

Mr. A. Alan Blind
 Vice President, Nuclear Power
 Consolidated Edison Company
 of New York, Inc.
 Broadway and Bleakley Avenue
 Buchanan, NY 10511

February 29, 2000

Template NRR-056

SUBJECT: INDIAN POINT NUCLEAR GENERATING UNIT NO. 2 RE: ISSUANCE OF
 AMENDMENT TO MODIFY OPERATIONS MANAGEMENT REQUIREMENTS
 AND CORRECT EDITORIAL ERRORS (TAC NO. MA4654)

Dear Mr. Blind:

The Commission has issued the enclosed Amendment No. 2070 to Facility Operating License No. DPR-26 for the Indian Point Nuclear Generating Unit No. 2. The amendment consists of changes to the Technical Specifications (TSs) in response to your application transmitted by letter dated January 22, 1999.

The amendment revises TS 3.7.D.1 to correct an editorial error, TS 6.2.2 to change the senior reactor operator license requirement for the Operations Manager, and TS 6.3.1 to modify the qualification requirement for the Operations Manager.

A copy of the related Safety Evaluation is enclosed. A Notice of Issuance will be included in the Commission's next regular biweekly Federal Register notice.

Sincerely,

/RA/

Jefferey F. Harold, Project Manager, Section 1
 Project Directorate I
 Division of Licensing Project Management
 Office of Nuclear Reactor Regulation

Docket No. 50-247

Enclosures: 1. Amendment No. 2070 to DPR-26
 2. Safety Evaluation

cc w/encls: See next page

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UNITED STATES
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

February 29, 2000

Mr. A. Alan Blind
Vice President, Nuclear Power
Consolidated Edison Company
of New York, Inc.
Broadway and Bleakley Avenue
Buchanan, NY 10511

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AMENDMENT TO MODIFY OPERATIONS MANAGEMENT REQUIREMENTS
AND CORRECT EDITORIAL ERRORS (TAC NO. MA4654)

Dear Mr. Blind:

The Commission has issued the enclosed Amendment No. 207 to Facility Operating License No. DPR-26 for the Indian Point Nuclear Generating Unit No. 2. The amendment consists of changes to the Technical Specifications (TSs) in response to your application transmitted by letter dated January 22, 1999.

The amendment revises TS 3.7.D.1 to correct an editorial error, TS 6.2.2 to change the senior reactor operator license requirement for the Operations Manager, and TS 6.3.1 to modify the qualification requirement for the Operations Manager.

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Jefferey F. Harold, Project Manager, Section 1
Project Directorate I
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Office of Nuclear Reactor Regulation

Docket No. 50-247

Enclosures: 1. Amendment No. 207 to DPR-26
2. Safety Evaluation

cc w/encls: See next page

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Units 1/2

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

CONSOLIDATED EDISON COMPANY OF NEW YORK, INC.

DOCKET NO. 50-247

INDIAN POINT NUCLEAR GENERATING UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 207
License No. DPR-26

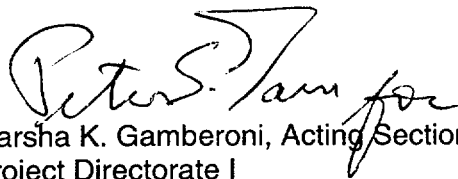
1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Consolidated Edison Company of New York, Inc. (the licensee) dated January 22, 1999, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-26 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 207 , are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance to be implemented within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION



Marsha K. Gamberoni, Acting Section Chief, Section 1
Project Directorate I
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: February 29, 2000

ATTACHMENT TO LICENSE AMENDMENT NO. 207

FACILITY OPERATING LICENSE NO. DPR-26

DOCKET NO. 50-247

Replace the following pages of the Appendix A Technical Specifications with the attached revised pages. The revised pages are identified by amendment number and contain marginal lines indicating the areas of change.

Remove Pages

3.7-4

6-2

6-3

Insert Pages

3.7-4

6-2

6-3

3. If the requirements of 3.7.C.1 or 3.7.C.2 cannot be met, then, within the next seven (7) days, either the inoperable condition shall be corrected or an alternate independent power system shall be established.
4. If the requirements of 3.7.C.3 cannot be satisfied, the reactor shall be placed in the hot shutdown condition utilizing normal operating procedures. If the requirements of 3.7.C.3 cannot be met within an additional 48 hours, the reactor shall be placed in the cold shutdown condition utilizing normal operating procedures.

The requirements of Specification 3.7.A may be modified for an emergency "Black Start" of the unit by using the requirements of either Specification 3.7.D.1 or 3.7.D.2 below:

- D.1.
 - a. all 138 kV lines to Buchanan de-energized,
 - b. the 13.8 kV line de-energized,
 - c. the 6.9 kV buses 5 and 6 energized from the onsite gas turbine through the 13.8/6.9 kV transformer,
 - d. the four 480-volt buses 2A, 3A, 5A and 6A energized from the diesels and the tie breakers between buses 5A and 2A and between buses 3A and 6A open,
 - e. three diesel generators operable with a minimum onsite supply of 6,334 gallons of fuel available in each of the individual storage tanks and 29,000 gallons of fuel available at the Buchanan Substation, or onsite other than the normal supply tanks,
 - f. station batteries Nos. 21, 22, 23 & 24 and their associated battery chargers and dc distribution systems operable, and
 - g. the 480-volt supply breakers 52/2A, 52/3A, 52/5A and 52/6A open.
- D.2.
 - a. establish 138 kV bus sections at Buchanan with at least 37 MW power (nameplate rating) from any combination of gas turbines at Buchanan and onsite,

6.2.2 Facility Staff

- a. Each on duty shift shall be composed of at least the minimum shift crew composition shown in Table 6.2-1.
- b. At least one licensed Operator shall be in the control room when fuel is in the reactor.
- c. At least two licensed Operators shall be present in the control room during reactor startup, scheduled reactor shutdown, and during recovery from reactor trips.
- d. An individual qualified in radiation protection procedures shall be onsite when fuel is in the reactor.
- e. All core alterations after the initial fuel loading shall be directly supervised by either a licensed Senior Reactor Operator or Senior Reactor Operator Limited to Fuel Handling. This individual shall have no other concurrent responsibilities during this operation.
- f. DELETED
- g. Administrative procedures shall be developed and implemented to limit the working hours of unit staff who perform safety-related functions (e.g., licensed Senior Operators, licensed Operators, health physicists, auxiliary operators, and key maintenance personnel).

The amount of overtime worked by unit staff members performing safety-related functions shall be limited in accordance with the NRC Policy Statement on working hours (Generic Letter No. 82-12).

- h. The Operations Manager or Assistant Operations Manager shall hold a senior reactor operator license.

6.3 FACILITY STAFF QUALIFICATIONS

6.3.1 Each member of the facility staff shall meet or exceed the minimum qualifications of ANSI N18.1-1971 for comparable positions, except for (1) the Operation Manager's and the Assistant Operations Manager's SRO license requirement which shall be in accordance with Technical Specification 6.2.2.h, and, (2) the Radiation Protection Manager who shall meet or exceed the minimum qualifications of Regulatory Guide 1.8, September 1975.

6.3.2 The Plant Manager shall meet or exceed the minimum qualifications specified for Plant Manager in ANSI N18.1-1971.

6.3.3 The Watch Engineer shall have a bachelor's degree or equivalent in a scientific or engineering discipline with specific training in plant design, and response and analysis of the plant for transients and accidents.

6.4 TRAINING

6.4.1 A retraining and replacement training program for the facility staff shall be maintained under the direction of the Nuclear Training Manager and shall meet or exceed the requirements and recommendations of Section 5.5 of ANSI N18.1-1971 and Appendix A of 10 CFR Part 55.

6.4.2 DELETED

6.5 REVIEW AND AUDIT

6.5.1 Station Nuclear Safety Committee (SNSC)

Function

6.5.1.1 The Station Nuclear Safety Committee shall function to advise the Vice President-Nuclear Power on all matters related to nuclear safety.



UNITED STATES
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 207 TO FACILITY OPERATING LICENSE NO. DPR-26

CONSOLIDATED EDISON COMPANY OF NEW YORK, INC.

INDIAN POINT NUCLEAR GENERATING UNIT NO. 2

DOCKET NO. 50-247

1.0 INTRODUCTION

By letter dated January 22, 1999, Consolidated Edison Company of New York, Inc. (the licensee), submitted a request to modify the Technical Specifications (TSs) for Indian Point Nuclear Generating Unit No. 2 (IP2). The proposed amendment would revise (1) TS 3.7.D.1.g to add a word to clarify the function of the supply breakers, (2) TS 6.2.2.h to change the senior reactor operator (SRO) license requirement for the Operations Manager (OM), and (3) TS 6.3.1 to modify the qualification requirement for the Operations Manager.

2.0 EVALUATION

TS Section 3.7.D.1 or 3.7.D.2 state the requirements for emergency "Black Start" of the Gas Turbine Generators. The requirements consist of the needed plant electrical configuration prior to performing the evolution. TS 3.7.D.1.g requires that the 480-volt tie breakers 52/2A, 52/3A, 52/5A and 52/6A be open. Con Ed proposes to change the word "tie" to the word "supply," which would reflect the actual function of the breakers.

TS Section 6.2.2.h contains a requirement for the OM to have an SRO license. The proposed change would allow for either the OM or the Assistant Operations Manager (AOM) to have an SRO license.

Current practice at IP2 is to have two SRO licensed positions in charge of shift activities. These positions are the Senior Reactor Operator and the Shift Watch Supervisor. Both of these positions report to the OM through the AOM. Allowing either the OM or the AOM to hold an SRO license gives the licensee greater flexibility and allows the OM to concentrate on broader issues than day-to-day plant operations. This change is consistent with the Improved Standard Technical Specifications and the staff finds it acceptable.

TS Section 6.3.1 requires that each member of the facility staff meet the requirements of ANSI standard ANSI N18.1-1971. This standard requires that, at the time of appointment, the OM have an SRO license. The licensee proposes to modify this specification to require that each member of the facility staff meet the requirements of ANSI N18.1-1971, except for the OM and AOM's SRO license requirement which will be in accordance with TS 6.2.2.h.

As stated above, it is acceptable for either the OM or AOM to have an SRO license. This change to Section 6.3.1 is necessary to implement that policy. The staff finds this change acceptable.

3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the New York State official was notified of the proposed issuance of the amendment. The State official had no comments.

4.0 ENVIRONMENTAL CONSIDERATION

This amendment changes recordkeeping, reporting, or administrative procedures or requirements. Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(10). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: G. Wunder

Date: February 29, 2000