

March 6, 2000

Mr. Robert P. Powers, Senior Vice President
Indiana Michigan Power Company
Nuclear Generation Group
500 Circle Drive
Buchanan, MI 49107

SUBJECT: DONALD C. COOK NUCLEAR PLANT, UNITS 1 AND 2 - SUPPLEMENTAL
RESPONSE TO NRC BULLETIN 88-04, "POTENTIAL SAFETY-RELATED PUMP
LOSS" (TAC NOS. MA5409 AND MA5410)

Dear Mr. Powers:

By letter dated December 3, 1998, you provided a supplemental response to NRC Bulletin 88-04, "Potential Safety-Related Pump Loss." During a 1997 review of the residual heat removal (RHR) system, your staff identified a potential for operating the weaker of two RHR pumps below its minimum flow requirement. The condition was calculated to occur during the transfer to cold leg recirculation following a loss-of-coolant accident and exists when the difference in degradation between the two RHR pumps exceeds 8%.

Your corrective action to preclude a weaker RHR pump from operating below its minimum flow requirement is to monitor the degradation and provide assurance that the difference between the two RHR pumps does not exceed 8%. The in-service testing program for the RHR pumps has been augmented to include the additional requirement. Currently, the pump test data indicate that the differential degradation between the two RHR pumps in each unit is less than 4%, so the weaker pump is capable of operating above its minimum flow requirement.

This will acknowledge receipt of your December 3, 1998, letter providing a supplemental response to NRC Bulletin 88-04. The staff has no further concerns at this time. Your corrective actions may be examined during future NRC inspections.

Please contact me at (301) 415-1345 if you have any questions.

Sincerely,

/RA/

John F. Stang, Senior Project Manager, Section 1
Project Directorate III
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket Nos. 50-315 and 50-316

cc: See next page

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