition entry is allowed for each unborated water source isolation valve.

CONDITION	REQUIRED ACTION	COMPLETION TIME
<p>A. -----NOTE----- Required Action A.3 must be completed whenever Condition A is entered. ----- One or more valves not secured in closed position.</p>	<p>A.1 Suspend CORE ALTERATIONS.</p>	<p>Immediately</p>
	<p><u>AND</u></p>	
	<p>A.2 Initiate actions to secure valve in closed position.</p>	<p>Immediately</p>
	<p><u>AND</u></p>	
	<p>A.3 Perform SR 3.9.1.1.</p>	<p>4 hours</p>

SURVEILLANCE REQUIREMENTS

SURVEILLANCE		FREQUENCY
SR 3.9.7.1	Verify each valve that isolates unborated water sources is secured in the closed position.	31 days

5.6 Reporting Requirements

5.6.5 CORE OPERATING LIMITS REPORT (COLR) (continued)

6. Nuclear Overpower Flux/Flow/Imbalance and RCS Variable Low Pressure allowable value limits for Specification 3.3.1;
 7. RCS Pressure, Temperature, and Flow Departure from Nucleate Boiling (DNB) Limits for Specification 3.4.1
 8. Core Flood Tanks Boron concentration limits for Specification 3.5.1;
 9. Borated Water Storage Tank Boron concentration limits for Specification 3.5.4;
 10. Spent Fuel Pool Boron concentration limits for Specification 3.7.12;
 11. RCS and Transfer Canal boron concentration limits for Specification 3.9.1; and
 12. AXIAL POWER IMBALANCE protective limits and RCS Variable Low Pressure protective limits for Specification 2.1.1.
- b. The analytical methods used to determine the core operating limits shall be those previously reviewed and approved by the NRC, specifically those described in the following documents:
- (1) DPC-NE-1002A, Reload Design Methodology II, Rev. 1, (SER dated October 1, 1985);
 - (2) NFS-1001A, Reload Design Methodology, Rev. 4, (SER dated July 29, 1981);
 - (3) DPC-NE-2003P-A, Oconee Nuclear Station Core Thermal Hydraulic Methodology Using VIPRE-01, (SER dated July 19, 1989);
 - (4) DPC-NE-1004P-A, Nuclear Design Methodology Using CASMO-3/SIMULATE-3P, (SER dated November 23, 1992);
 - (5) DPC-NE-2008P-A, Fuel Mechanical Reload Analysis Methodology Using TACO3, (SER dated April 3, 1995);
 - (6) BAW-10192-PA, BWNT LOCA - BWNT Loss of Coolant Accident Evaluation Model for Once-Through Steam Generator Plants, (SER dated February 18, 1997);

5.6 Reporting Requirements

5.6.5 CORE OPERATING LIMITS REPORT (COLR) (continued)

- (7) DPC-NE-3000P-A, Thermal Hydraulic Transient Analysis Methodology, Rev. 2, (SER dated October 14, 1998);
 - (8) DPC-NE-2005P-A, Thermal Hydraulic Statistical Core Design Methodology, Rev. 1, (SER dated November 7, 1996); and
 - (9) DPC-NE-3005-PA, UFSAR Chapter 15 Transient Analysis Methodology, Rev. 1, (SER dated May 25, 1999).
- c. The core operating limits shall be determined such that all applicable limits (e.g., fuel thermal mechanical limits, core thermal hydraulic limits, Emergency Core Cooling System (ECCS) limits, nuclear limits such as SDM, transient analysis limits, and accident analysis limits) of the safety analysis are met.
- d. The COLR, including any midcycle revisions or supplements, shall be provided upon issuance for each reload cycle to the NRC.

5.6.6 Post Accident Monitoring (PAM) and Main Feeder Bus Monitor Panel (MFPMP) Report

When a report is required by Condition B or G of LCO 3.3.8, "Post Accident Monitoring (PAM) Instrumentation" or Condition D of LCO 3.3.23, "Main Feeder Bus Monitor Panel," a report shall be submitted within the following 14 days. The report shall outline the preplanned alternate method of monitoring (PAM only), the cause of the inoperability, and the plans and schedule for restoring the instrumentation channels of the Function to OPERABLE status.

5.6.7 Tendon Surveillance Report

Any abnormal degradation of the containment structure detected during the tests required by the Pre-stressed Concrete Containment Tendon Surveillance Program shall be reported to the NRC within 30 days. The report shall include a description of the tendon condition, the condition of the concrete (especially at tendon anchorages), the inspection procedures, the tolerances on cracking, and the corrective action taken.

5.6 Reporting Requirements (continued)

5.6.8 Steam Generator Tube Inspection Report

The steam generator tube inspection report shall comply with the following:

- a. The number of tubes plugged or repaired in each steam generator shall be reported to the NRC within 30 days following the completion of the plugging or repair procedure.
 - b. The results of the steam generator tube inservice inspection shall be reported to the NRC within 3 months following completion of the inspection. This report shall include:
 1. Number and extent of tubes inspected.
 2. Location and percent of wall-thickness penetration for each indication of a degraded tube.
 3. Identification of tubes plugged or repaired.
 4. Number of tubes repaired by rerolling and number of indications detected in the new roll area of the repaired tubes.
 - c. Results of steam generator tube inspections which fall into Category C-3 and require notification to the NRC shall be reported prior to resumption of plant operation. The written report shall provide the results of investigations conducted to determine cause of the tube degradation and corrective measures taken to prevent recurrence.
 - d. The designation of affected and unaffected areas will be reported to the NRC when they are determined.
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