

NRR-
Template 058

February 29, 2000

Mr. Michael D. Wadley, President
Northern States Power Company
414 Nicollet Mall
Minneapolis, MN 55401

SUBJECT: PRAIRIE ISLAND NUCLEAR GENERATING PLANT, UNITS 1 AND 2 -
ISSUANCE OF AMENDMENTS RE: RELOCATION OF CERTAIN PORTIONS OF
DESIGN FEATURES INFORMATION TO UPDATED SAFETY ANALYSIS
REPORT (TAC NOS. MA8191 AND MA8192)

Dear Mr. Anderson:

The Commission has issued the enclosed Amendment No. 148 to Facility Operating License No. DPR-42 and Amendment No. 139 to Facility Operating License No. DPR-60 for the Prairie Island Nuclear Generating Plant, Units 1 and 2, respectively. The amendments consist of changes to the Technical Specifications in response to your application dated November 6, 1996, as supplemented April 10 and October 1, 1997, and March 4, 1998.

The amendments revise TS Section 5.0, "DESIGN FEATURES," by relocating certain portions of the design features information to the Updated Safety Analysis Report (USAR), consistent with NUREG-1431, "Standard Technical Specifications, Westinghouse Plants," Revision 1. The relocated information in the USAR will be subject to the provisions of 10 CFR 50.59.

A copy of our related safety evaluation is also enclosed. The Notice of Issuance will be included in the Commission's biweekly *Federal Register* notice.

Sincerely,

/RA/
Tae Kim, Senior Project Manager, Section 1
Project Directorate III
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket Nos. 50-282 and 50-306

- Enclosures: 1. Amendment No. 148 to DPR-42
- 2. Amendment No. 139 to DPR-60
- 3. Safety Evaluation

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NAME	TKim: <i>JK</i>		RBouling <i>RB</i>		WBeckner	<i>Wadley</i>	CCraig <i>CC</i>	
DATE	2/14/00		2/14/00		02/10/00	2/10/00	2/25/00	

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NAME	TKim: <i>TJK</i>		RBouling <i>RB</i>		WBeckner	<i>WBeckner</i>	CCraig <i>CC</i>	
DATE	2/14/00		2/14/00		02/10/00	2/10/00	2/25/00	

DOCUMENT NAME: G:\PDIII-1\PRAIRIE\AMDMA8191.WPD
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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

February 29, 2000

Mr. Michael D. Wadley, President
Northern States Power Company
414 Nicollet Mall
Minneapolis, MN 55401

SUBJECT: PRAIRIE ISLAND NUCLEAR GENERATING PLANT, UNITS 1 AND 2 -
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Sincerely,

Tae Kim, Senior Project Manager, Section 1
Project Directorate III
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket Nos. 50-282 and 50-306

Enclosures: 1. Amendment No. 148 to DPR-42
2. Amendment No. 139 to DPR-60
3. Safety Evaluation

cc w/encl: See next page

Prairie Island Nuclear Generating Plant,
Units 1 and 2

cc:

J. E. Silberg, Esquire
Shaw, Pittman, Potts and Trowbridge
2300 N Street, N. W.
Washington, DC 20037

Plant Manager
Prairie Island Nuclear Generating Plant
Northern States Power Company
1717 Wakonade Drive East
Welch, MN 55089

Adonis A. Neblett
Assistant Attorney General
Office of the Attorney General
455 Minnesota Street
Suite 900
St. Paul, MN 55101-2127

U.S. Nuclear Regulatory Commission
Resident Inspector's Office
1719 Wakonade Drive East
Welch, MN 55089-9642

Regional Administrator, Region III
U.S. Nuclear Regulatory Commission
801 Warrenville Road
Lisle, IL 60532-4351

Mr. Stephen Bloom, Administrator
Goodhue County Courthouse
Box 408
Red Wing, MN 55066-0408

Commissioner
Minnesota Department of Commerce
121 Seventh Place East
Suite 200
St. Paul, MN 55101-2145

Site Licensing
Prairie Island Nuclear Generating Plant
Northern States Power Company
1717 Wakonade Drive East
Welch, MN 55089

Tribal Council
Prairie Island Indian Community
ATTN: Environmental Department
5636 Sturgeon Lake Road
Welch, MN 55089

Site General Manager
Prairie Island Nuclear Generating Plant
Northern States Power Company
1717 Wakonade Drive East
Welch, MN 55089



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

NORTHERN STATES POWER COMPANY

DOCKET NO. 50-282

PRAIRIE ISLAND NUCLEAR GENERATING PLANT, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 148
License No. DPR-42

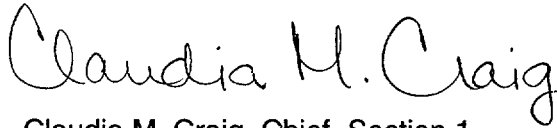
1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Northern States Power Company (the licensee) dated November 6, 1996, as supplemented April 10 and October 1, 1997, and March 4, 1998, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-42 is hereby amended to read as follows:

Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 148 , are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance and shall be implemented within 30 days. In addition, the licensee shall include the relocated information in the Updated Final Safety Analysis Report submitted to the NRC, pursuant to 10 CFR 50.71(e), as was described in the licensee's application dated November 6, 1996, as supplemented April 10 and October 1, 1997, and March 4, 1998, and evaluated in the NRC staff's safety evaluation dated February 29 , 2000.

FOR THE NUCLEAR REGULATORY COMMISSION



Claudia M. Craig, Chief, Section 1
Project Directorate III
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical Specifications

Date of Issuance: February 29, 2000

ATTACHMENT TO LICENSE AMENDMENT NO. 148

FACILITY OPERATING LICENSE NO. DPR-42

DOCKET NO. 50-282

Replace the following pages of the Appendix A Technical Specifications with the attached revised pages. The revised pages are identified by amendment number and contain marginal lines indicating the areas of change.

REMOVE

TS-vii
TS.5.1-1
TS.5.1-2
TS.5.4-1

INSERT

TS-vii
TS.5.1-1
Deleted
Deleted

TABLE OF CONTENTS (Continued)

<u>TS SECTION</u>	<u>TITLE</u>	<u>PAGE</u>
5.0	DESIGN FEATURES	TS.5.1-1
5.1	Site Location	TS.5.1-1
5.2	A. Containment Structures	TS.5.2-1
	1. Containment Vessel	TS.5.2-1
	2. Shield Building	TS.5.2-2
	3. Auxiliary Building Special Ventilation Zone	
	B. Special Ventilation Systems	TS.5.2-2
	C. Containment System Functional Design	TS.5.2-3
5.3	Reactor	TS.5.3-1
	A. Reactor Core	TS.5.3-1
	B. Reactor Coolant System	TS.5.3-1
	C. Protection Systems	TS.5.3-1
5.4	Deleted	
5.5	Deleted	
5.6	Fuel Handling	TS.5.6-1
	A. Criticality Consideration	TS.5.6-1
	B. Spent Fuel Storage Structure	TS.5.6-1
	C. Fuel Handling	TS.5.6-2
	D. Spent Fuel Storage Capacity	TS.5.6-3

5.0 DESIGN FEATURES

5.1 SITE LOCATION

The site for the Prairie Island Nuclear Generating Plant is located on the west bank of the Mississippi River, approximately 6 miles northwest of the city of Red Wing, Minnesota. The site exclusion area boundary has a minimum radius of 715 meters from the center line of either reactor.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

NORTHERN STATES POWER COMPANY

DOCKET NO. 50-306

PRAIRIE ISLAND NUCLEAR GENERATING PLANT, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 139
License No. DPR-60

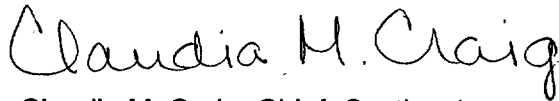
1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Northern States Power Company (the licensee) dated November 6, 1996, as supplemented April 10 and October 1, 1997, and March 4, 1998, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-60 is hereby amended to read as follows:

Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 139, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance and shall be implemented within 30 days. In addition, the licensee shall include the relocated information in the Updated Final Safety Analysis Report submitted to the NRC, pursuant to 10 CFR 50.71(e), as was described in the licensee's application dated November 6, 1996, as supplemented April 10 and October 1, 1997, and March 4, 1998, and evaluated in the NRC staff's safety evaluation dated February 29, 2000.

FOR THE NUCLEAR REGULATORY COMMISSION



Claudia M. Craig, Chief, Section 1
Project Directorate III
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical Specifications

Date of Issuance: February 29, 2000

ATTACHMENT TO LICENSE AMENDMENT NO. 139

FACILITY OPERATING LICENSE NO. DPR-60

DOCKET NO. 50-306

Replace the following pages of the Appendix A Technical Specifications with the attached revised pages. The revised pages are identified by amendment number and contain marginal lines indicating the areas of change.

REMOVE

TS-vii
TS.5.1-1
TS.5.1-2
TS.5.4-1

INSERT

TS-vii
TS.5.1-1
Deleted
Deleted

TABLE OF CONTENTS (Continued)

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	A. Reactor Core	TS.5.3-1
	B. Reactor Coolant System	TS.5.3-1
	C. Protection Systems	TS.5.3-1
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5.5	Deleted	
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	A. Criticality Consideration	TS.5.6-1
	B. Spent Fuel Storage Structure	TS.5.6-1
	C. Fuel Handling	TS.5.6-2
	D. Spent Fuel Storage Capacity	TS.5.6-3

5.0 DESIGN FEATURES

5.1 SITE LOCATION

The site for the Prairie Island Nuclear Generating Plant is located on the west bank of the Mississippi River, approximately 6 miles northwest of the city of Red Wing, Minnesota. The site exclusion area boundary has a minimum radius of 715 meters from the center line of either reactor.



UNITED STATES
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 148

TO FACILITY OPERATING LICENSE NO. DPR-42

AND AMENDMENT NO. 139 TO FACILITY OPERATION LICENSE NO. DPR-60

NORTHERN STATES POWER COMPANY

PRAIRIE ISLAND NUCLEAR GENERATING PLANT, UNITS 1 AND 2

DOCKET NOS. 50-282 AND 50-306

1.0 INTRODUCTION

By application dated November 6, 1996, as supplemented April 10 and October 1, 1997, and March 4, 1998, Northern States Power Company (NSP or the licensee) requested changes to the Technical Specifications (TSs) for the Prairie Island Nuclear Generating Plant, Units 1 and 2. The proposed changes would revise TS Section 5.0, "DESIGN FEATURES," by relocating certain portions of the design features information to the Updated Safety Analysis Report (USAR), consistent with NUREG-1431, "Standard Technical Specifications, Westinghouse Plants," Revision 1. The relocated information in the USAR will be subject to the provisions of 10 CFR 50.59. The April 10 and October 1, 1997, and March 4, 1998, supplemental letters were within the scope of the original *Federal Register* notice and did not change the initial proposed no significant hazards consideration determination.

2.0 BACKGROUND

The NRC staff issued Amendment Nos. 131 and 123 on October 21, 1997, for Prairie Island, Units 1 and 2, respectively, in response to NSP's application dated November 6, 1996, as supplemented April 10 and October 1, 1997. Amendment Nos. 131 and 123 adopted the proposed changes to TS 3.3-7, TS 4.5-3, and part of the proposed changes to TS 5.4-1, but did not include the proposed changes to TS 5.1. The staff noted in Amendment Nos. 131 and 123 that the proposed changes to TS 5.1 and the remaining changes to TS 5.4 would be addressed in a separate licensing action. Subsequently, the licensee submitted a letter dated March 4, 1998, with supplemental information regarding the proposed changes to TS 5.1 and TS 5.4. With this March 4, 1998, letter, the staff initiated a separate licensing action to address the proposed changes to TS 5.1 and TS 5.4.

The proposed changes, in accordance with the licensee's application dated November 6, 1996, as supplemented April 10 and October 1, 1997, and March 4, 1998, would revise Section 5.0, "Design Features," of the current TSs (CTS), adopting, for the most part, the form and content of NUREG-1431, Revision 1, "Standard Technical Specifications [STS] for Westinghouse Plants," for the changes requested. The proposed changes include the relocation of certain

portions of the design features section to other licensee-controlled documents (i.e., USAR). The relocated requirements will be subject to the appropriate level of regulatory authority and control in accordance with 10 CFR 50.59.

2.0 EVALUATION

When converting a plant's CTS (or portion thereof) to the STS format, a licensee may, for each individual specification, at its discretion, either (1) adopt the technical requirements of the corresponding STS, (2) retain the existing CTS, or (3) propose a different specification. The staff classifies selections (1) and (2) as "in-scope," while selection (3) is referred to as "beyond-scope." In-scope changes are generically accepted based on consistency with the staff's policy on STS conversions. Beyond-scope changes are evaluated on an individual basis.

Each of the licensee's proposed changes is discussed and evaluated below, and the proposed changes are identified as either "in-scope" or "beyond-scope." Appropriate justification is provided for each beyond-scope change.

2.1 Proposed Changes to TS-vii, "Table of Contents"

The Table of Contents page, TS-vii of the CTS, would be changed to reflect the changes to the design features section titles and page numbers consistent with the proposed changes authorized in these amendments.

Administrative (nontechnical) changes that are intended to incorporate human factors principles into the form and structure of the STS so that plant operations personnel can use them more easily are considered in-scope changes and are acceptable on that basis. The proposed change to TS-vii, "Table of Contents," is considered in-scope and is, therefore, acceptable.

2.2 Proposed Changes to TS Section 5.1, "SITE"

CTS Section 5.1, "SITE," would be replaced with STS Section 5.1, "SITE LOCATION," which provides a text description of the location for the site and the exclusion area boundary. This is accomplished by revising the first paragraph of CTS Section 5.1, which defines the location of the site and the exclusion area boundary, relocating the remaining paragraphs of CTS Section 5.1 to the USAR, and changing the title of this section to "SITE LOCATION," consistent with the STS format in NUREG-1431.

The revision to the first paragraph of CTS Section 5.1 involves minor editorial changes to clarify the existing text description of the location for the site and the exclusion area boundary. The revision also deletes references to "the low population zone" and the "nearest population center." Up-to-date information regarding both the low population zone and the nearest population center is currently located in USAR Section 2.2. The existence of this information in the USAR will ensure that any change to either the low population zone or the nearest population center will have to be evaluated using the 10 CFR 50.59 process.

The remaining paragraphs of CTS Section 5.1 provide information including areas within the exclusion area boundary that are controlled by the U.S. Army Corp of Engineers, meteorological considerations, flood and flood protection, and seismic information. This information is either currently included in the USAR or will be relocated to USAR by the

licensee. (These license amendments require full implementation, including relocating the above mentioned portion of CTS Section 5.1 to the USAR, within 30 days from the date of issuance of this amendment.) The existence of this information in the USAR will ensure that any future change will have to be evaluated using the 10 CFR 50.59 process. The staff also notes that previously issued Amendment Nos. 141 and 132, dated December 7, 1998, deleted references in TS 5.1 to the emergency procedures for floods and earthquakes.

Based on the above discussion, the NRC staff finds the proposed changes to TS Section 5.1 to be consistent with the STS format in NUREG-1431, and are, therefore acceptable.

2.3 Proposed Changes to TS Section 5.4, "ENGINEERED SAFETY FEATURES"

The licensee also proposed to delete CTS Section 5.4, "ENGINEERED SAFETY FEATURES," in its entirety, and relocate it to the USAR. CTS Section 5.4 contains information (text description) regarding a general overview of the engineered safety features (ESFs), ESFs that are shared between the two units, and the emergency cooling water system.

This information is either currently included in USAR or will be relocated to USAR by the licensee. (These license amendments require full implementation, including relocating the above mentioned portion of CTS Section 5.4 to USAR, within 30 days from the date of issuance.) The existence of this information in the USAR will ensure that any future change will have to be evaluated using the 10 CFR 50.59 process. The staff also notes that the reference to the surveillance requirement for the emergency cooling water pumps in CTS 5.4.2 is covered by TS 4.5.B.1.b.

Based on the above discussion, the NRC staff finds the proposed changes to TS Section 5.4 to be consistent with the STS format in NUREG-1431, and are, therefore acceptable.

3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Minnesota State official was notified of the proposed issuance of the amendments. The State official had no comments.

4.0 ENVIRONMENTAL CONSIDERATION

The amendments change a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The NRC staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendments involve no significant hazards consideration and there has been no public comment on such finding (62 FR 4338). Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: T. Kim

Date: February 29, 2000

DATED: February 29, 2000

AMENDMENT NO. 148 TO FACILITY OPERATING LICENSE NO. DPR-42-PRAIRIE ISLAND UNIT 1
AMENDMENT NO. 139 TO FACILITY OPERATING LICENSE NO. DPR-60-PRAIRIE ISLAND UNIT 2

Docket File (50-282, 50-306)

PUBLIC

PDIII-1 Reading

J. Zwolinski/S. Black

S. Bajwa/C. Craig

R. Bouling

T. J. Kim (2)

OGC

G. Hill (4)

W. Beckner

ACRS

R. Lanksbury, RIII

~~RGN (SE)~~