UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

February 28, 2000

Mr. L. W. Myers
Senior Vice President
Beaver Valley Power Station
Post Office Box 4
Shippingport, PA 15077

SUBJECT: BEAVER VALLEY 1 AND 2 - ISSUANCE OF AMENDMENT RE: RELOCATION OF SEISMIC MONITORING INSTRUMENTATION REQUIREMENTS PER GUIDANCE OF GENERIC LETTER 95-10 (TAC NOS. MA5624 AND MA5625)

Dear Mr. Myers:

The Commission has issued the enclosed Amendment No. 228 to Facility Operating License No. DPR-66 and Amendment No. 107 to Facility Operating License No. NPF-73 for the Beaver Valley Power Station, Unit Nos. 1 and 2 (BVPS-1 and 2). These amendments consist of changes to the Technical Specifications (TSs) in response to Duquesne Light Company's (DLC) application dated May 27, 1999, as the then licensee for BVPS-1 and 2, which submitted License Amendment Request Nos. 268 and 143.

On the date of the May 27, 1999, letter, DLC was the licensed operator for BVPS-1 and BVPS-2. On December 3, 1999, DLC's ownership interests in both BVPS-1 and BVPS-2 were transferred to the Pennsylvania Power Company, and DLC's operating authority for BVPS-1 and BVPS-2 was transferred to FirstEnergy Nuclear Operating Company (FENOC). By letter dated December 13, 1999, FENOC requested that the Nuclear Regulatory Commission continue to review and act upon all requests before the Commission which had been submitted by DLC.

These amendments relocate the seismic monitoring instrumentation requirements contained in Technical Specification (TS) 3/4.3.3.3 to the Licensing Requirements Manual (LRM) based on the guidance provided in Generic Letter 95-10, "Relocation of Selected Technical Specifications Requirements Related to Instrumentation." The Bases section for Specification 3/4.3.3.3 is also relocated to the LRM. The appropriate Index pages, Table Index page (Unit No. 1 only), TS pages and Bases pages are revised to reflect the removal of the seismic monitoring instrumentation specification from the TSs. An additional specification page is added to reflect that TS Number 3/4.3.3.4 is not used. This additional page also denotes the number of the following page. Finally, the Bases section is modified to denote that TS Number 3/4.3.3.4 is not used.

D101

A copy of our safety evaluation is also enclosed. The Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

Daniel S. Collins, Project Manager, Section 1
Project Directorate I
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket Nos. 50-334 and 50-412

Enclosures: 1. Amendment No. 228 to DPR-66
2. Amendment No. 107 to NPF-73
3. Safety Evaluation

cc w/encls: See next page

Template #NRR-056

A copy of our safety evaluation is also enclosed. The Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

/RA/

Daniel S. Collins, Project Manager, Section 1
Project Directorate I
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket Nos. 50-334 and 50-412

Enclosures: 1. Amendment No.228 to DPR-66
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3. Safety Evaluation

cc w/encls: See next page

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** SE Input dated 1/2/00 was provided and no major technical changes were made.

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

PENNSYLVANIA POWER COMPANY

OHIO EDISON COMPANY

FIRSTENERGY NUCLEAR OPERATING COMPANY

DOCKET NO. 50-334

BEAVER VALLEY POWER STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 228
License No. DPR-66

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by FirstEnergy Nuclear Operating Company, et al. (the licensee) dated May 27, 1999, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-66 is hereby amended to read as follows:

   (2) Technical Specifications

   The Technical Specifications contained in Appendix A, as revised through Amendment No. 228 , are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance and shall be implemented within 60 days. Implementation of this amendment shall include the relocation of these Technical Specification requirements to the appropriate documents as described in the licensee's application dated May 27, 1999, and evaluated in the staff's safety evaluation attached to this amendment.

FOR THE NUCLEAR REGULATORY COMMISSION

Marsha Gamberoni

Marsha Gamberoni, Acting Chief, Section 1
Project Directorate I
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical Specifications

Date of Issuance: February 28, 2000

ATTACHMENT TO LICENSE AMENDMENT NO. 228

FACILITY OPERATING LICENSE NO. DPR-66

DOCKET NO. 50-334

Replace the following pages of Appendix A Technical Specifications with the attached revised pages. The revised pages are identified by amendment number and contain marginal lines indicating the areas of change.

Remove	Insert
IV	IV
X	X
XVI	XVI
3/4 3-38	3/4 3-38
3/4 3-39	3/4 3-39
3/4 3-40	- - - - - -
B 3/4 3-2	B 3/4 3-2

INDEX

LIMITING CONDITIONS FOR OPERATION AND SURVEILLANCE REQUIREMENTS

INDEX

BASES

TABLE INDEX

3/4.3.3.3 (This Specification number is not used.)

3/4.3.3.4 (This Specification number is not used.)

BASES

3/4.3.3 MONITORING INSTRUMENTATION

3/4.3.3.1 RADIATION MONITORING INSTRUMENTATION

The OPERABILITY of the radiation monitoring channels ensures that: 1) the radiation levels are continually measured in the areas served by the individual channels; 2) the alarm or automatic action is initiated when the radiation level trip setpoint is exceeded; and 3) sufficient information is available on selected plant parameters to monitor and assess these variables following an accident. This capability is consistent with the recommendations of NUREG-0737, "Clarification of TMI Action Plan Requirements," October, 1980.

3/4.3.3.2 MOVABLE INCORE DETECTORS

The OPERABILITY of the movable incore detectors with the specified minimum complement of equipment ensures that the measurements obtained from use of this system accurately represent the spatial neutron flux distribution of the reactor core. The OPERABILITY of this system is demonstrated by irradiating each detector used and determining the acceptability of its voltage curve.

For the purpose of measuring $F_Q(Z)$ or $F^N_{\Delta H}$, a full incore flux map is used. Quarter-core flux maps, as defined in WCAP-8648, June 1976, may be used in recalibration of the excore neutron flux detection system, and full incore flux maps or symmetric incore thimbles may be used for monitoring the Quadrant Power Tilt Ratio when one Power Range Channel is inoperable.

3/4.3.3.3 (This Specification number is not used.)

3/4.3.3.4 (This Specification number is not used.)

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

PENNSYLVANIA POWER COMPANY

OHIO EDISON COMPANY

THE CLEVELAND ELECTRIC ILLUMINATING COMPANY

THE TOLEDO EDISON COMPANY

FIRSTENERGY NUCLEAR OPERATING COMPANY

DOCKET NO. 50-412

BEAVER VALLEY POWER STATION, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 107
License No. NPF-73

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by FirstEnergy Nuclear Operating Company, et al. (the licensee) dated May 27, 1999, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-73 is hereby amended to read as follows:

   (2) Technical Specifications

   The Technical Specifications contained in Appendix A, as revised through Amendment No. 107 , and the Environmental Protection Plan contained in Appendix B, both of which are attached hereto are hereby incorporated in the license. FENOC shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of the date of its issuance and shall be implemented within 60 days. Implementation of this amendment shall include the relocation of these Technical Specification requirements to the appropriate documents as described in the licensee's application dated May 27, 1999, and evaluated in the staff's safety evaluation attached to this amendment.

FOR THE NUCLEAR REGULATORY COMMISSION

Marsha Gamberoni

Marsha Gamberoni, Acting Chief, Section 1
Project Directorate I
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical Specifications

Date of Issuance: February 28, 2000

ATTACHMENT TO LICENSE AMENDMENT NO. 107

FACILITY OPERATING LICENSE NO. NPF-73

DOCKET NO. 50-412

Replace the following pages of Appendix A Technical Specifications with the attached revised pages. The revised pages are identified by amendment number and contain marginal lines indicating the areas of change.

Remove	Insert
IV	IV
X	X
3/4 3-46	3/4 3-46
3/4 3-47	3/4 3-47
3/4 3-48	- - - - - -
B 3/4 3-11	B 3/4 3-11

INDEX

LIMITING CONDITION FOR OPERATION AND SURVEILLANCE REQUIREMENTS

INDEX

BASES

3/4.3.3.3 (This Specification number is not used.)

3/4.3.3.4 (This Specification number is not used.)

BASES

3/4.3.3.3 (This Specification number is not used.)

3/4.3.3.4 (This Specification number is not used.)

3/4.3.3.5 REMOTE SHUTDOWN INSTRUMENTATION

The OPERABILITY of the remote shutdown instrumentation ensures that sufficient capability is available to permit shutdown and maintenance of HOT STANDBY of the facility from locations outside of the control room. This capability is required in the event control room habitability is lost and is consistent with General Design Criteria 19 of 10 CFR 50.

3/4.3.3.6 (This Specification number is not used).

3/4.3.3.7 CHLORINE DETECTION SYSTEMS

The OPERABILITY of the chlorine detection systems ensures that sufficient capability is available to promptly detect and initiate protective action in the event of an accidental chlorine release. This capability is required to protect control room personnel and is consistent with the recommendations of Regulatory Guide 1.95, "Protection of Nuclear Power Plant Control Room Operators Against an Accidental Chlorine Release," January 1977.

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NOS. 228 AND 107 TO FACILITY OPERATING

LICENSE NOS. DPR-66 AND NPF-73

PENNSYLVANIA POWER COMPANY

OHIO EDISON COMPANY

THE CLEVELAND ELECTRIC ILLUMINATING COMPANY

THE TOLEDO EDISON COMPANY

FIRSTENERGY NUCLEAR OPERATING COMPANY

BEAVER VALLEY POWER STATION, UNIT NOS. 1 AND 2

DOCKET NOS. 50-334 AND 50-412

1.0 INTRODUCTION

By letter dated May 27, 1999, the Duquesne Light Company (DLC) submitted a request for changes to the Beaver Valley Power Station, Unit Nos. 1 and 2 (BVPS-1 and 2), Technical Specifications (TSs). The requested changes would relocate the seismic monitoring instrumentation requirements contained in TS 3/4.3.3.3 to the Licensing Requirements Manual (LRM) based on the guidance provided in Generic Letter (GL) 95-10, "Relocation of Selected Technical Specifications Requirements Related to Instrumentation." The Bases section for Specification 3/4.3.3.3 will also be relocated to the LRM. The appropriate Index pages, Table Index page (Unit No. 1 only), TS pages and Bases pages will be revised to reflect the removal of the seismic monitoring instrumentation specification from the TSs. An additional specification page will be added to reflect that TS Number 3/4.3.3.4 is not used. This additional page will also denote the number of the following page. The Bases section will also be modified to denote that TS Number 3/4.3.3.4 is not used.

On the date of the May 27, 1999, letter, DLC was the licensed operator for BVPS-1 and BVPS-2. On December 3, 1999, DLC's ownership interests in both BVPS-1 and BVPS-2 were transferred to the Pennsylvania Power Company, and DLC's operating authority for BVPS-1 and BVPS-2 was transferred to FirstEnergy Nuclear Operating Company (FENOC). By letter dated December 13, 1999, FENOC requested that the Nuclear Regulatory Commission (NRC) continue to review and act upon all requests before the Commission which had been submitted by DLC.

2.0 BACKGROUND

Per GL 95-10, licensees were allowed to request a license amendment to relocate selected instrumentation requirements from their TSs. Licensees that decided to follow GL 95-10 were asked to submit license amendment requests to relocate the affected TSs.

GL 95-10 addresses the relocation of selected TS requirements related to instrumentation as a result of applying the 10 CFR 50.36 criteria (specifies criteria that TS sections should address). The NRC staff determined that, in accordance with the 10 CFR 50.36 criteria, several specifications did not warrant inclusion in the TSs. The staff also concluded that the instrumentation addressed by these specifications is not related to dominant contributors to plant risk. The TS for seismic monitoring instrumentation represented one of several entities which were recommended for relocation to licensee-controlled documents.

3.0 EVALUATION

The relocation of the seismic requirements to the LRM may be viewed as a two-fold process which calls for: (1) the deletion of the information as found in said seismic TSs from the plant TSs, and (2) the addition of said information to the LRM. In order to carry out the relocation of the seismic requirements, the licensee submitted proposed revisions to the plant TSs.

3.1. The licensee proposes that: (1) the Seismic Monitoring Instrumentation requirements, TS 3/4.3.3.3, for BVPS-1 and 2, and (2) the Bases section for TS 3/4.3.3.3, for BVPS-1 and 2, be relocated from the plant TSs to the LRM.

Unit 1

The licensee proposes that TS 3/4.3.3.3 (located on page 3/4 3-38) and its associated tables, Table 3.3-7 (located on page 3/4 3-39), "Seismic Monitoring Instrumentation," and Table 4.3-4 (located on page 3/4 3-40), "Seismic Monitoring Instrumentation Surveillance Requirements," be relocated from the TSs to the LRM.

The licensee proposes that the Bases section (located on page B3/4 3-2) which corresponds to TS 3/4.3.3.3 be relocated from the TSs to the LRM.

Unit 2

The licensee proposes that TS 3/4.3.3.3 (located on page 3/4 3-46) and its associated tables, Table 3.3-7 (located on page 3/4 3-37), "Seismic Monitoring Instrumentation," and Table 4.3-4 (located on page 3/4 3-48), "Seismic Monitoring Instrumentation Surveillance Requirements," be relocated from the TSs to the LRM.

The licensee proposes that the Bases section (located on page B3/4 3-11) which corresponds to TS 3/4.3.3.3 be relocated from the TSs to the LRM.

Seismic monitoring instrumentation is not used to detect degradation of the reactor coolant pressure boundary; it is not a process variable, design feature, or operating restriction that is an initial condition of a design basis accident or transient analysis; it performs no protective functions assumed in a safety analysis to mitigate a design basis accident or transient; and there is no operating experience or probabilistic risk assessment which shows it to be significant to public health and safety. Therefore, seismic monitoring instrumentation does not meet any of the four criteria specified in 10 CFR 50.36(c)(2)(ii) regarding items for which a limiting condition for operation must be established. Hence, it is acceptable to relocate these TS requirements to the LRM. Such a relocation of these requirements is consistent with the guidance of GL 95-10.

3.2 The following revisions proposed by the licensee are editorial in nature and correspond to the proposed relocation of the seismic monitoring instrumentation requirements from the TSs to the LRM.

3.2.1 The licensee requests that the appropriate Index pages, TS pages, and Bases pages, for BVPS-1 and 2, be revised to reflect the removal of the seismic monitoring instrumentation requirements from the TSs.

Unit 1

Index Pages

Index pages IV, X, and XVI are revised to delete references to TS 3/4.3.3.3 and the associated Tables 3.3-7 and 4.3-4.

Technical Specification Pages

TS page 3/4 3-38 is revised to state, "3.4.3.3.3 This Specification number is not used."

TS page 3/4 3-39 is revised not only to remove Table 3.3-7 but also to state that TS number 3/4.3.3.4 is not used, and to indicate that the next page is 3/4 3-44.

TS page 3/4 3-40 is deleted.

Bases Pages

Bases page B 3/4 3-2 is revised to state: "3.4.3.3.3 This Specification number is not used."

Unit 2

Index Pages

Index pages IV and X are revised to delete references to TS 3/4.3.3.3

Technical Specification Pages

TS page 3/4 3-46 is revised to state: "3.4.3.3.3 This Specification number is not used."

TS page 3/4 3-47 is revised not only to remove Table 3.3-7 but also to state that TS number 3/4.3.3.4 is not used, and to indicate that the next page is 3/4 3-52.

TS page 3/4 3-48 is deleted.

Bases Pages

Bases page B 3/4 3-11 is revised to state: "3.4.3.3.3 This Specification number is not used."

The licensee's proposed TS modifications which will relocate the seismic monitoring instrumentation requirements to the LRM as well as the corresponding editorial changes, as detailed above, have been found acceptable by the NRC staff.

4.0 SUMMARY

The NRC staff reviewed the licensee's submittal which addresses relocation of the seismic monitoring instrumentation requirements contained in TS 3/4.3.3.3, and editorial changes, per GL 95-10. The staff also reviewed the request pertaining to TS 3/4.3.3.4. Based on its review, the NRC staff finds the proposed TS changes acceptable.

5.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Pennsylvania State official was notified of the proposed issuance of the amendments. The State official had no comments.

6.0 ENVIRONMENTAL CONSIDERATION

The amendments change requirements with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 and change surveillance requirements. The NRC staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendments involve no significant hazards consideration, and there has been no public comment on such finding (64 FR 35203). Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

7.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributors: D. Spaulding
D. Collins

Date: February 28, 2000