

Tennessee Valley Authority, Post Office Box 2000, Soddy-Daisy, Tennessee 37379

March 3, 2000

U.S. Nuclear Regulatory Commission ATTN: Document Control Desk Washington, D.C. 20555

Gentlemen:

In the Matter of) Doc Tennessee Valley Authority)

Docket No. 50-327

SEQUOYAH NUCLEAR PLANT (SQN) - UNIT 1 CORE OPERATING LIMITS REPORT (COLR)

In accordance with SQN Unit 1 Technical Specification 6.9.1.14.c, enclosed is the Unit 1 Cycle 11 COLR.

Please direct questions concerning this issue to me at (423) 843-7170 or J. D. Smith at (423) 843-6672.

Sincerel Ped

Pedro Salas Licensing and Industry Affairs Manager

Enclosure cc: See page 2 U.S. Nuclear Regulatory Commission Page 2 March 3, 2000

cc (Enclosure): Mr. R. W. Hernan, Project Manager Nuclear Regulatory Commission One White Flint, North 11555 Rockville Pike Rockville, Maryland 20852-2739

> NRC Resident Inspector Sequoyah Nuclear Plant 2600 Igou Ferry Road Soddy-Daisy, Tennessee 37379-3624

Regional Administrator U.S. Nuclear Regulatory Commission Region II 61 Forsythe St., SW, Suite 23T85 Atlanta, Georgia 30323-2711

ENCLOSURE

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SEQUOYAH NUCLEAR PLANT UNIT 1 CYCLE 11 CORE OPERATING LIMITS REPORT REVISION 0

LICENSING TRANSMITTAL TO NRC SUMMARY AND CONCURRENCE SHEET

THE PURPOSE OF THIS CONCURRENCE SHEET IS TO ASSURE THE ACCURACY AND COMPLETENESS OF TVA SUBMITTALS TO THE NRC.

DATE DATE DUE NRC <u>04/15/2000 - C</u>				
SUBMITTAL PREPARED BY _J. W. Proffitt				
SUBJECT: Sequoyah Nuclear Plant (SQN) - Unit 1 Core OperatingLimits Report (COLR)				
PURPOSE/SUMMARY Transmit to NRC the latest COLR revision.				
RESPONDS TO (RIMS NO.)				
NEW COMMITMENTSYESX_NO				
INDEPENDENT REVIEW DATE:				
LICENSING BASIS CHANGE - If this submittal requires a change to the licensing basis, a change has been initiated in accordance with NADP-7.				
A concurrence signature reflects that the signatory has assured that the submittal is appropriate and consistent with TVA Policy, applicable commitments are approved for implementation and supporting documentation for submittal completeness and accuracy has been prepared.				
CONCURRENCE (3)				
NAME ORGANIZATION SIGNATURE DATE				
D. L Koehl SQN Plant Manager				
10CFR 50.54(f) oath or affirmation required [] Yes [] No [X] N/A				
J. D. Smith SQN Licensing Supv.				
J. W. Proffitt SQN Lic & IA				

I: license / worddoc / Unit 1 Cycle 11 COLR JWP:PMB

L36 000217 800

COLR FOR SEQUOYAH UNIT 1 CYCLE 11

QA RECORD

SEQUOYAH NUCLEAR PLANT UNIT 1, CYCLE 11

REVISION 0

February 2000

Prepared: Muclear Fuel 1 2-22-2000 Date

Reviewed:

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Reactor Engineering Supervisor

Operations Manager

PORC Chairman

2-22-2000 Date

<u>Z-ZZ-2000</u> Date

2-24-2000 Date

Revision 0

Pages affected _____

Reason for Revision _

SEQUOYAH - UNIT 1

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1.0 CORE OPERATING LIMITS REPORT

This Core Operating Limits Report (COLR) for Sequoyah Unit 1 Cycle 11 has been prepared in accordance with the requirements of Technical Specification (TS) 6.9.1.14.

The TSs affected by this report are listed below:

- TABLE 2.2-1 f_1 (ΔI) trip reset function for OT ΔT Trip (QTNL, QTPL) and rates of trip setpoint decrease per percent ΔI (QTNS, QTPS)
- TABLE 2.2-1 f_2 (ΔI) trip reset function for OP ΔT Trip (QPNL, QPPL) and rates of trip setpoint decrease per percent ΔI (QPNS, QPPS)
- 3/4.1.1.3 Moderator Temperature Coefficient (MTC)
- 3/4.1.3.5 Shutdown Rod Insertion Limit
- 3/4.1.3.6 Control Rod Insertion Limits
- 3/4.2.1 Axial Flux Difference (AFD)
- 3/4.2.2 Heat Flux Hot Channel Factor (F_Q (X,Y,Z))
- 3/4.2.3 Nuclear Enthalpy Rise Hot Channel Factor ($F_{\Delta H}$ (X,Y))
- 2.0 OPERATING LIMITS

The cycle-specific parameter limits for the specifications listed in section 1.0 are presented in the following subsections. These limits have been developed using the NRC approved methodologies specified in TS 6.9.1.14.

The following abbreviations are used in this section:

BOL stands for Beginning of Cycle Life ARO stands for All Rods Out HZP stands for Hot Zero THERMAL POWER EOL stands for End of Cycle Life RTP stands for RATED THERMAL POWER

2.1 <u>Moderator Temperature Coefficient - MTC</u> (Specification 3/4.1.1.3)

2.1.1 The MTC limits are:

The BOL/ARO/HZP-MTC shall be less positive than $0 \Delta k/k/^{\circ}F$ (BOL limit). With the measured BOL/ARO/HZP-MTC more positive than $0 \Delta k/k/^{\circ}F$ (as-measured MTC limit), establish control rod withdrawal limits to ensure the MTC remains less positive than $0 \Delta k/k/^{\circ}F$ for all times in core life.

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The EOL/ARO/RTP-MTC shall be less negative than or equal to -4.5 x $10^{-4} \Delta k/k/^{\circ}F$.

2.1.2 The 300 ppm surveillance limit is:

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The measured 300 ppm/ARO/RTP-MTC should be less negative than or equal to -3.75 x $10^{-4} \Delta k/k/^{\circ}F$.

2.2 Shutdown Rod Insertion Limit (Specification 3/4.1.3.5)

2.2.1 The shutdown rods shall be withdrawn to a position as defined below:

Cycle Burnup (MWD/MTU)	Steps Withdrawn
<u><</u> 4,000	<u>></u> 225 to <u><</u> 231
> 4,000 to < 14,000	<u>> 222 to < 231</u>
<u>≥</u> 14,000	<u>></u> 225 to <u><</u> 231

2.3 Control Rod Insertion Limits (Specification 3/4.1.3.6)

2.3.1 The control rod banks shall be limited in physical insertion as shown in Figure 1.

2.4 <u>Axial Flux Difference - AFD</u> (Specification 3/4.2.1)

2.4.1 The axial flux difference (AFD) limits (AFD^{Limit}) are provided in Figure 2.

2.5 <u>Heat Flux Hot Channel Factor - Fo (X,Y,Z)</u> (Specification 3/4.2.2)

 $F_{Q}(X,Y,Z)$ shall be limited by the following relationships:

$$F_{Q}(X,Y,Z) \leq \frac{F_{Q}^{RTP}}{P} * K(Z) \quad \text{for } P > 0.5$$

$$F_{Q}(X,Y,Z) \leq \frac{F_{Q}^{RTP}}{0.5} * K(Z) \quad \text{for } P \leq 0.5$$

$$THERMAL POWER$$

where P = ______ RATED THERMAL POWER

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2.5.1 $F_Q^{RTP} = 2.50$

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2.5.2 K(Z) is provided in Figure 3

The following parameters are required for core monitoring per the Surveillance Requirements of Specification 3/4.2.2:

2.5.3	NSLOPE ^{AFD} = 1.15		
	where NSLOPE ^{AFD} =	Negative AFD limit adjustment required to compensate for each 1% that F_{Q} (X,Y,Z) exceeds BQDES.	
2.5.4	PSLOPE ^{AFD} = 1.08		
	where PSLOPE ^{AFD} =	Positive AFD limit adjustment required to compensate for each 1% that F_{α} (X,Y,Z) exceeds BQDES.	
2.5.5	$NSLOPE^{f_2(\DeltaI)} = 1.40$	·	
	where NSLOPE $f_{2^{(\Delta I)}}$	Adjustment to negative OP Δ T f ₂ (Δ I) limit required to compensate for each 1% that F _Q (X,Y,Z) exceeds BCDES.	
2.5.6	$PSLOPE^{f_2(\Delta^{\mathbf{i}})} = 1.63$		
	where PSLOPE ^{f2(△I)}	Adjustment to positive OP Δ T f ₂ (Δ I) limit required to compensate for each 1% that F _Q (X,Y,Z) exceeds BCDES.	
2.5.7	BQNOM(X,Y,Z) =	Nominal design peaking factor, increased by an allowance for the expected deviation between the nominal design power distribution and the measurement.	
2.5.8	BQDES(X,Y,Z) =	Maximum allowable design peaking factor which ensures that the F_{Q} (X,Y,Z) limit will be preserved for operation within the LCO limits, including allowances for calculational and measurement uncertainties.	
2.5.9	BCDES(X,Y,Z) =	Maximum allowable design peaking factor which ensures that the centerline fuel melt limit will be preserved for operation within the LCO limits, including allowances for calculational and measurement uncertainties.	
	BQNOM(X,Y,Z), BQDES(X,Y,Z), and BCDES(X,Y,Z) data bases are provided for input		

BQNOM(X,Y,Z), BQDES(X,Y,Z), and BCDES(X,Y,Z) data bases are provided for input to the plant power distribution analysis codes on a cycle specific basis and are determined using the methodology for core limit generation described in the references in Specification 6.9.1.14.

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2.5.10 The increase in $F_{Q}^{M}(X,Y,Z)$ for compliance with the 4.2.2.2.e Surveillance Requirements is defined as follows:

For cycle burnups ≤ 7128 MWd/MTU 2.0	1%
For cycle burnups > 7128 MWd/MTU to \leq 7774 MWd/MTU 3.0)%
For cycle burnups > 7774 MWd/MTU to \leq 9171 MWd/MTU 4.0)%
For cycle burnups > 9171 MWd/MTU to \leq 9494 MWd/MTU 3.0)%
For cycle bumups > 9494 MWd/MTU 2.0)%

Nuclear Enthalpy Rise Hot Channel Factor - FAH (X,Y) (Specification 3/4.2.3)

 F_{AH} (X,Y) shall be limited by the following relationship:

 $F_{AH}(X,Y) \leq MAP(X,Y,Z) / AXIAL(X,Y)$

2.6.1 MAP(X,Y,Z) is provided in Table 1.

AXIAL(X,Y) is the axial peak from the normalized axial power shape.

The following parameters are required for core monitoring per the Surveillance Requirements of Specification 3/4.2.3:

 $F \Delta H R^{M}(X,Y) < B H N O M(X,Y)$

where $F \Delta H R^M (X, Y) = F_{AH} (X, Y) / MAP^M / AXIAL(X, Y)$

 $F_{\Delta H}(X,Y)$ is the measured radial peak at location X,Y.

MAP^M is the value of MAP(X,Y,Z) obtained from Table 1 for the measured peak.

- 2.6.2 BHNOM(X,Y) = nominal design radial peaking factor, increased by an allowance for the expected deviation between the nominal design power distribution and the measurement.
- maximum allowable design radial peaking factor which ensures that 2.6.3 BHDES(X,Y) = the $F_{AH}(X, Y)$ limit will be preserved for operation within the LCO limits, including allowances for calculational and measurement uncertainties.
- maximum allowable design radial peaking factor which ensures that 2.6.4 BRDES(X,Y) = the steady state DNBR limit will be preserved for operation within the LCO limits, including allowances for calculational and measurement uncertainties.

BHNOM(X,Y), BHDES(X,Y) and BRDES(X,Y) data bases are provided for input to the plant power distribution analysis computer codes on a cycle specific basis and are determined using the methodology for core limit generation described in the references in Specification 6.9.1.14.

2.6.5 RRH = 3.34 when $0.8 < P \le 1.0$

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2.6

RRH = 1.67 when $P \le 0.8$

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where RRH = Thermal power reduction required to compensate for each 1% that $F_{AH}(X,Y)$ exceeds its limit.

P = Thermal Power / Rated Thermal Power

2.6.6 TRH = 0.0334 when $0.8 < P \le 1.0$

TRH = 0.0167 when $P \le 0.8$

where TRH = Reduction in OT Δ T K₁ setpoint required to compensate for each 1% F_{AH}(X,Y) exceeds its limit.

- 2.6.7 All cycle burnups shall use a 2% increase in $F_{\Delta H}^{M}$ (X,Y) margin for compliance with the 4.2.3.2.d.1 Surveillance Requirement.
- 3.0 REACTOR CORE PROTECTIVE LIMITS

3.1 Trip Reset Term [$f_1(\Delta I)$] for Overtemperature Delta T-Trip (Specification 2.2.1)

The following parameters are required to specify the power level-dependent $f_1(\Delta I)$ trip reset term limits for the Overtemperature Delta-T trip function:

3.1.1 QTNL = -20%

where QTNL = the maximum negative Δl setpoint at rated thermal power at which the trip setpoint is not reduced by the axial power distribution.

3.1.2 QTPL = +5%

where QTPL = the maximum positive ΔI setpoint at rated thermal power at which the trip setpoint is not reduced by the axial power distribution.

- 3.1.3 QTNS = 2.50%
 - where QTNS = the percent reduction in Overtemperature Delta-T trip setpoint for each percent that the magnitude of ΔI exceeds its negative limit at rated thermal power (QTNL).

3.1.4 QTPS = 1.40%

where QTPS = the percent reduction in Overtemperature Delta-T trip setpoint for each percent that the magnitude of ΔI exceeds its positive limit at rated thermal power (QTPL).

3.2 Trip Reset Term [$f_2(\Delta I)$] for Overpower Delta-T Trip (Specification 2.2.1)

The following parameters are required to specify the power level-dependent $f_2(\Delta I)$ trip reset term limits for the Overpower Delta-T trip function:

3.2.1 QPNL = -25%

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where QPNL = the maximum negative ∆I setpoint at rated thermal power at which the trip setpoint is not reduced by the axial power distribution.

3.2.2 QPPL = +25%

where QPPL = the maximum positive ΔI setpoint at rated thermal power at which the trip setpoint is not reduced by the axial power distribution.

3.2.3 QPNS = 1.70%

where QPNS = the percent reduction in Overpower Delta-T trip setpoint for each percent that the magnitude of ∆I exceeds its negative limit at rated thermal power (QPNL).

3.2.4 QPPS = 1.70%

where QPPS = the percent reduction in Overpower Delta T trip setpoint for each percent that the magnitude of ΔI exceeds its positive limit at rated thermal power (QPPL).

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Table 1

Maximum Allowable Peaking Limits MAP(X,Y,Z)

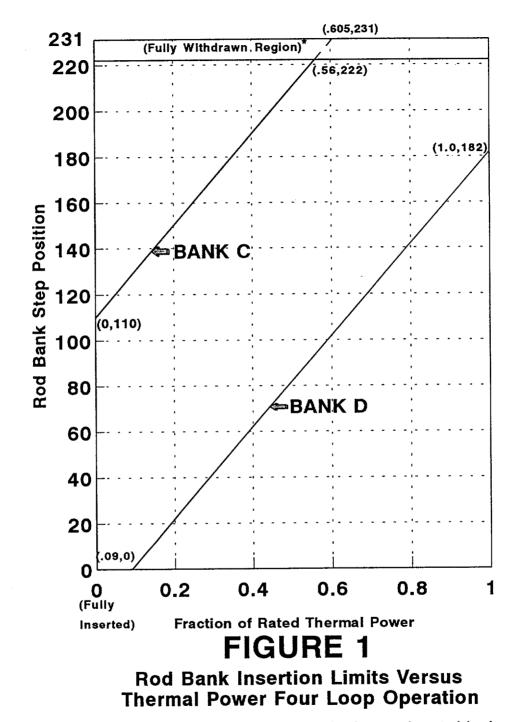
Elevation (ft)	AXIAL (X,Y)	<u>MAP (X,Y,Z)</u>
2 4 6 8 10	1.1	1.970 1.966 1.958 1.945 1.917
2 4 6 8 10	1.2	2.208 2.197 2.180 2.150 2.072
2 4 6 8 10	1.3	2.453 2.434 2.406 2.315 2.185
2 4 6 8 10	1.4	2.702 2.672 2.572 2.429 2.288
2 4 6 8 10	1.5	2.956 2.826 2.683 2.529 2.381
2 4 6 8 10	1.7	3.162 3.007 2.850 2.690 2.542
2 4 6 8 10	1.9	3.283 3.133 2.982 2.821 2.685

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COLR For Sequoyah Unit 1 Cycle 11



^{*} Fully withdrawn region shall be the condition where shutdown and control banks are at a position within the interval of \geq 222 and \leq 231 steps withdrawn, inclusive.

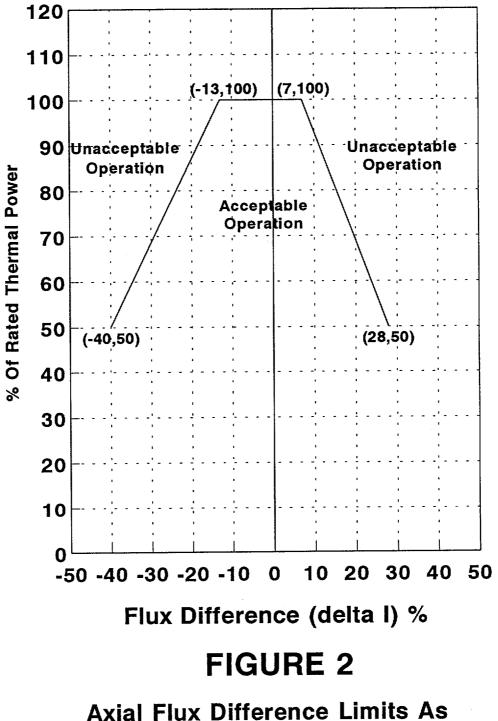
Fully withdrawn shall be the position as defined below,

<u>Cycle Burnup (MWd/MTU)</u>	<u>Step Withdrawn</u>	
≤ 4000	≥ 225 to <u><</u> 231	
> 4000 to < 14,000	≥ 222 to <u><</u> 231	
≥ 14,000	≥ 225 to <u><</u> 231	

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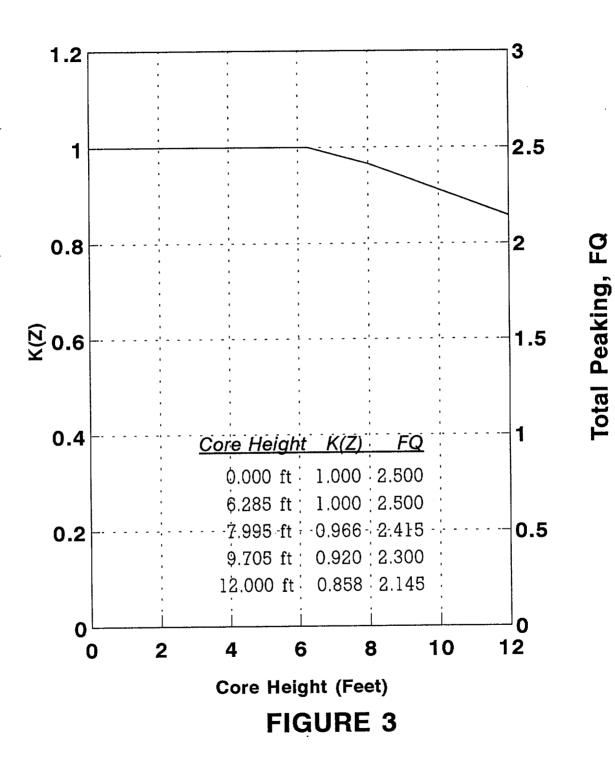
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K(Z) - Normalized FQ(X,Y,Z) as a Function of Core Height

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