

Tennessee Valley Authority, Post Office Box 2000, Spring City, Tennessee 37381-2000

FFR 2 9 2000

U.S. Nuclear Regulatory Commission ATTN: Document Control Desk Washington, D.C. 20555

Gentlemen:

In the Matter of) Docket No. 50-390 Tennessee Valley Authority)

WATTS BAR NUCLEAR PLANT (WBN) UNIT 1 - TECHNICAL SPECIFICATION (TS) CHANGE NO. 98-005 - CHANGE TO A MORE NEGATIVE MODERATOR TEMPERATURE COEFFICIENT (MTC) - REVISED PAGE

Reference:

TVA LETTER TO NRC DATED SEPTEMBER 30, 1999, "WATTS BAR NUCLEAR PLANT (WBN) - UNIT 1 - TECHNICAL SPECIFICATION (TS) CHANGE NO. 98-005 - CHANGE TO A MORE NEGATIVE MODERATOR TEMPERATURE COEFFICIENT (MTC)"

As recently discussed with NRC's Mr. Robert E. Martin, this letter provides a revised replacement page for the referenced WBN license amendment to reflect an administrative change. Page number 5.0-32 of the proposed TS added an Item No. 5 to reference the new analytical methodology (WCAP-15088-P, Rev 1) for the determination of MTC. As discussed with Mr. Martin, the Staff considers that the Item 5 reference to WCAP-15088-P, R1, should include a reference to NRC's forthcoming issuance of their safety evaluation for the subject license amendment. Accordingly, TVA has provided an Errata page 5.0-32 (both mark-up and revised pages) in the Enclosure to this letter. TVA has used shading in the mark-up page to distinguish the additional text from our original mark-up provided in the referenced letter.



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If you have any questions about this change, please contact me at (423) 365-1824.

Sincerely,

P. L. Pace, Manager

Licensing and Industry Affairs

Enclosure

cc: See page 3

Subscribed and sworn to before me on this 294 day of fully 2000

Notary Public

My Commission Expires_

June 27, 2001

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cc (Enclosures):

NRC Resident Inspector Watts Bar Nuclear Plant 1260 Nuclear Plant Road Spring City, Tennessee 37381

Mr. Robert E. Martin, Senior Project Manager U.S. Nuclear Regulatory Commission One White Flint North 11555 Rockville Pike Rockville, Maryland 20852

U.S. Nuclear Regulatory Commission Region II Atlanta Federal Center 61 Forsyth St., SW, Suite 23T85 Atlanta, Georgia 30303

Mr. Michael H. Mobley, Director Division of Radiological Health 3rd Floor L & C Annex Nashville, Tennessee 37423

ENCLOSURE

TENNESSEE VALLEY AUTHORITY WATTS BAR NUCLEAR PLANT (WBN) UNIT 1

PROPOSED TECHNICAL SPECIFICATION (TS) CHANGE TS-98-005
MARKED AND REVISED REPLACEMENT PAGES

I. AFFECTED PAGE LIST

Page 5.0-32

II. MARKED PAGE/REVISED PAGE

Attached

5.9.5 CORE OPERATING LIMITS REPORT (COLR)

- a. Core operating limits shall be established prior to the initial and each reload cycle, or prior to any remaining portion of a cycle, and shall be documented in the COLR for the following:
 - LCO 3.1.4 Moderator Temperature Coefficient
 - LCO 3.1.6 Shutdown Bank Insertion Limit
 - LCO 3.1.7 Control Bank Insertion Limits
 - LCO 3.2.1 Heat Flux Hot Channel Factor
 - LCO 3.2.2 Nuclear Enthalpy Rise Hot Channel Factor
 - LCO 3.2.3 Axial Flux Difference
 - LCO 3.9.1 Boron Concentration
- b. The analytical methods used to determine the core operating limits shall be those previously reviewed and approved by the NRC, specifically those described in the following documents:
 - 1. WCAP-9272-P-A, WESTINGHOUSE RELOAD SAFETY EVALUATION METHODOLOGY", July 1985 (W Proprietary). (Methodology for Specifications 3.1.4 Moderator Temperature Coefficient, 3.1.6 Shutdown Bank Insertion Limit, 3.1.7 Control Bank Insertion Limits, 3.2.1 Heat Flux Hot Channel Factor, 3.2.2 Nuclear Enthalphy Rise Hot Channel Factor, 3.2.3 Axial Flux Difference, and 3.9.1 Boron Concentration.
 - 2. WCAP-10266-P-A, Rev 2, "The 1981 VERSION OF WESTINGHOUSE EVALUATION MODEL USING BASH CODE," March 1987. (W Proprietary). (Methodology for Specification 3.2.1 Heat Flux Hot Channel Factor).
 - 3. WCAP-10216-P-A, Revision 1A, "RELAXATION OF CONSTANT AXIAL OFFSET CONTROL F(Q) SURVEILLANCE TECHNICAL SPECIFICATION," February 1994 (W Proprietary). (Methodology for Specifications 3.2.1 Heat Flux Hot Channel Factor (W(Z) Surveillance Requirements For F(Q) Methodology) and 3.2.3 Axial Flux Difference (Relaxed Axial Offset Control).)
 - 4. WCAP-12610-P-A, "VANTAGE + FUEL ASSEMBLY REFERENCE CORE REPORT," April 1995. (W Proprietary). (Methodology for Specification 3.2.1 Heat Flux Hot Channel Factor).

INSERT

WCAP-15088-P, Rev.1, "Safety Evaluation Supporting A More Negative EOL Moderator Temperature Coefficient Technical Specification for the Watts Bar Nuclear Plant," July 1999, (W Proprietary), as approved by the NRC staff's Safety Evaluation accompanying the issuance of Amendment No. (Methodology for Specification 3.1.4- Moderator Temperature Coefficient.)

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 - LCO 3.1.7 Control Bank Insertion Limits
 - LCO 3.2.1 Heat Flux Hot Channel Factor
 - LCO 3.2.2 Nuclear Enthalpy Rise Hot Channel Factor LCO 3.2.3 Axial Flux Difference

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 - WCAP-10266-P-A, Rev 2, "The 1981 VERSION OF WESTINGHOUSE EVALUATION MODEL USING BASH CODE," March 1987. (W Proprietary). (Methodology for Specification 3.2.1 - Heat Flux Hot Channel Factor).
 - WCAP-10216-P-A, Revision 1A, "RELAXATION OF CONSTANT AXIAL OFFSET CONTROL F(Q) SURVEILLANCE TECHNICAL SPECIFICATION, February 1994 (W Proprietary). (Methodology for Specifications 3.2.1 - Heat Flux Hot Channel Factor (W(Z) Surveillance Requirements For F(Q) Methodology) and 3.2.3 - Axial Flux Difference (Relaxed Axial Offset Control).)
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 - WCAP-15088-P, Rev.1, "Safety Evaluation Supporting A More Negative EOL Moderator Temperature Coefficient Technical Specification for the Watts Bar Nuclear Plant, "July 1999, (W Proprietary), as approved by the NRC staff's Safety Evaluation accompanying the issuance of Amendment No. (Methodology for Specification 3.1.4- Moderator Temperature Coefficient.)

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